

# Current Design and R&D Progress of CN HCCB TBS

**Xiaoyu WANG**

**Southwestern Institute of Physics (SWIP), China**

**On behalf of CN TBM team**

***CNTBM program: Helium Cooled Ceramic Breeder Test Blanket System (HCCB TBS)***

***Leaded by CN DA***

***Supporting Institutes:***

- 1). Southwestern Institute of Physics (SWIP), China*
- 2). China Academy of Engineering Physics (CAEP), China*
- 3). Institute of Nuclear Energy Safety Technology (INEST), China*



# Outline

- **Introduction**
- **System Overview**
- **Design Optimization**
- **R&D Progress**
- **Safety**
- **Summary**



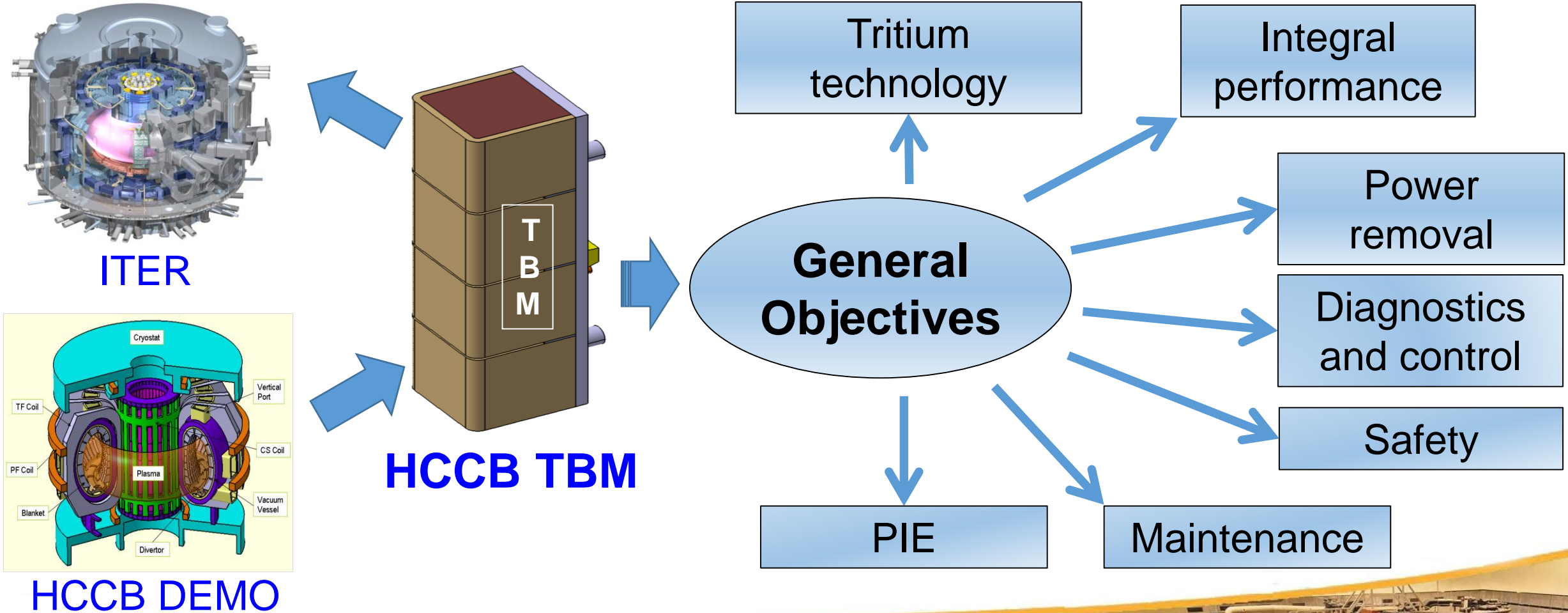
# Introduction

- The ITER facility could offer a **unique opportunity** to demonstrate the feasibility to test **tritium breeding blanket technology** in a tokamak reactor and to test Tritium producing components.
- **Verification of tritium breeding technology** by Test Blanket Module (TBM) program is one of the engineering goals for ITER.
- **CN TBM Program** was established by CN DA in 2009 and the **Helium Cooled Ceramic Breeder (HCCB) TBM** concept was selected.
- CN TBM Program is the **first step** toward the future breeding blanket for CFETR and DEMO.



# General Objectives of CN HCCB-TBM Program

The objectives of CN HCCB TBS is to test the tritium breeding blanket technology in the tokamak operation conditions provided by ITER.





# TBM Concepts and Port-Sharing

## TBM Port allocation

Port No. and PM	TBM Concept	TBM Concept
#2 (PM : CN)	HCCB (TL : CN)	LLCB (TL : IN)
#16 (PM : EU)	HCLL (TL : EU)	HCPB (TL : EU)
#18 (PM : JA)	WCCB (TL : JA)	HCCR (TL : KO)

PM : Port Master, TL : TBM Leader

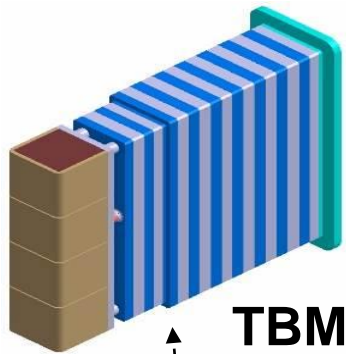


**The signature of CN  
HCCB TBMA**

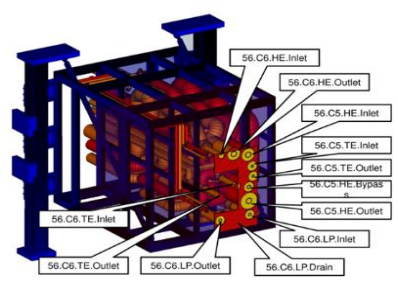
- The [HCCB-TBS TBM Arrangement](#) (TBMA) was signed on Feb. 13<sup>th</sup> 2014 by ITER Organization and CN DA. This is a fundamental step forward for the Chinese TBM Program.
- The [CDR](#) has been hold in July 2014 and approved in September 2015 by ITER. Now HCCB-TBS is at the preliminary design phase.



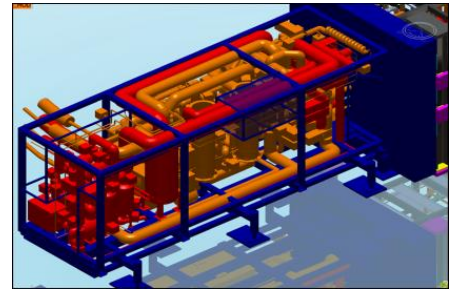
# Subsystems and Configuration of HCCB TBS



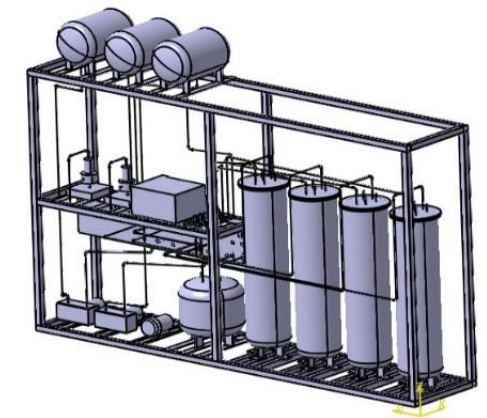
TBM set



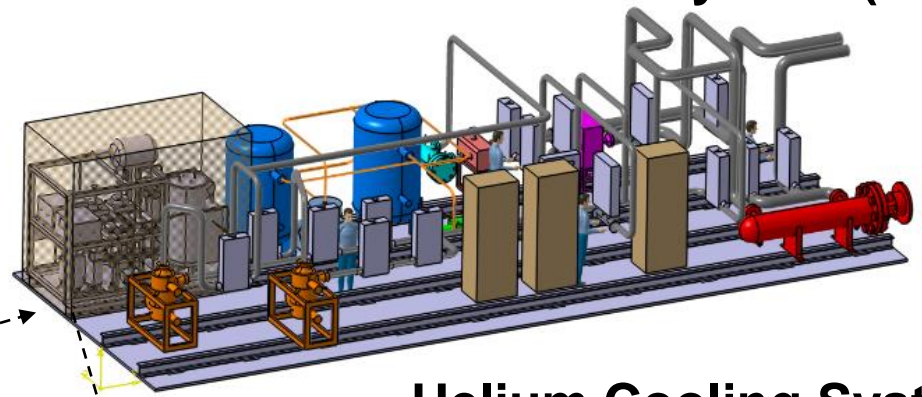
Pipe Forest (PF)



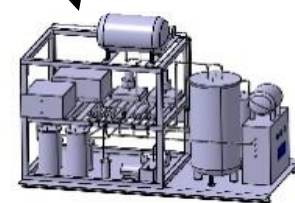
Ancillary Equipment Unit



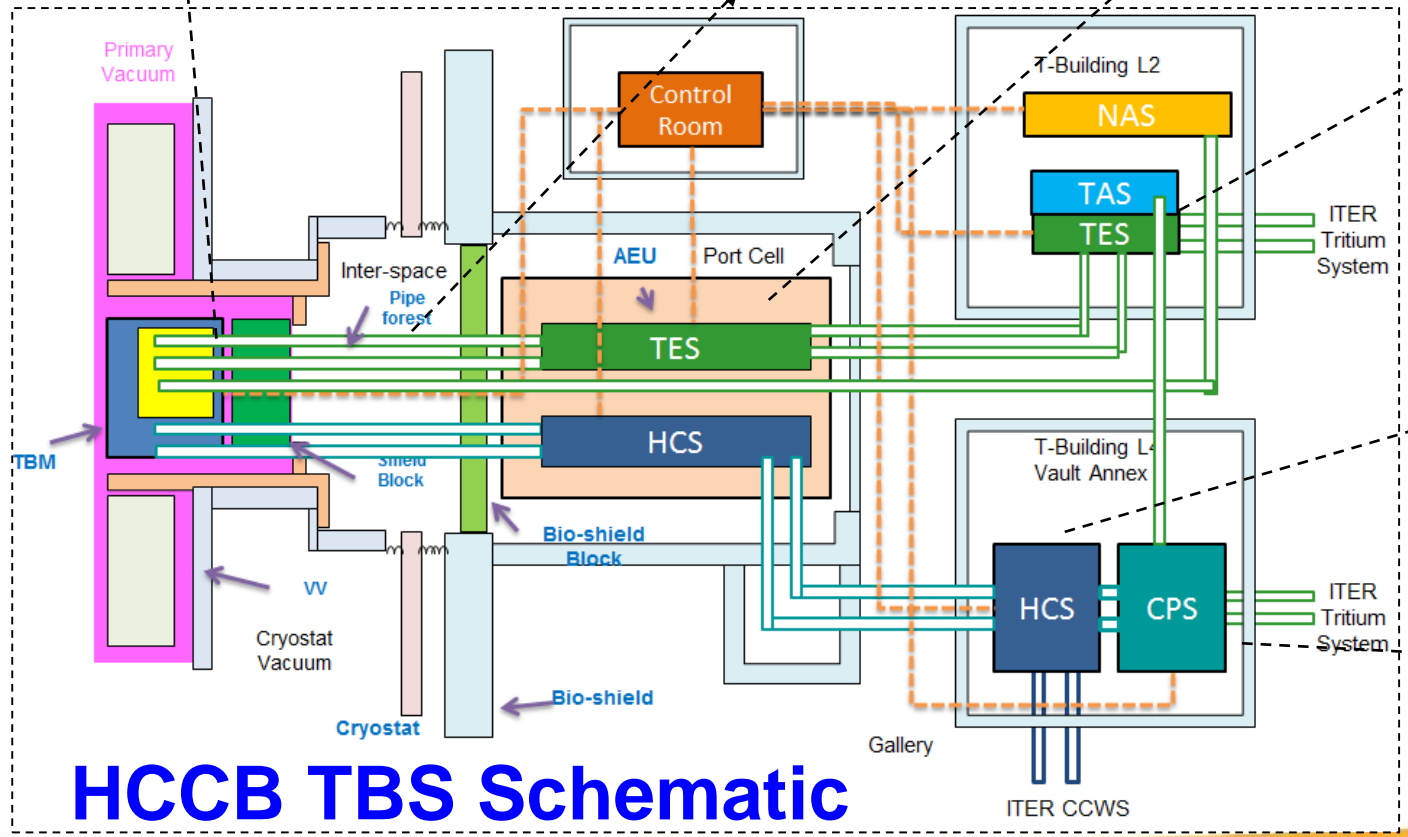
Tritium Extraction System (TES)



Helium Cooling System (HCS)



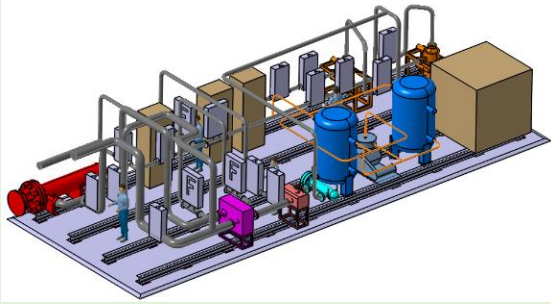
Coolant Purification System (CPS)



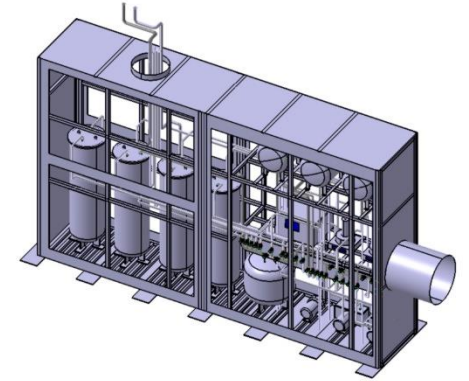


# Subsystems and Configuration of HCCB TBS

**Room: 14-L4-20**  
HCS & CPS

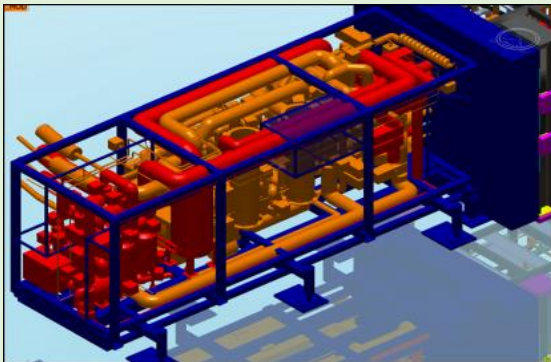


**Room: 14-L2-24**  
TES & TAS & NAS

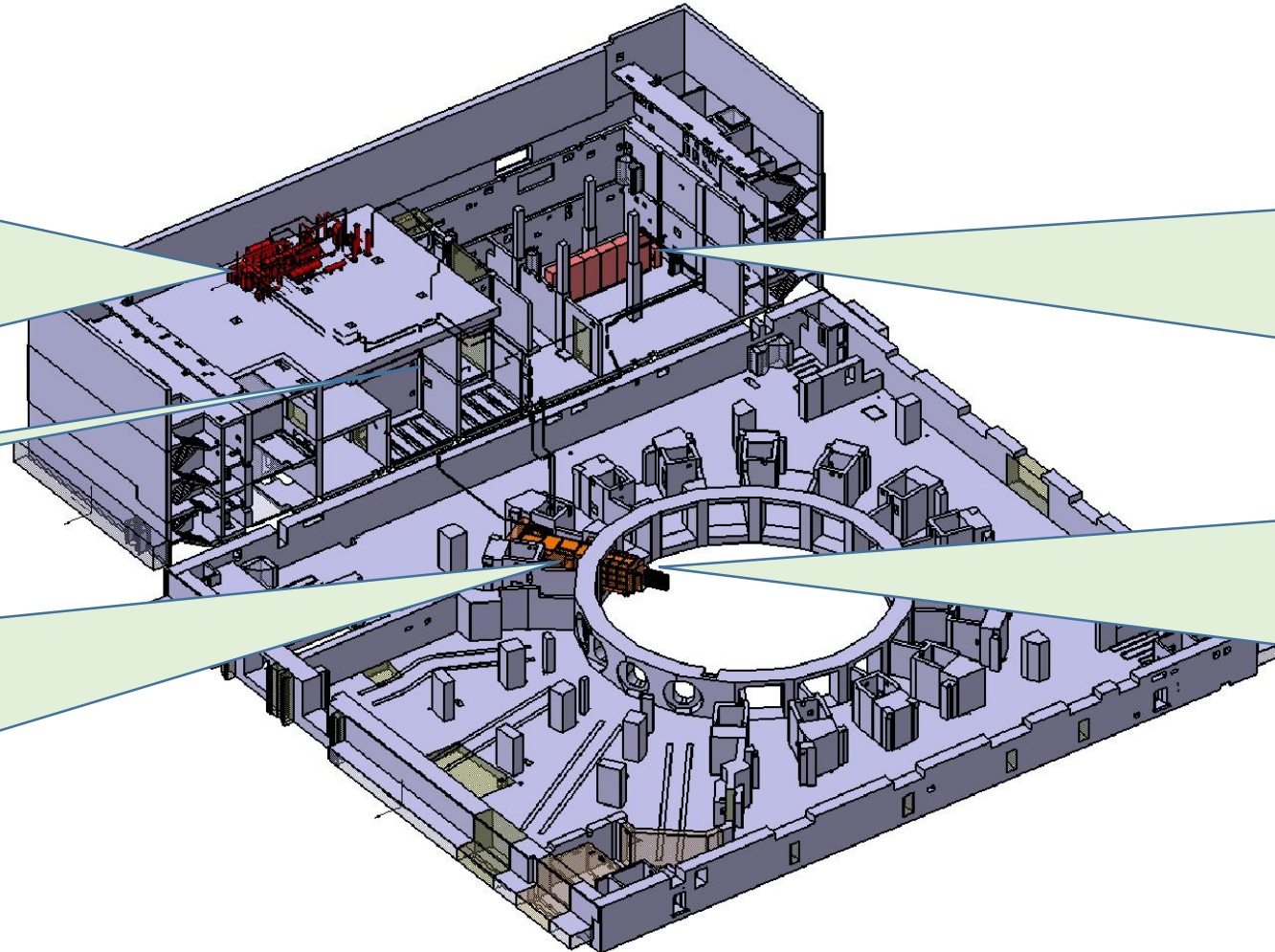
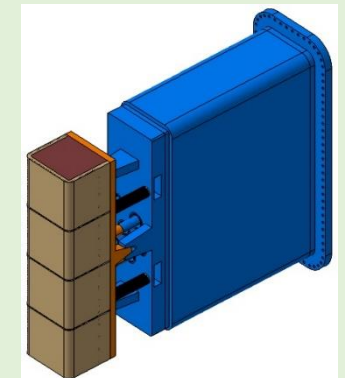


Connection Pipes

**Room: 11-L1-C02**  
AEU & PF



**Tokamak Port #2**  
TBM-set



## HCCB TBS Layout Configuration



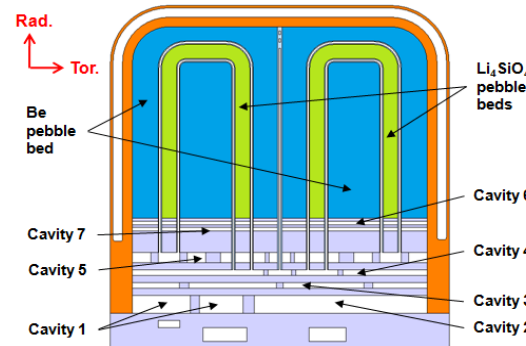
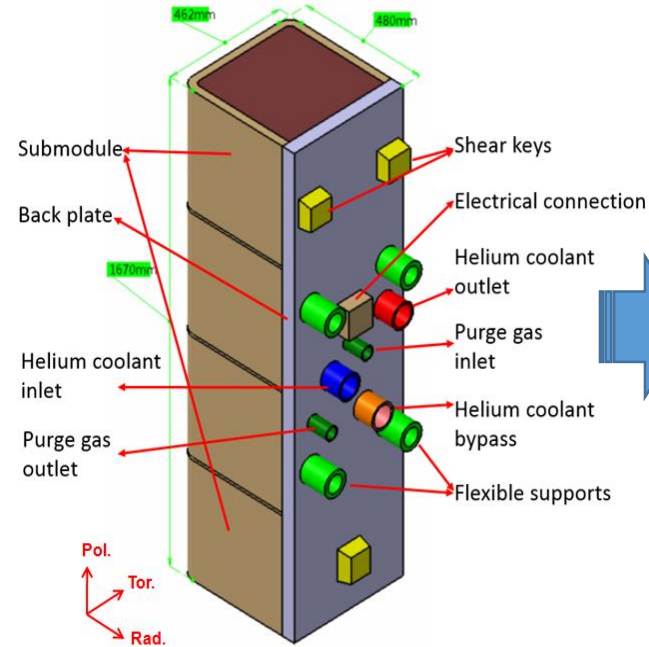
# Design Optimization for HCCB TBM-set

Since conceptual design, the HCCB TBM was significantly simplified considering the material performance, manufacturability and ALARA principle.

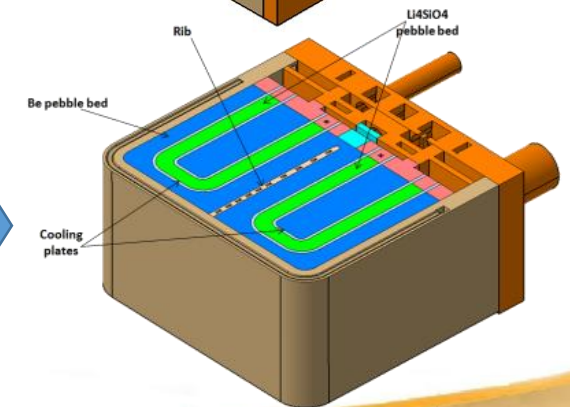
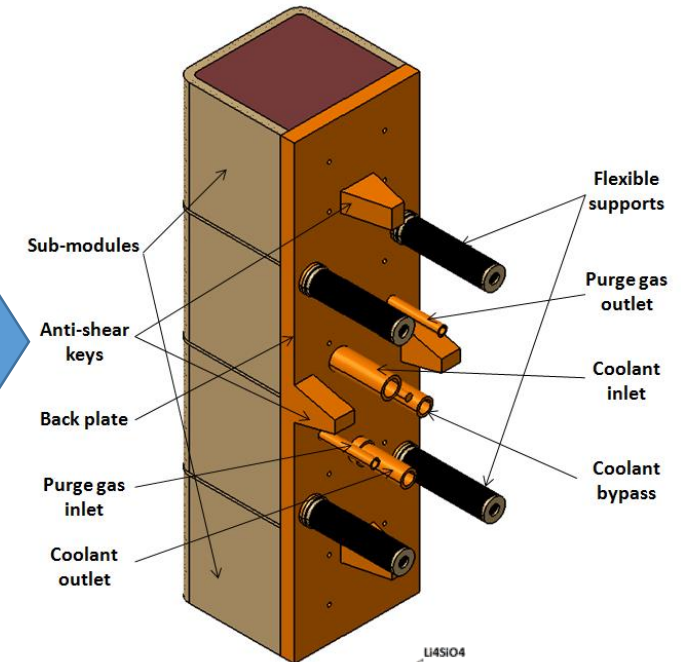
## Main optimization:

- Simplification of back plate
- Multiplier pebble bed: Binary  $\rightarrow$  Unitary
- Enrichment Li6 increase
- Design configuration update
- TPR is slightly increased to  $\sim 0.061\text{g/FPD}$
- TES parameters: 0.3MPa, 0.3g/s

## Conceptual design



## Optimized design

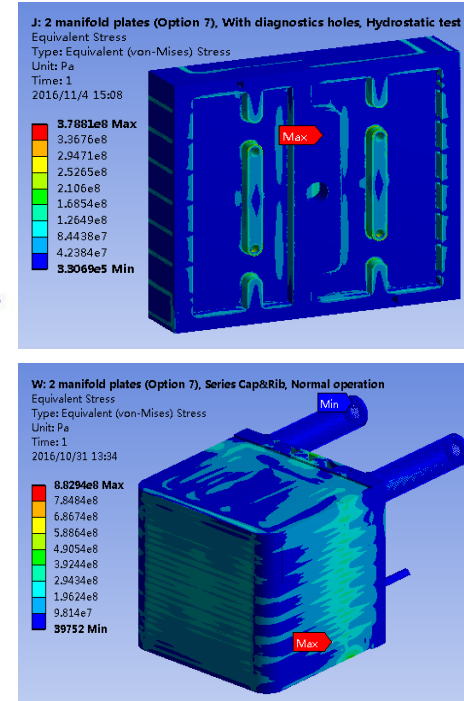
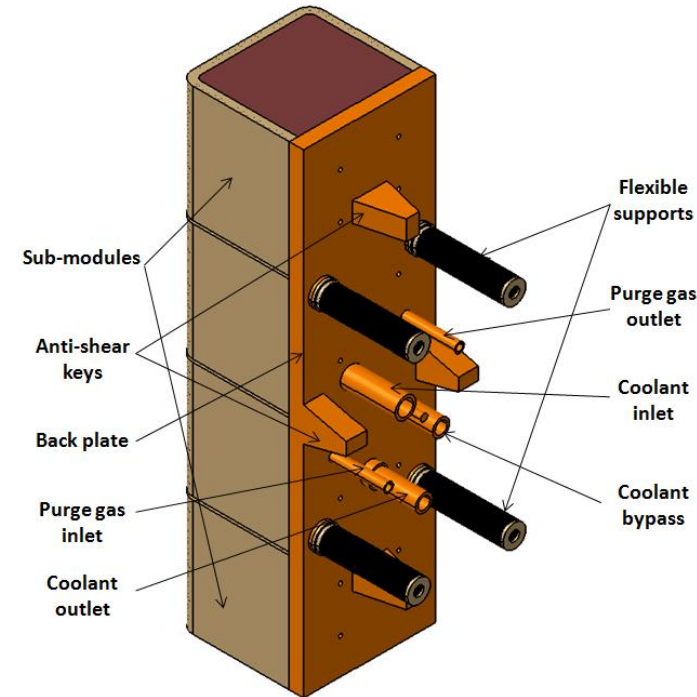




# Design Optimization for HCCB TBM-set

## Main design parameters

Parameters	Values
Neutron wall load	0.78 MW/m <sup>2</sup>
Surface heat flux	0.3 MW/m <sup>2</sup>
Structural material	CLAM/CLF-1 ~1.2ton (<550°C)
Tritium Breeder	Li <sub>4</sub> SiO <sub>4</sub> pebble bed (<900°C)
Neutron Multiplier	Beryllium pebble bed (<650°C)
Coolant	Helium (8MPa) 1.04 kg/s (300°C/500°C)
Purge gas	Helium (0.3MPa) with 0.1% H <sub>2</sub>
TPR	0.061g/FPD



## Analysis validation

	Pm	Pm+Pb	Pm+Pb+ΔQ
<b>Normal</b>	<84MPa	127MPa	500MPa
<b>Hydrostatic test</b>	<162MPa	311MPa	-
<b>In-box LOCA</b>	<174.7MPa	224MPa	-



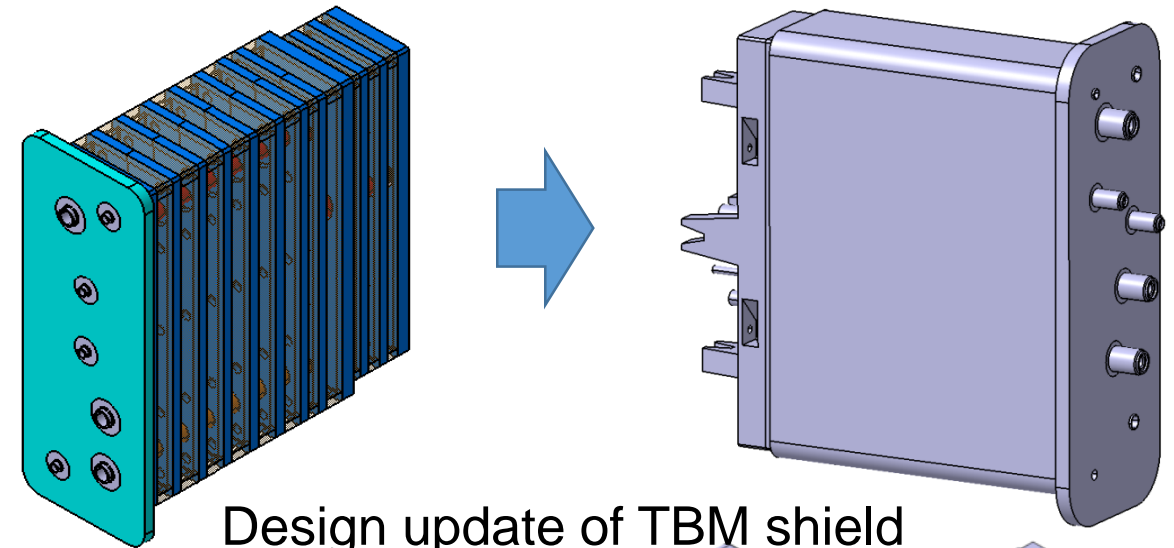
# Design Optimization for HCCB TBM-set

Based on fabrication technology R&D and engineering analyses, HCCB TBM shield was further optimized:

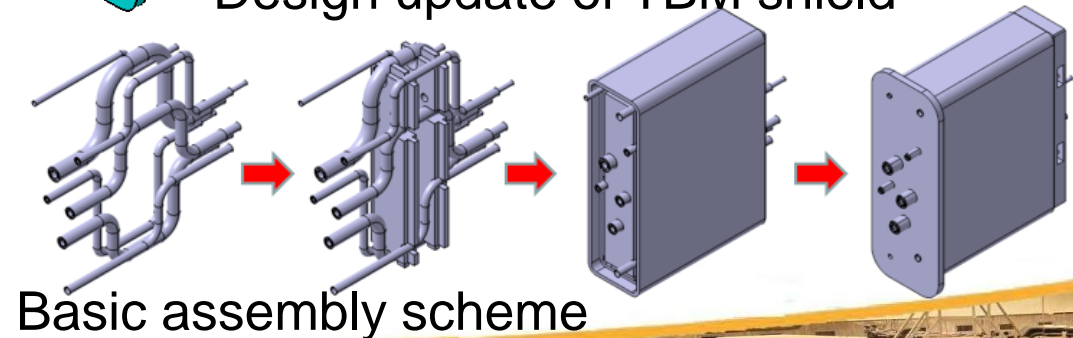
- Reduce the total length to increase the interspace with TBM with similar shielding capability
- Optimize pipe configuration, thickness of shell, supporting structure to simplify the manufacturing

## Main design parameters

Parameters	Values
Structural material	SS316LN-IG
Coolant	Water (4MPa) 0.1 kg/s 70°C/125°C
Dead weight	~5 tons
Water volume	0.98 m <sup>3</sup>
Water fraction	~40%
Nuclear heating	20.3 kW



Design update of TBM shield



Basic assembly scheme



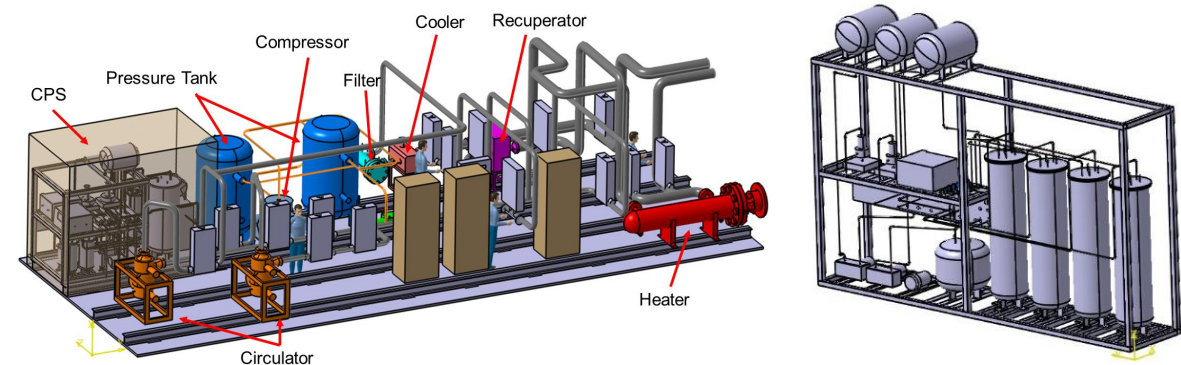
# Design Optimization for HCCB TBS Ancillary Systems

## Main design parameters

Parameters	Values (HCS)
Main structural material	SS316L
Supporting structure material	SS304
<b>Primary coolant circuit</b>	Helium
- Pressure	8 MPa
- Total flow rate	1.04 kg/s
- Pressure drop	~0.5 MPa
- Inlet/outlet temperature	500°C/300°C
<b>Interface with CCWS</b>	Water
- Pressure	0.8 MPa
- Total low rate	21.3 kg/s
- Inlet/outlet temperature	31°C/43°C
<b>Tritium related system</b>	<b>Values (TES, CPS)</b>
- Purge gas	He with 0.1% H <sub>2</sub>
- T purification efficiency	≥ 95%
- Impurity removal efficiency	≥ 90%
- T extraction efficiency	≥ 90%

The design of all ancillary systems have been optimized considering the review comments, safety and interface requirements:

- Configuration update based on equipment investigation, PFD and PID diagrams update
- System performance assessment, structural analysis



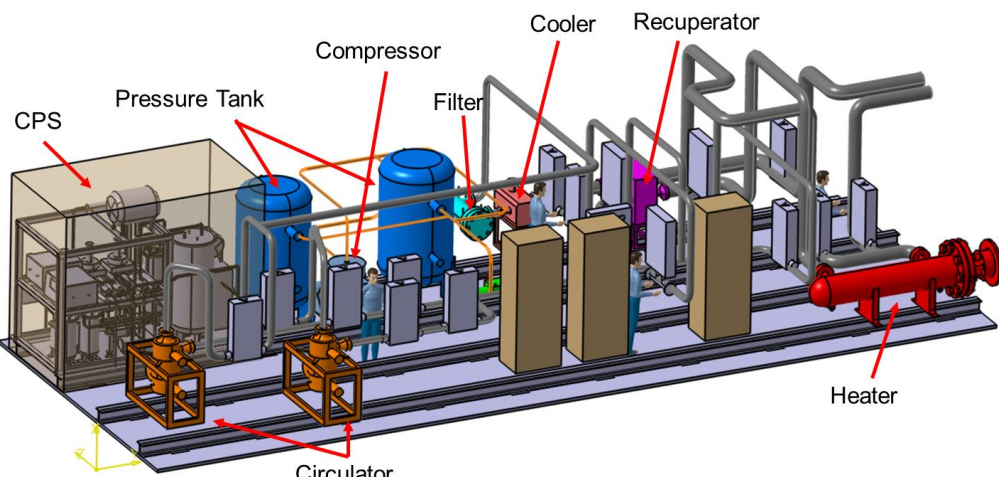
HCS Design

TES Design

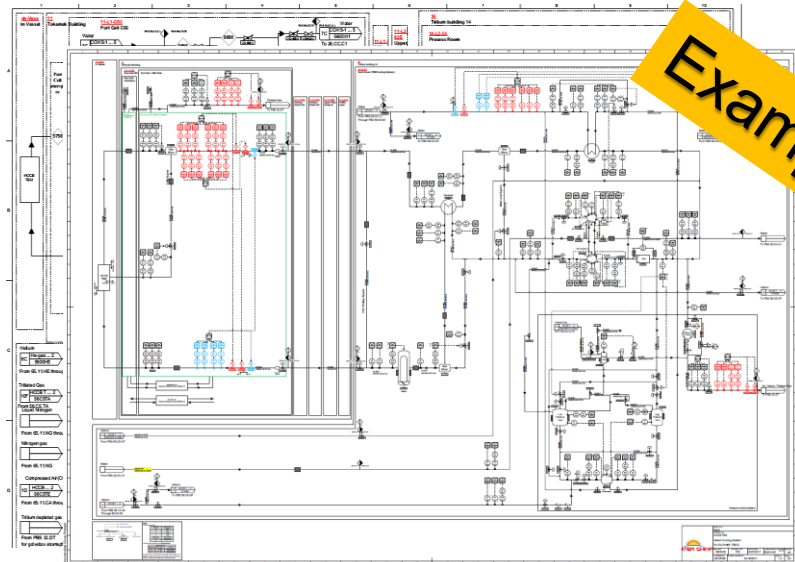




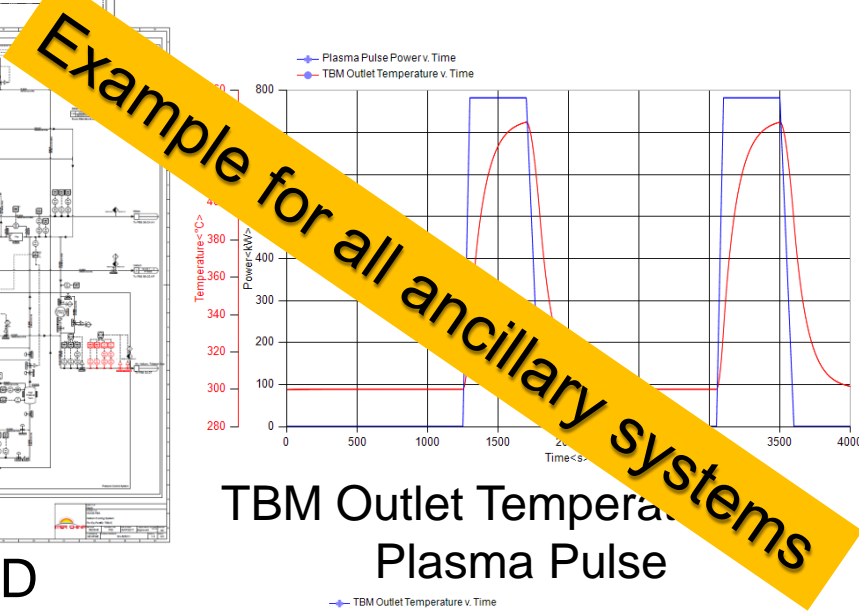
# Design Optimization for HCCB TBS Ancillary Systems



New configuration of HCS

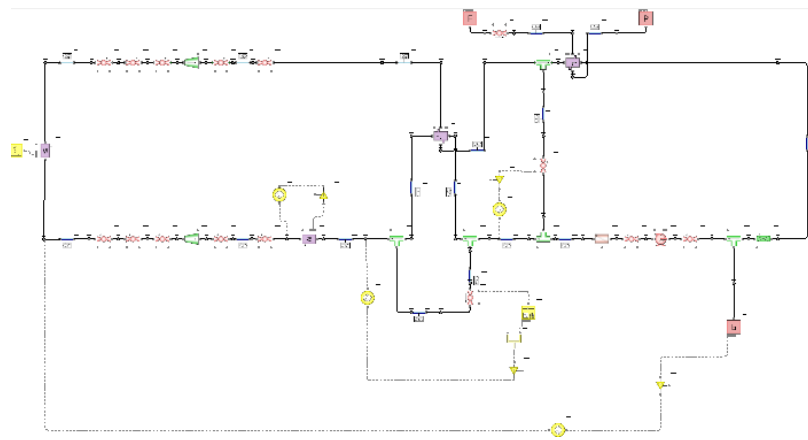
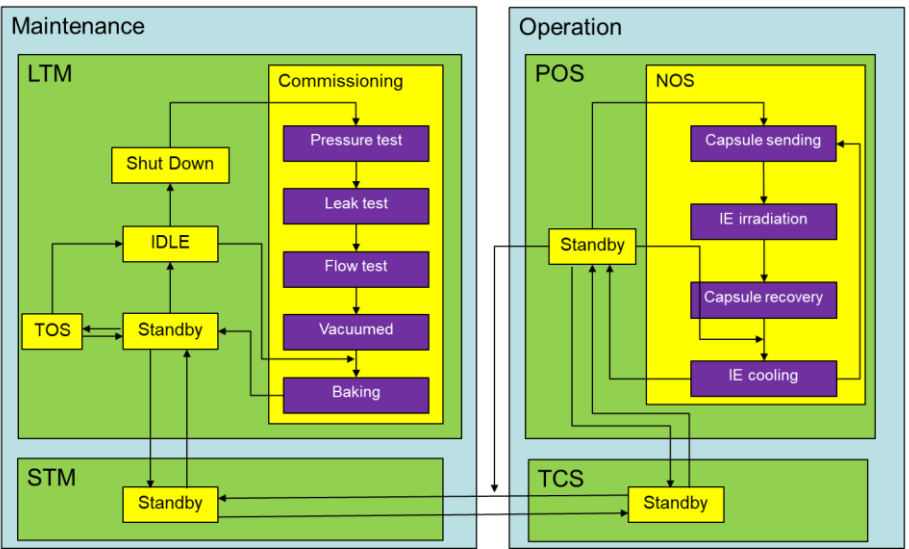


Updated PFD and PID

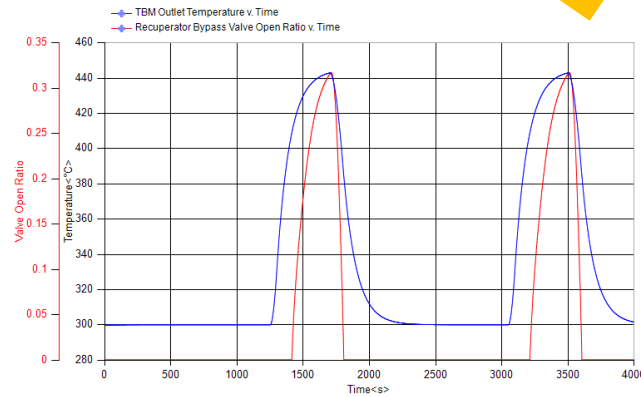


TBM Outlet Temperature vs. Plasma Pulse

Operation states and transition path



Performance analysis model for HCS



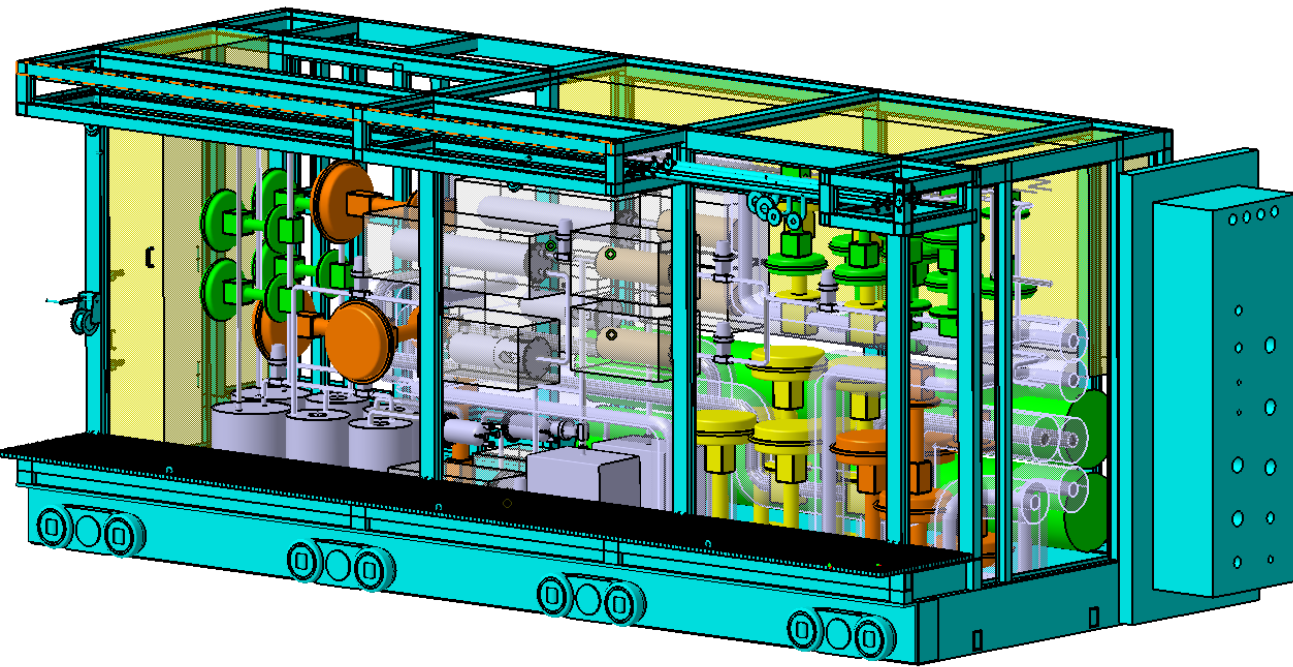
Bypass Valve Open Ratio vs. TBM Outlet Temperature



# HCCB TBS System integration

## AEU integration

- Component design for HCCB TBS in AEU has been updated.
- Uncertainty of IN withdraw → two sets of HCCB TBS components for AEU.



Component configuration in AEU

## Components and equipment in TBS 5

Components	Dimension	insulation layer/mm	Mass/kg	Material	Work Under Maximum ambient temperature 60°C , (Y/N)	
Cooler 1	D200xH700mm	100	40	316L	Y	
Cooler 2	D200xH700mm	100	40	316L	Y	
Ionization chamber 1	D500xL700mm	/	40	316L	Uncertain,need further evaluation	
Ionization chamber 2	D500xL700mm	/	40	316L	Uncertain,need further evaluation	
Molecular Sieve Bed	D150xH400mm	/	35	316L	Y	
Measurement unit	400x400x500mm	/	30	316L	Uncertain,need further evaluation	
Reduction Bed	D380xH800mm	120	110	316L	Y	
Relief Tank	2m <sup>3</sup>	/	300	316L	Y	
Particle Filter 1	D100xH400mm	100	10	316L	Y	
Particle Filter 2	D100xH400mm	100	10	316L	Y	
Compressor	D550xL750mm	/	100	316L	Uncertain,need further evaluation	
Pipes(2)	DN25/80s	100(only outlet)	/	316L	Y	
NAS	Pipes(2)	DN6/40s	/	316L	Y	
DA	Pipes(1)	DN50/10s	/	316L	Y	
HCS	Pipes(3)	DN80/80s	100	/	316L	Y
	Mixer	D120xL200mm	100	TBD	316L	Y





# R&D Progress – material development

- ◆ Improved the material specification of RAFM steel and completed the fabrication of 4 ingots (5-tons) and 10 tons section bar of CLF-1. Prepared the two standards for CLF-1 steel and one for CLAM.
- ◆ Completed the trial certificate with NB and obtained the 3.2 certificate based on RCC-MR.
- ◆ Completed ion implantation experiment and He(0.1%H) compatibility experiment for CLF-1.



VCM ingot



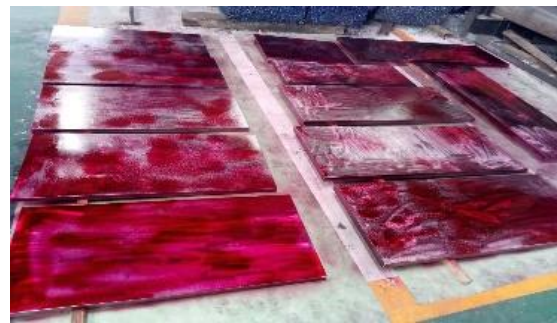
VAR Ingot



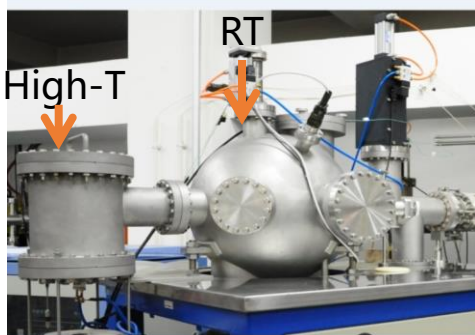
Bar and forging



NDT for plates



Mechanical properties testing



Ion implantation experiment



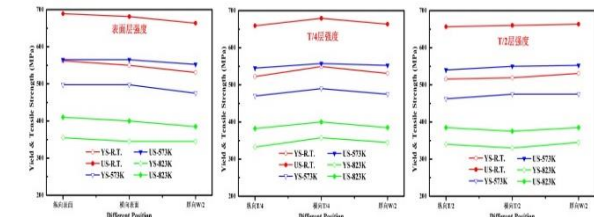
Compatibility experiment



Standards for forging and plate



3.2 certificate



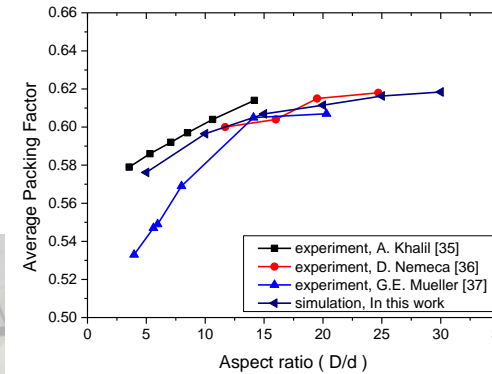
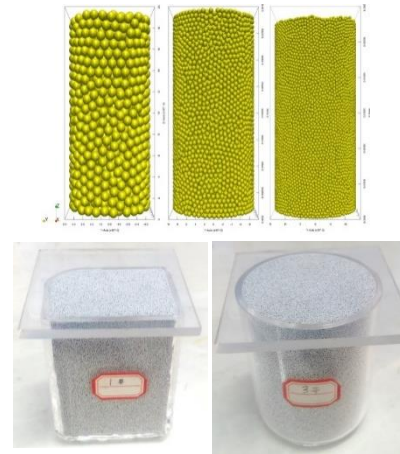
Tensile at Surface, T/4, T/2

Refer: FIP/P3-22  
H.LIAO

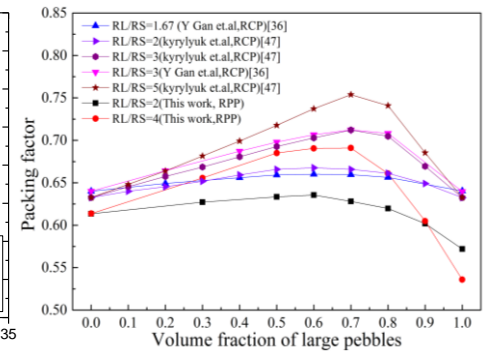


# R&D Progress – material development

- ◆ Completed the beryllium pebble fabrication facility improvement and achieved the production capability of **10kg scale**; Completed He<sup>4</sup> implantation experiment for beryllium;
- ◆ Studied unitary/binary pebble bed (PB), U-shape and compressed PB by DEM and experiments.

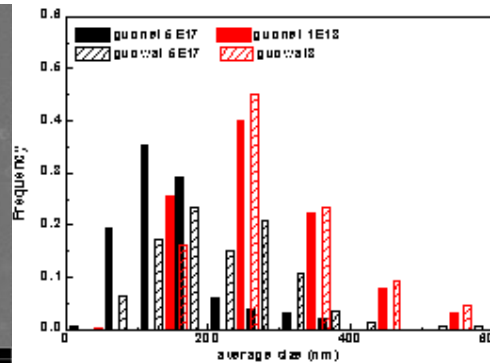
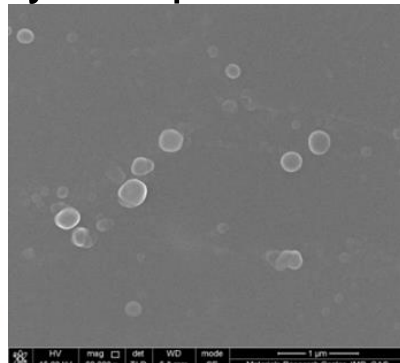
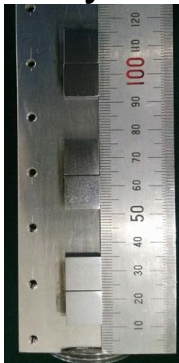


Unitary pebble bed

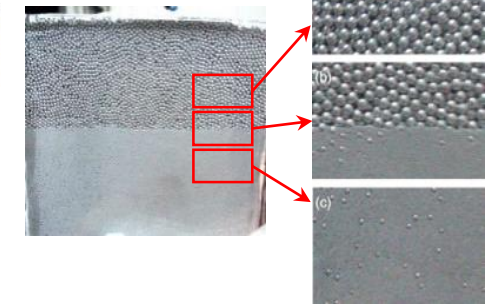
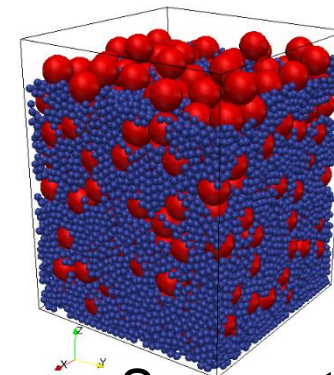


Binary pebble bed

Beryllium pebble fabrication facility and product



Bubble distribution



Segregation in binary PB

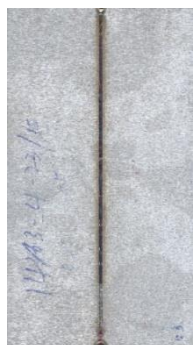
Be sample & He<sup>4</sup> irradiation facility





# R&D Progress – TBM fabrication technology

- ◆ After a large amount of testing and consideration of TBM design, the **LB welding** and **EB welding** have been selected as the main welding method, and the **TIG welding** as supplementary method.
- ◆ Based on the TBM updated design and the selected welding methods, the **preliminary fabrication procedure** of has been prepared.



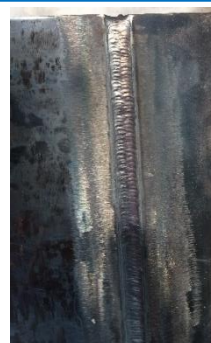
LB (13mm)



EB (55mm)



EB (32mm, T)



TIG (10mm)



LB (13mm)



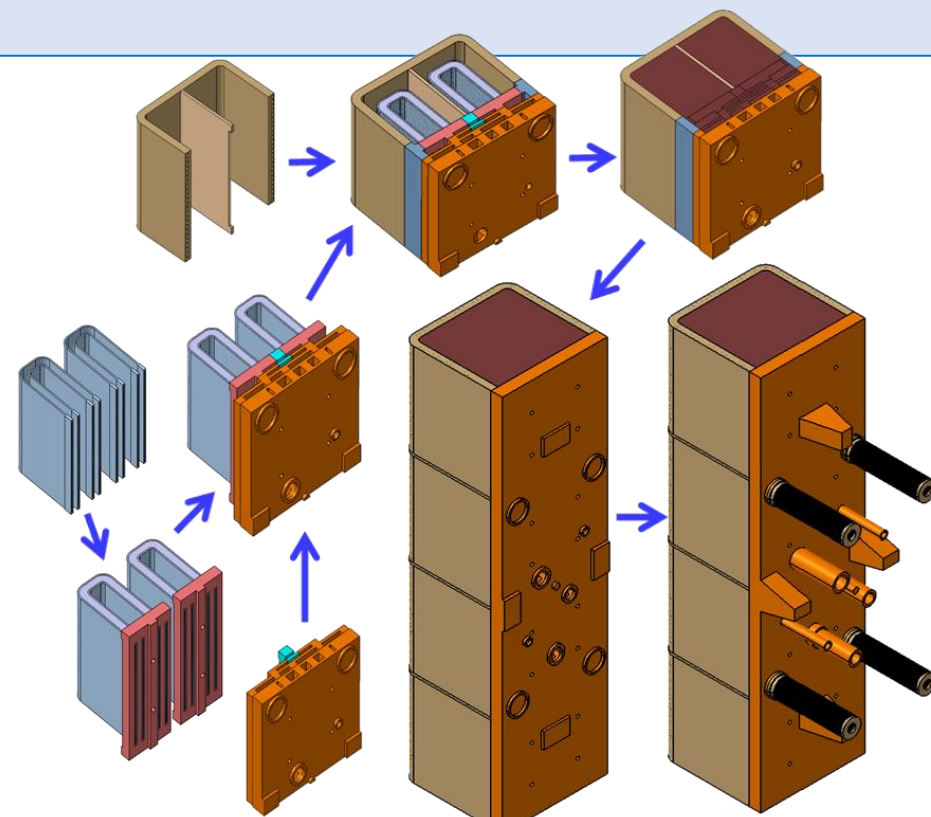
LB (30mm )  
with filler wire



EB (35mm )



TIG dissimilar welding  
(10mm CLF-1/SS316L)





# R&D Progress – TBM fabrication technology

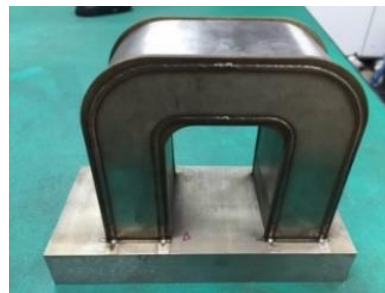
◆ The sample of key components and semi-prototype have been fabricated and tested.



FW sample



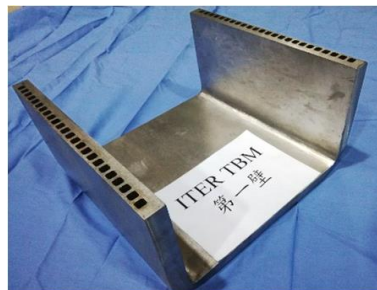
CP sample



Breeding unit sample



Prototype mockup of double-layer pipe for shield



FW mockup



Cooling plate (CP) mockup



Submodule mockup welding



Back plate mockup



Cover plate mockup

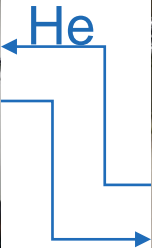




# R&D Progress – HCCB TBS Ancillary Systems

- ◆ Several testing facilities/loops have been constructed, the related experiments are ongoing to verify the design and operation of ancillary systems. The construction of new testing facilities are under plan.

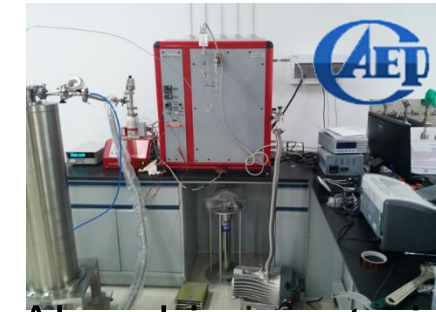
HHFT facility  
EMS-60



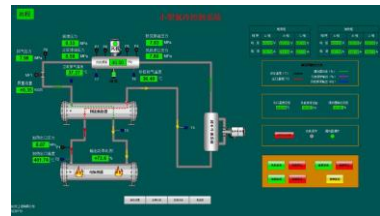
HeCEL-1 (0.1kg/s @ 8MPa & 400°C)



CPS testing facility

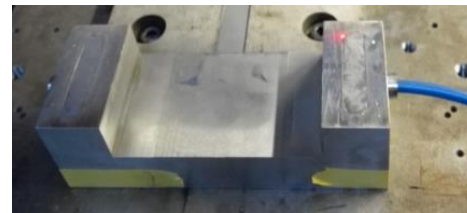


Absorbing material testing facility



Control system and  
ITER Mini-CODAC

Hydraulic testing sample



HHFT testing sample



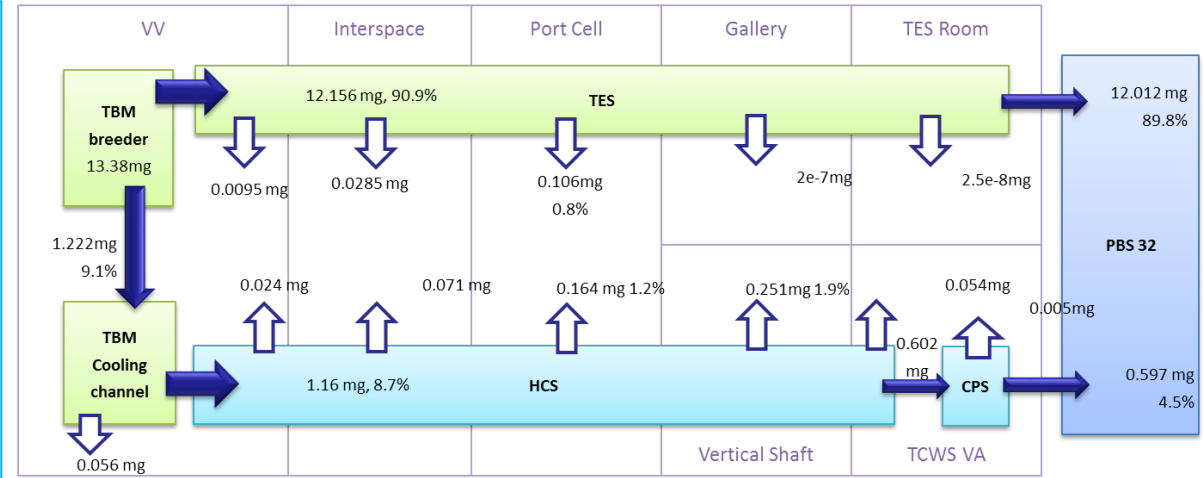
TES testing facility



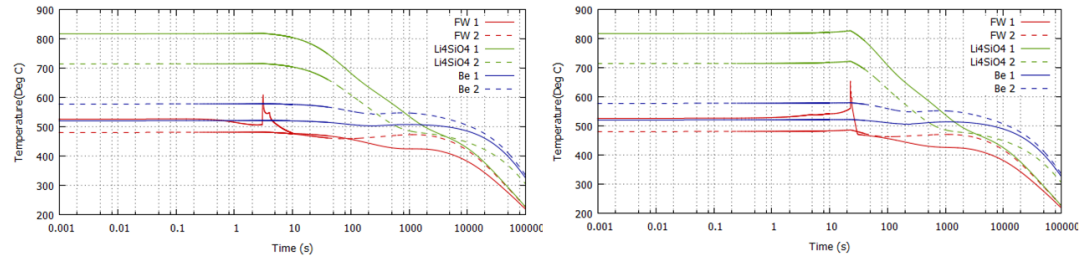
# Safety Aspect

The safety work focused on preparing the answering reports of **Engagement 9.1**.

- Design description update and provisions
- Nuclear analysis (incl. benchmark with KO&IN)
- Tritium analysis (incl. benchmark with KO)
- Accident analysis
- Other analysis

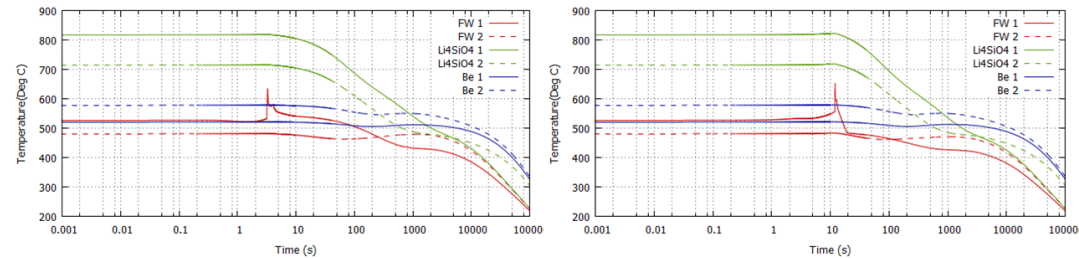


## Example of tritium release results



Temperature of CN HCCB TBM during large break area (LOCA in Port Cell)

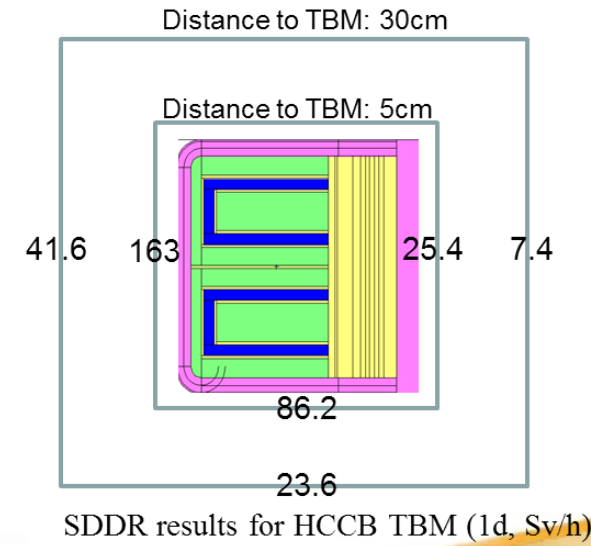
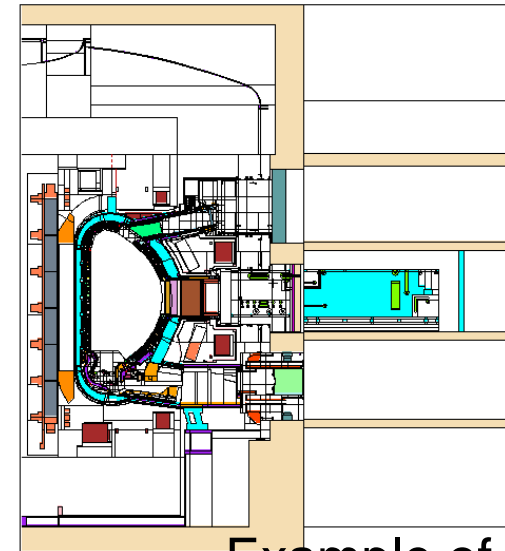
Temperature of CN HCCB TBM during small break area (LOCA in Port Cell)



Temperature of CN HCCB TBM during large break area (LOCA in TCWS Vault)

Temperature of CN HCCB TBM during small break area (LOCA in TCWS Vault)

## Example of accident analysis results



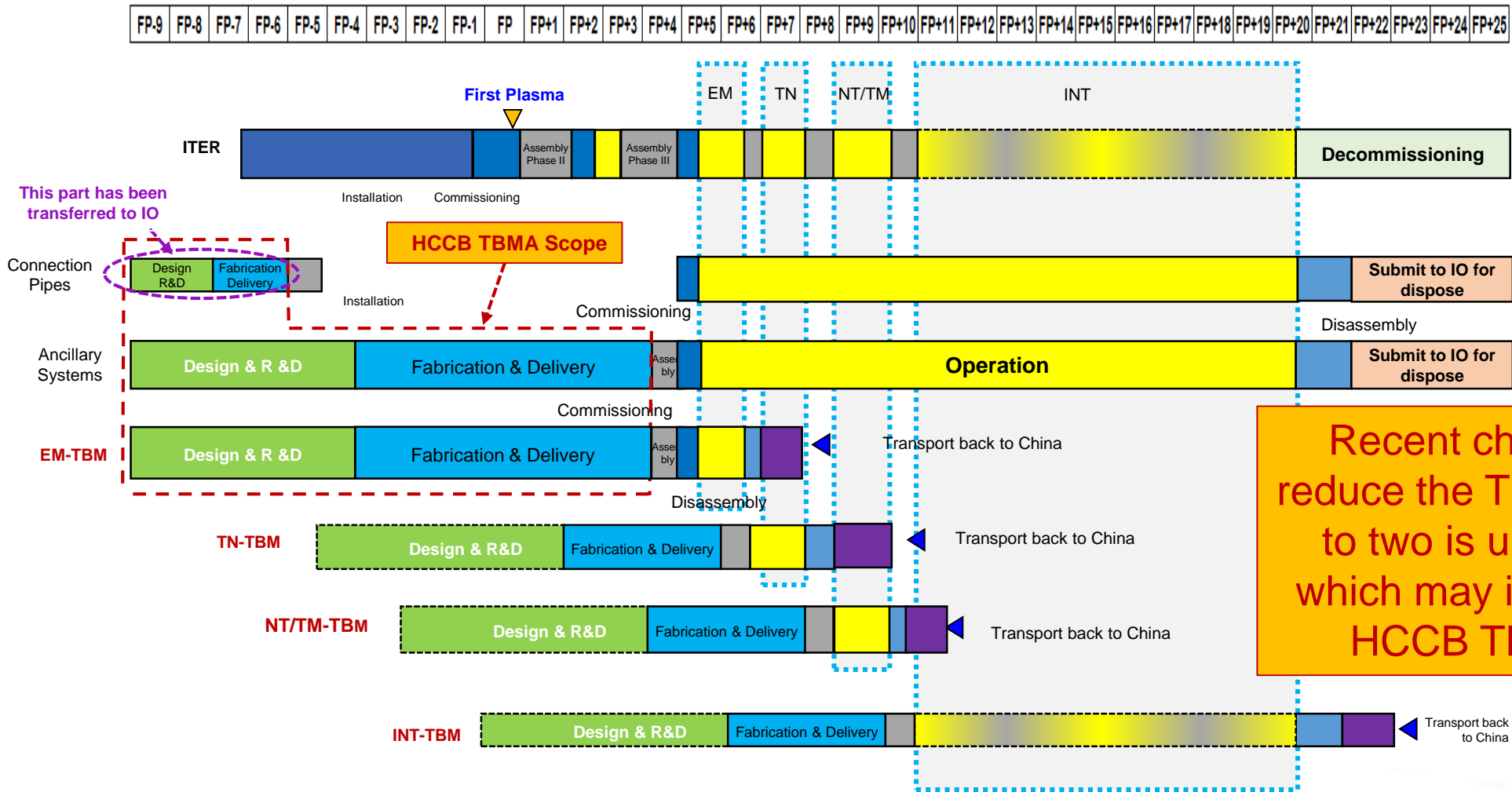
## Example of dose rate results





# HCCB-TBS Overall Schedule

## ITER Commissioning and Operations



# Summary

- CN HCCB TBS is **one of the most important part of China fusion development strategy** toward DEMO.
- After the conceptual design approval in 2015, the design of the HCCB TBS has been **significantly optimized** and developed in detail according to the schedule and requirements.
- In order to support the design optimization, a lot of **R&D activities** have been implemented, including structure material and function materials development, fabrication of TBM mock-up, construction of the testing loops and so on.
- Still **many challenges** remain and need to be solved during the design phase.





# Thank you for your attention !

