



FEASIBILITY STUDY OF SMALL SODIUM COOLED FAST EACTORS

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Previous Studies

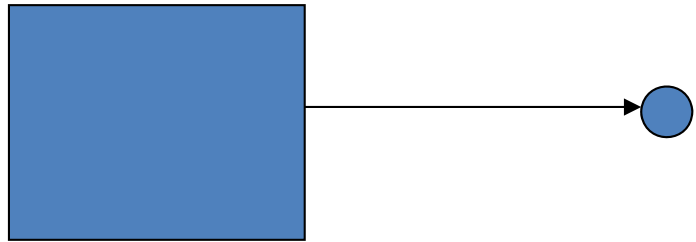
JAEA has studied about small sodium cooled reactors in the Feasibility Study.

-Modular Reactor for base load power source

-Remote Place Power Source (<50MWe)

-Multi-purpose Reactor (Hydrogen Production)

Today's Topic



Large Scale Reactor

- High economic performance with scale effect
- High R&D risk

-IRIS, IMR, small BWR

M.D. Carelli, et al., ICAPP'05, No. 5059, Seoul, Korea, May, (2005).

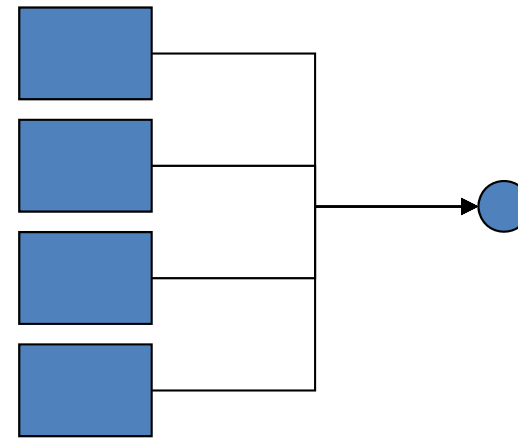
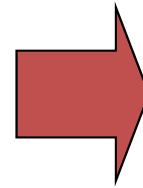
T. Kanagawa, et al., ICAPP'05, No. 5204, Seoul, Korea, May, (2005).

H. Heki, et al., ICAPP'05, No. 5174, Seoul, Korea, May, (2005).

-IFR

Y.I. Chang, "The Integrated Fast Reactor", Nucl. Tech., Vol. 88, (1989).

Sector of Fast Reactor and Advanced Reactor Research and Development



Modular Reactor

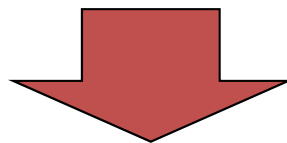
- High economic performance with standardization & Common use between units
- Low R&D risk

Purpose

Energy Resource Sustainability,
Low Environmental Burden with
MA transmutation

Proposal of a commercialize **fast reactor** with low R&D risk
(major requirement)

- Economical competitiveness with large scale reactors
- Demonstration of reactor technologies
- Demonstration of **economical performance** including a whole fuel cycle system



Important for
Commercialization

- Further cost reduction of modular fast reactor
- Evaluation of construction cost of a first kind of plant (FOAK)
(demonstration plant) considering a whole fuel cycle system

Major Design Conditions

-Electric Output: 300MW-electric

Low construction cost
in the small fuel cycle
facility

-Fuel Type: U-TRU-Zr

MA transmutation

-Reactor Type: Loop Type

Potential of Cost Reduction

-Spent Fuel Storage: In-vessel Storage (IVS)

Simple ex-vessel fuel
handling system

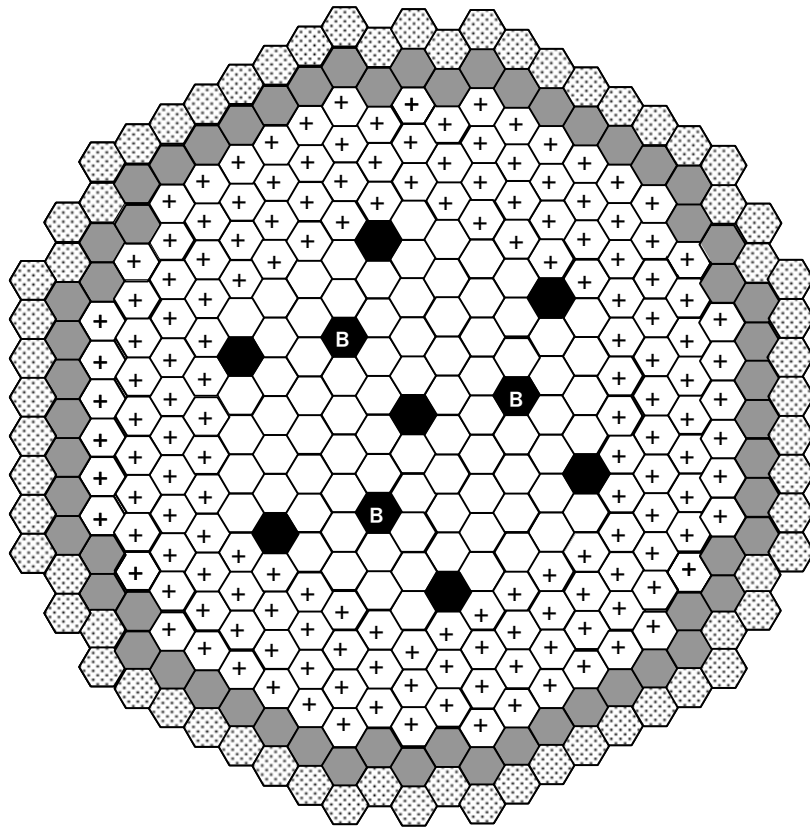
-Main Cooling System: One-Loop Main Cooling System







Cost Reduction

-Main Pump: Electro Magnetic Pump (EMP)

Technology for one-loop
main cooling system

Single Pu enrichment plural Zr content Regions Core



	Inner core fuel S/A	81
	Outer core fuel S/A	162
	S.S. shielding	60
	Zr-H shielding	66
	Primary control rod	7
	Backup control rod	3

-Output: 714MW
(300MW-electric)

-Operation Cycle:
2years (4batches)

-Burnup: 80GWd/t

-Outlet Temp.:
550deg-C

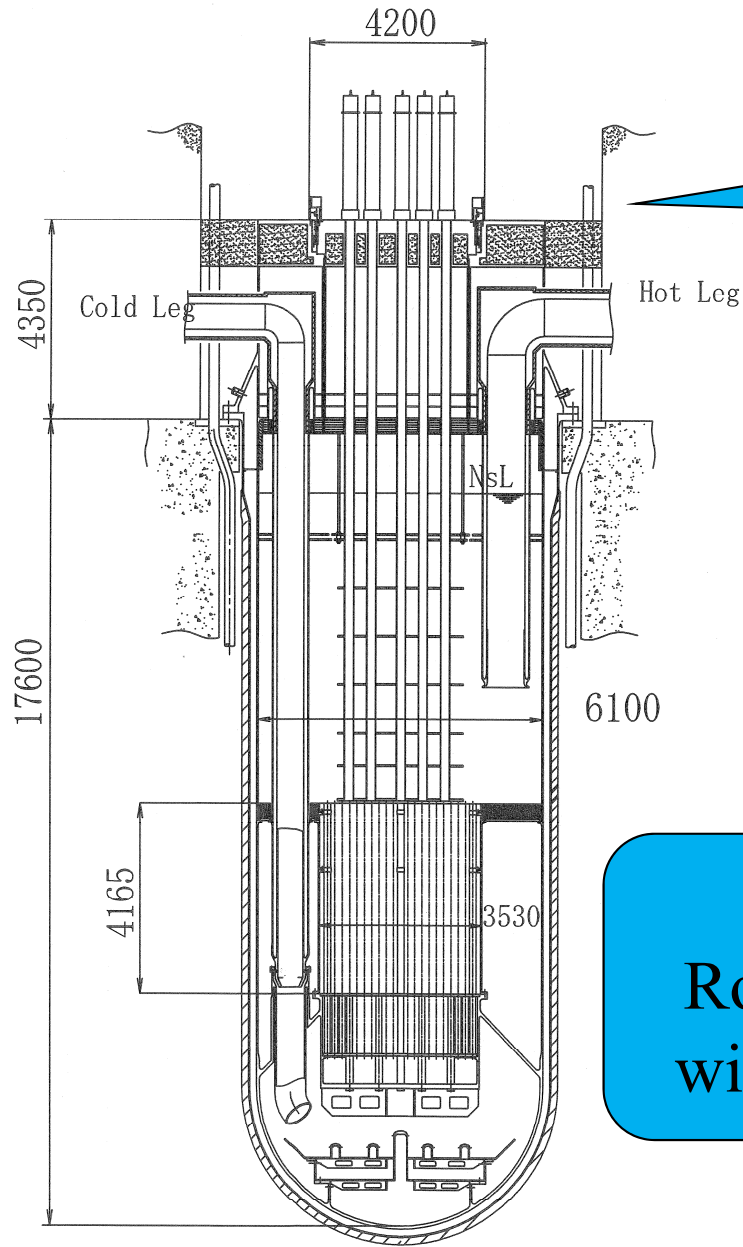
Core Diameter:2.63m Pin Diameter:8.5mm
Core Height:1m S/A pitch:157.2mm

S. Sugino, T. Ogawa, Y. Okano and T. Mizuno, GLOBAL 2005, No. 399, Oct., (2005).

T. Mizuno, T. Ogawa, K. Sugino and M. Naganuma, ICAPP'05, No.5195, Seoul, Korea,
May, (2005).

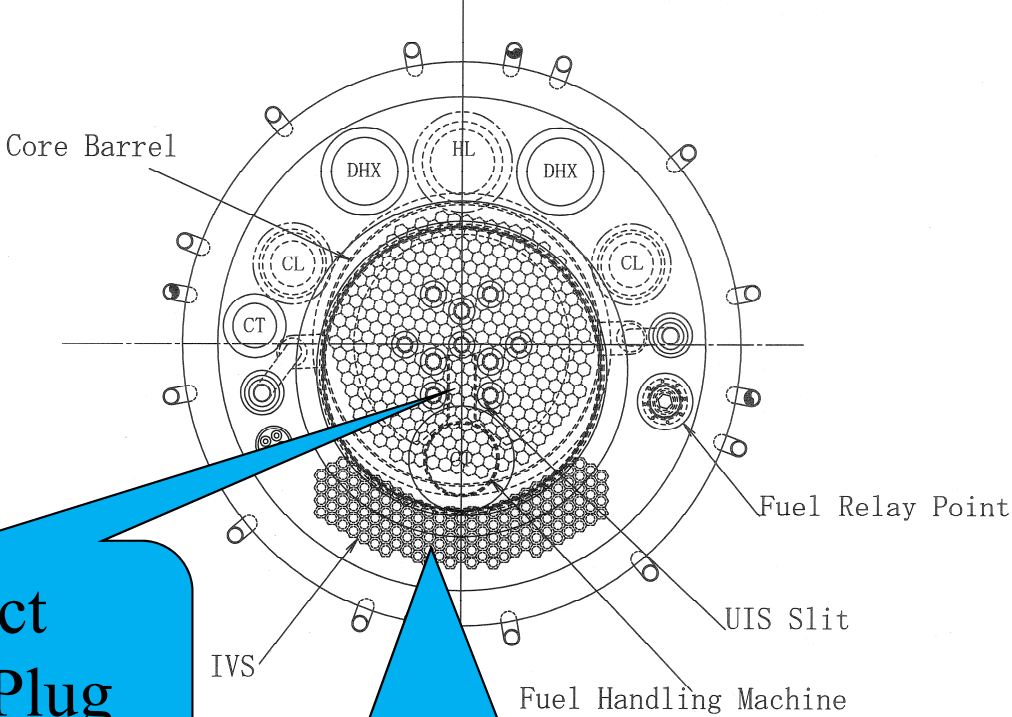
Sector of Fast Reactor and Advanced Reactor Research and Development

Reactor Vessel



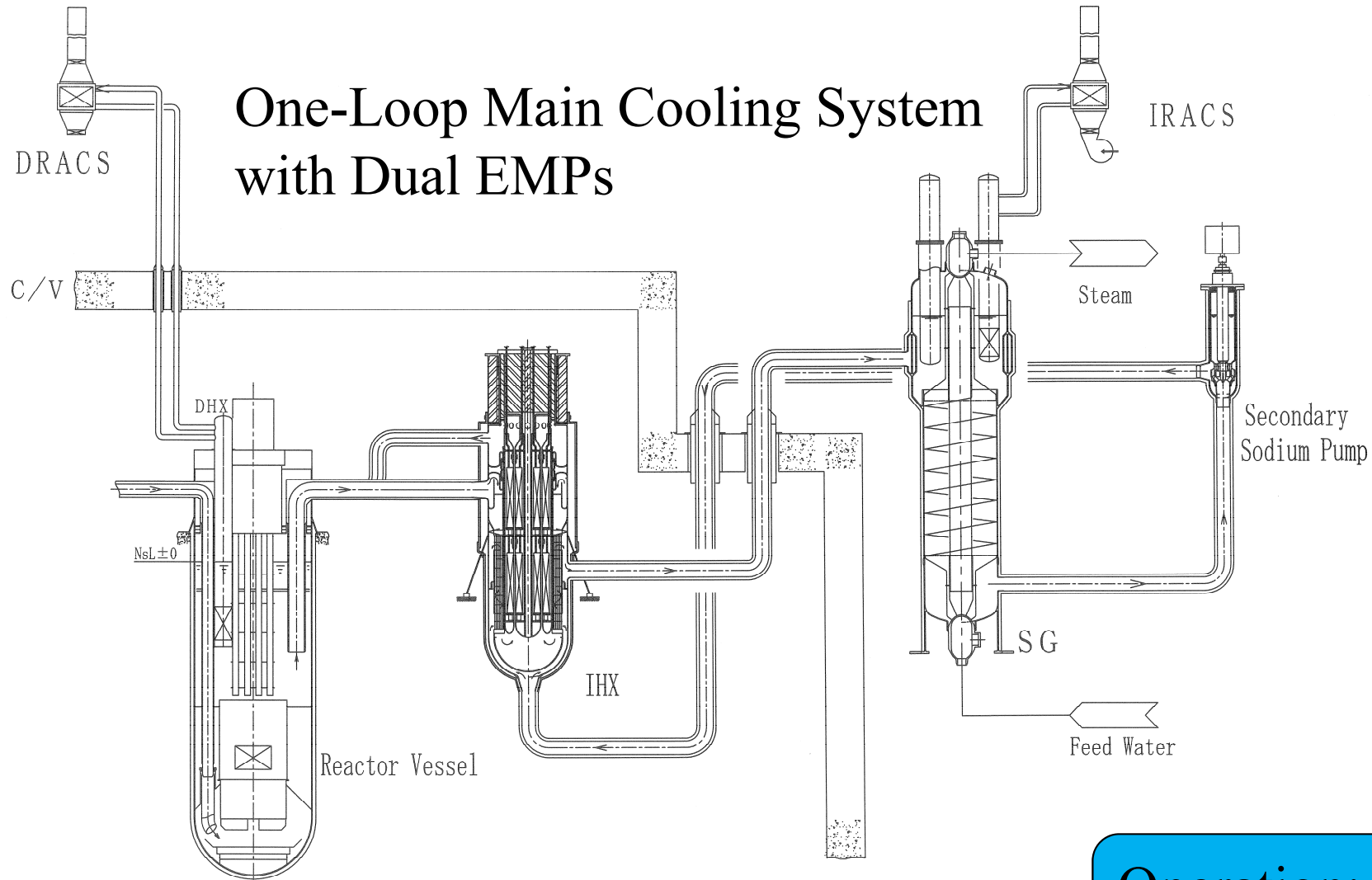
L shape piping with high chromium steel*

Compact Rotating Plug with Slit UIS*



IVS with capacity for 4 years storage

*: Same technologies from JSFR



Annular Linear Induction Pump Experience

1m³/min: A. Oto. et al, Nucl. Tech. Vol. 110, p159, (1995)

44m³/min: W. Kwant et al., ICON-5, No. 2553, (1997)

160m³/min H. Ota, et al, J. of Nucl. Sci. and Tech., vol. 41, No. 4, p511-523, (2004)

Operation: 2550h
Efficiency: 40%

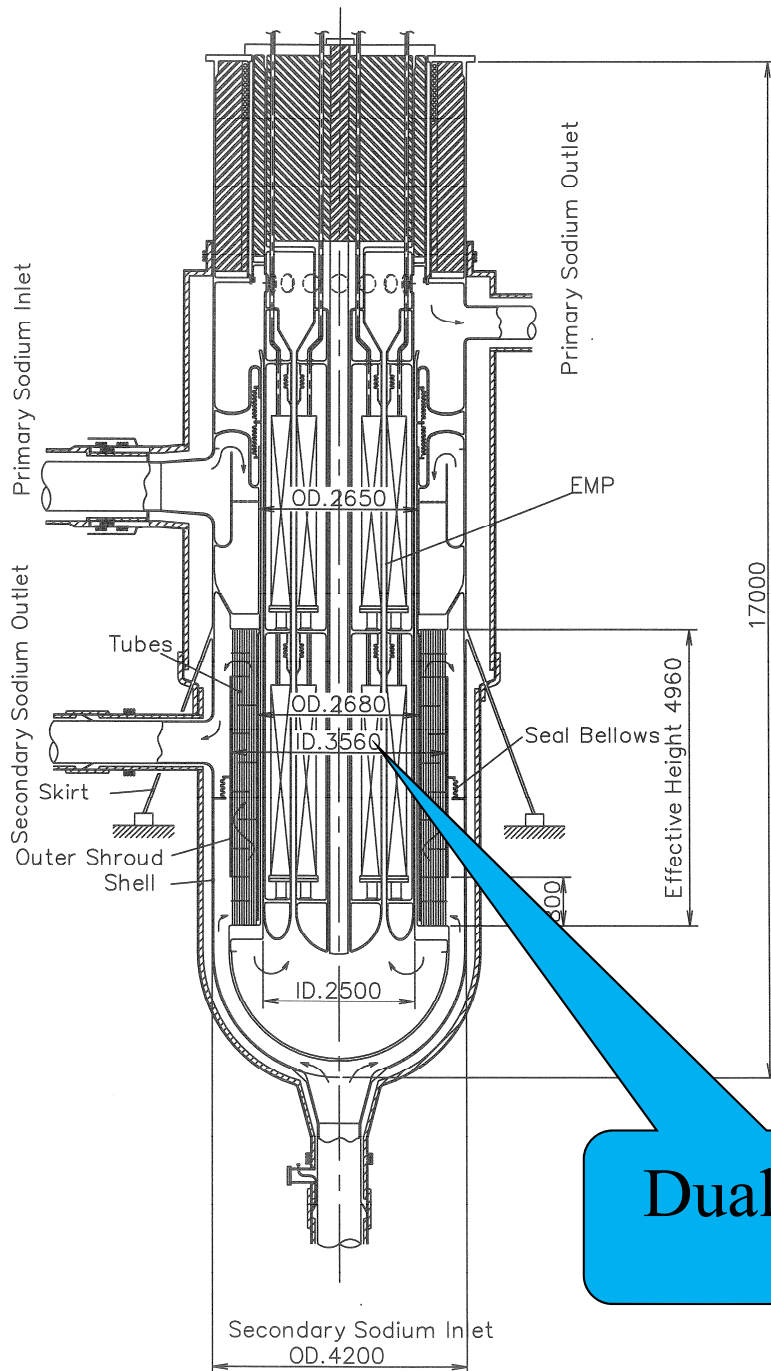
IHX with Internal EMP

Internal EMP

Items	Value
Temperature	395deg-C
Flow Rate	255m ³ /min
Pump Head	0.4MPa

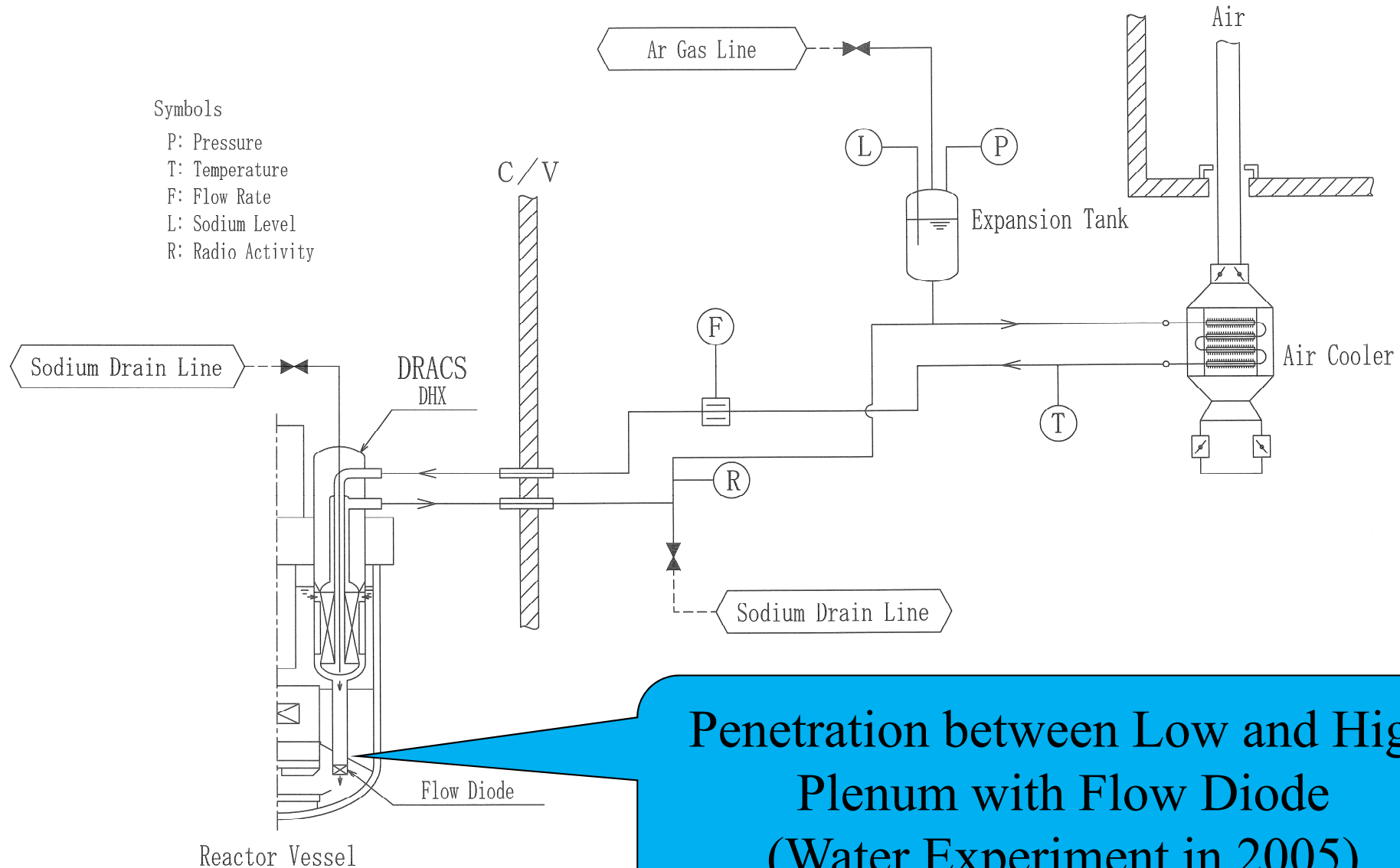
IHX

Items	Value
Capacity	714MW
Tube Outer Diameter	25.4mm
Tube Thickness	1.1mm
Tube Length	5m
Tube Quantity	4412
Tube Arrangement	Triangle
Tube Pitch	32mm
Heat Transfer Area	1700m ²
Material	12Cr

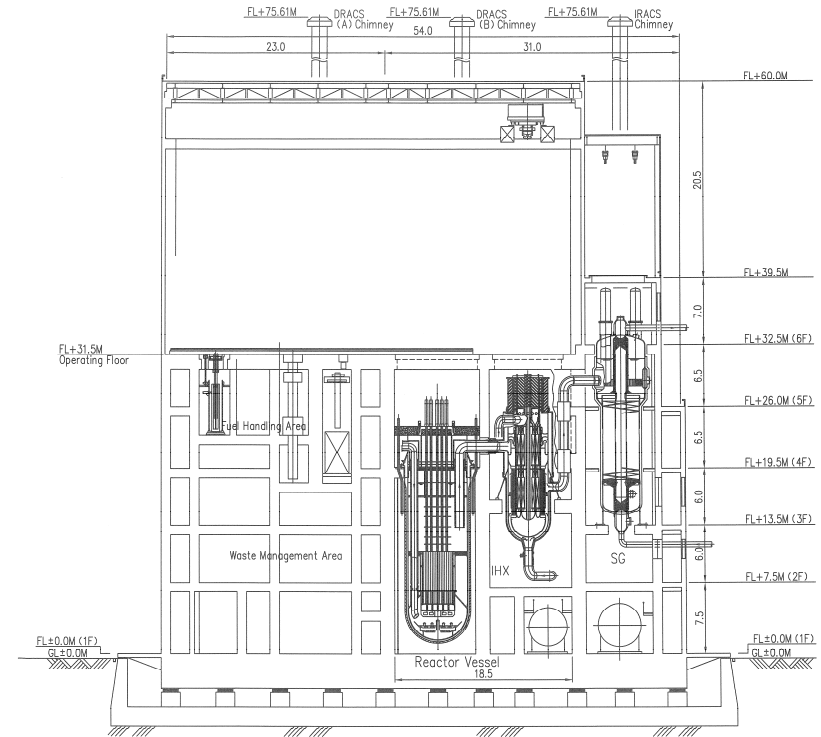
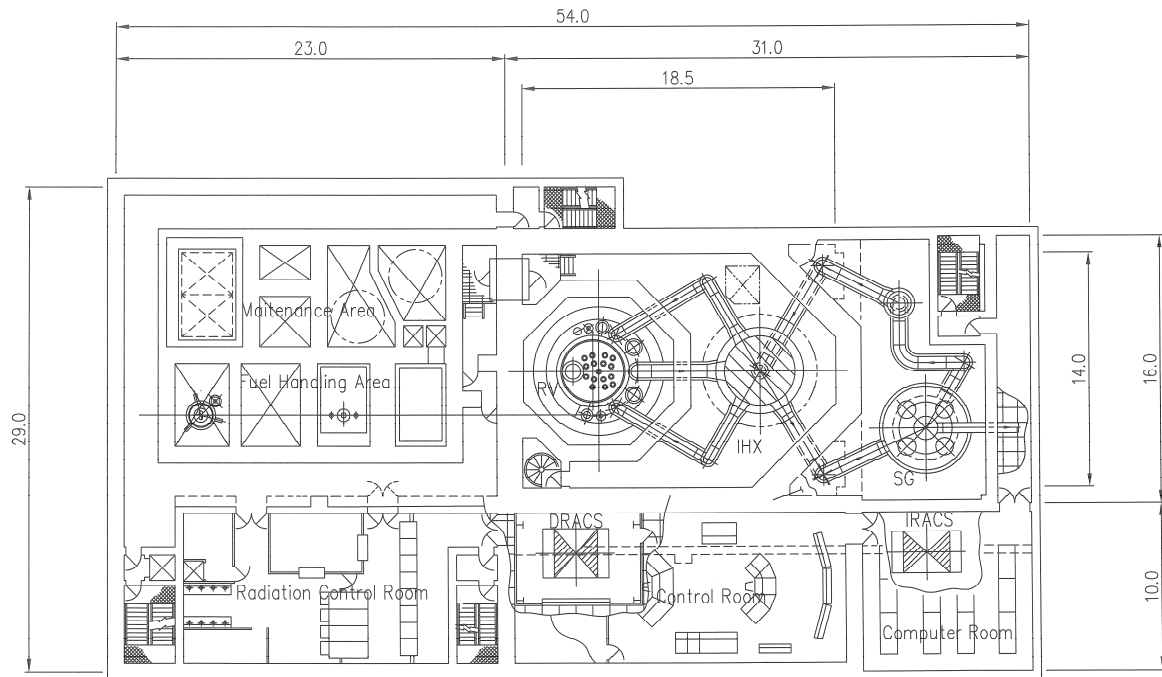


Dual EMPs

Two DRACS and one IRACS with Natural Convection



Penetration between Low and High Plenum with Flow Diode (Water Experiment in 2005)

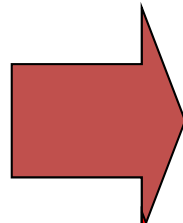


Total Volume **65,100m³** for 300MW-electric
 (Reactor Building of MONJU **207,000m³** for 280MW-electric)

Item	Mass (ton)
Reactor Structure	107
IHX	125
Primary Circuit Pump	122
Primary Circuit Piping	72
SG	276
Secondary Circuit Pump	330
Total	1037

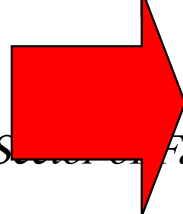
*Except EMP stator 154tonne

FOAK



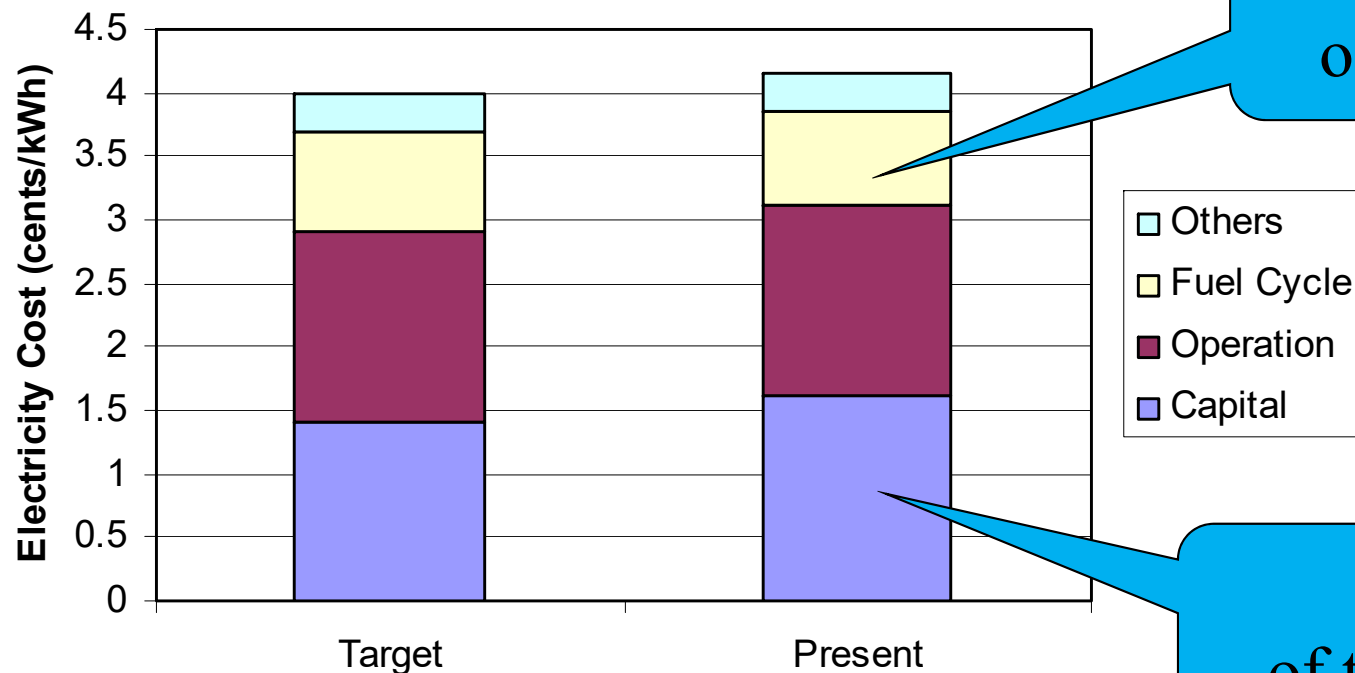
190% of Target (2,000\$/kW, 1USD=100JPY)

NOAK



115%

Supporting Fast Reactor and Advanced Reactor Research and Development



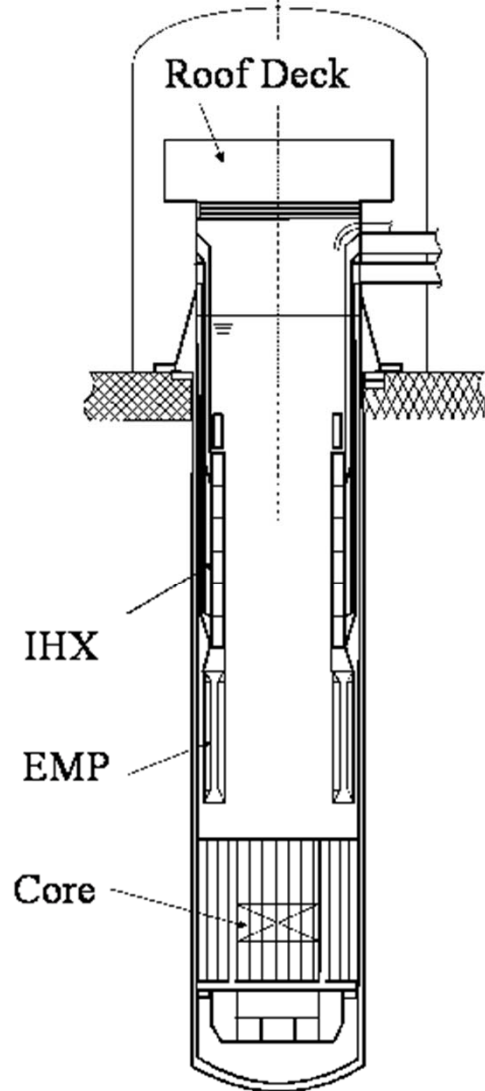
94%
of the target

115%
of the target

Target for Large Scale Reactor
in Future Japan 4cents/kWh

300MWe SFR-SMR has a potential for economic competitiveness versus large scale plant.

JNC Tank Type SMFR

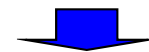


Reactor Vessel

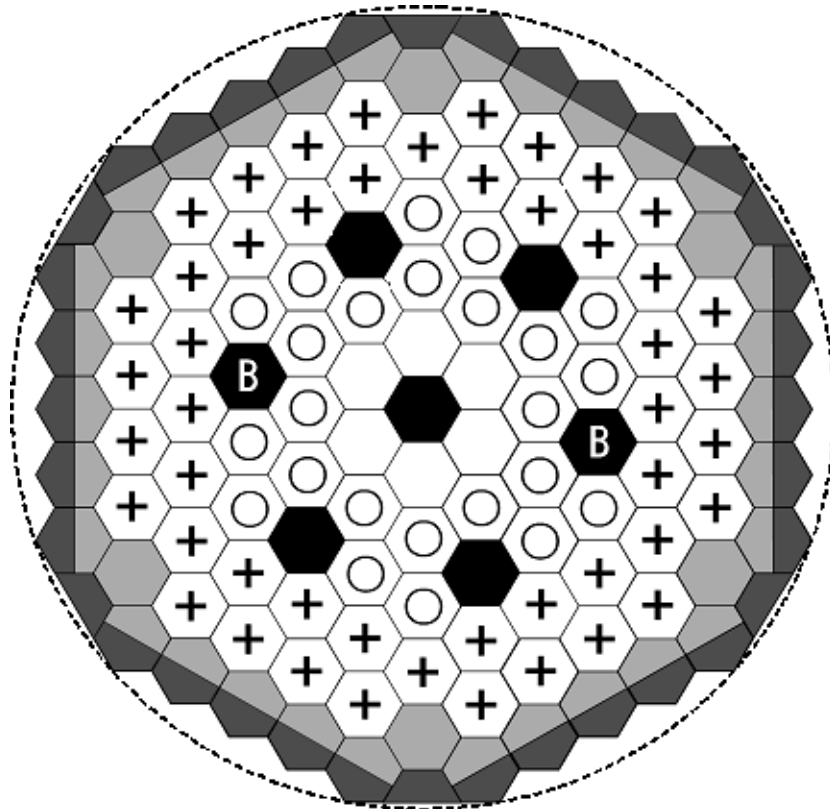
-JNC has studied about small sodium cooled Reactors since FY2001.

Design Condition in the present study

- Electric power 50MW
- Core life time 30y
- Without Refueling



Simple Compact Reactor Vessel

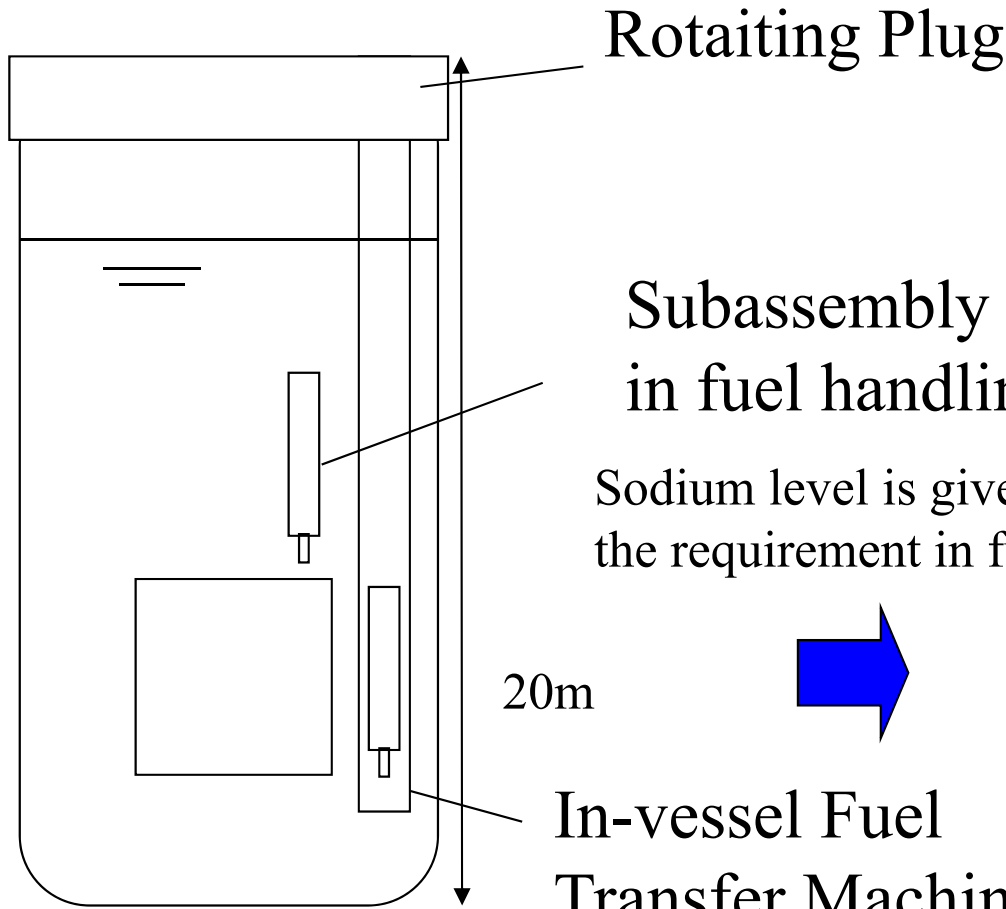


- Thermal output: 120MWt
- Electric output: 50MWe
- Fuel type: U-Pu-Zr ternary metal
- Outlet temperature: 550°C
- Core life time: 30年
- Burnup reactivity: 1.2%Δk/kk'
- Core height: 1.18m
- Core equivalent diameter: 1.82m
- Average discharge burnup: 74GW/dt

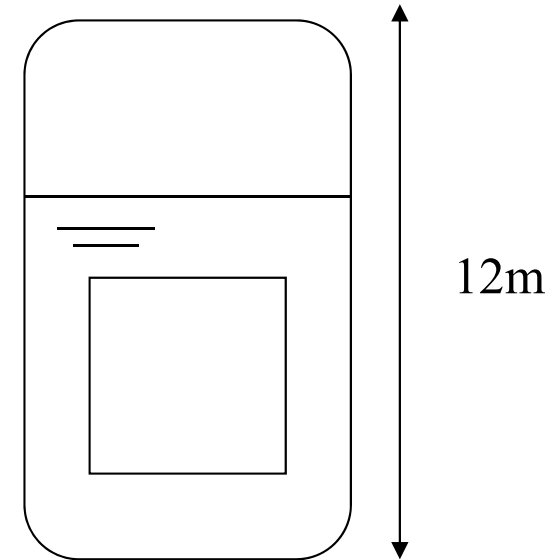
	Inner	Middle	Outer
Zr content	10%	10%	6%
Smear density	70%	79%	85%

Simplified Compact Reactor Vessel

Ordinary Loop Type



Loop Type Without Refueling

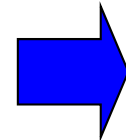


Rotating Plug

Subassembly
in fuel handling

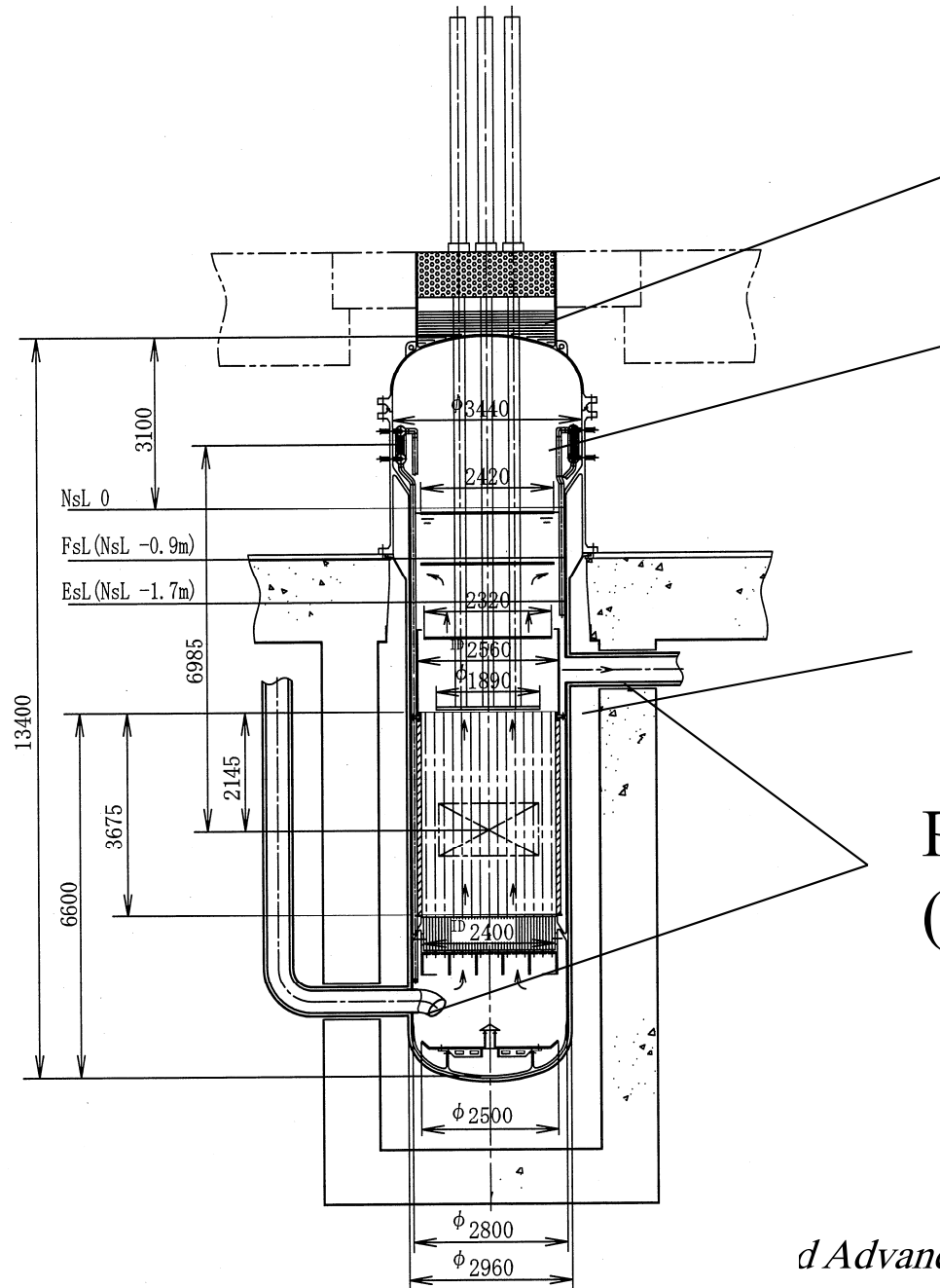
Sodium level is given by
the requirement in fuel handling

20m



In-vessel Fuel
Transfer Machine

- Sodium level is restricted only in operating condition
- Rotating plug and in-vessel Fuel Transfer Machine can be removed



Simplified upper structure without rotating plug

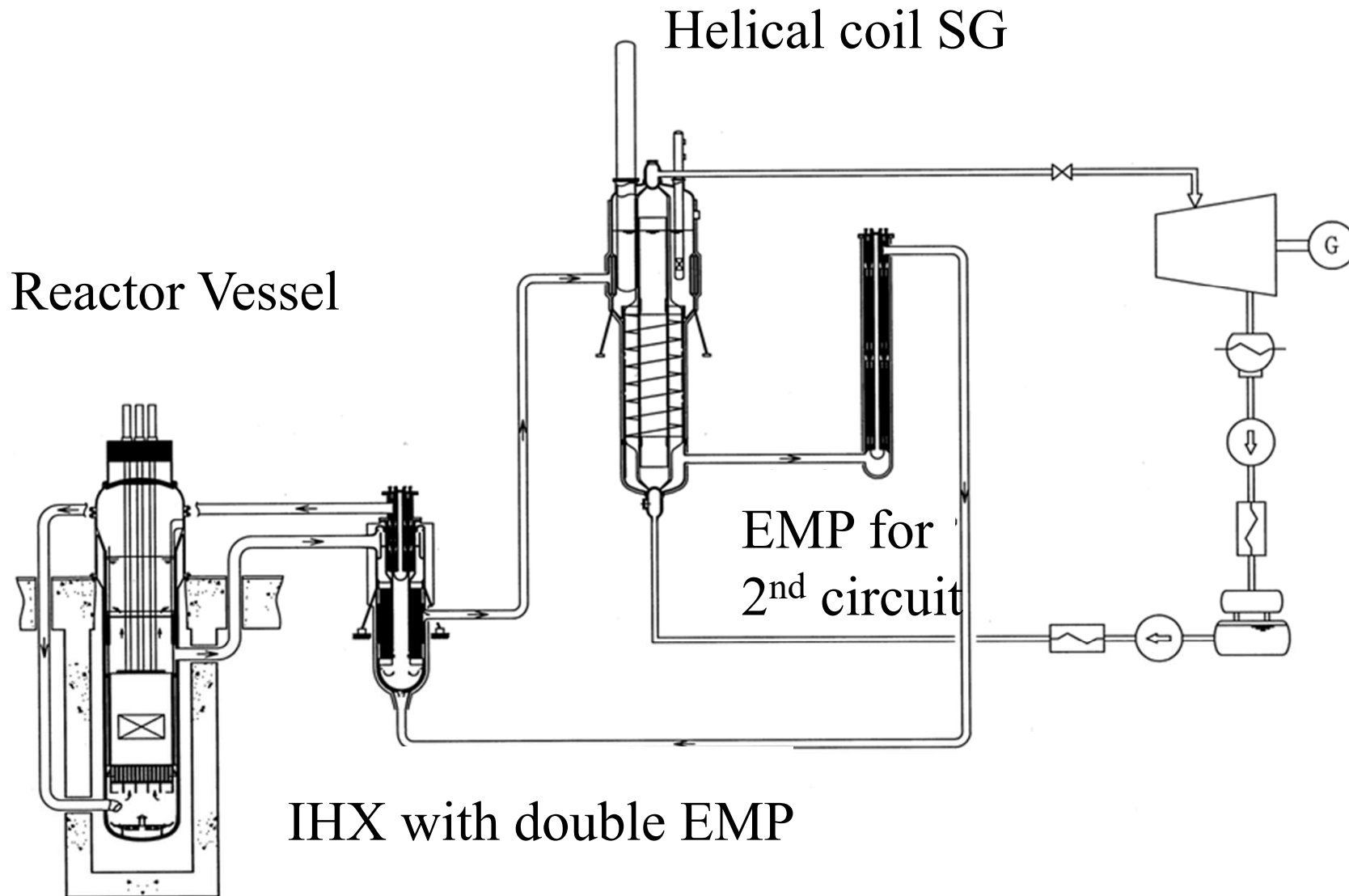
DHX at cover gas (reduction of RV height)

Direct subassembly support by RV (removing core barrel)

Piping with nozzles (reduction of RV diameter)

{ Diameter 2.8m
 Height 13.4m

Total system (1 loop)



Mass Evaluation

Item	Previous Pool type design	Loop design	(ton)
RV	70	38.5	
In-vessel Structures	65	28	
Roof Deck	40	18	
GV	40	20.5	
IHX	90	41.5	
Primary Circuit Pump	40	13	
Primary Circuit Piping	0	10	
SG	90	97	
Secondary Circuit Pump	35	35	
Secondary Circuit Pump	7	7.1	
Total	477	308.6	

Simplified Loop type design has a advantage in construction cost.



Present Study in JAEA

Innovation objectives:

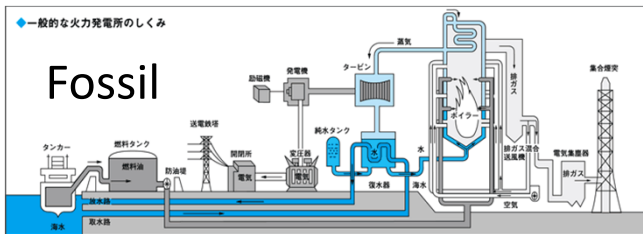
- Co-existence with renewable energy
- 80% CO2 reduction in 2050

Current



Innovative energy system

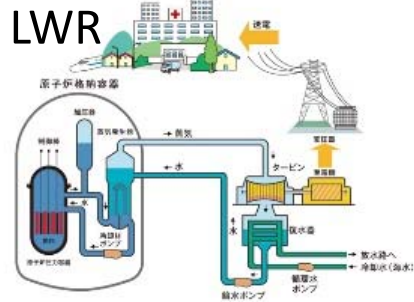
<https://www.fepec.or.jp/index.html>



HTGR

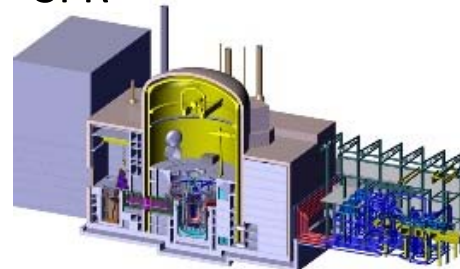


- ✓ No fossil fuel use
- ✓ LWR → SFR/HTGR
- ✓ Load following to VRE



<https://www.enecho.meti.go.jp/>

SFR



1) Significant cost reduction

- ✓ Reduce manufacturing cost
- ✓ Raise fuel performance w/ extended burnup
- ✓ Raising thermal efficiency to 50-60%
- ✓ Standardization and non-nuclearization

2) Fuel cycle synergy

- ✓ SFR + HTGR closed fuel cycle - reduce or eliminate U use
- ✓ Supply sufficient Pu required for zero emission grid
- ✓ MA burning, etc.

3) Zero emission grid

- ✓ Capability to follow variable renewables while operating reactor at baseload
- ✓ Capability to produce hydrogen and heat at low demand/price of electricity
- ✓ Smart grid system integrating requirements of environment, market, safety, maintenance, grid resilience, regulation, etc.

4) Significant nuclear safety improvement

- ✓ Order of magnitude reduction or elimination of risk of core melt, radioactive material release, combustion chemicals

- Set vision goals
- Evaluate R&D needs
- Implement R&D activities

HTGR roles:

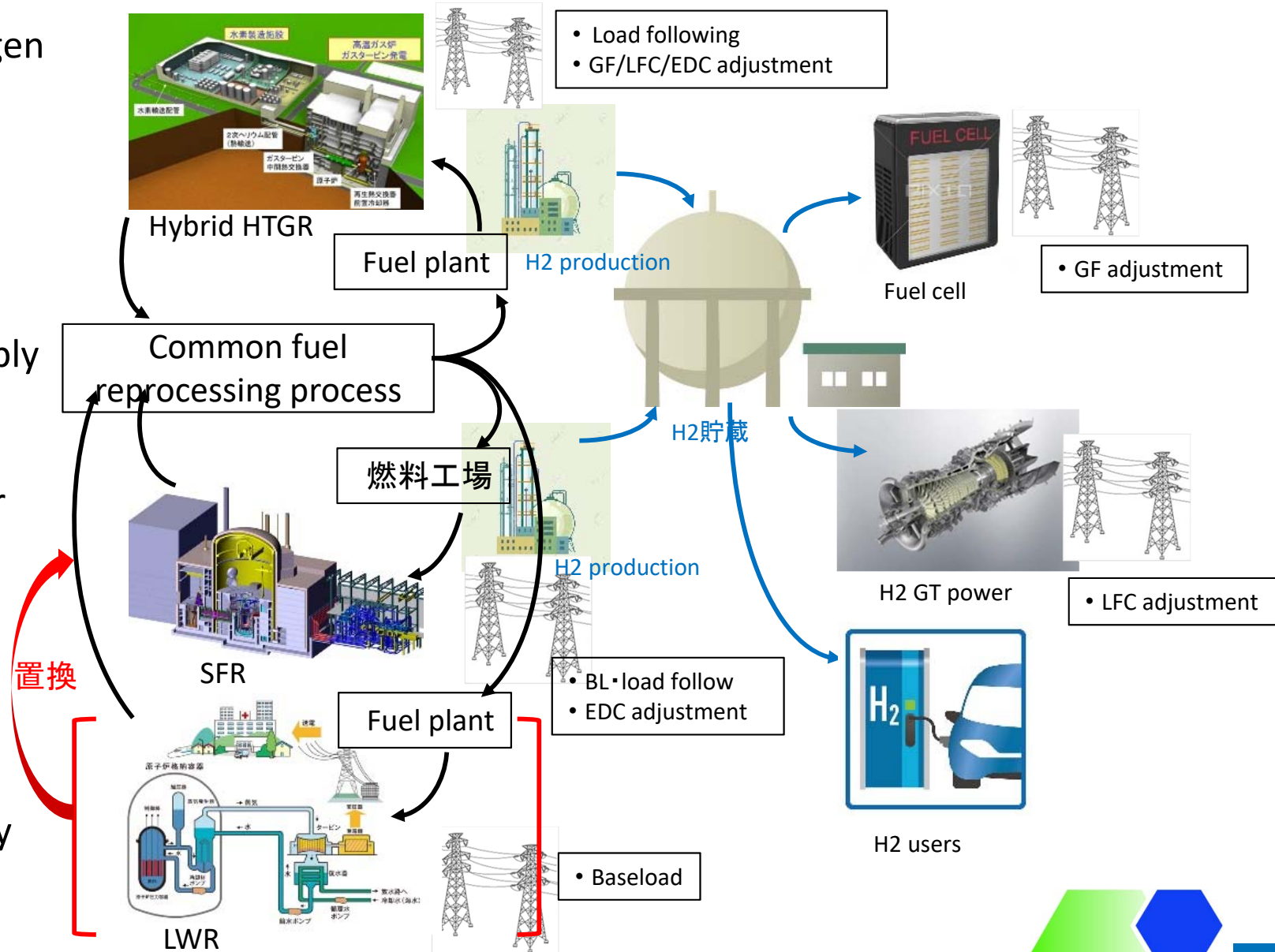
- Process heat and hydrogen supply
- Load following
- GF/LFC/EDC power adjustment

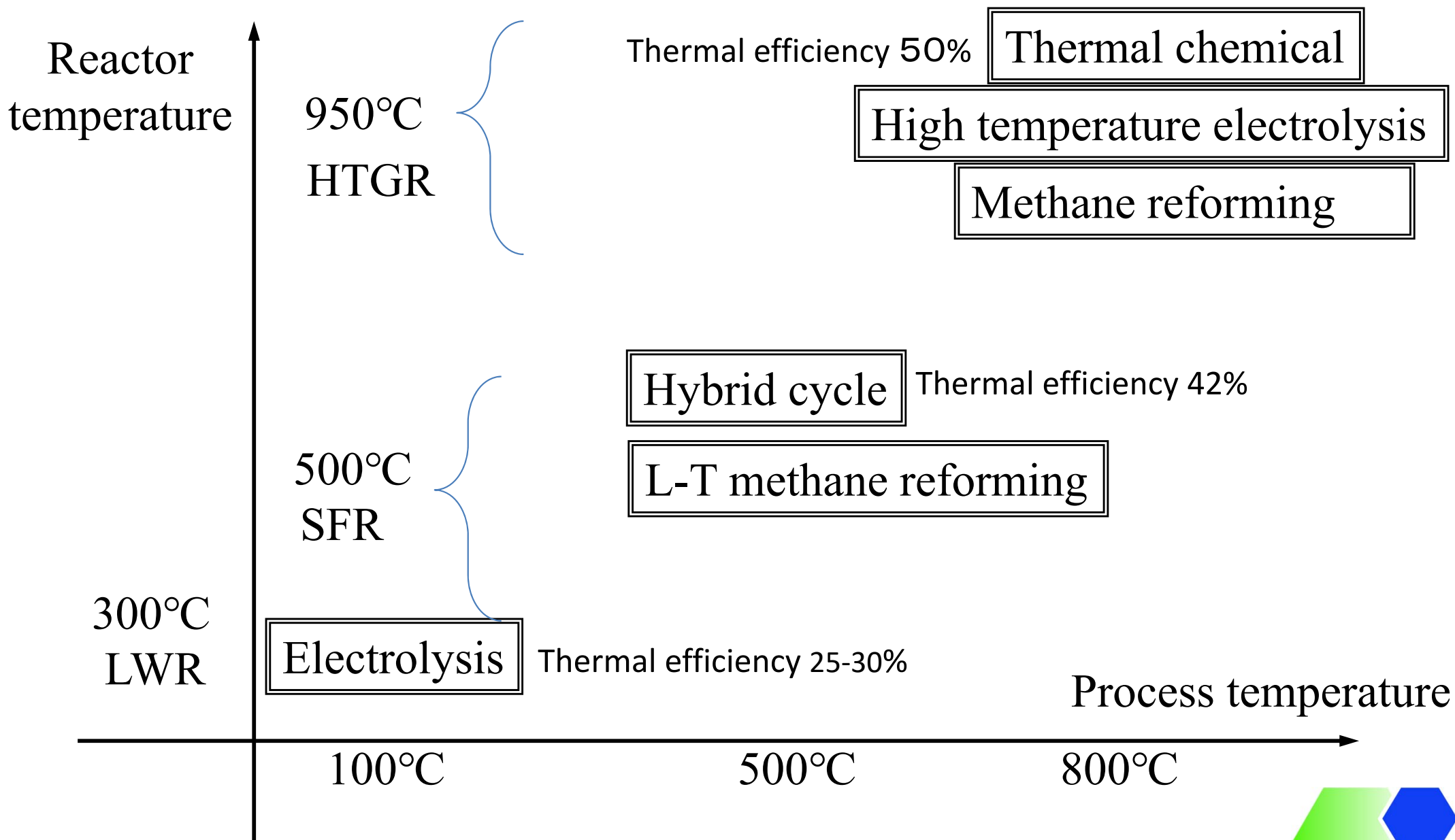
SFR roles:

- Pu burning and fuel supply
- Pu cleanup
- MA burning
- Hydrogen production for electricity storage
- BL and load follow
- EDC power adjustment

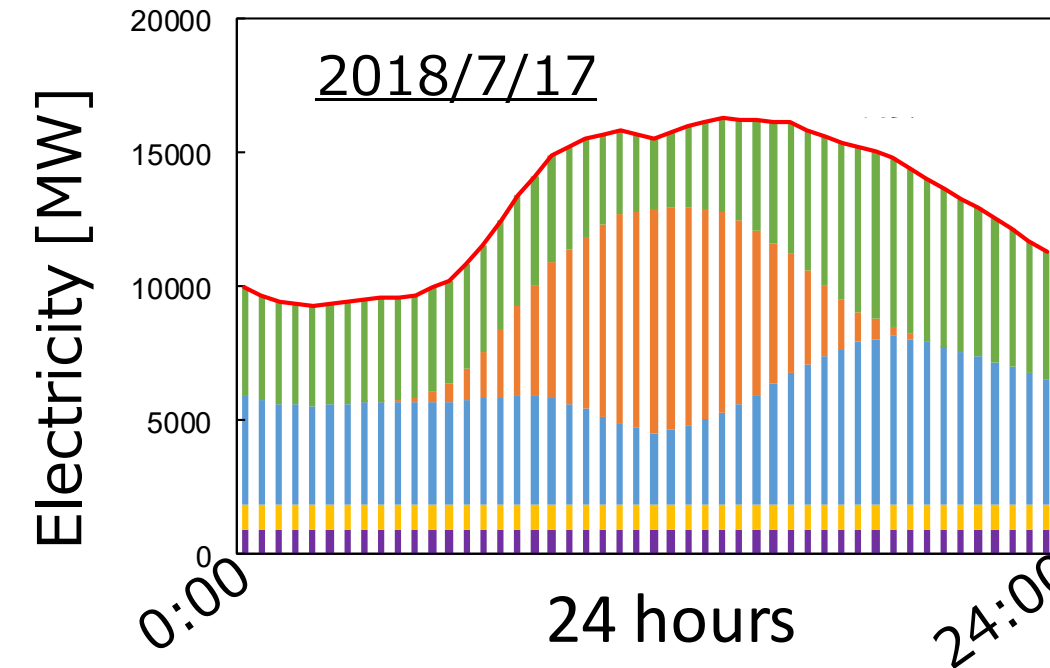
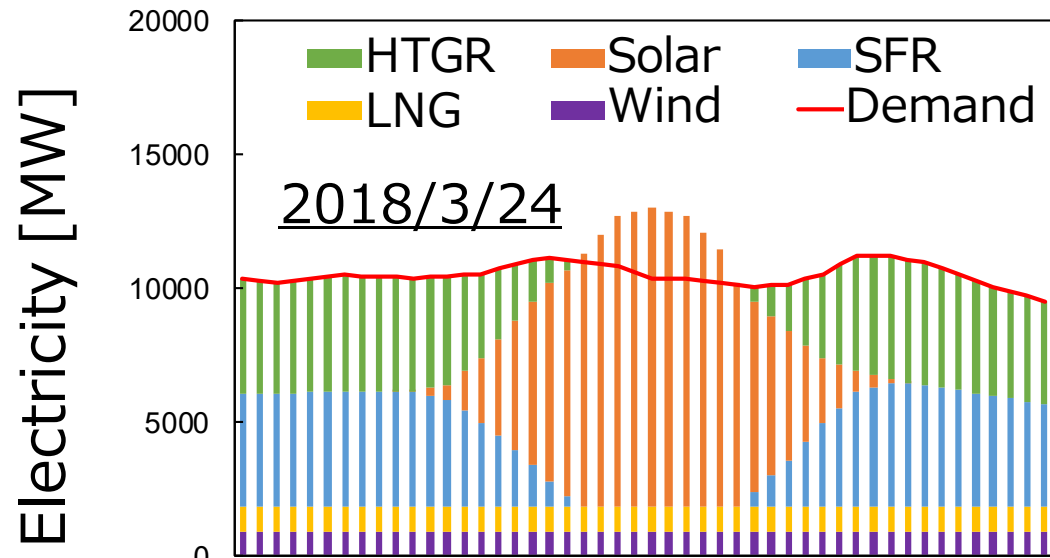
LWR role:

- Baseload (BL) electricity





Japan 9 regional utilities' average in 2018



Sector

- Flexible introduction
 - SFR: Pu management flexibility (Pu burning/breeding, and Pu cleanup from LWRs)
 - HTGR: Flexibility of co-existence with renewable energy
 - SMR: Lower investment cost and lower R&D risks
- Safety enhancement
 - SFR: Lower void coefficient for Pu management core
 - HTGR: High passive safety ability with large heat capacity of the core
- Requirement from the society
 - Hydrogen energy...etc.