

Lithium Vapor Box Divertor: Modeling & Plans

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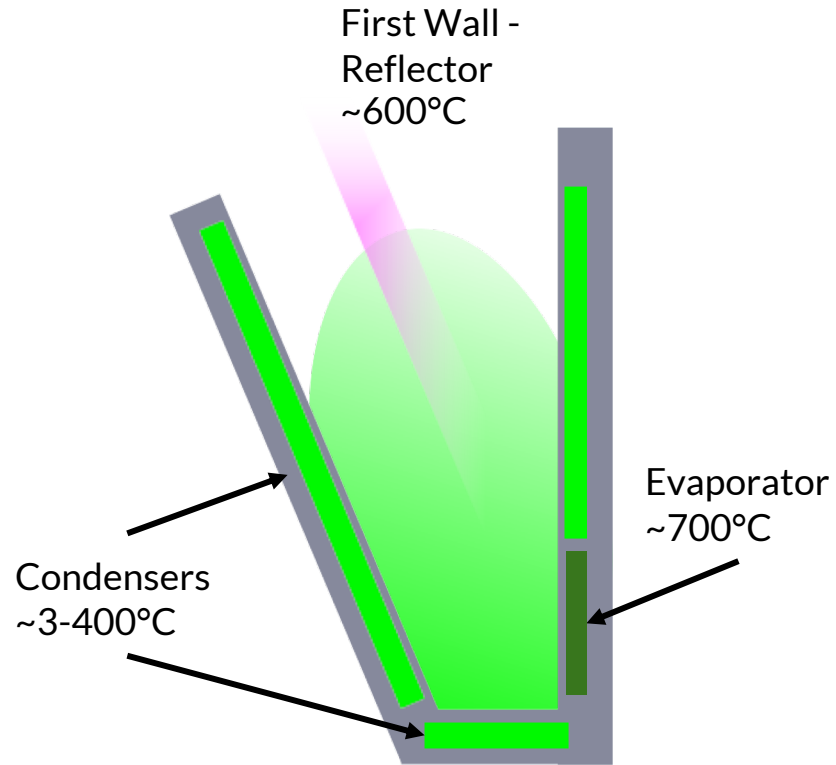
Outline

- Why a Lithium Vapor Box?
- Modeling
 - UEDGE - SPARTA
 - SOLPS - ITER
- Plans
 - Magnum PSI
 - NSTX-U
 - Off-Line R&D



The Lithium Vapor Box Divertor

- Localizes lithium vapor in divertor
- Evaporation close to target, condensation elsewhere in slot.
- Exploring cases without baffles.
- Cannot be achieved with gaseous impurities: pumping is too weak
- Target resilient to reattachment.



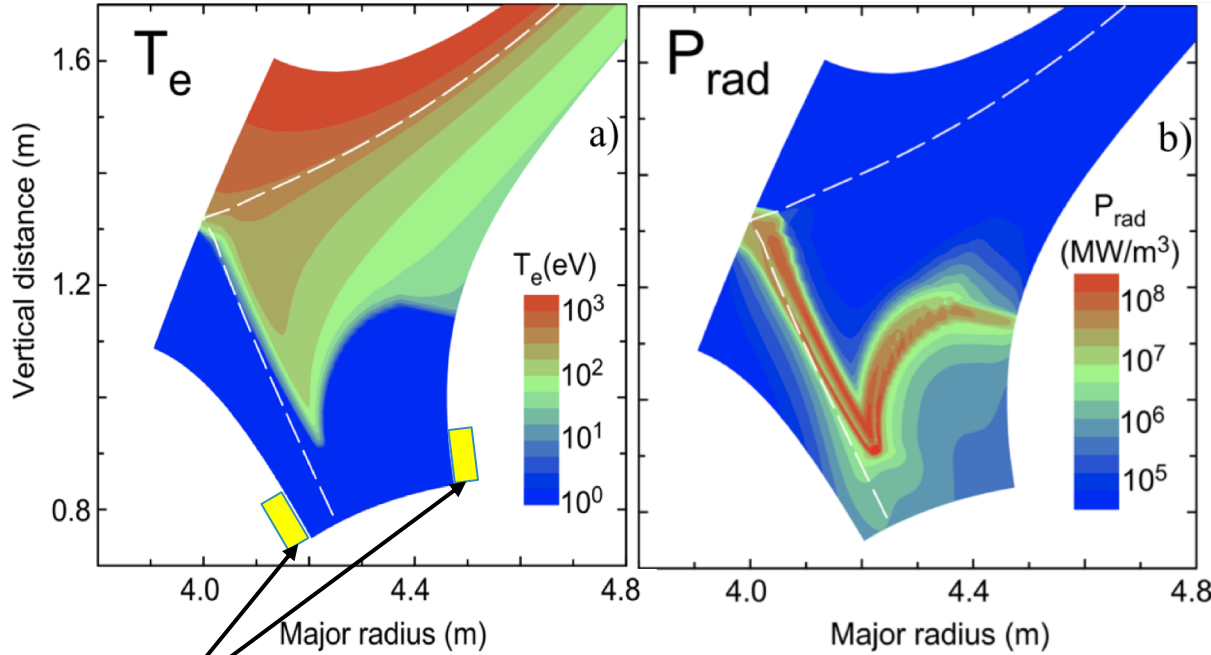
Why a Lithium Vapor Box?

- Gaseous impurities easily lead to X-point MARFEs, degrading pedestal performance (OK in a reactor?).
 - Lithium can be pumped by condensation, like in heat pipe
- Want $P_{\text{sep}} \geq P_{\text{LH}}$ and $P_{\text{target}} \ll P_{\text{LH}}$
 - Need separate control of mantle & SOL radiation
- Lithium has low ionization potential (5.6 eV), well contained by hydrogen isotope recycling (13.6 eV).
 - Nitrogen: 14.5 eV, Neon 21.6 eV



UEDGE Achieves Stable Detachment

- High levels of recombination before hitting the target, temperatures < 0.2 eV
- No baffles required

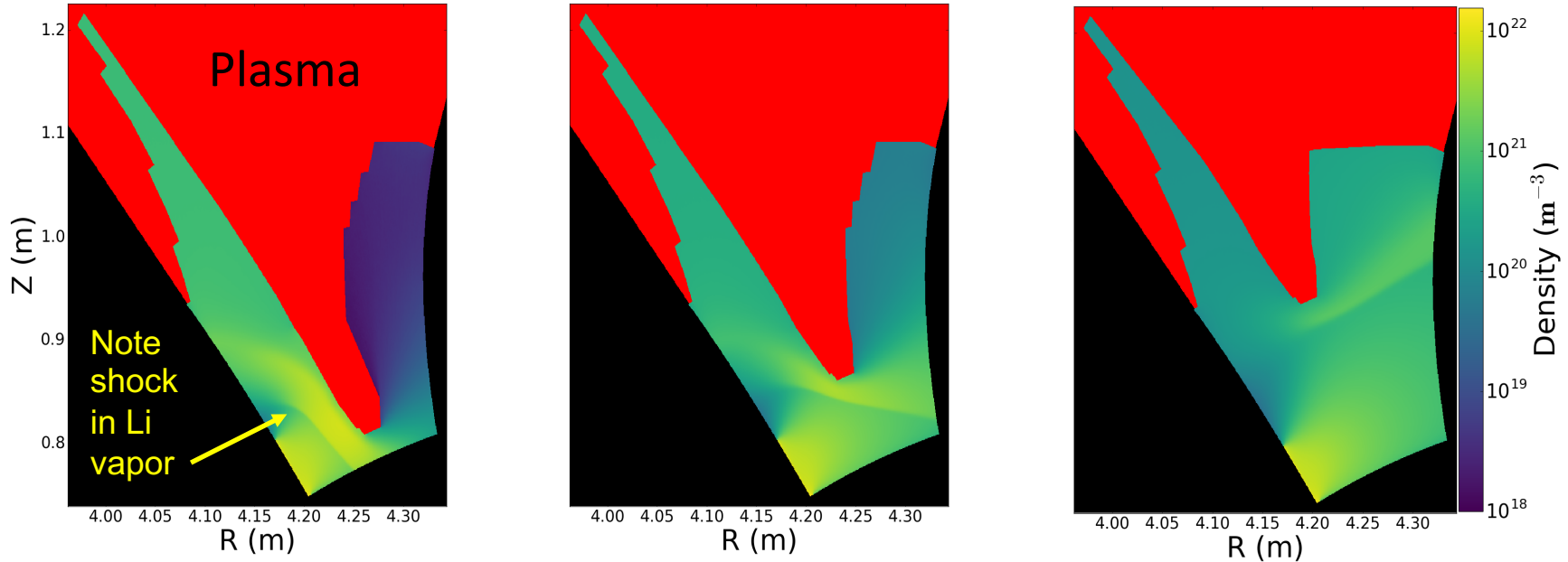


Lithium
Evaporaters

T.D. Rognlien et al. Nuclear Materials
and Energy 18 (2019) 233-238



SPARTA Shows Detachment Stability



Factor of 6 reduction in lithium ionization as plasma retreats from vapor region \Rightarrow stable detachment front



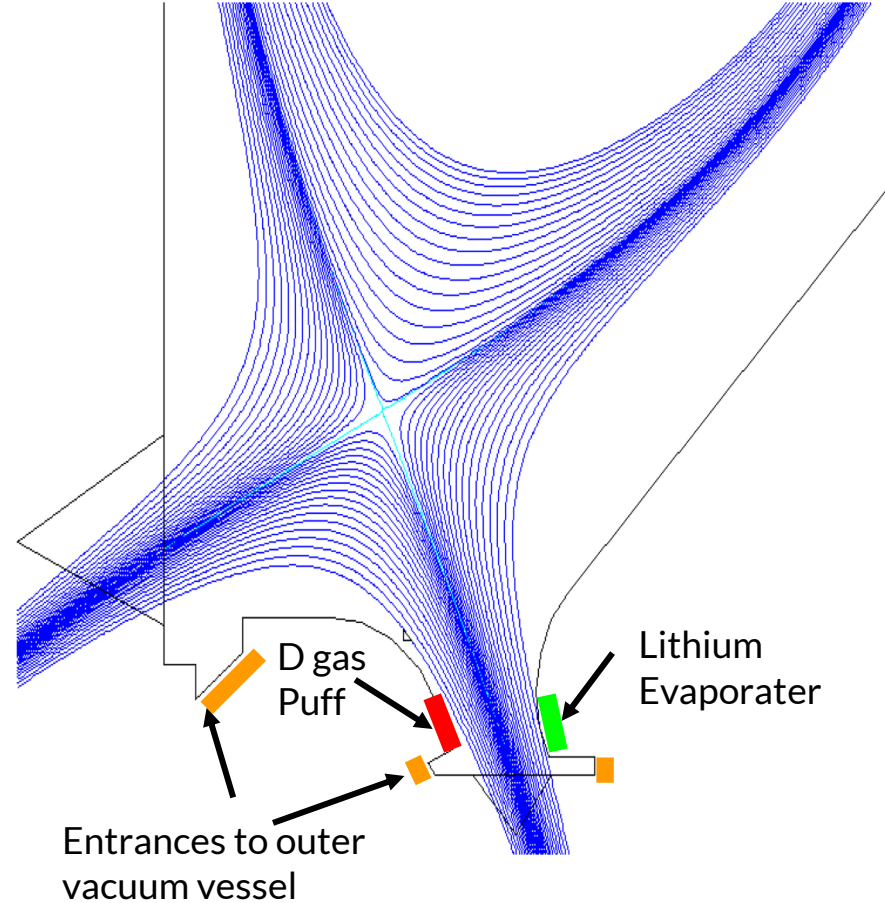
SOLPS-ITER

- Monte-Carlo modeling of neutrals
 - More sophisticated than UEDGE, less so than SPARTA
- More modern thermal force model than UEDGE
 - Sytova et al. (2018) Contrib. Plasma Phys.
- Allows more self-consistent simulations



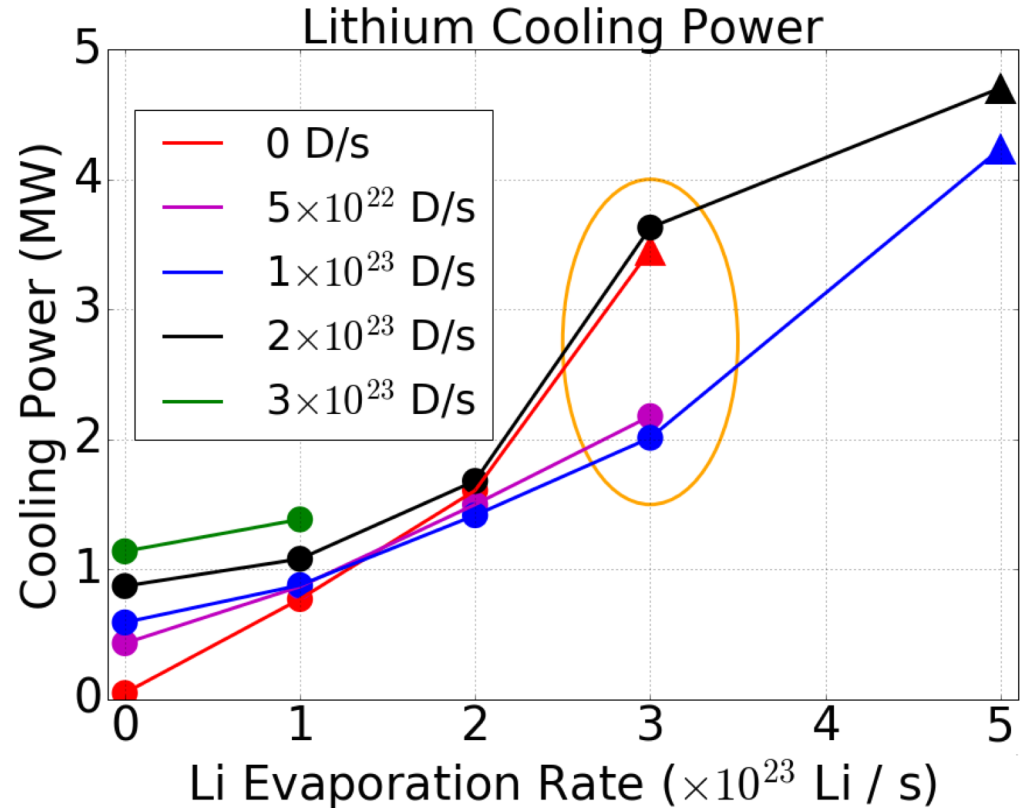
Using SOLPS-ITER

- Applied to a realistic EAST PFC geometry and equilibrium
- Added in lithium evaporation from the far scrape off layer region and a deuterium gas puff injection from the private flux region
- Deuterium entirely recycling while lithium was being entirely absorbed.
- 4MW of input power



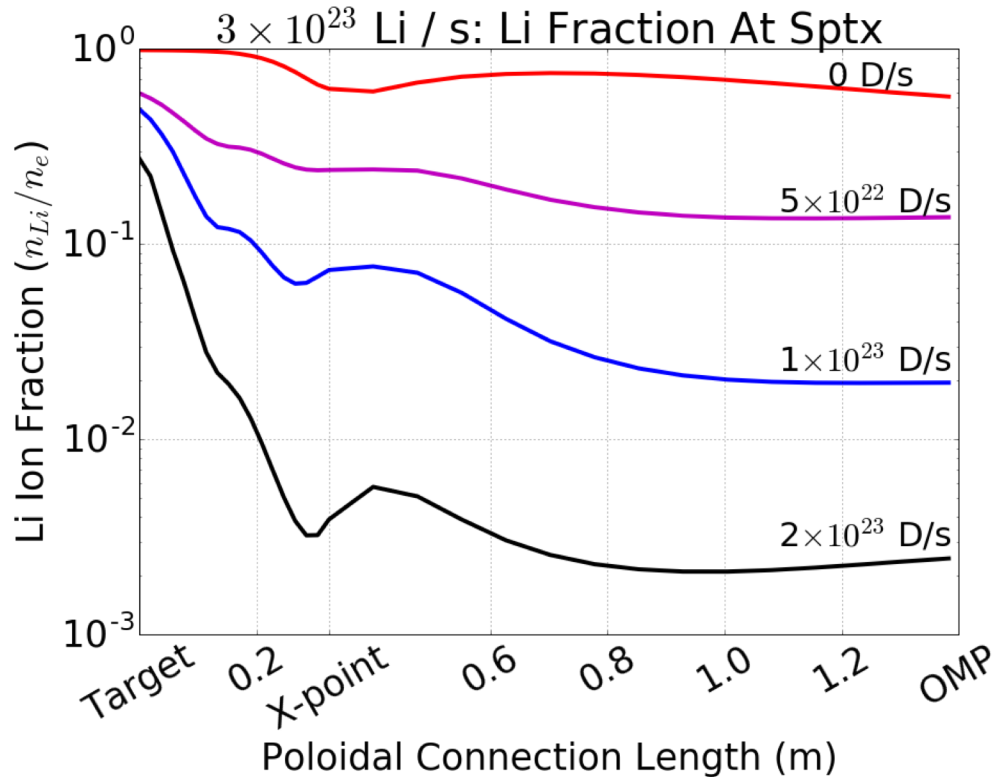
Lithium Causes Large Radiation Fraction

- Without lithium, radiation only gets to ~1 MW
- 90% of input power can be radiated while maintaining high upstream pressure

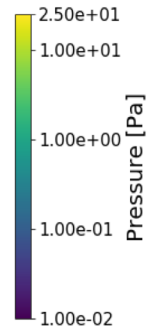
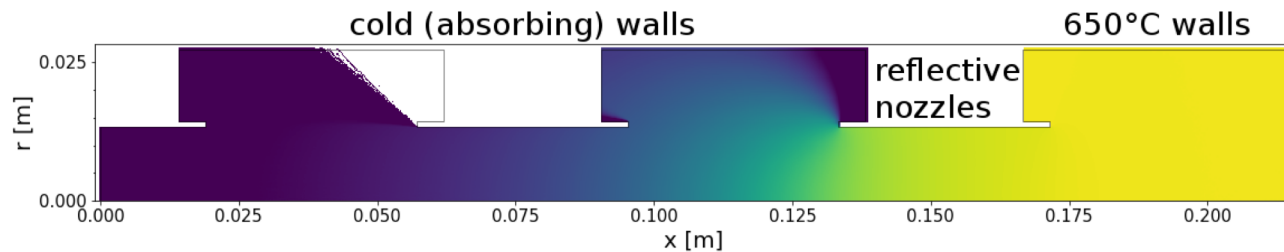
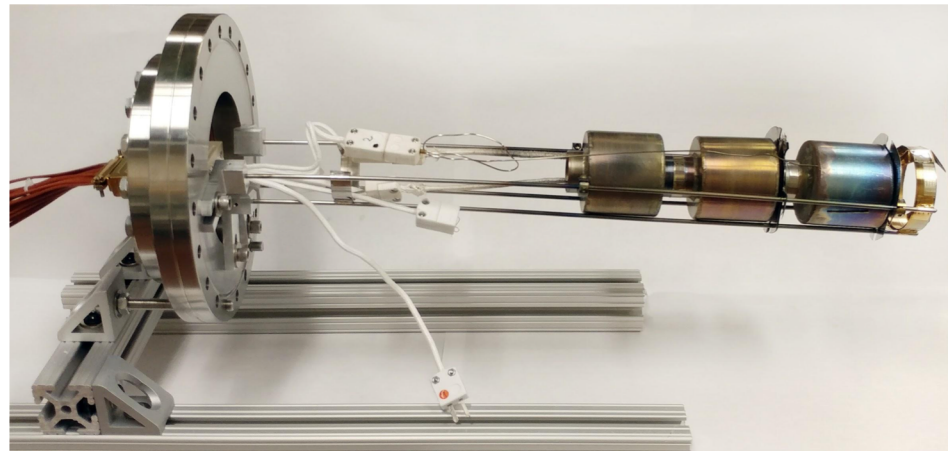
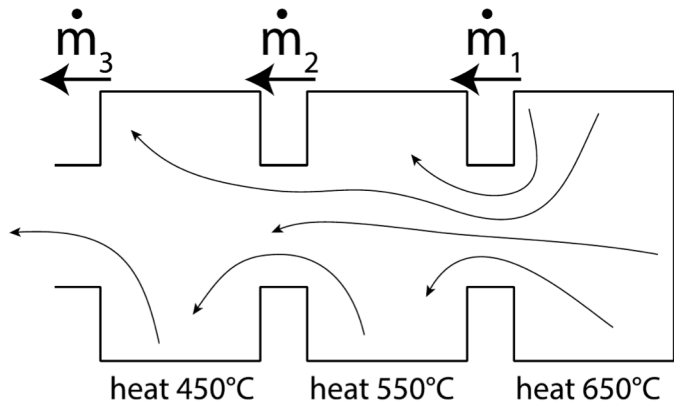


Lithium Can be Contained to Divertor

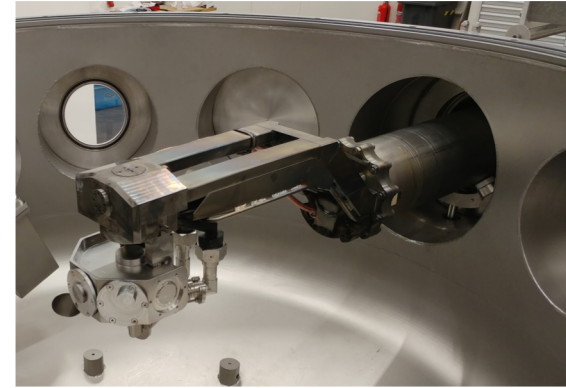
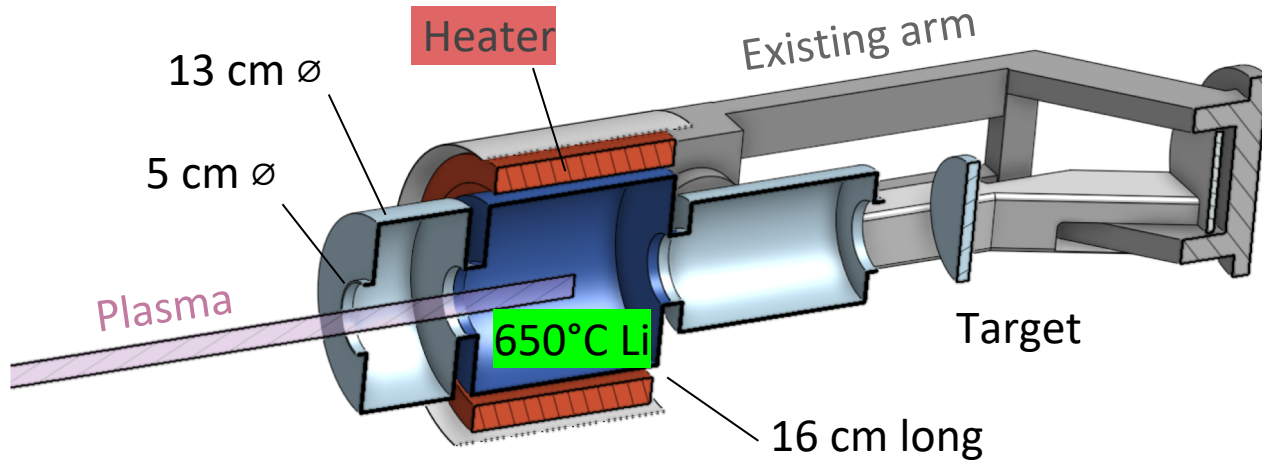
- Higher deuterium puffs pushes the lithium into the divertor
- OMP Lithium fraction reduces from 14% to 0.25% across a factor of 4 increase in the deuterium gas puff.



Validate Li vapor model at PPPL



Device to demonstrate detachment, volumetric recombination at Magnum



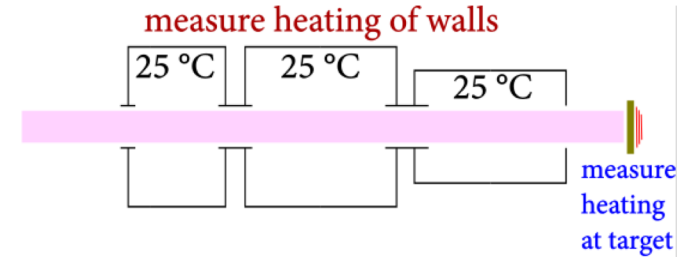
- Li vapor in central box
- Demonstrate volumetric energy loss by Li



Demonstrate detachment via Li vapor

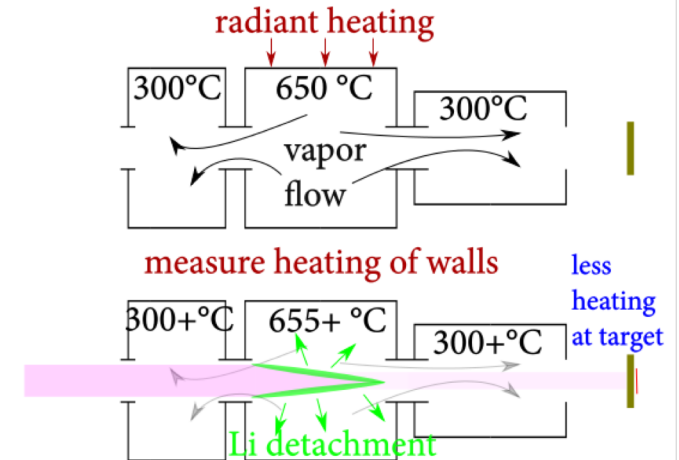
Without Li:

- Turn on plasma
- Measure dT/dt at walls and target



With Li:

- Heat box to make vapor cloud
- 10 second plasma pulse
- Measure dT/dt at walls and target



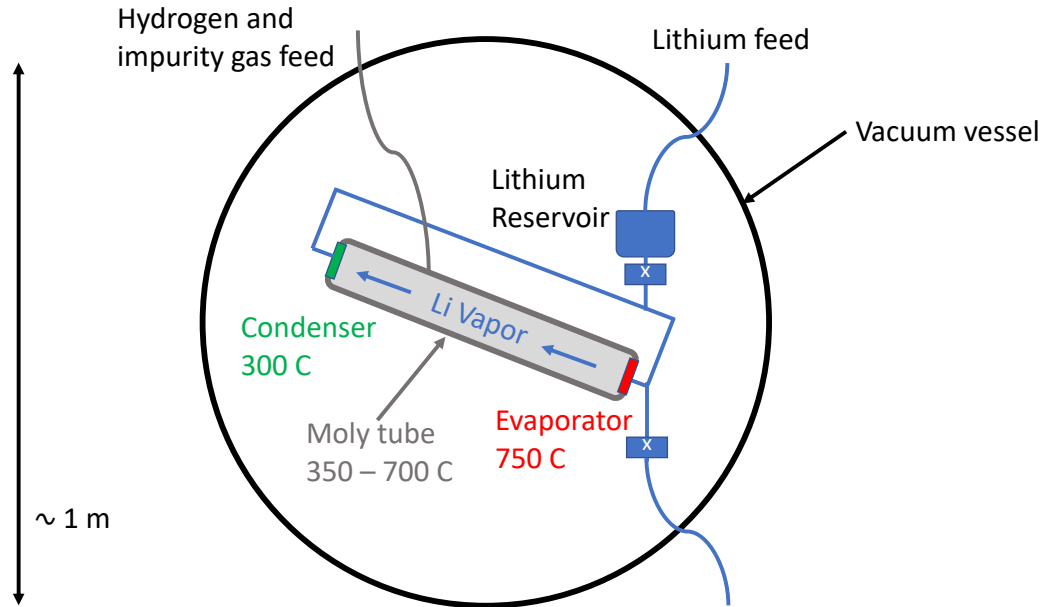
Options for NSTX-U

- 50 cm toroidal segment of 3-D printed tungsten
 - No divertor slot configuration
- = Technology test in tokamak environment.
 - Can we maintain clean plasma-facing surface with carbon walls, or do we require high-Z walls?
 - Can we maintain desired Li temperature?
 - Can we refill reservoir between shots?
- Move to full toroidal ring with slot
 - Slot configuration optimized with modeling



Needed Off-Line R&D

- Inject D & Impurities.
- Add glow probe to produce ionization.
- Measure D retention on tube surface.
- Recirculate Li using capillary force.
- Add B field across flow.
- Clean lithium flow.



Conclusions

- Lithium Vapor Box is promising
 - To maintain stable detachment front
 - To provide SOL-only power loss (other impurities in core?)
 - To retain impurities in divertor
- Next steps
 - Test lithium vapor containment at PPPL
 - Test volumetric energy loss at Magnum-PSI
- Potential Future steps
 - Divertor segment at NSTX-U
 - Full vapor box at NSTX-U
 - D retention & lithium recirculation studies in test stand

