

Development of a Systematic, Self-consistent Algorithm for the K-DEMO Steady-state Operation Scenario

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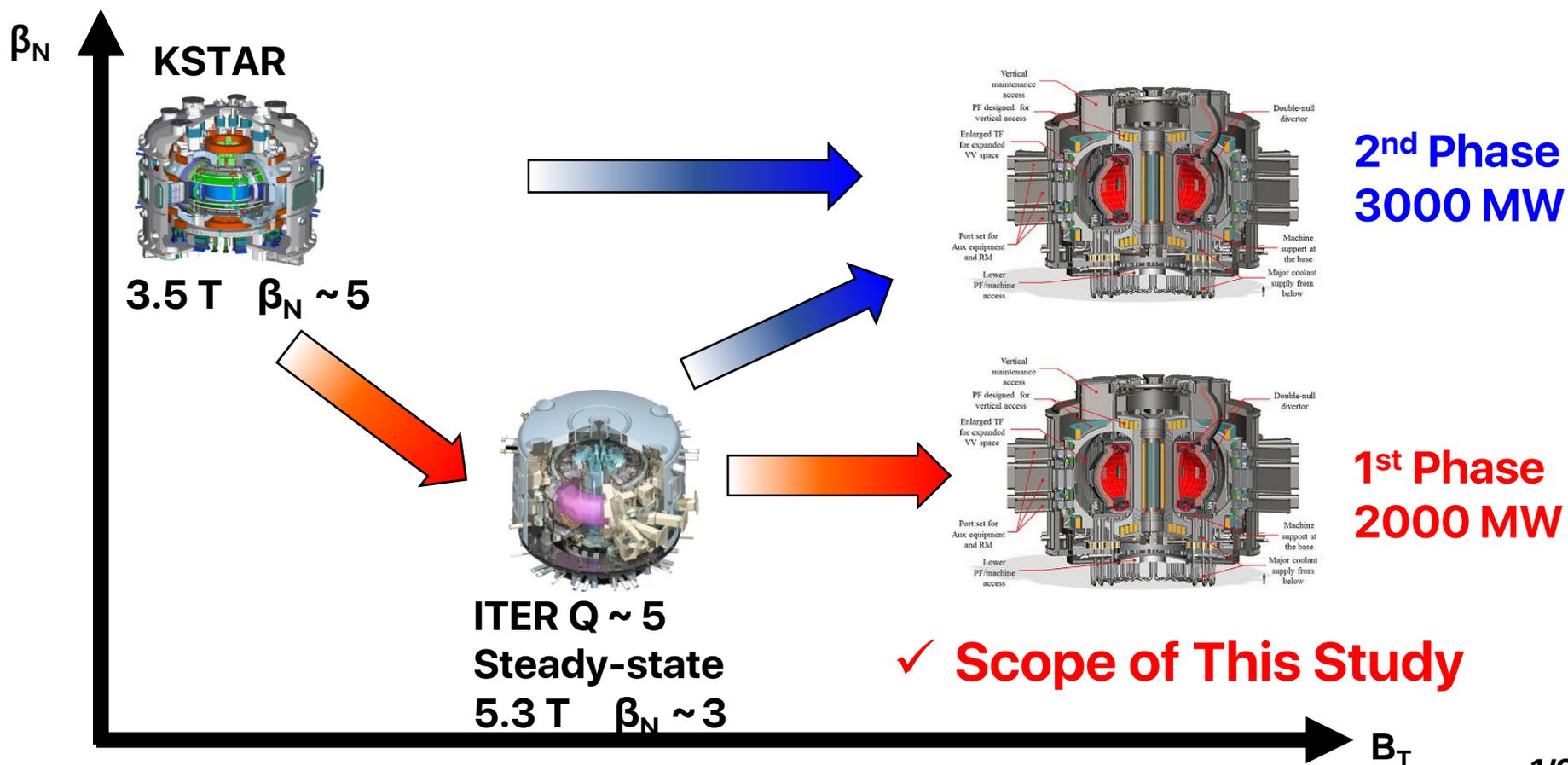
NFRI National Fusion
Research Institute

K-DEMO Goal: Demonstrate a Net Electricity Generation



- 1st phase High B_T
- 2nd phase High B_T & β_N

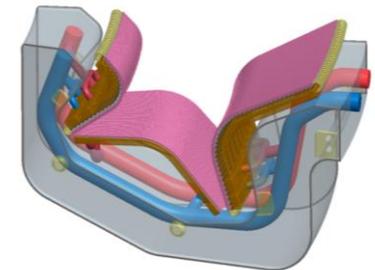
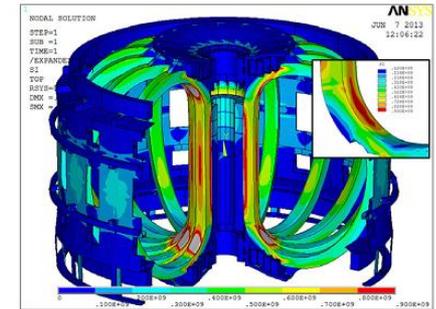
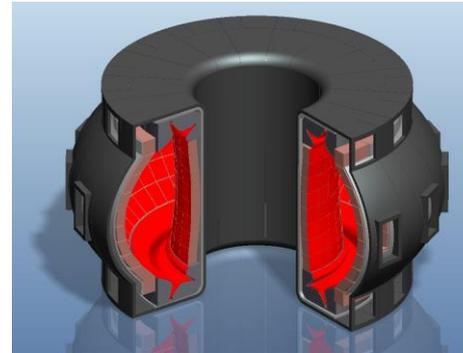
- Pressure Limit $\propto B_T \cdot \beta_N \cdot I_p$
- $f_{BS} \propto B_T \cdot \beta_N / I_p$



K-DEMO Design Parameters



Major Radius (R)	6.8 m
Minor Radius (a)	2.1 m
Toroidal Magnetic Field	7.4 T
Elongation / Triangularity	2.0 / 0.6
β_N	< 4
Fusion Power (P_F)	2000 MW (1 st) 3000 MW (2 nd)
Fusion Gain	20
Divertor Operation	Double-null



< K-DEMO Magnet & Divertor Analysis [1]>

- Size & aspect ratio similar to those of ITER & KSTAR.
- Nb₃Sn configuration/stress analysis/test fabrication.
- Tungsten mono-block type divertor, RAFM cooling tube, high pressure water-cooling.
- Ceramic pebble type breeder blanket, high pressure water cooling.

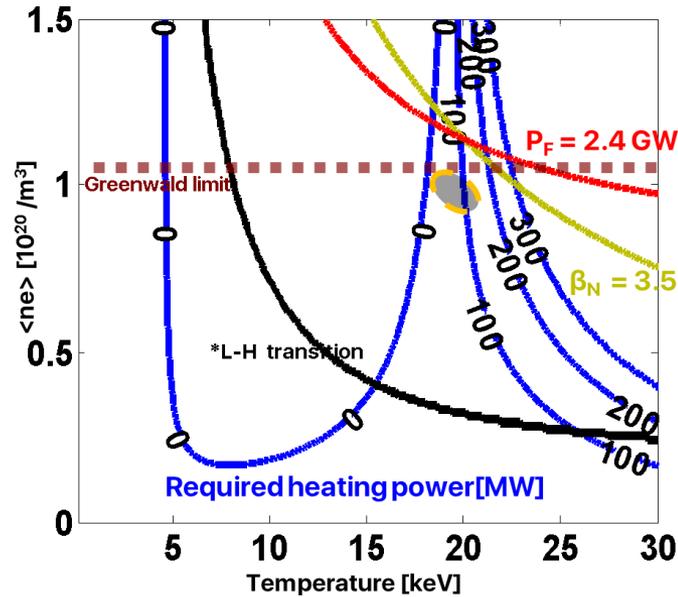
[1] Kim, K., et al, "Design concept of K-DEMO for near-term implementation", Nucl. Fusion 55 (2015) 053027.

0-D System Analysis to Identify Fusion Performance & Operating Regime



<Operation Regime for K-DEMO[1]>

[1] Kang, J.S., et al, Fusion. Eng. Des. 109-111, Part A (2016) 724.



✓ 0-D power balance

$P_F = 2000$ MW, $Q = 20$

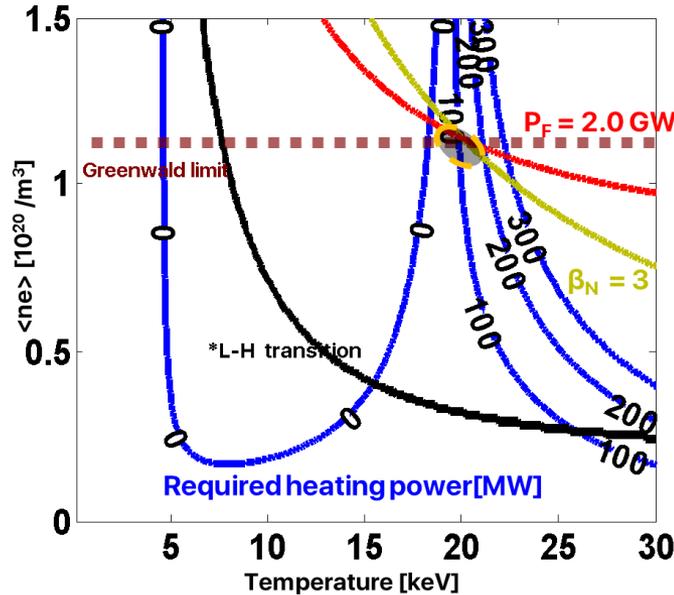
$n_{eavg} = 0.87 \times 10^{20} \text{ m}^{-3}$, $T_e = 20.0$ keV

0-D System Analysis to Identify Fusion Performance & Operating Regime



<Operation Regime for K-DEMO[1]>

[1] Kang, J.S., et al, Fusion. Eng. Des. 109-111, Part A (2016) 724.

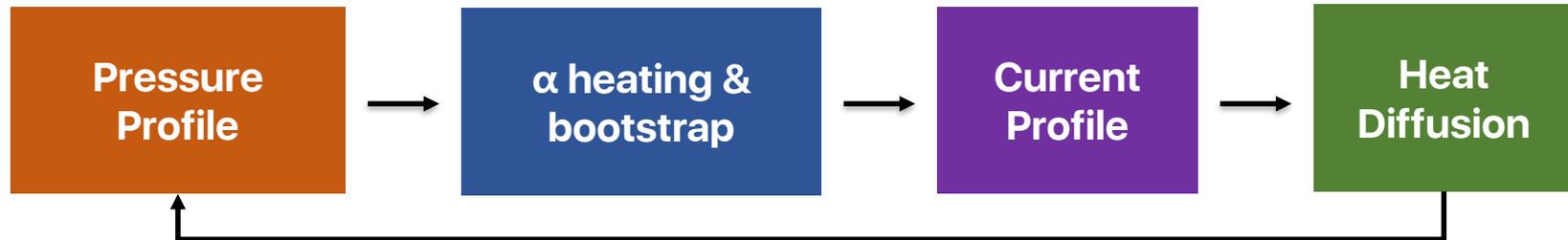


✓ **0-D power balance**

$P_F = 2000 \text{ MW}, Q = 19$

$n_{\text{eavg}} = 1.2 \times 10^{20} \text{ m}^{-3}, T_e = 20.0 \text{ keV}$

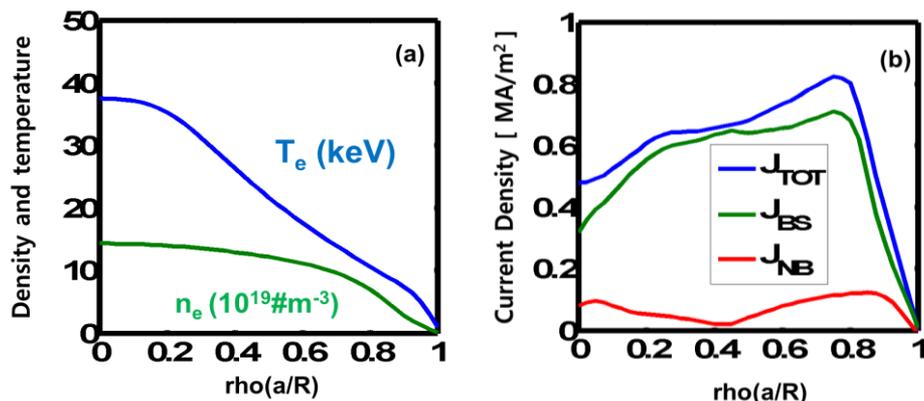
✓ **Burning plasma profile effect (self H/CD) change optimum operation regime.**



➔ **Need a self-consistent integrated modeling!**

- ✓ Target pressure and current profile are achieved intuitively.

<1-D KDEMO n, T, J profile[1]>



Modelling with prescribed heating schemes

- (1) 1 beam 1 MeV 100 MW on-axis
- (2) 2 beams, on-axis (1.5 MeV 50 MW) off-axis (0.6 MeV 50 MW)
- (3) 2 beams, on-axis (1.3 MeV 40 MW) off axis (0.8 MeV 80 MW)
- (4) ■
- (5) ■
- (6) ■

Try ⇒ Select the maximum fusion gain one

➔ **Systematic approach is required!**

A Systematic Algorithm for K-DEMO Burning Plasma Design is Developed



- Find p & j profiles and corresponding H&CD specification to maximize Q , systematically and self-consistently.

$$f(p, j) = \max \{Q\} \quad p: \text{pressure profile} \quad j: \text{current profile}$$

High performance steady state operation needs optimization of P&J profiles.

Burning plasma algorithm must integrate various physics/engineering.

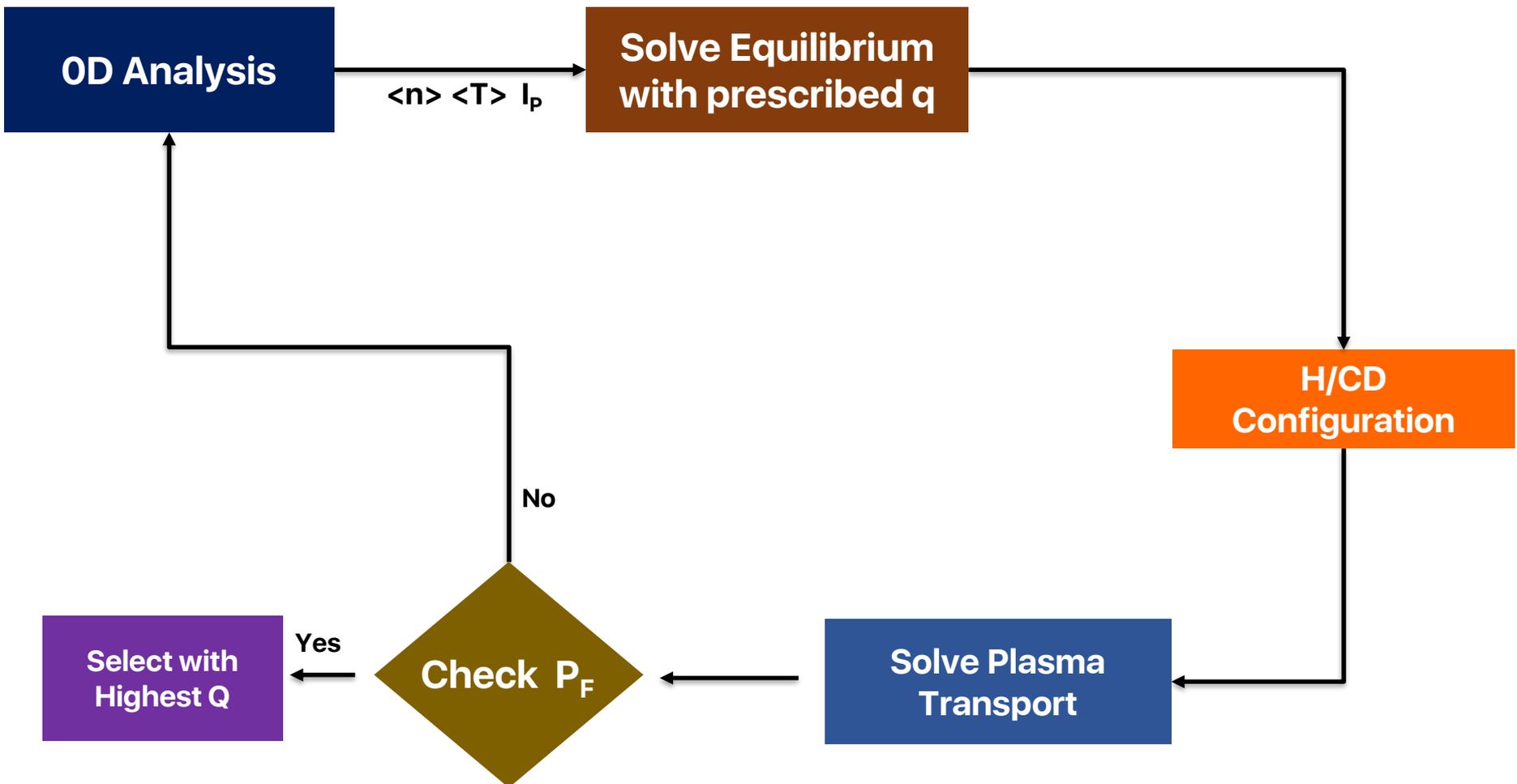
Optimization Algorithm Consider Constraints

Confinement

MHD stability

H/CD

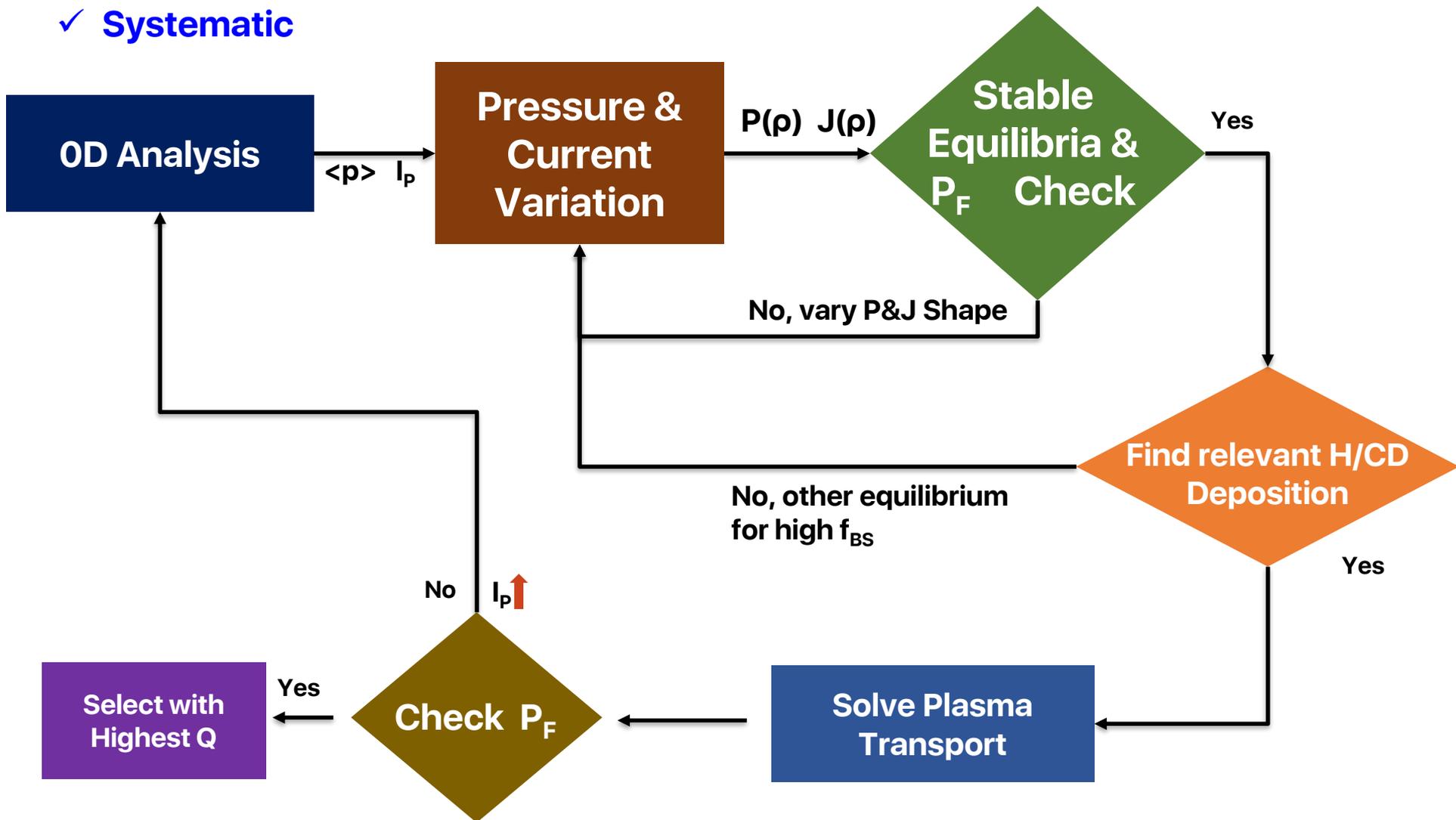
Previous Flowchart of K-DEMO Target P&J



K-DEMO Burning Plasma Target Scenario Algorithm Flowchart



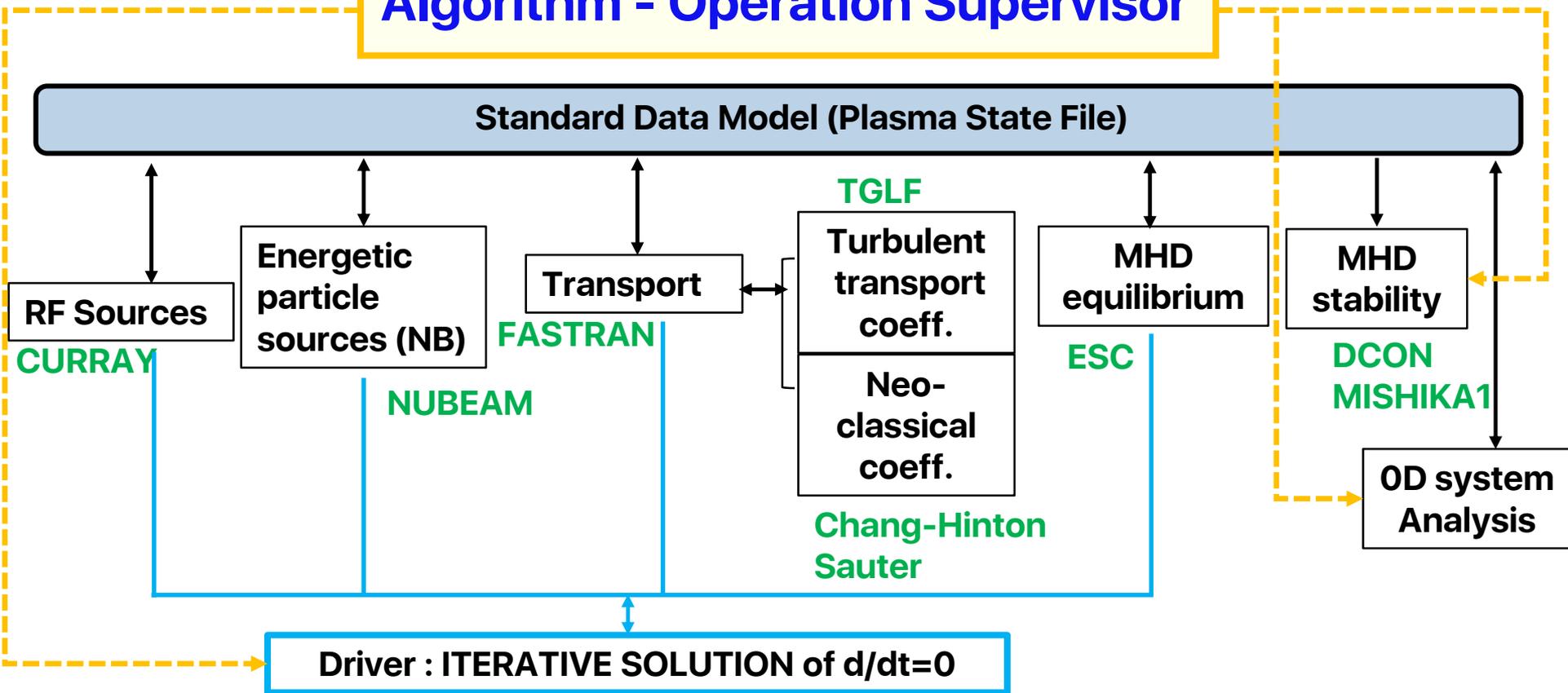
- ✓ Confinement, Stability, and H/CD are simultaneously satisfied.
- ✓ Systematic



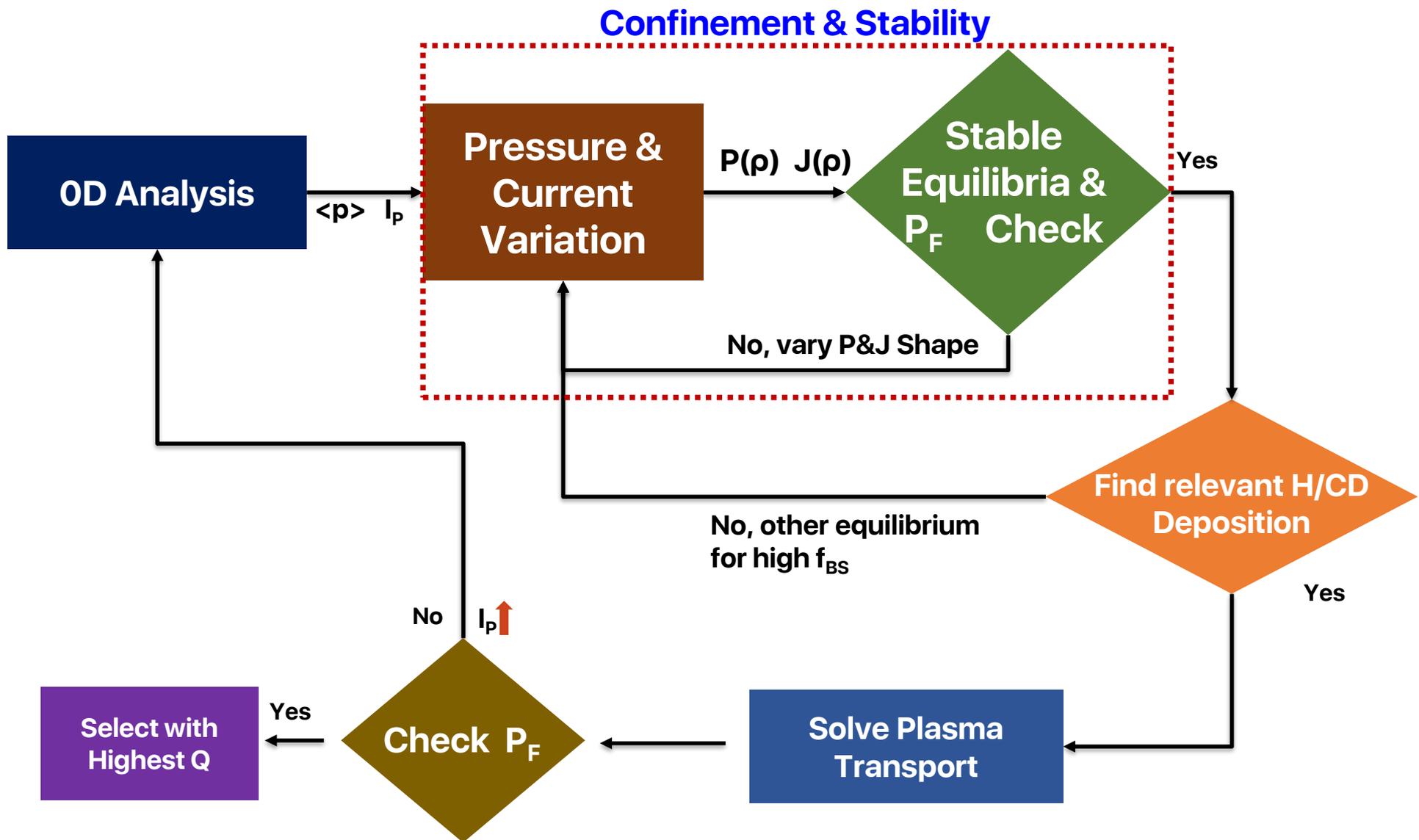
FASTRAN/IPS is Adopted for Numerical Apparatus to Implement Algorithm



Algorithm - Operation Supervisor

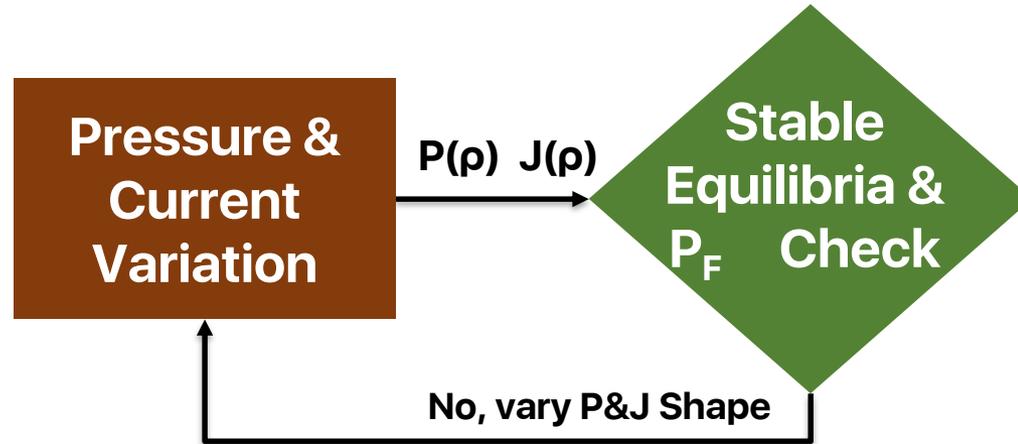


Loop 1: Find Stable Equilibrium Satisfying P_F

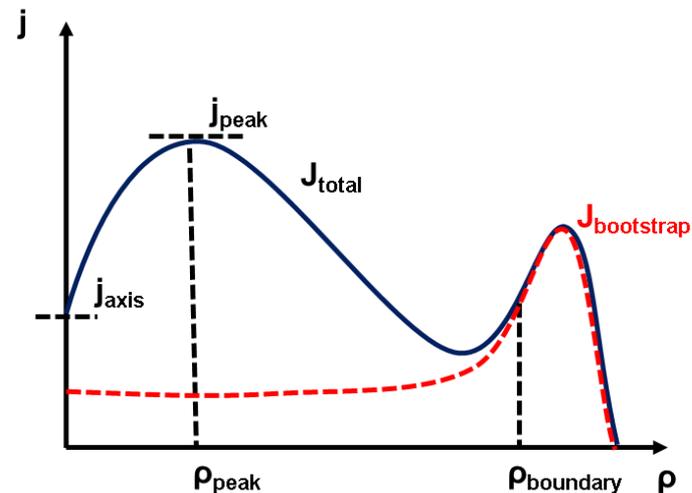
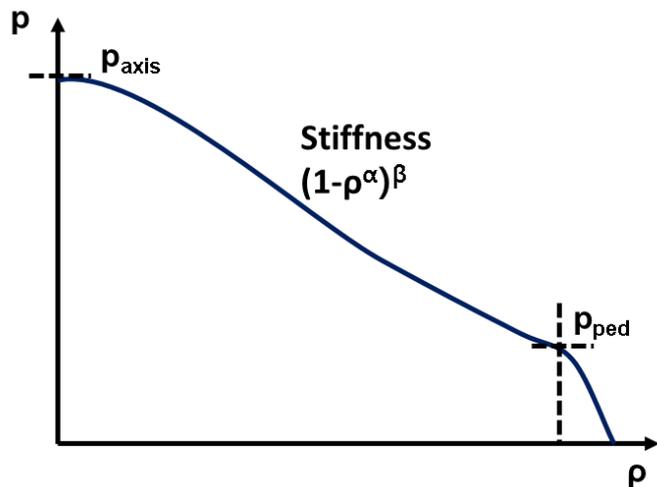


Loop 1: Find Stable Equilibrium Satisfying P_F

Target P&J Profiles are Explored.



Control Knobs – p_{axis} , α , β , p_{ped} , j_{axis} , j_{peak} , ρ_{peak} , $\rho_{boundary}$

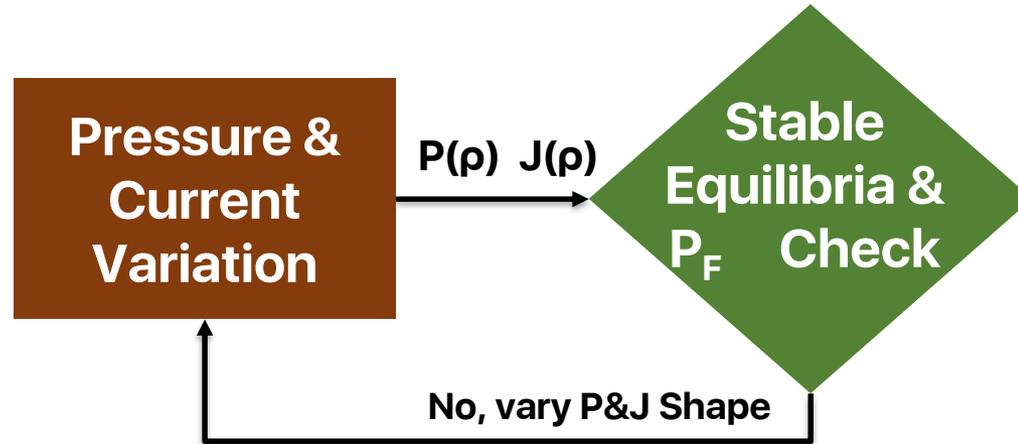


$\Delta p_{ped} = 0.1$ benchmark from ITER.

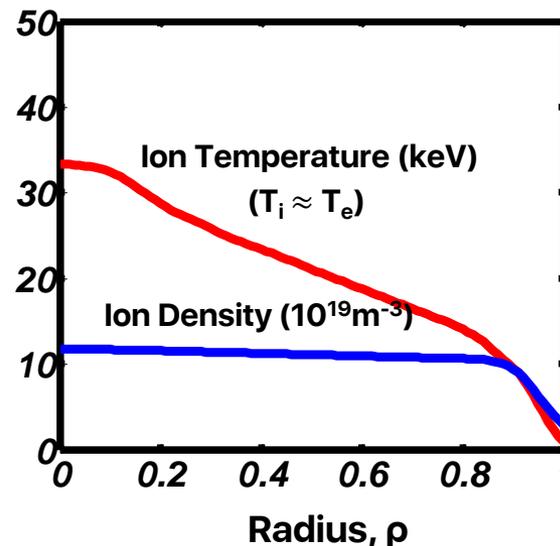
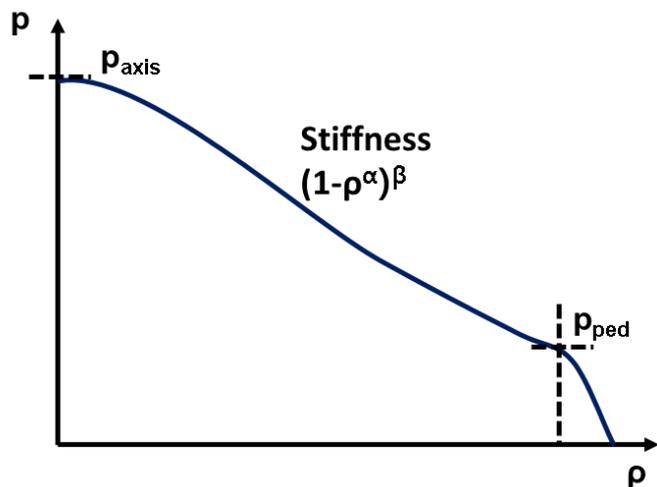


Loop 1: Find Stable Equilibrium Satisfying P_F

Target P&J Profiles are Explored.



✓ P_F Evaluation with T_i / n_i

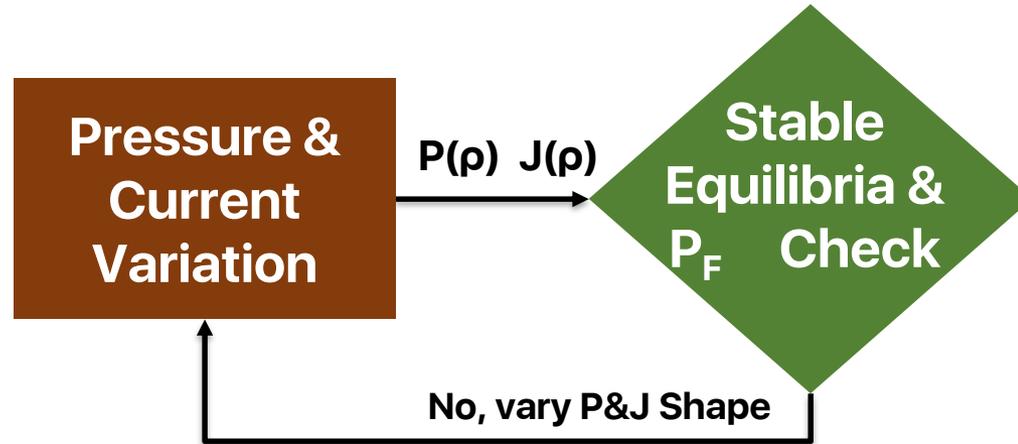


$Z_{eff} Z_{imp} f_{imp}$ from ITER

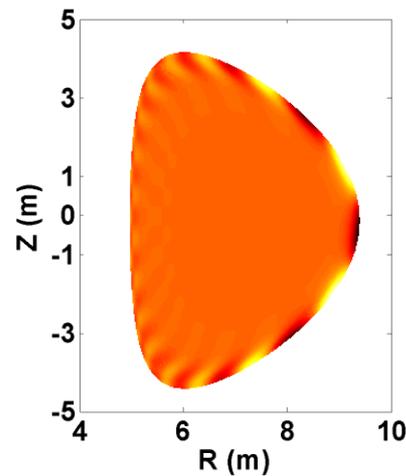
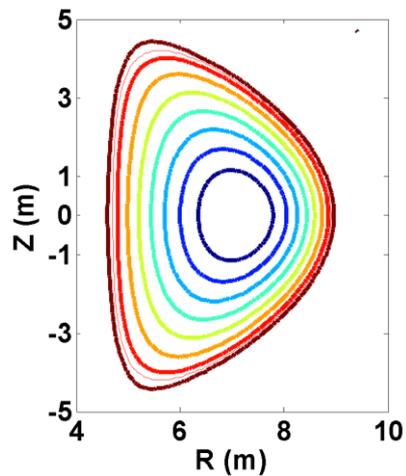
Prescribed n_e ($f_{GW} = 1$)
 $n_{axis}/n_{ped} = 0.8$
 $n_{ped}/n_{sep} = 0.3$

Loop 1: Find Stable Equilibrium Satisfying P_F

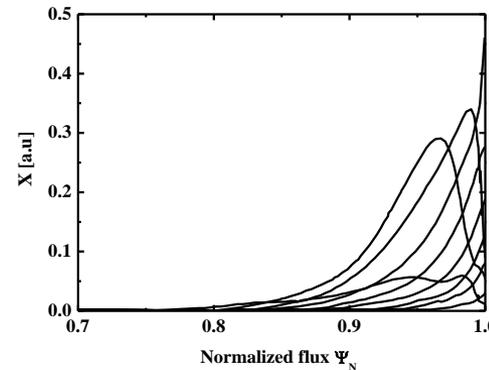
Target P&J Profiles are Explored.



✓ Linear Ideal MHD Evaluation with DCON[1] & MISHIKA1[2]



- $T_{ped} < \text{ideal MHD P-B mode [3] limit.}$
- $n = 3$ mode structure.

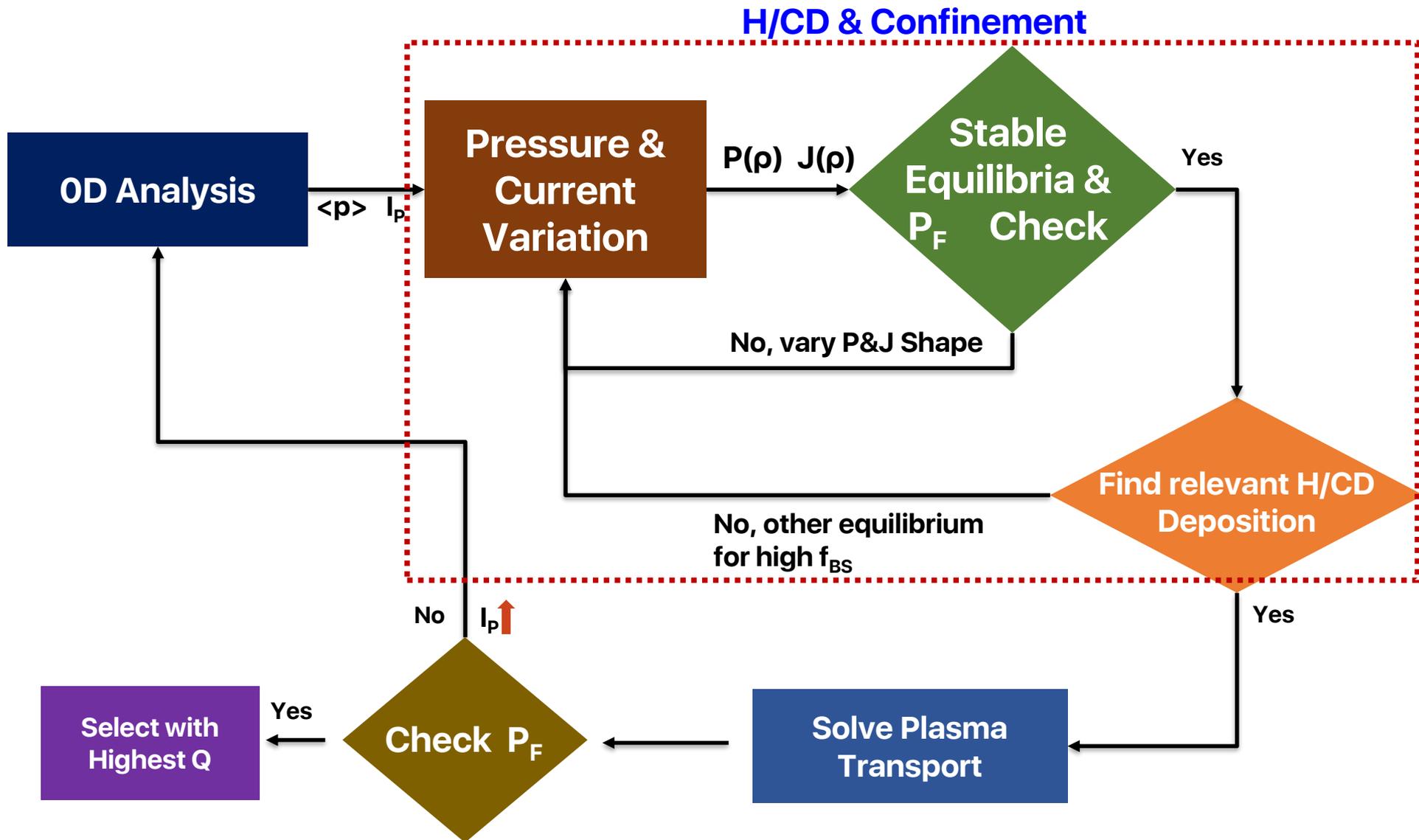


[1] Glasser, A. H., et al., Bulletin of the American Physical Society Vol. 42, p. 1848 (1997).

[2] Mikhailovskii, A. B., et al., Plasma physics reports, 23(10), p. 844 (1997).

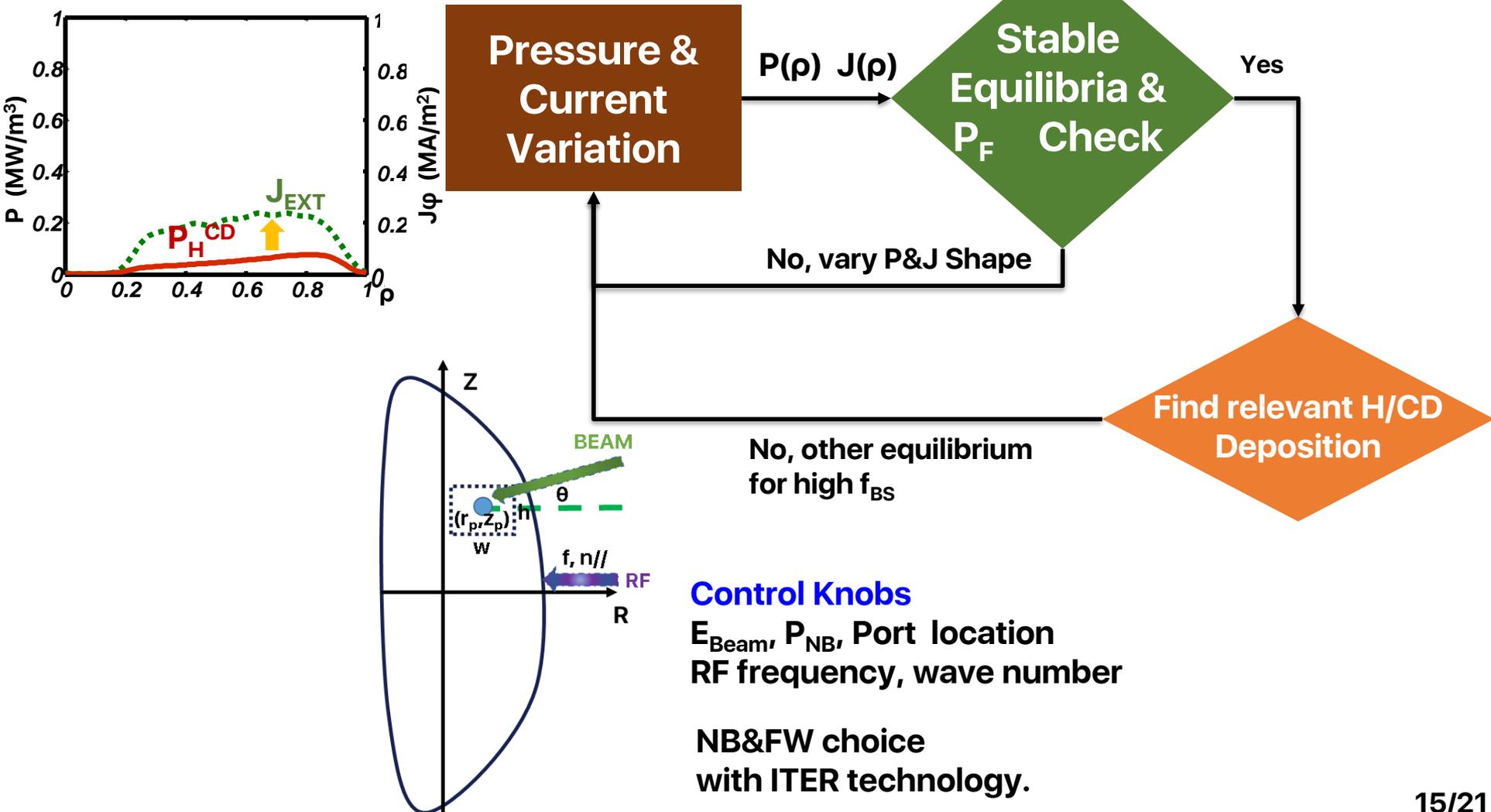
[3] Snyder, P. B., et al., Physics of Plasmas 9 (2002) 2037.

Loop 2: Determine External H/CD Configuration



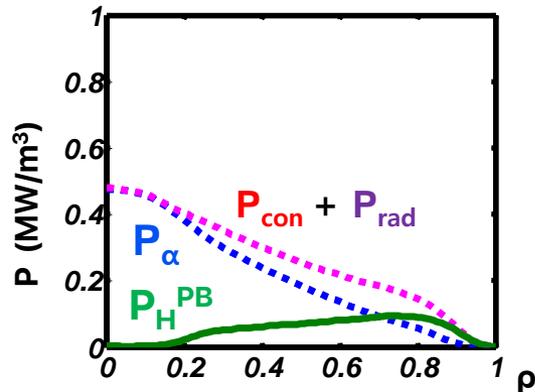
Loop 2: Determine External H/CD Configuration

1. Scanning H/CD control knobs to match $J_{EXT} = J_{TOT} - J_{BS}$



Loop 2: Determine External H/CD Configuration

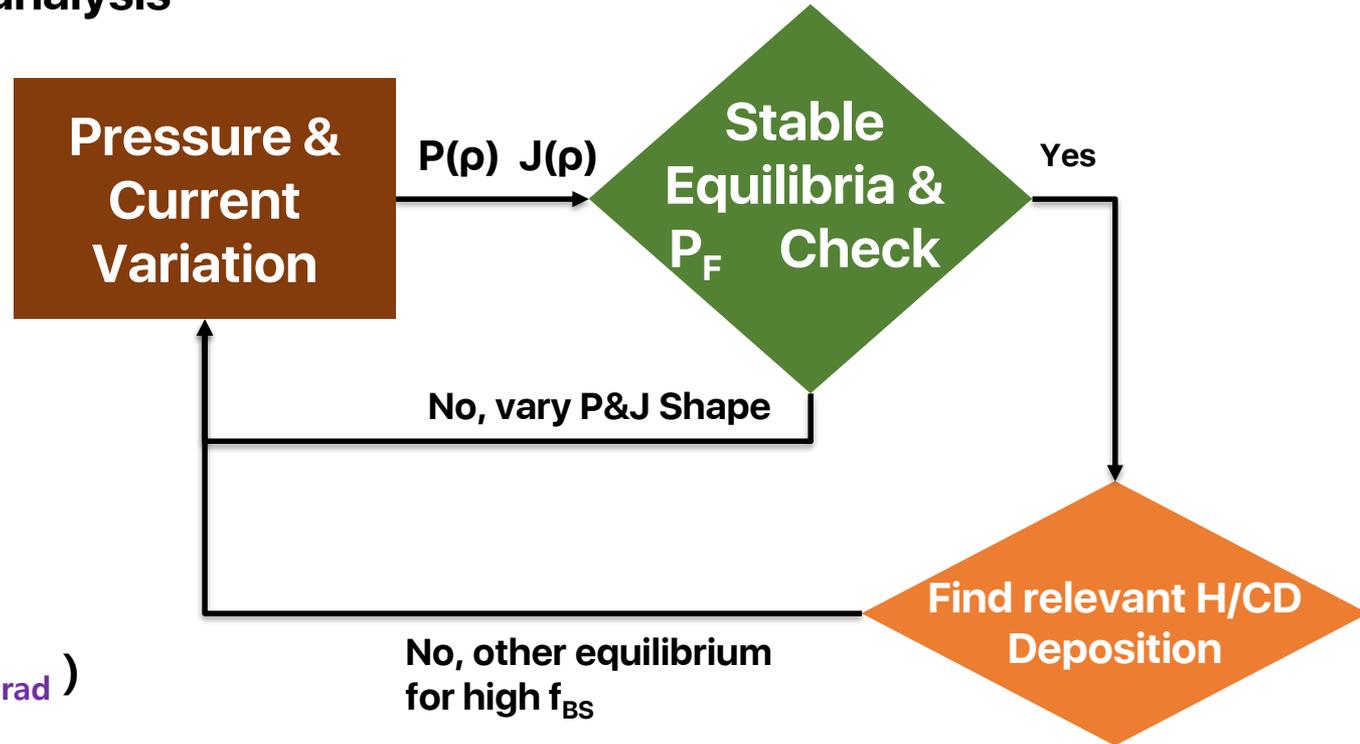
2. 1-D power balance analysis



Calculate P_H^{PB} to match 1-D power balance.

$$P_H^{PB} = P_\alpha - (P_{con} + P_{rad})$$

P_{con} from diffusion model & n, T profile
 P_α P_{RAD} from n, T profile (Loop 1 Result)



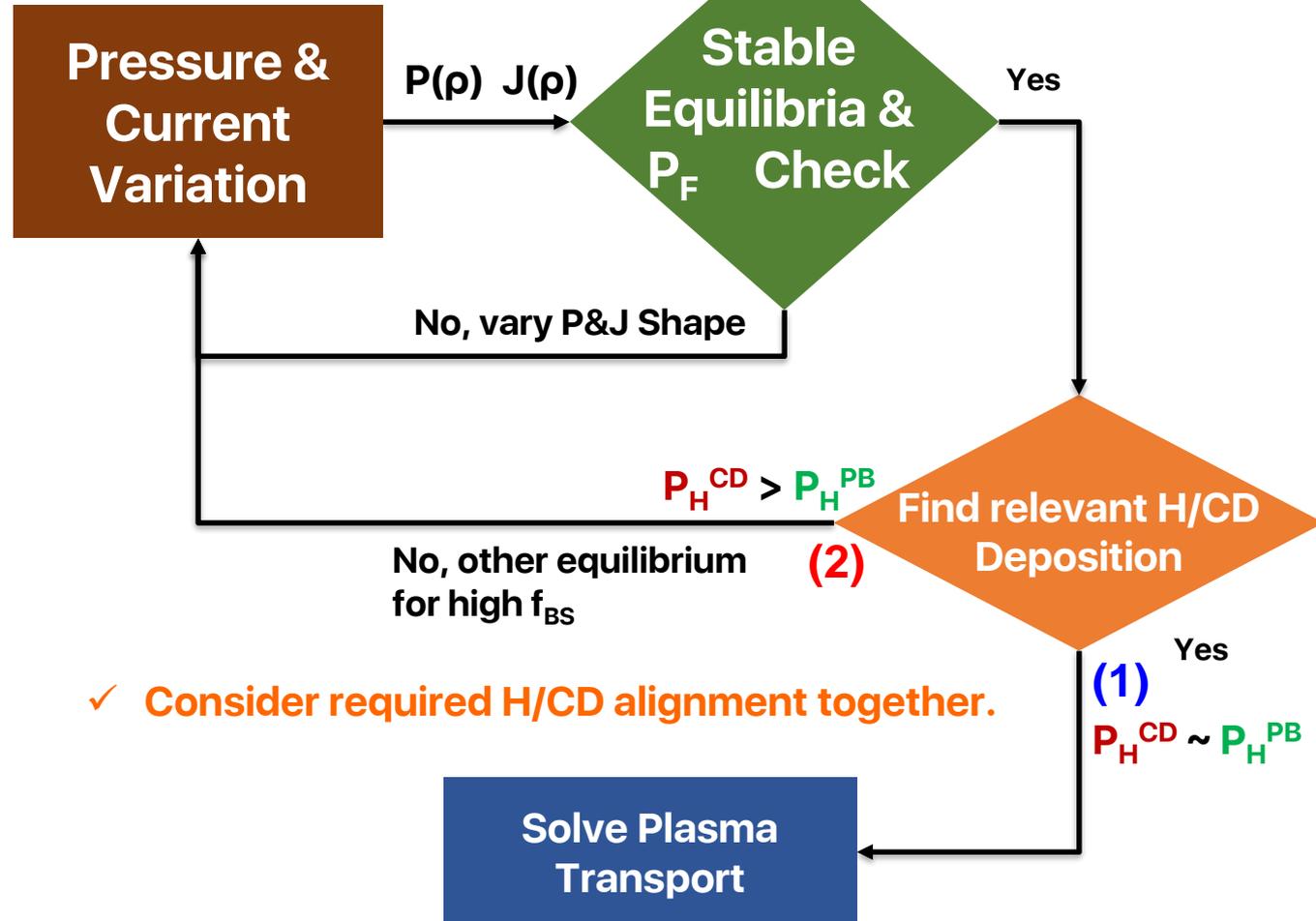
Loop 2: Determine External H/CD Configuration

3. Compare P_H^{CD} & P_H^{PB}

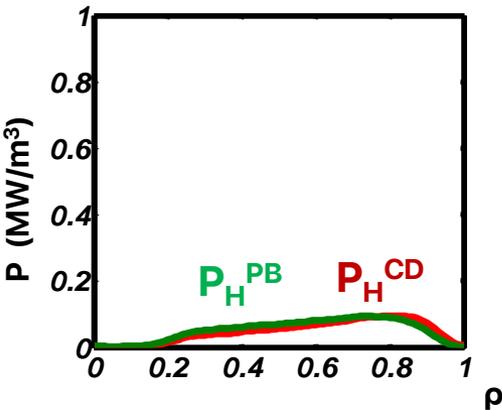
(1) $P_H^{CD} \sim P_H^{PB}$

(2) $P_H^{CD} > P_H^{PB}$

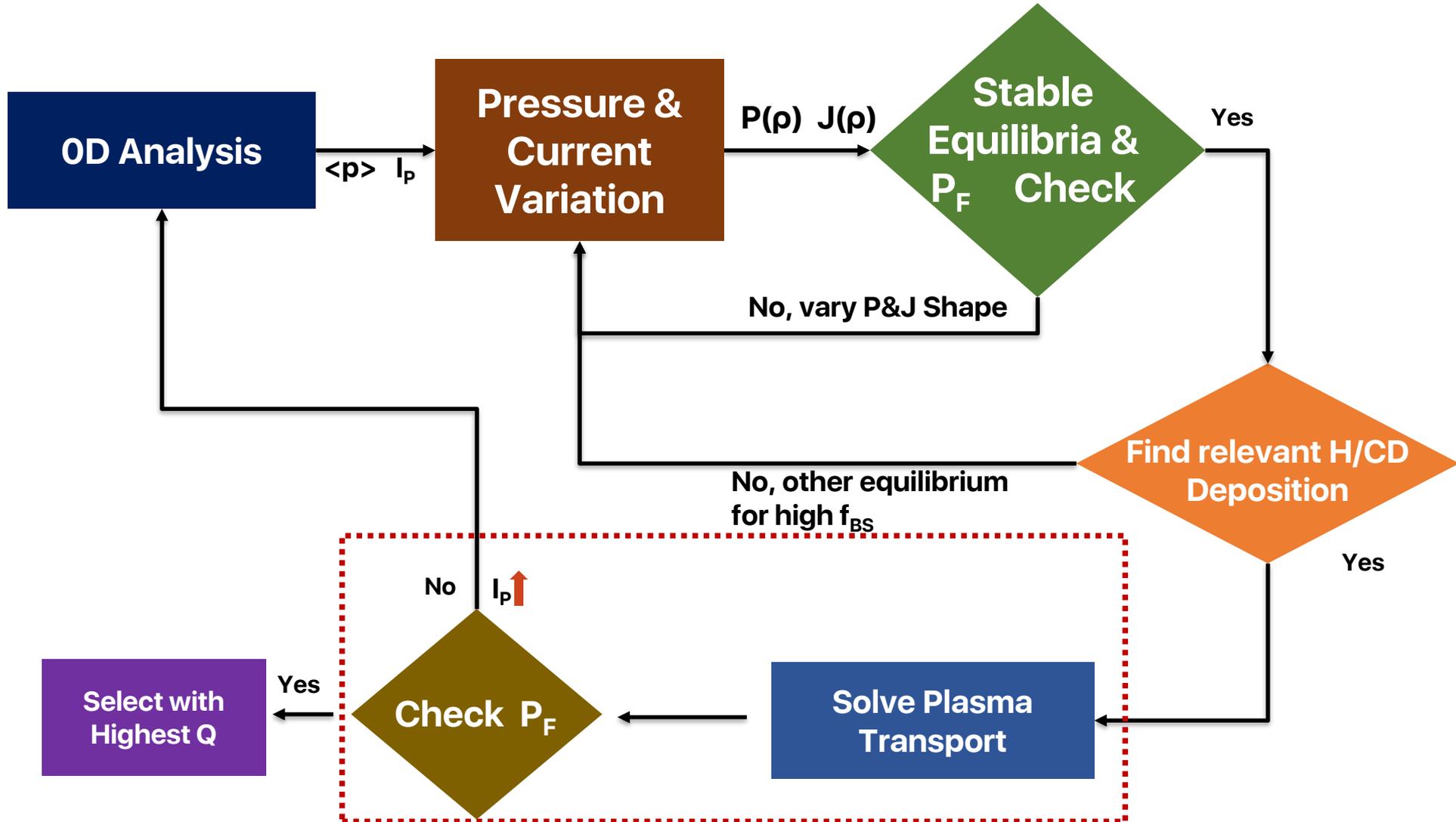
(3) $P_H^{CD} < P_H^{EXT}$ (Discard)



✓ Consider required H/CD alignment together.



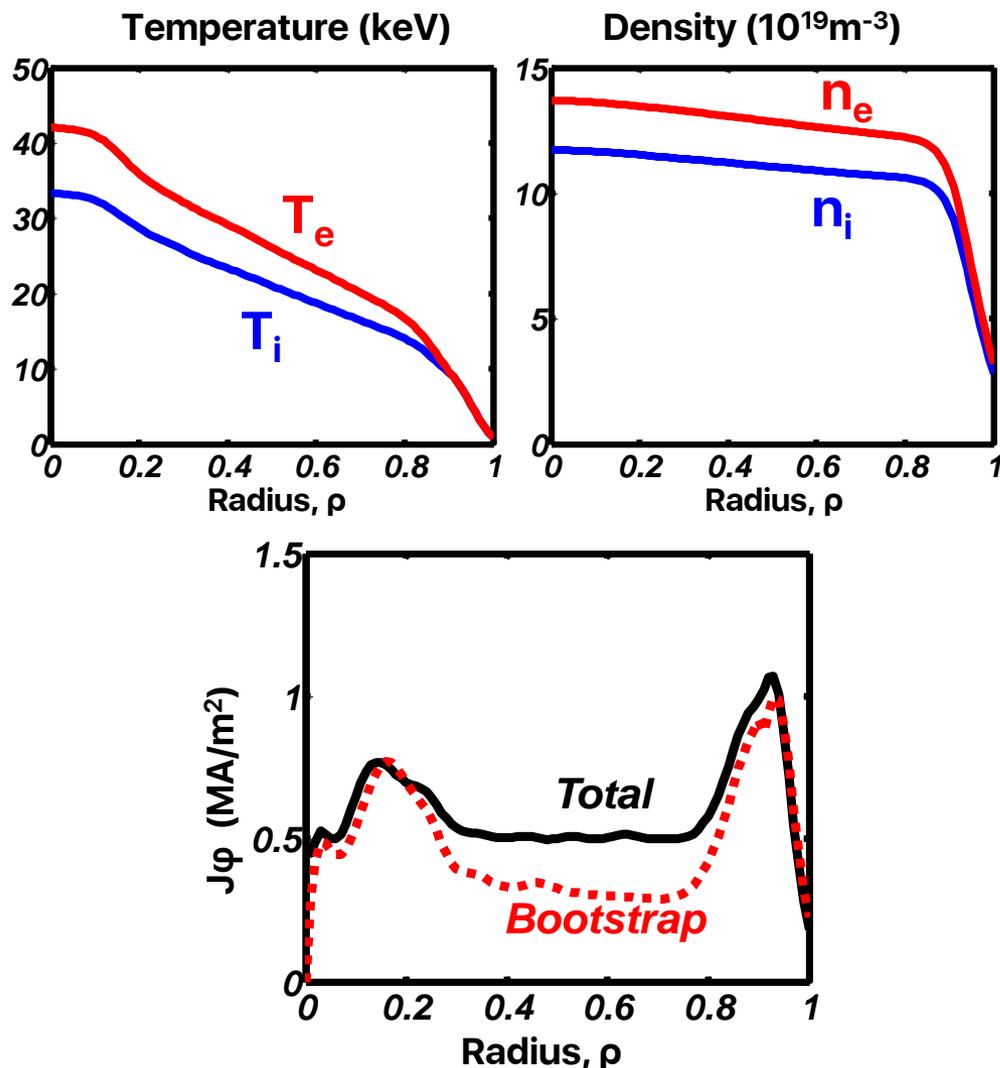
Loop 3: Check Self-consistency between Plasma Profiles and H/CD Configuration by Integrated Modelling



An Optimum Target P&J is Attained through Algorithm with Integrated Transport Modeling

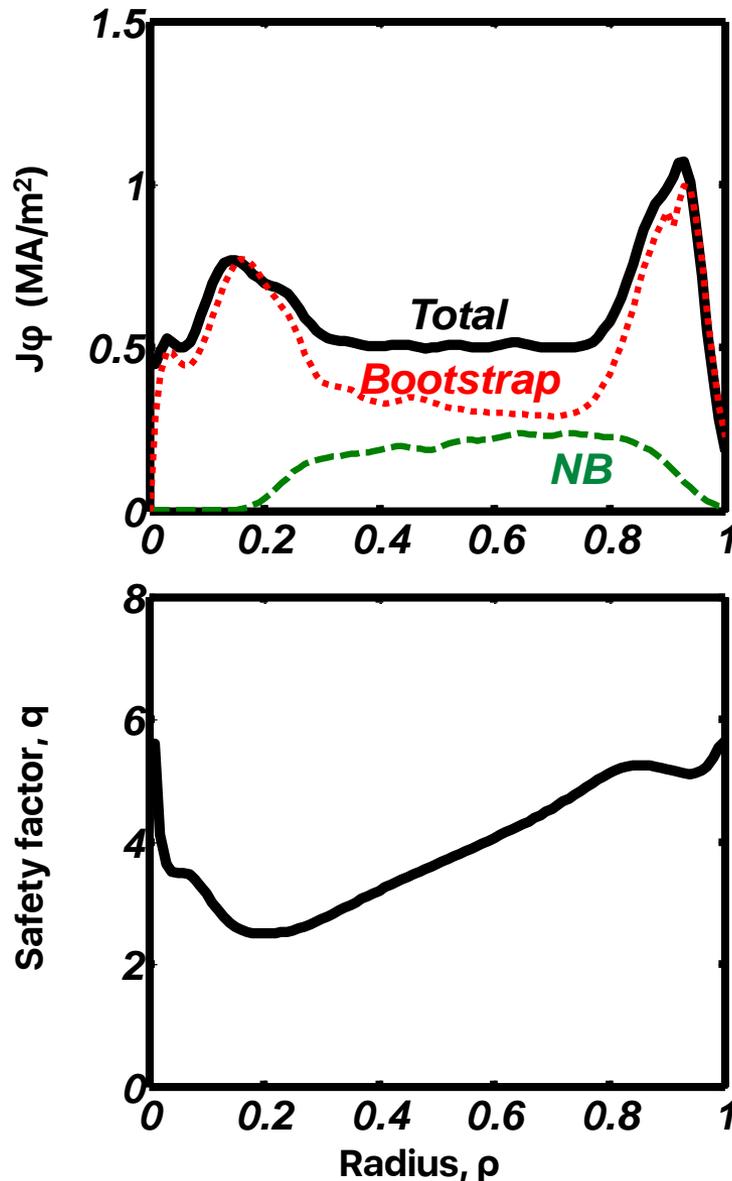


A fully non-inductive target plasma is finally achieved.



	Result	0 D
P_F (MW)	2080	2000
Q (Fusion Gain)	18.8	20
P_{NB} (MW)	110	100
I_p (MA)	15.5	12
f_{BS}	77 %	75 %
β_N	2.8	3.10
H_{98y2}	1.2	1.3
T_{ped}	8.3 keV	
n_{ped}	$9.9 \cdot 10^{19} \text{ m}^{-3}$	

Discussion of Derived H/CD Specification



- **Off-axis NB provide broad J Profile**
 - improved confinement.
- **Single 110MW 500keV NB injection**
 - Alignment with less-bootstrap region.
 - Even half of ITER NBI beam energy.
- **$q_{\min} > 2$ is set to avoid tearing modes.**

Conclusion & Future Work

- **A systematic scenario optimization algorithm maximizing the fusion gain is newly established by integrating equilibrium, confinement, stability, and current drive requirement, self-consistently.**
- **Target pressure & current profile for K-DEMO is derived with designed algorithm.**
 - ✓ fusion power of 2080 MW, fusion gain Q of 18.8 and normalized beta β_N of 2.81.
- **Particle Transport/plasma rotation/more detailed stability are planned.**
 - ✓ Particle transport and plasma rotation are also key control knobs for stable fusion power production and they will be updated to the systematic algorithm in the near future.
 - ✓ Pedestal structure and plasma shaping optimization are expected to address more efficient/accurate target scenario.

