

Overview of EAST Experiments on the Development of High-Performance Steady- State Scenario

**B.N. Wan on behalf of
EAST team & collaborators**

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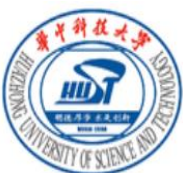
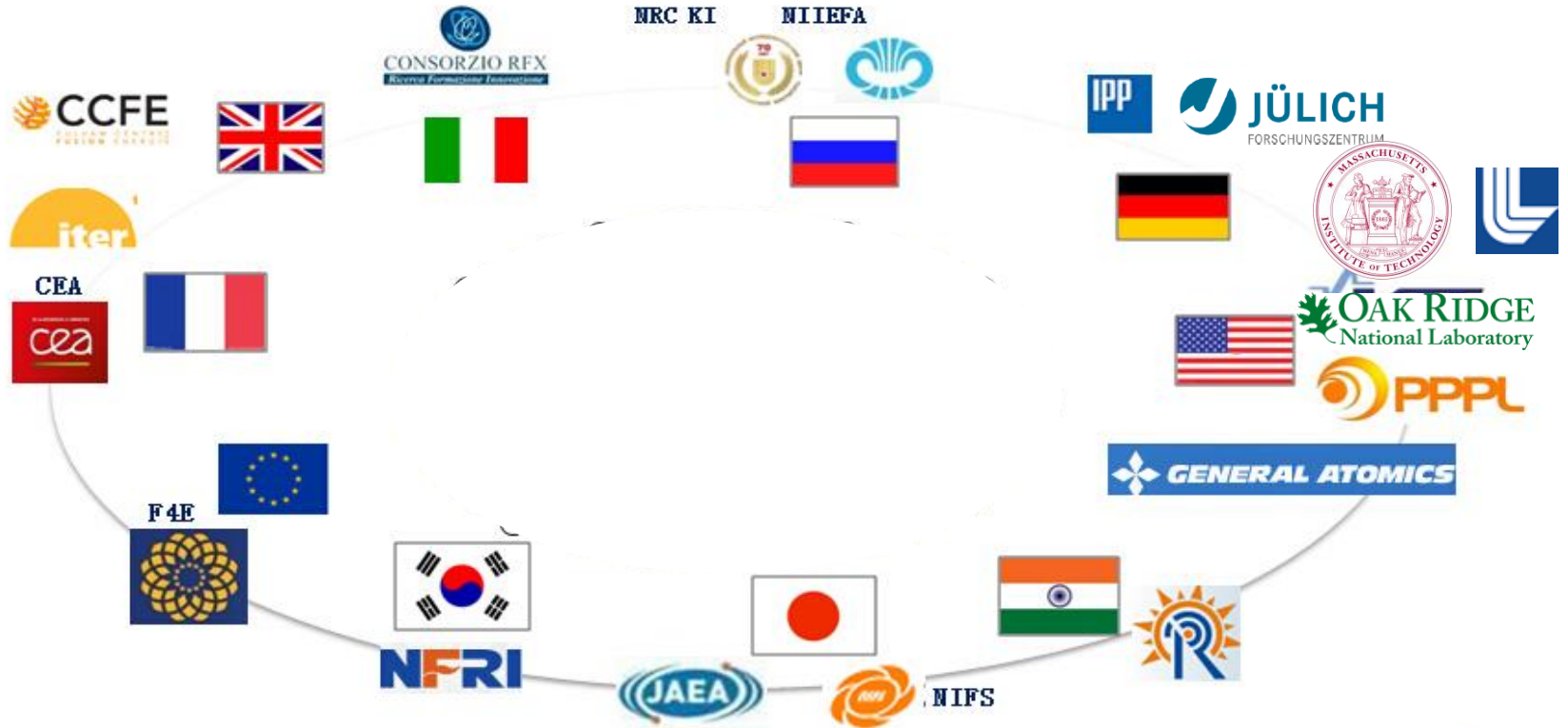
Institute of Plasma Physics, Chinese Academy of Sciences



Collaborators



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Scenario Developments on EAST



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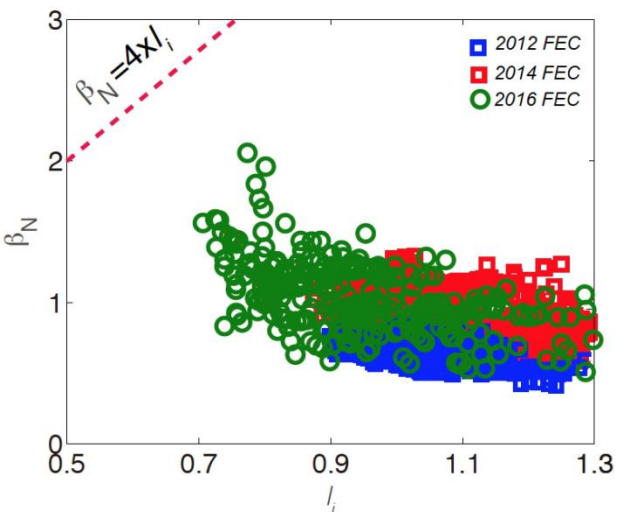
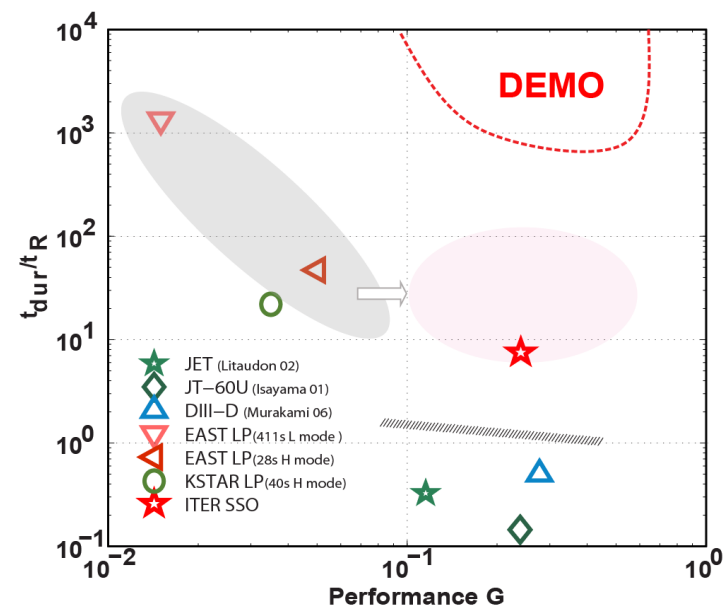
Step I

Improve heating efficiency and develop the SSO-relevant fundamental physics and key diagnostics;



Step II

Develop the SSO high performance plasma scenarios and demonstrate (≥ 100 s) long-pulse H-mode plasmas;



Step III

Optimize the SSO plasma and extend the EAST operation domain towards long-pulse, high beta, high power, high performance regime.

- **Introduction**
 - *Facility upgrade in support of steady-state long-pulse scenarios*
- **Exploration of Steady-State Plasma Operation with ITER-like Tungsten Divertor**
 - *Scenario development with RF dominated heating*
 - *Hot spot issues*
- **Progress of Key Physics Issues towards Steady-State Operation Regimes**
 - *LHCD at high density*
 - *RMP ELM control*
 - *Particle/power exhaust control*
 - *MCM physics at low collisionality*
- **Summary and Future Plans**

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- ◆ LHCD 4+6 MW (2.45/4.6GHz)
 - Fast Electron Source
 - Edge Current Drive /Profile
- ◆ ICRH 6+6 MW (25-75MHz)
 - Ion and Electron Heating
 - Central Current Drive
- ◆ **ECRH 2(4) MW (140GHz)**
 - **Dominant electron heating**
 - **Steering mirror, j_ϕ tailoring**
- ◆ NBI 4+4 MW (co/counter, 80kV)
 - Sufficient power to probe β limit
 - Variable rotation/ rot-shear

- **Elevated capabilities in last two years allow EAST to play a key role for developing advanced SS scenarios**
 - Fully non-inductive CD, high bootstrap current fraction (f_{bs}).
 - Active control of ELM and stationary heat load on divertors

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Fully non-inductive, high β_P long-pulse H-mode operations



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➤ The goal is to develop fully non-inductive scenarios

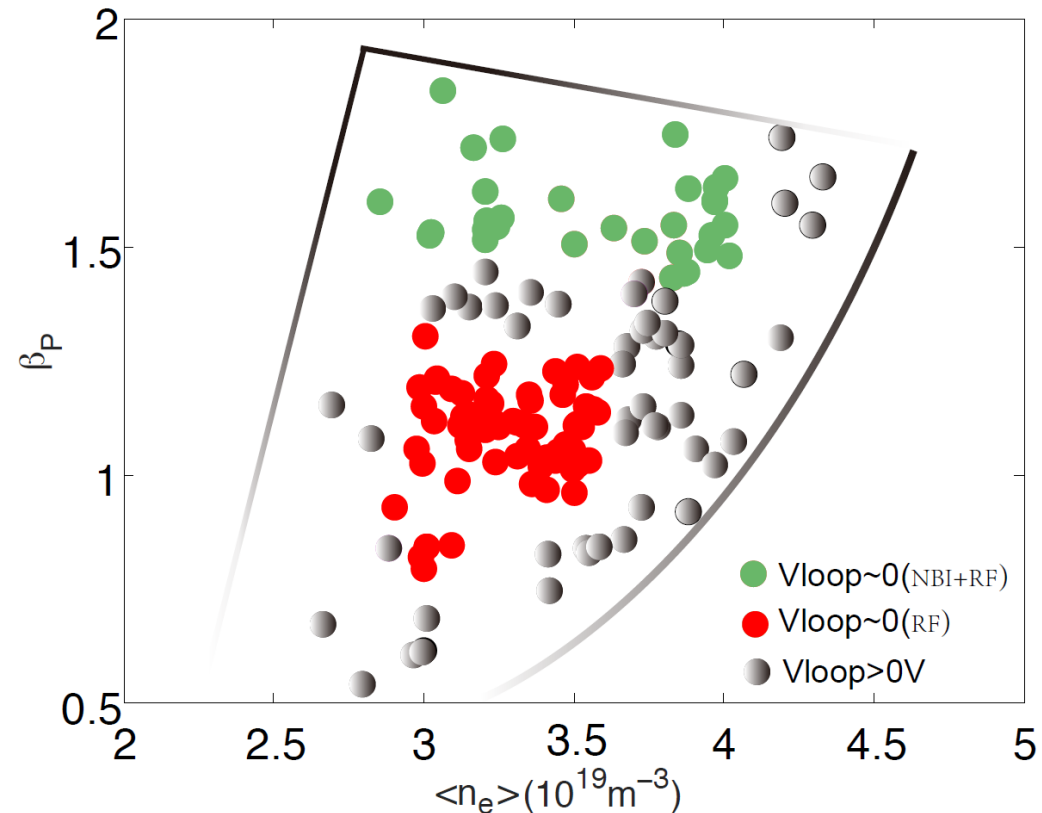
◆ High LHCD with moderate f_{bs}

➤ Recent EAST results show that

◆ Zero loop voltage is achieved at moderate density

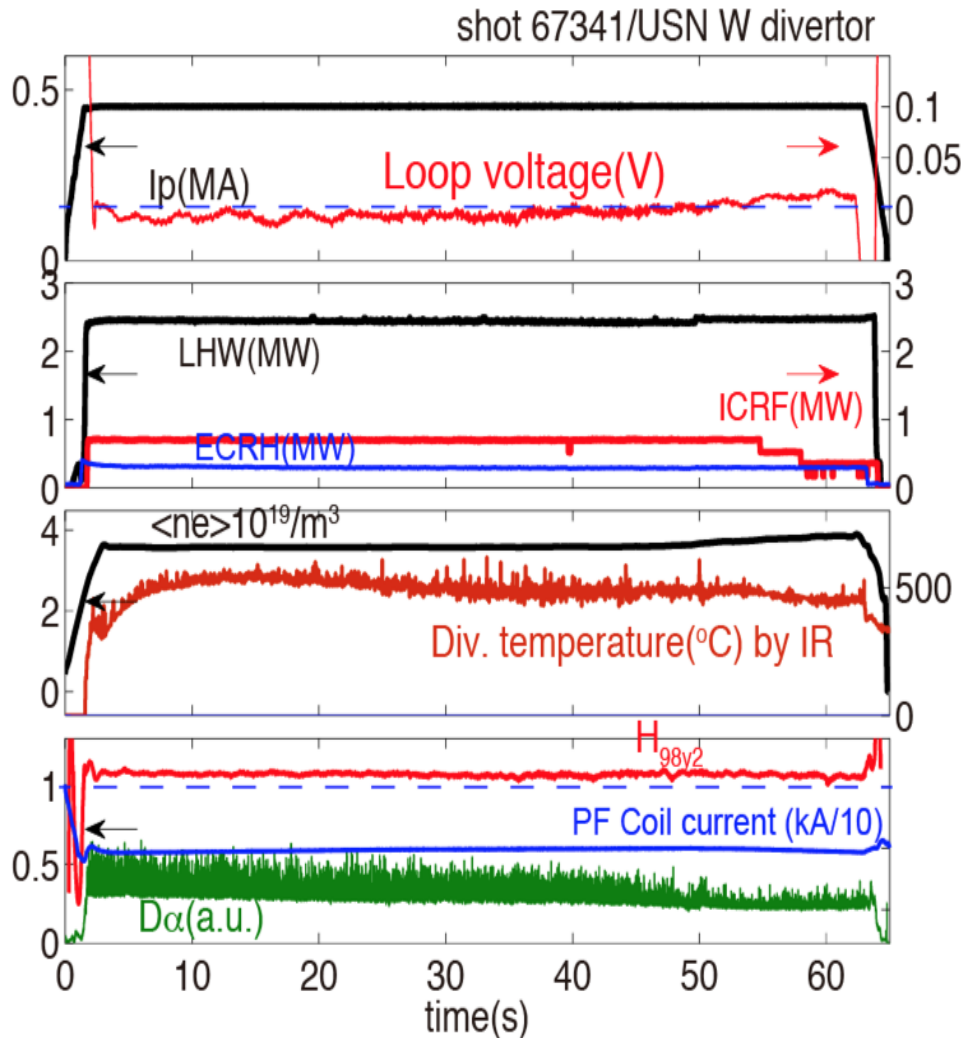
◆ Extension for ITER and CFTR

◆ Optimization of P_{CD} , η_{CD} and f_{bs}



Scatter plot of EAST β_P versus line-averaged density of low loop voltage plasmas

Minute-scale H-mode operation (>60s)!



➤ **Pure RF heating:**

$$P_{\text{LHW}, 2.45\text{GHz}} = 0.4\text{MW}$$

$$P_{\text{LHW}, 4.6\text{GHz}} = 2.1\text{MW}$$

$$P_{\text{ICRF}} = 0.8\text{MW},$$

$$P_{\text{ECRH}} = 0.3\text{MW};$$

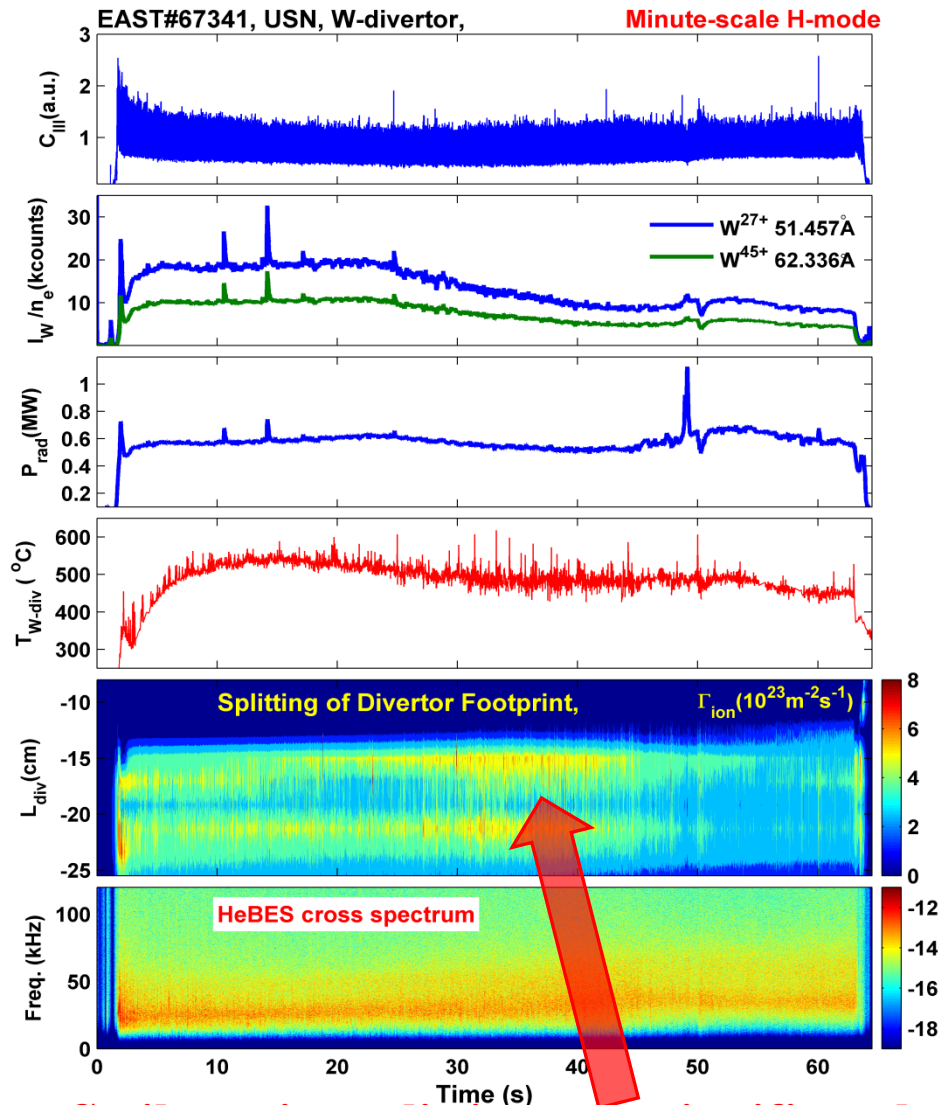
➤ **Good confinement with**

$$H_{98(y2)} \sim 1.1;$$

➤ **Good control of impurity level- assisted by ELMs and ECRH, and an edge coherent mode;**

➤ **Inter-ELM divertor heat flux $\sim 3 \text{ MW/m}^2$**

Minute-scale H-mode operation (>60s)!



Strike point splitting can significantly reduce peak heat load!

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$$P_{LHW, 4.6GHz} = 2.1MW$$

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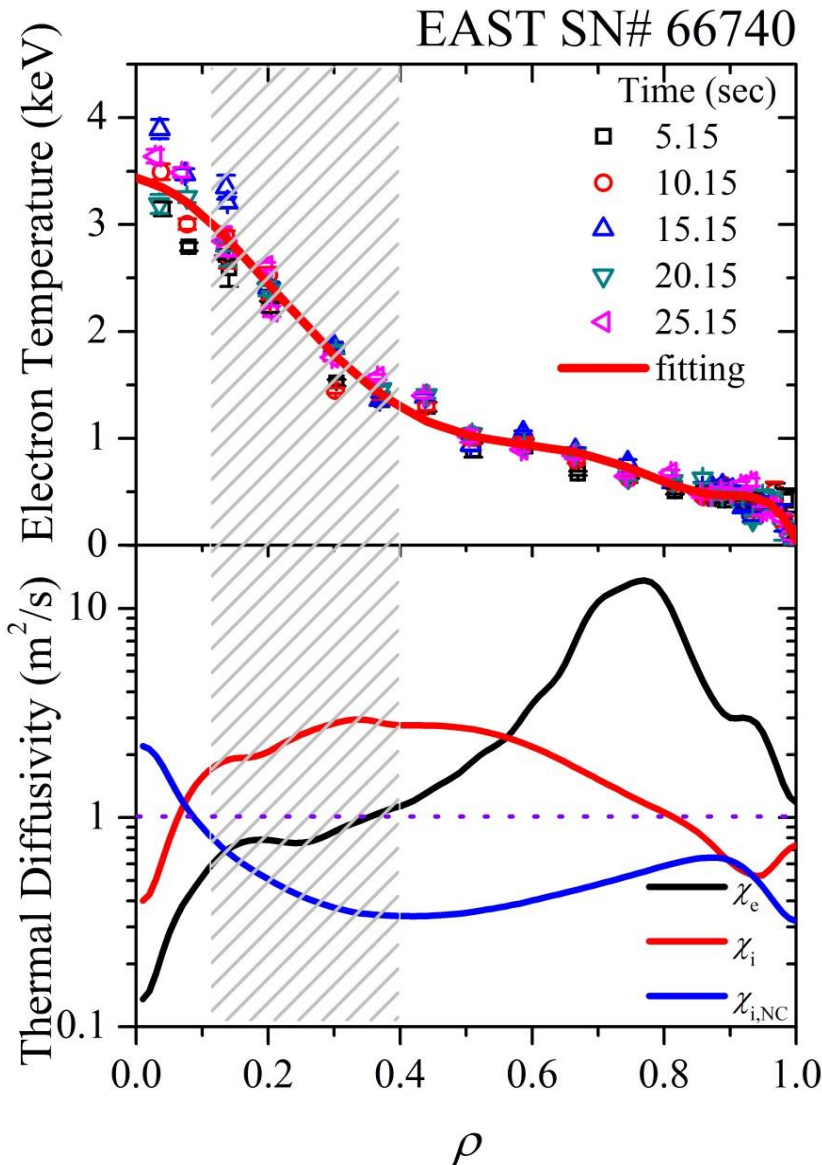
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Core confinement for RF heated long-pulse fully non-inductive H-mode plasmas



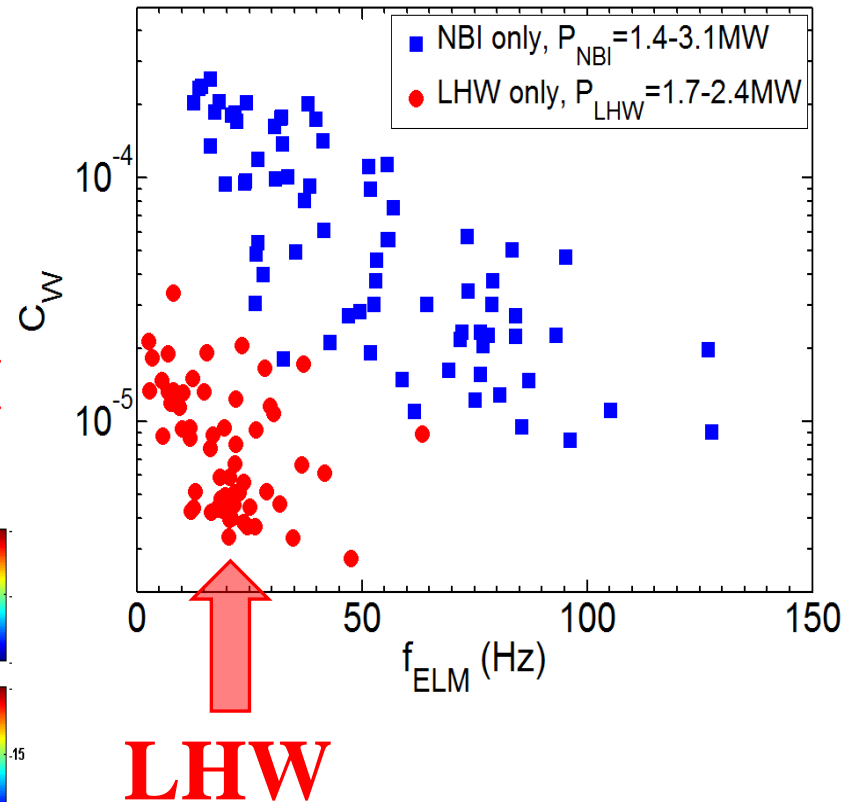
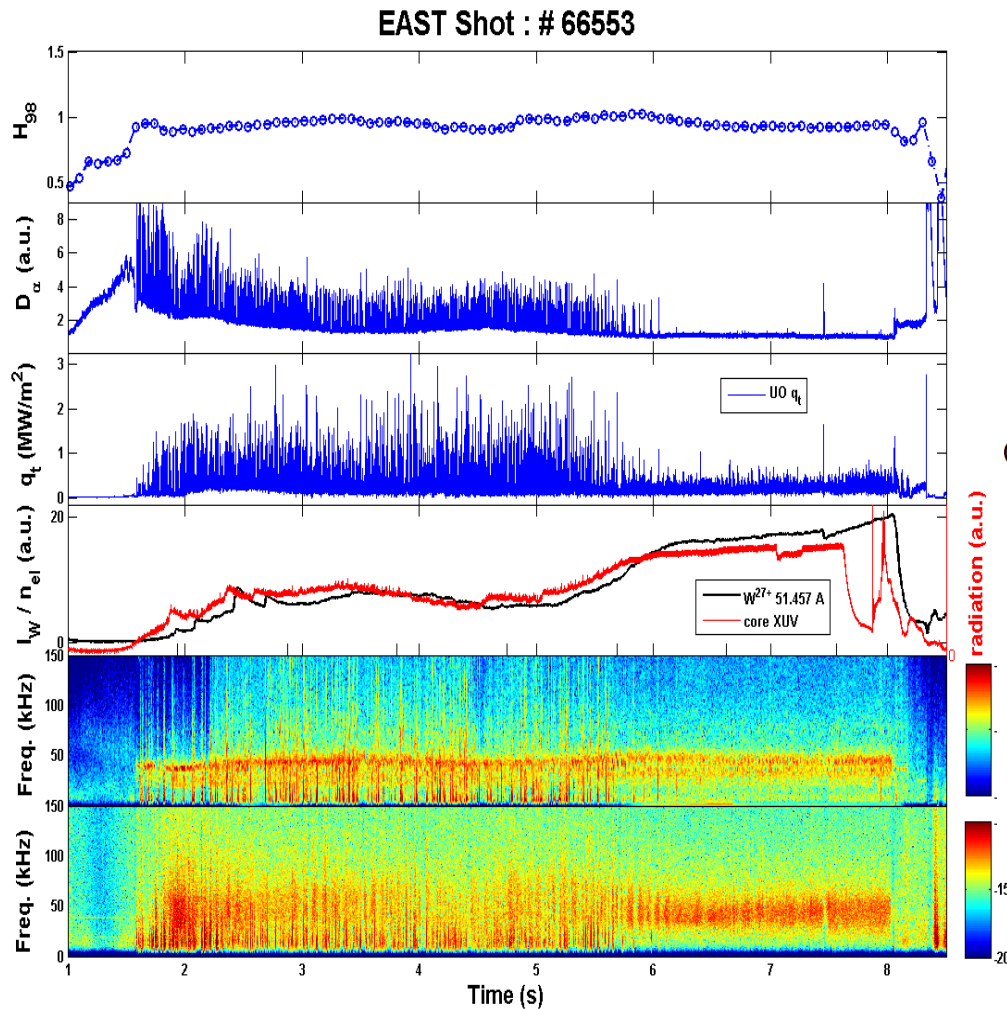
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- **Stationary peaked T_e profile was typically maintained in the series of long pulse modes on EAST**
- **Power balance analysis shows the significantly reduced χ_e in plasma core**
- **The core T_e profile meets the ITB criterion [G. Gresset, NF 2002]**
 - $\rho_{Te}^*(\text{max})=0.02 > \rho_{ITB}^* \sim 0.014$
- **The improved confinement was sustained very stably for tens of seconds!**



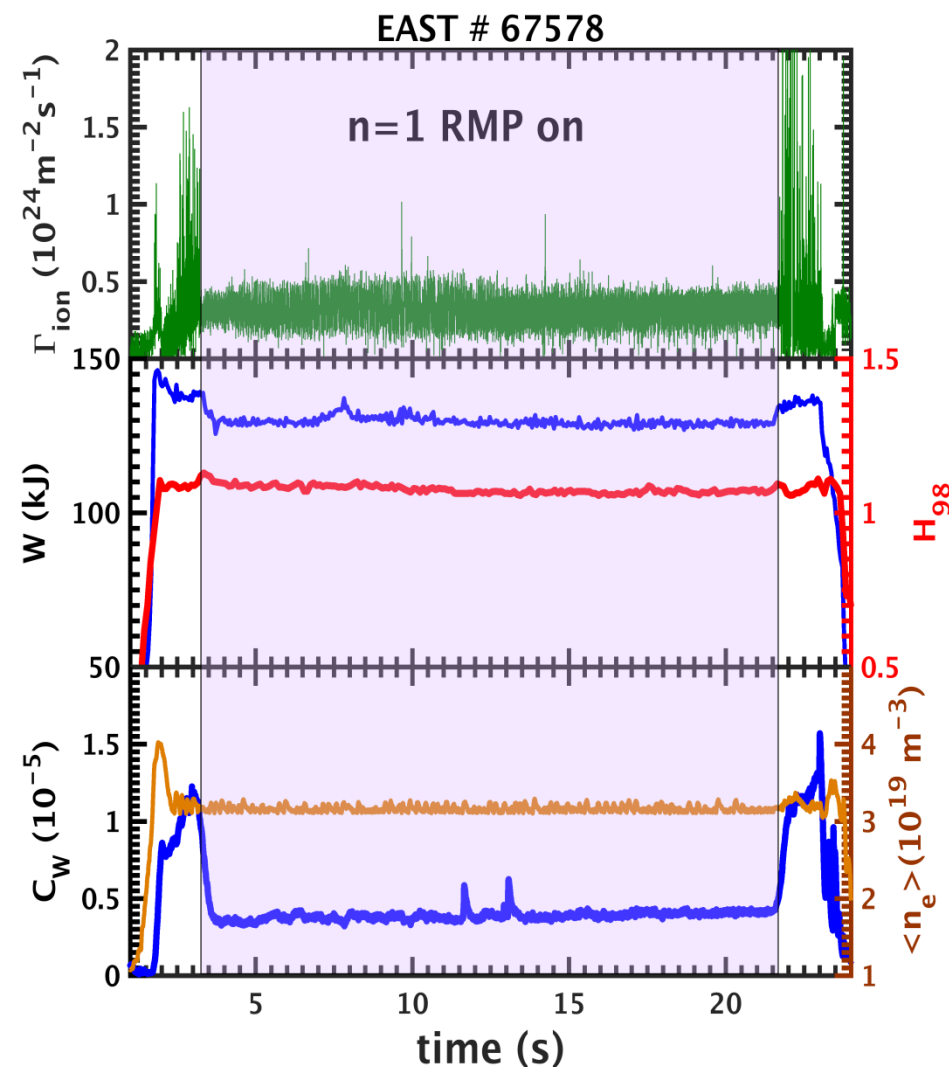
RF heating plays a crucial role in regulating impurity exhaust



ELM suppression by RMP in long pulse H-mode with W divertor operation!



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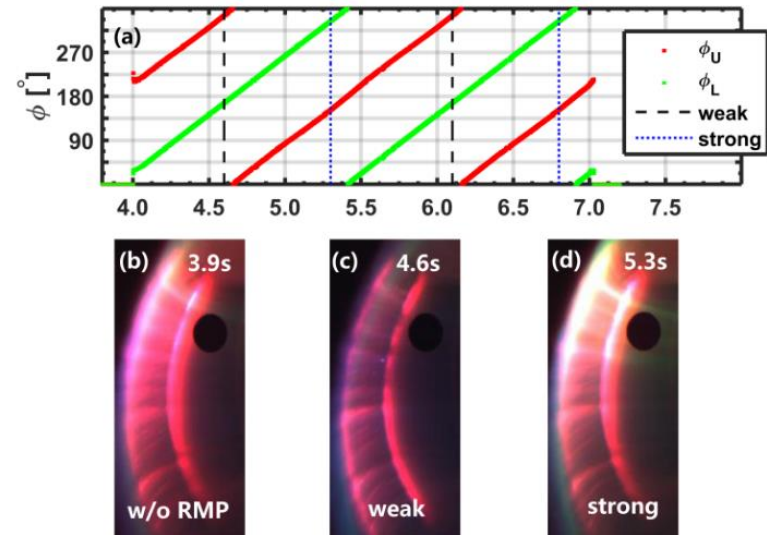
- ELM suppression in long-pulse ($\sim 20\text{s}$) operational scenario was realized with small effect on plasma performance ($H_{98} > 1$)
- Using $n=1$ RMP with optimized spectrum
- Clear pump-out effect on W , which is helpful for sustaining long-pulse high-performance
- Compatible with long-pulse RF-heated H-mode plasmas

Hot spot issue and solutions



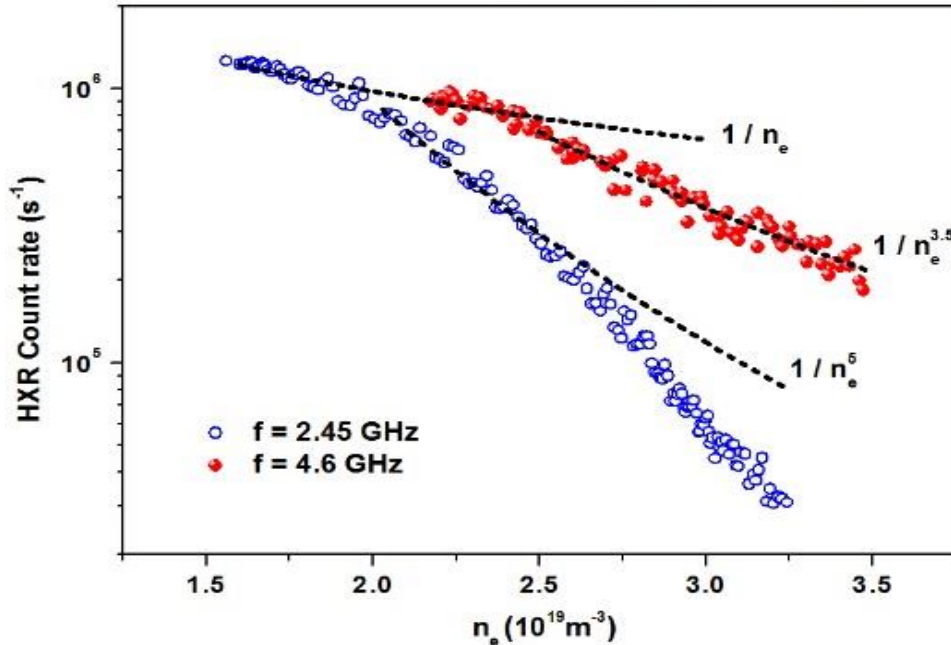
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- Strong hot spots observed on the guard limiter of 4.6GHz LHCD antenna: limiting the pulse length due to strong impurity influx and damage to the limiter
- Global parameter scan identified a threshold LHCD power was around 2.5~3.0MW
- Possible mitigation from rotating RMP was observed by tuning the particle flux hitting on the guard limiter
- New guard limiter design was proposed with inclined surface to lessen direct particle deposition on the surface

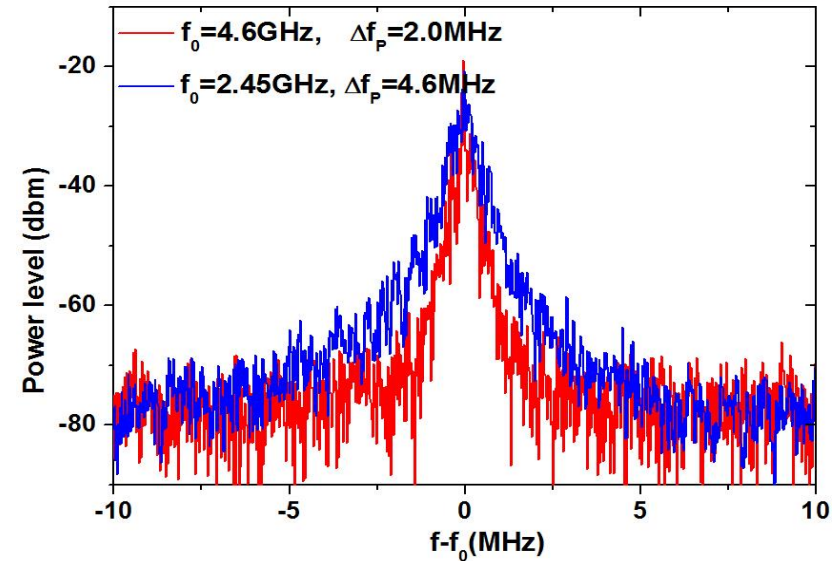


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$I_p = 500\text{kA}$, $P_{LH} = 1.0\text{MW}$, $B_t = 2.3\text{T}$



Shot No. 54439



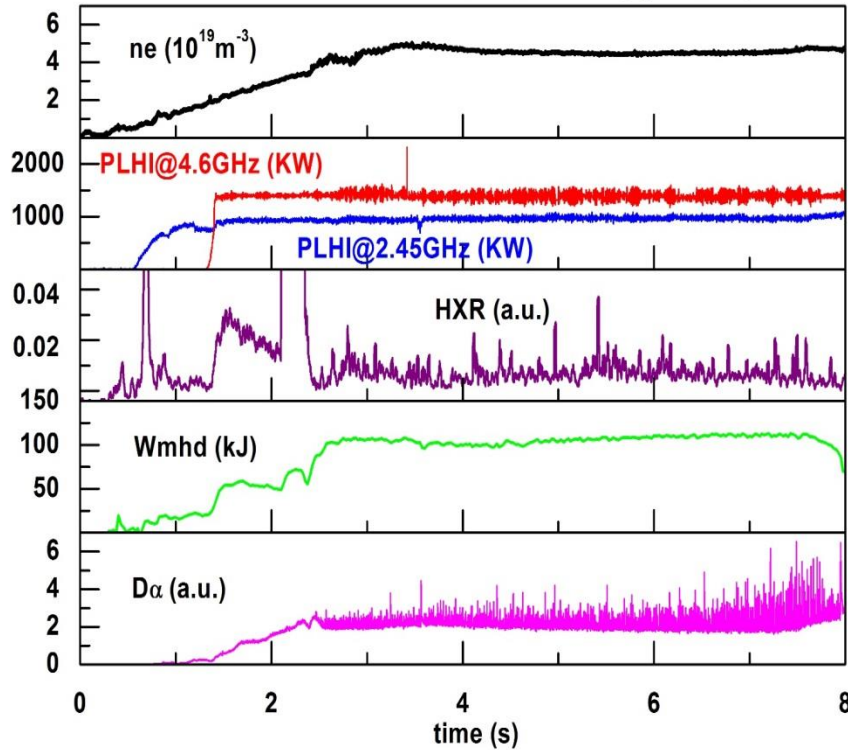
- HXR measurement suggested higher current drive capability at 4.6GHz than 2.45GHz;
- Less parametric instability (PI) behavior with 4.6GHz wave

LHCD still effective at high density in H-mode



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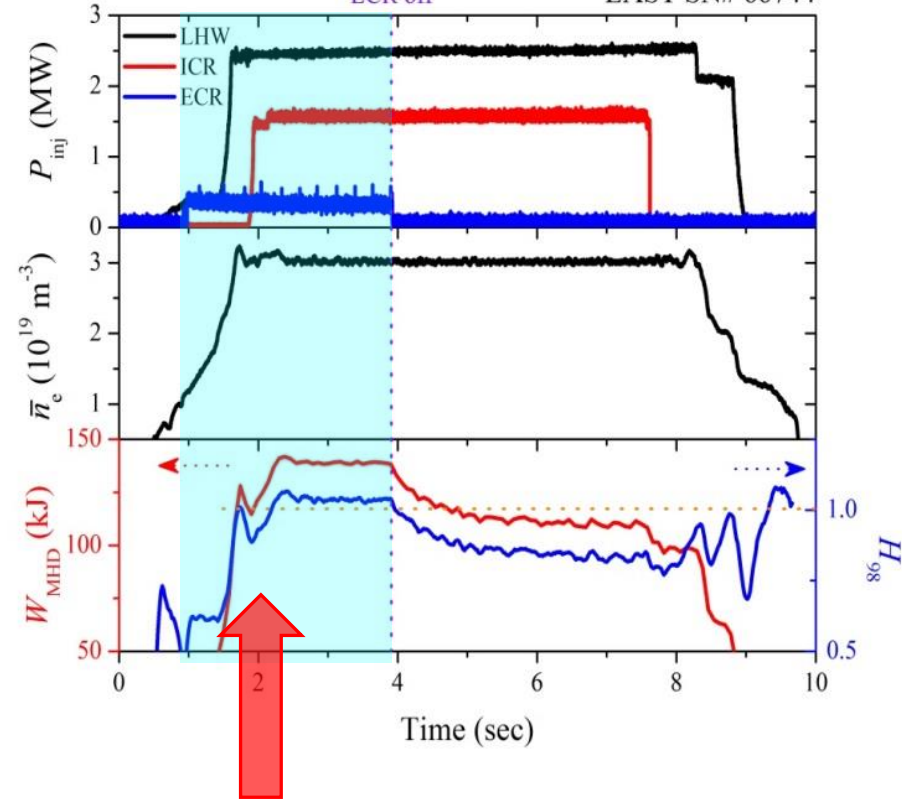
Shot No. 57115, $I_p=500\text{kA}$, $B_t=2.8\text{T}$



- Even if at $n_e \sim 4.5 \times 10^{19} \text{m}^{-3}$, part of current is still driven by LHW!
- Simulation show that N upshift improves accessibility of LHW at high density.

ECR off

EAST SN# 66744

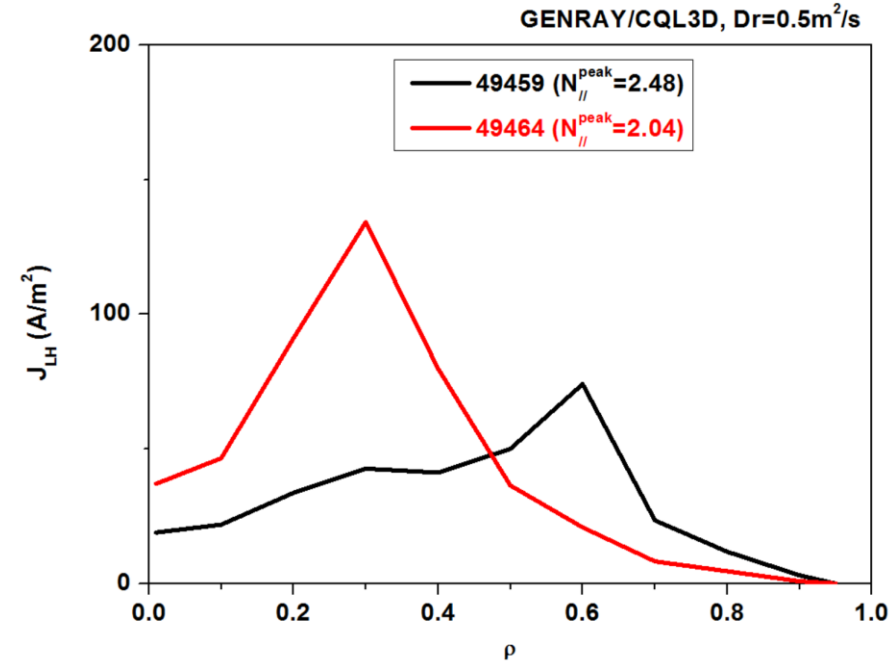
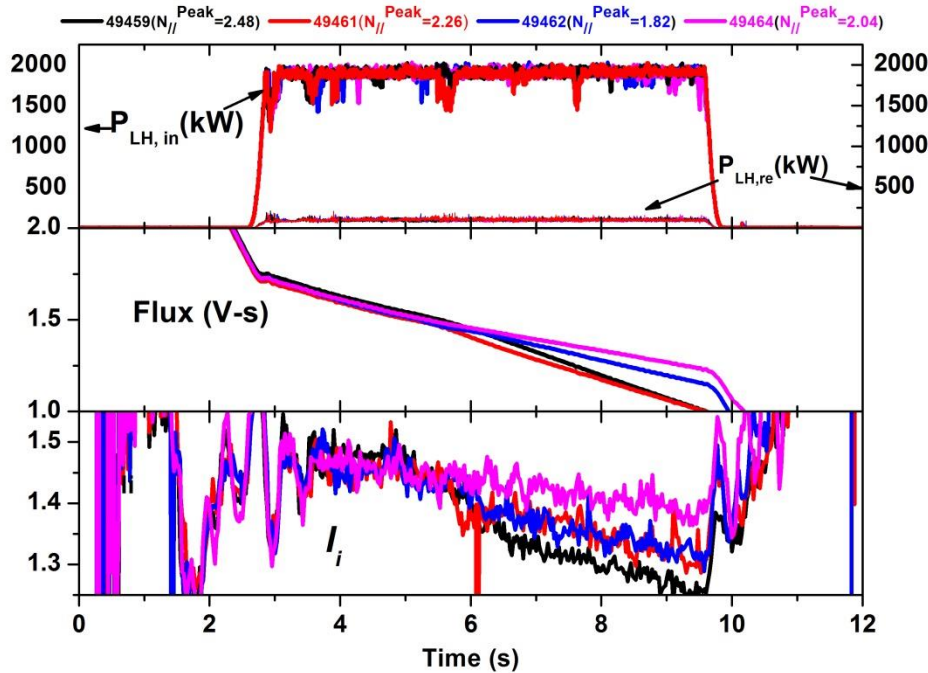


ECRH plays a crucial role for achieving high-performance H-mode plasmas on EAST

Profile control with LH spectrum tuning



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The best CD effect is obtained with $N_{||}^{peak}=2.04$

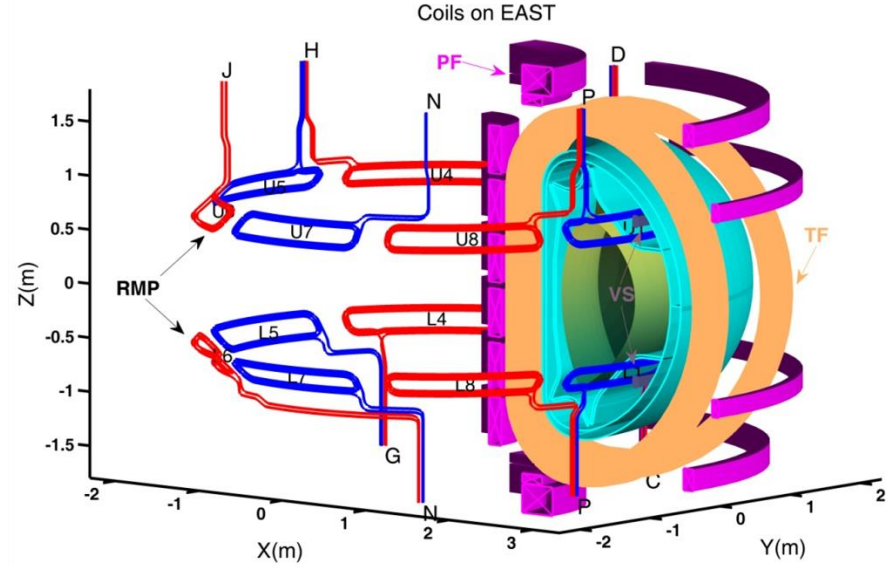
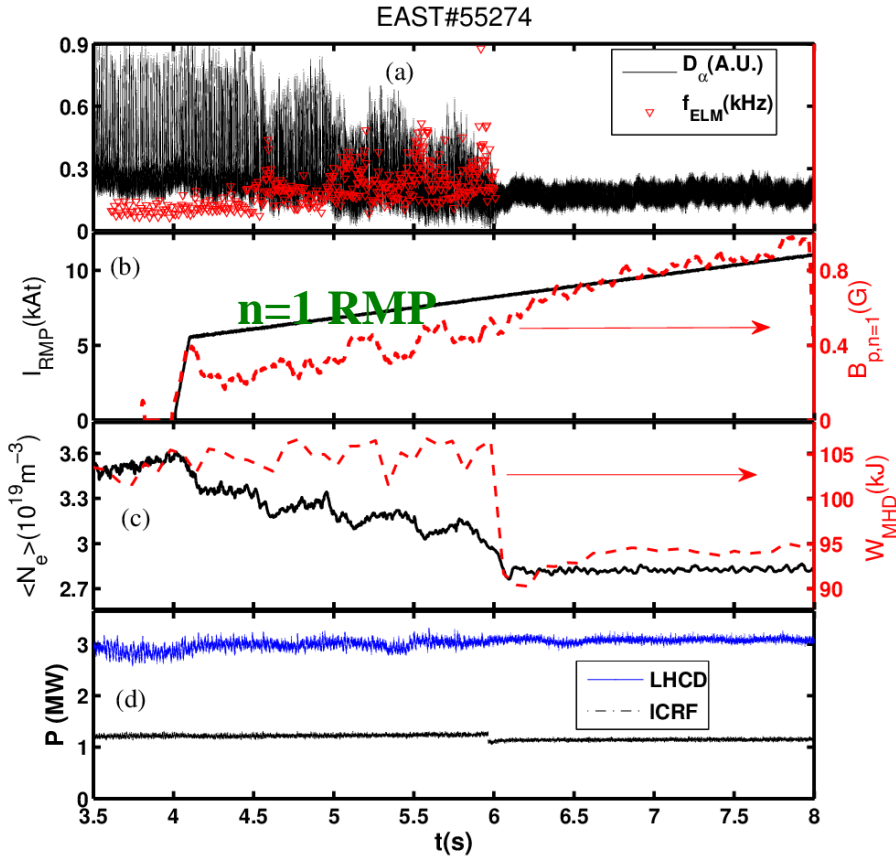
li in 49464 ($N_{||}^{peak}=2.04$) is the largest, indicating more current is driven in the core region compared to other cases.

➤ GENRAY/CQL3D simulation suggest different driven current profiles, qualitatively consistent with the experiments.

ELM suppression with low n RMP



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➤ Full ELM suppression was accessed using low n RMP in low rotating plasma with RF dominant heating in EAST

ELM suppression with $n=1$ RMP in slow-rotating RF heated plasmas

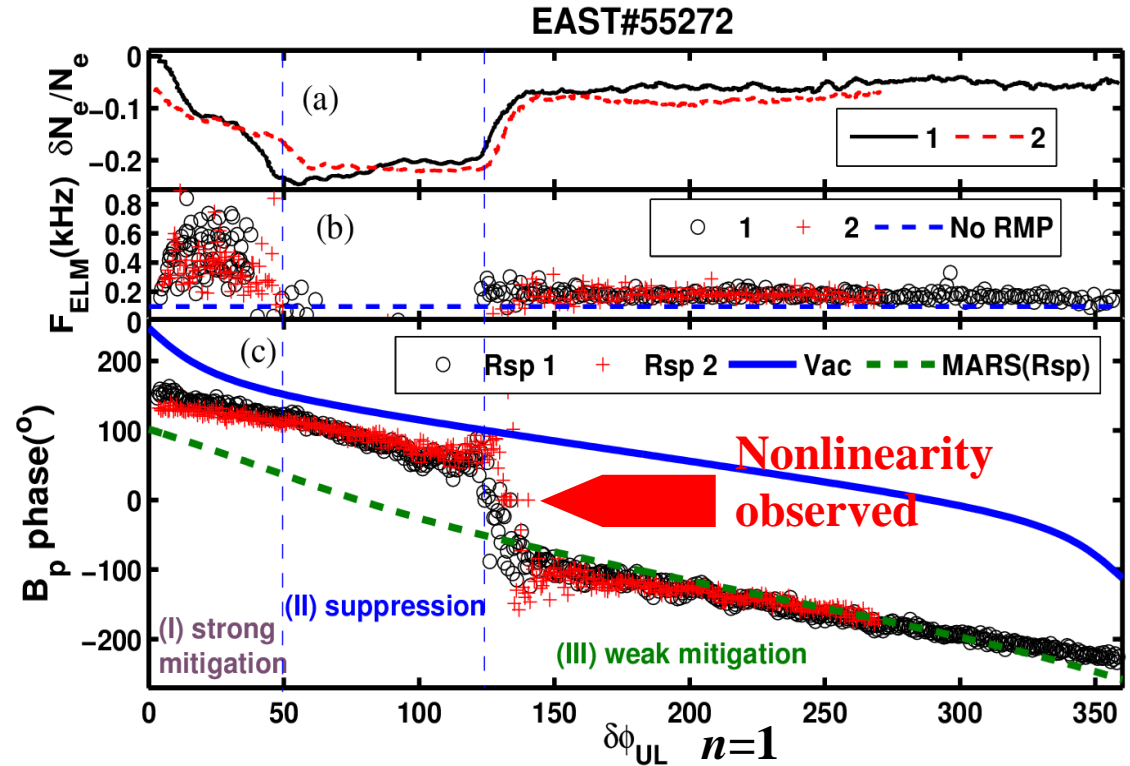
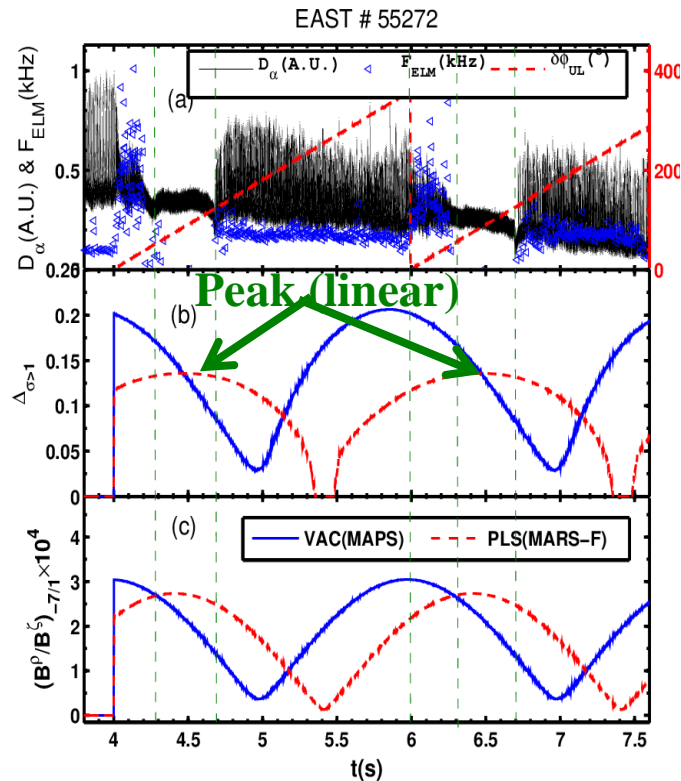
- ✓ 3MW LHCD + 1MW ICRF
- ✓ $n=1, 2$ RMP
- ✓ $\Omega_{\phi} \sim 0$

[Sun Y. *et al.*, Phys. Rev. Lett. 117, 115001 (2016)]

Nonlinear plasma response observed



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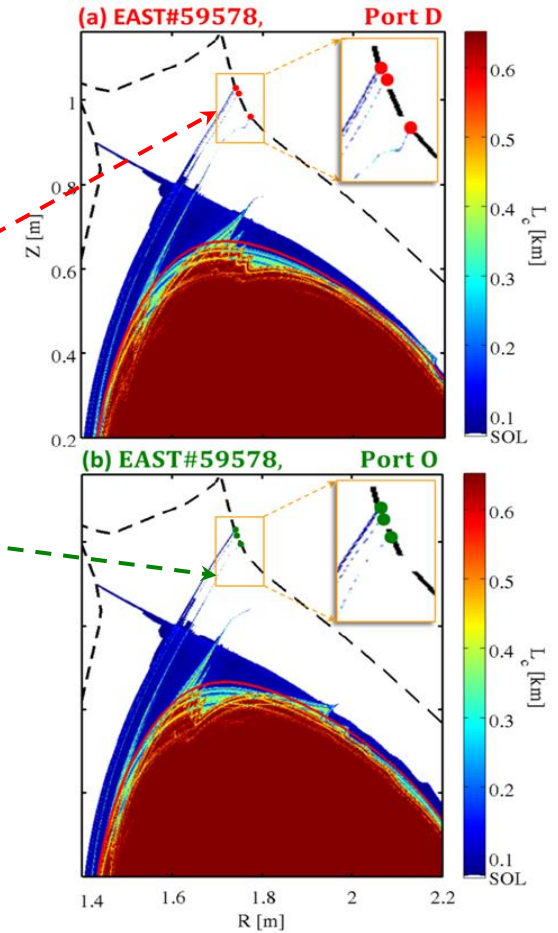
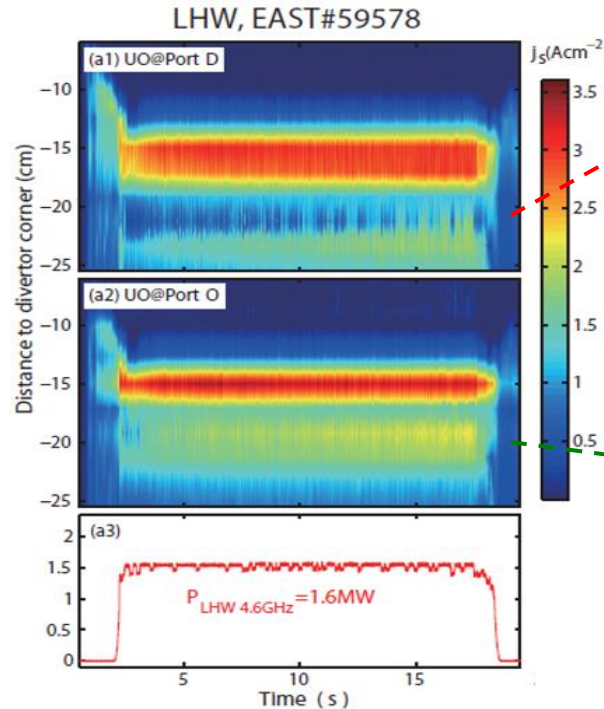
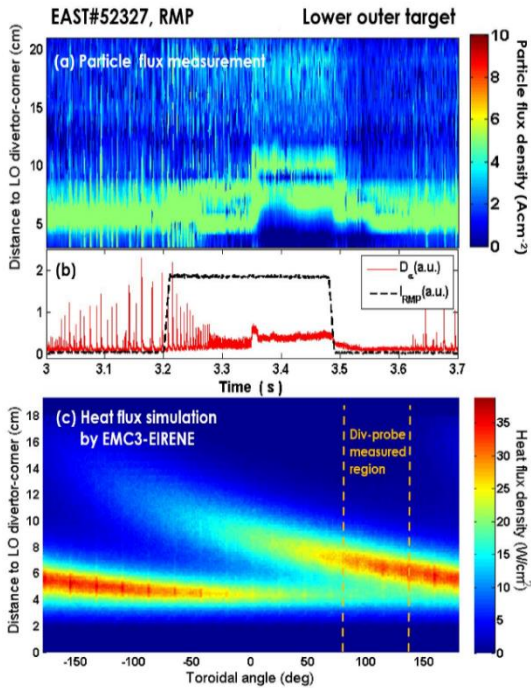
- Linear plasma response determines the best spectrum for ELM suppression.
- Nonlinear plasma response suggests that a critical level of magnetic topological change taking into account plasma response plays a key role in accessing final ELM suppression

Control of 3D divertor footprint



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- Both LHW & RMP coils can induce edge topology change → striated magnetic flux lobes → 3D divertor footprints



- The experimental and modeling results of different toroidal locations show good agreement.
- Simulations: EMC3-EIRENE (RMP), field line tracing taking into account helical current filaments in the SOL (LHW).

➔ **Allowing further heat flux control using 3D footprint with regulated divertor conditions.**



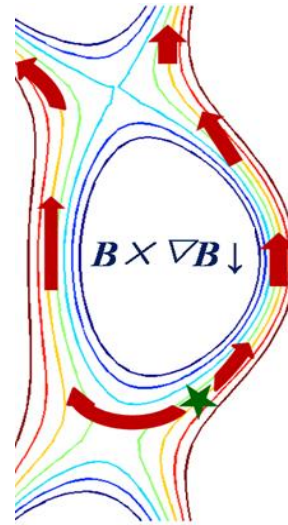
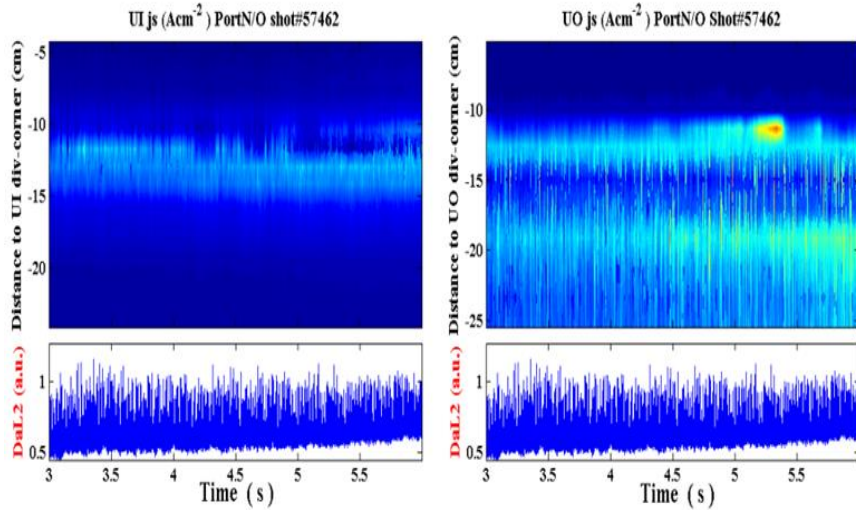
EX/P7-10; TH/P6-20

Active control of particle exhaust



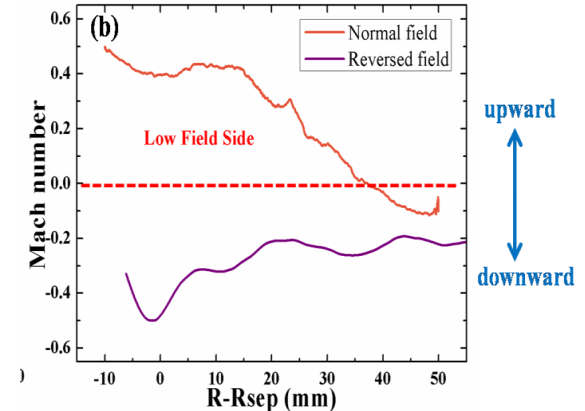
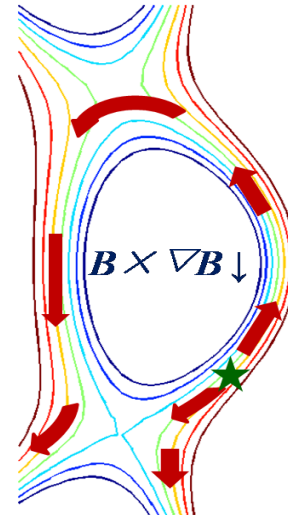
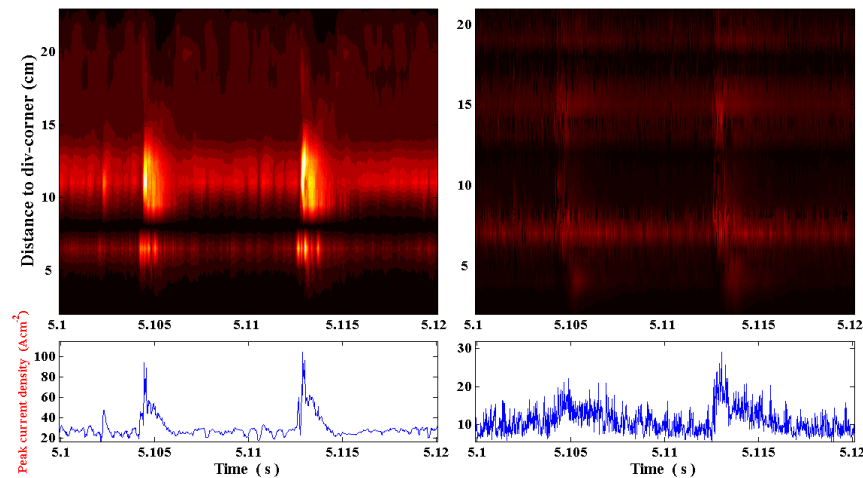
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➤ USN, W-divertor, $B \times \nabla B \downarrow$



- ELM particle flux favors outer divertor in USN, while favoring inner divertor in LSN.
- A reference for particle exhaust during long-pulse H-mode operation, together with cryo-pumps.
- Consistent with PS flow, as measured by LFS/HFS reciprocating Langmuir probes.
- Compatible with ICRF heating

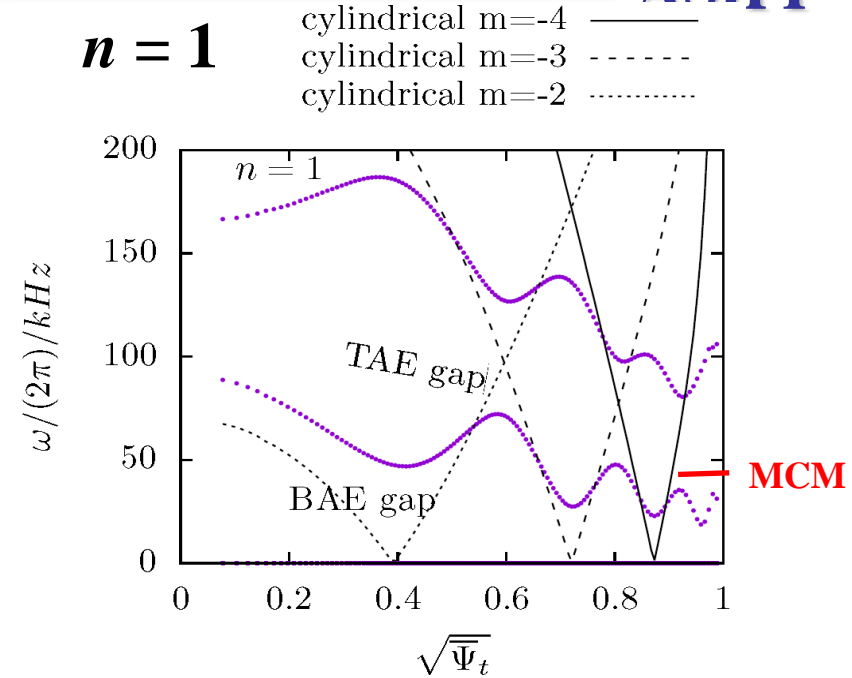
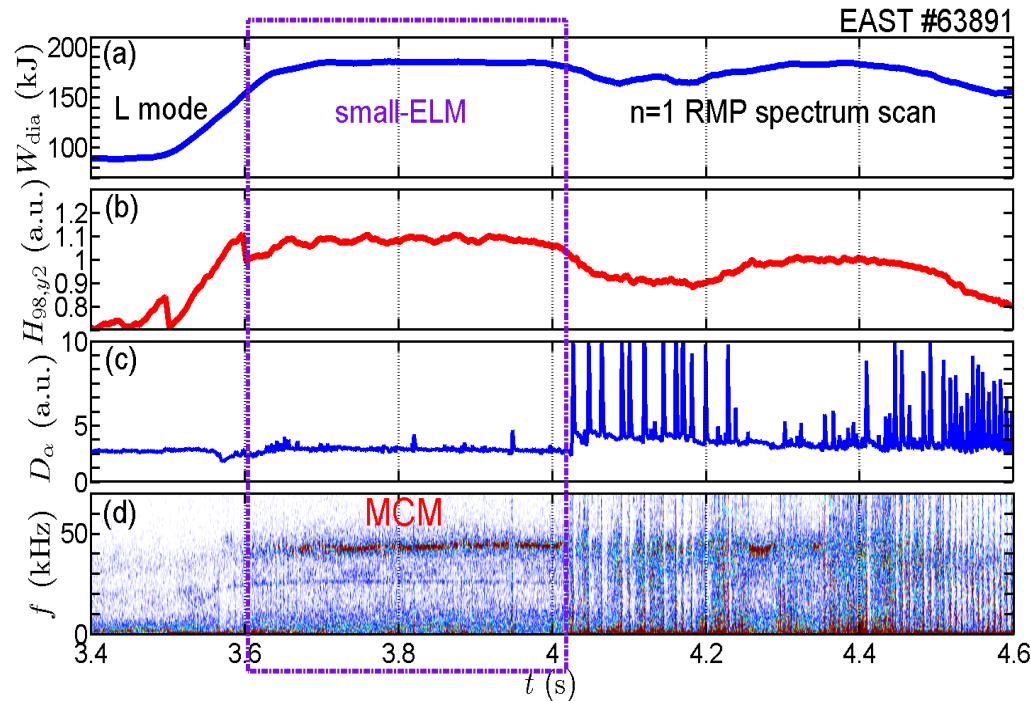
➤ LSN, C-divertor, $B \times \nabla B \downarrow$



New Stationary Small/No ELM H-Mode Regime at Low Collisionality



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- A new stationary small/no ELM H-mode at **low collisionality** ($\nu_e^* < 1$)
- Good energy confinement, $H_{98(y,2)} \gtrsim 1.1$,
- A low- n (mostly $n=1$ and sometimes $n=2$) electro-Magnetic Coherent Mode (MCM) at 30-60 kHz in the pedestal region.

- MCM frequency appears to be located in the TAE gap near the local trapped-thermal-electron bounce frequency
- Frequency scales linearly with the local Alfvén frequency, indicating the possibility of trapped-electron-driven TAE mode through bounce resonance with trapped thermal electrons.

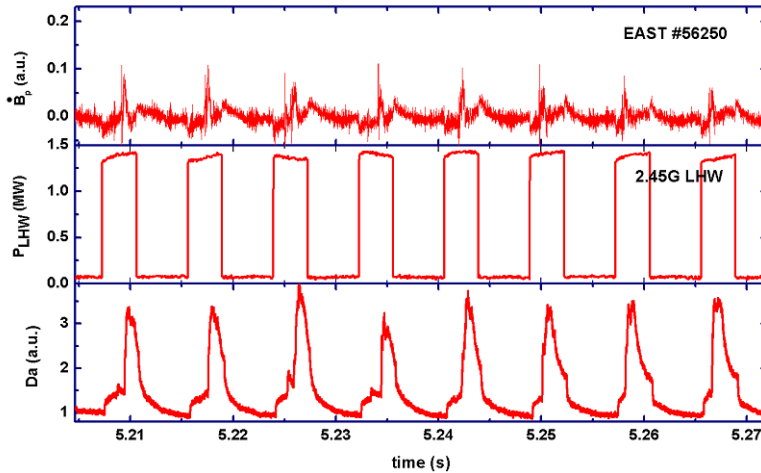
EX/10-2

ELM pacing with LHCD modulation



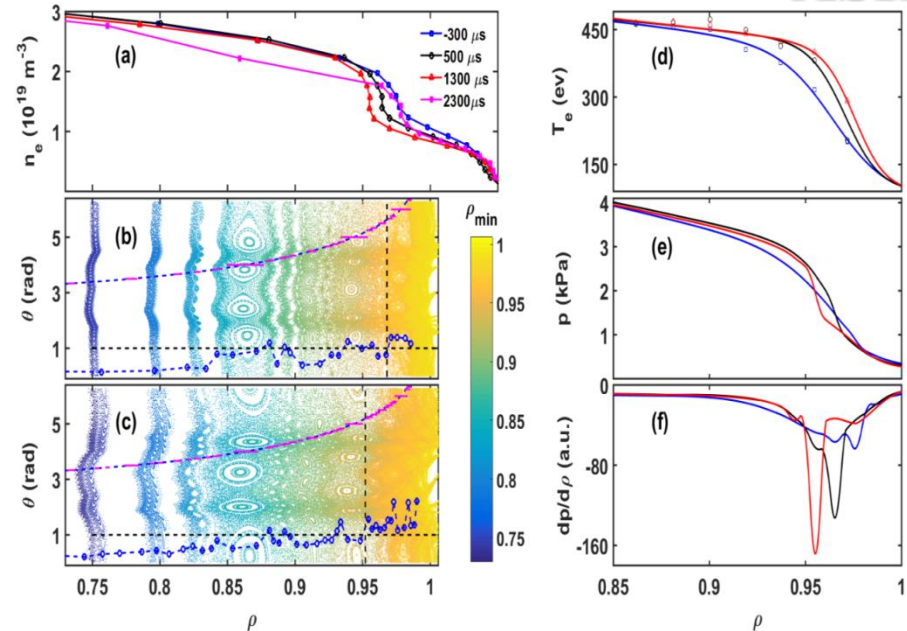
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120 Hz, 40% duty cycle



- > $I_p \sim 0.45 \text{ MA}$, $n_{ei} = 3.3 \sim 45\% n_{GW}$, $B_T \sim 2.33 \text{ T}$, $P_{LH,4.6} \sim 1.3 \text{ MW}$, $P_{NBI,1L} \sim 2.8 \text{ MW}$
- > 2.45G Modulation: $P_{LH,2.45} \sim 1.4 \text{ MW}$, 120 Hz, 40% duty cycle

- LHCD-induced flattening of density profile near the separatrix and pedestal density pump-out have been observed.
- Density gradient is steepened near the pedestal top, causing pedestal-pressure-gradient increase that may be responsible for the ELM triggering.



- Vacuum-field modeling of LHCD-induced 3D magnetic topology change indicates that the flat-density-profile region and its radial width expansion are largely consistent with those of the LHCD-induced edge stochastic magnetic field layer, which may explain the observed density profile change, similar to the effect of RMPs.

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- Stationary RF-heated long-pulse H-mode operations (**>60s !**) were achieved on EAST with progress in the *relevant physics* in support of SSO on *tungsten divertor*
- Extension of the *SSO towards high β_p regime (up to 1.8)*;
 - Achievement of *low-n (1, 2) RMP ELM suppression* in the RF dominant slow-rotating long-pulse H-mode plasma (*~20s*); Observation of the *first evidence of a nonlinear transition* from mitigation to suppression of the ELMs by using RMPs;
 - Extension of the *current drive in high-density domain* (up to $4.5 \times 10^{19} \text{m}^{-3}$) with 4.6 GHz and 2.45GHz LHCD systems together;
 - *Regulating heat deposition distribution and reducing transient peak heat fluxes* on the divertor and PFCs by applying *3D* magnetic perturbations at the plasma boundary.
 - Discovery of *a new stationary ELM-stable H-mode* ($H_{98(y,2)} \gtrsim 1.1$, $v_{e,ped}^* < 1$) *regime*, which exhibited a *low-n MCM* at the pedestal.

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- **OV/2-2 B.N. Wan:** Overview of EAST Experiments on the Development of High-Performance Steady-State Scenario
- **EX/4-3 A.M. Garofalo:** Development of High Poloidal β Steady-State Scenario with ITER-Like W Divertor on EAST
- **EX/10-2 G.S. Xu:** ELM Pace-Making and Long-Pulse ELM-Stable H-Mode Operation with LHCD in EAST
- **EX/P7-2 X. Gao:** Key Issues towards Long Pulse High Operation on EAST Tokamak
- **EX/P7-4 Y. Sun:** ELM Suppression Using Resonant Magnetic Perturbation in EAST
- **EX/P7-5 B. J. Ding:** Recent Experimental and Modelling Advances in the Understanding of Lower Hybrid Current Drive in ITER-Relevant Regimes
- **EX/P7-7 B. Lyu:** Experimental Study of Radio-Frequency Driven Spontaneous Rotation for High-Performance Plasmas on EAST
- **EX/P7-8 X.J. Zhang:** Heating and Confinements by the Waves in the Ion Cyclotron Range of Frequencies on EAST
- **EX/P7-10 L. Wang:** Evidence and Modelling of 3D Divertor Footprint Induced by Lower Hybrid Waves on EAST with Tungsten Divertor Operations
- **EX/P7-12 X.D. Zhang:** Fishtail Divertor: A New Divertor Concept on EAST for Active Control of Heat Load on Divertor Plate
- **EX/P7-15 G. Li:** Predictions of the Baseline Operation Scenario in Chinese Fusion Engineering Test Reactor
- **EX/P7-16 W. X. Ding:** Current Transport and Density Fluctuations at L-H Transition on EAST
- **TH/P6-19 T. Y. Xia:** Divertor Heat Flux Simulations in ELMy H-Mode Discharges of EAST and Other Tokamaks
- **TH/P6-20 J. Huang:** EMC3-EIRENE Simulations for the Impact of External Magnetic Perturbations on EAST Edge Plasma
- **FIP/1-1 P. Fu:** Recent Progress of ITER Package in ASIPP
- **FIP/P4-21 Z. Song:** Research and Development Progress of the ITER PF Converter System
- **MPT/1-2Ra G.-N. Luo:** Overview on Decade Development of Plasma-Facing Components at ASIPP

Welcome to the poster session for further discussions!



Thank you !



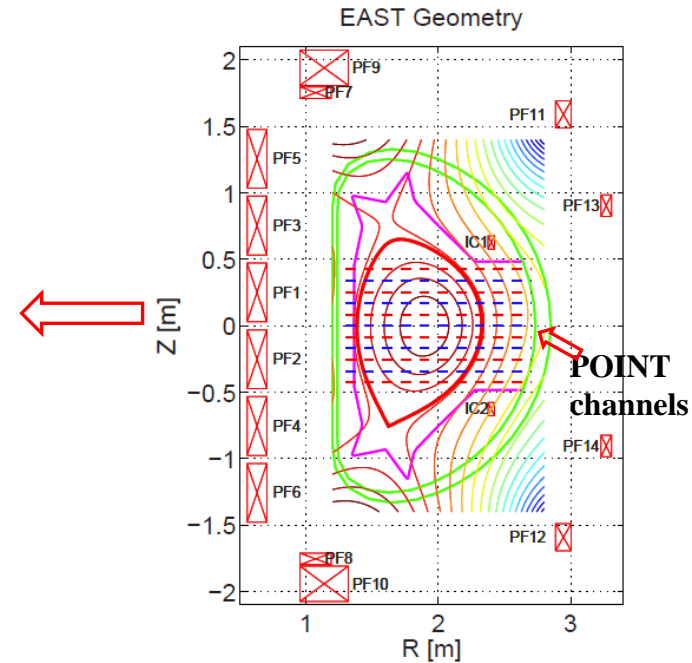
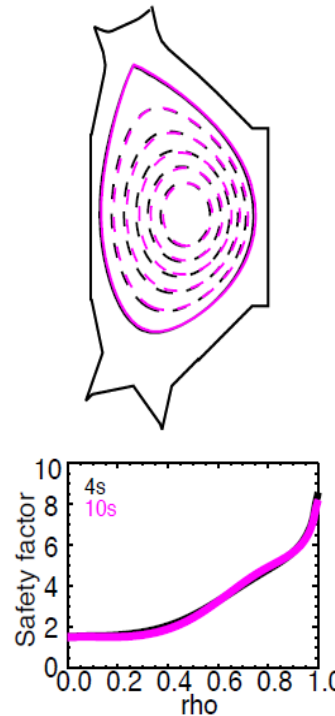
Backup slides

Diagnostics for key profiles

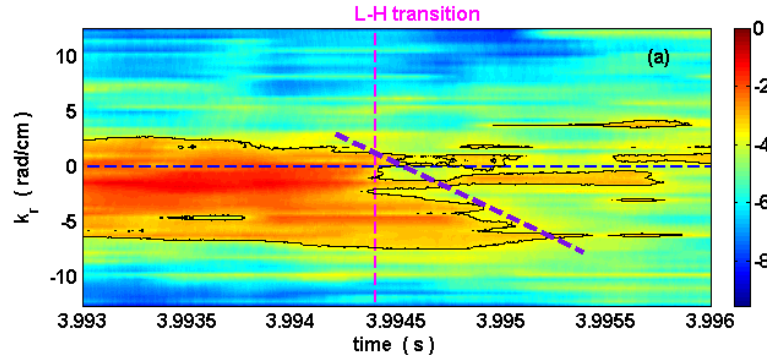
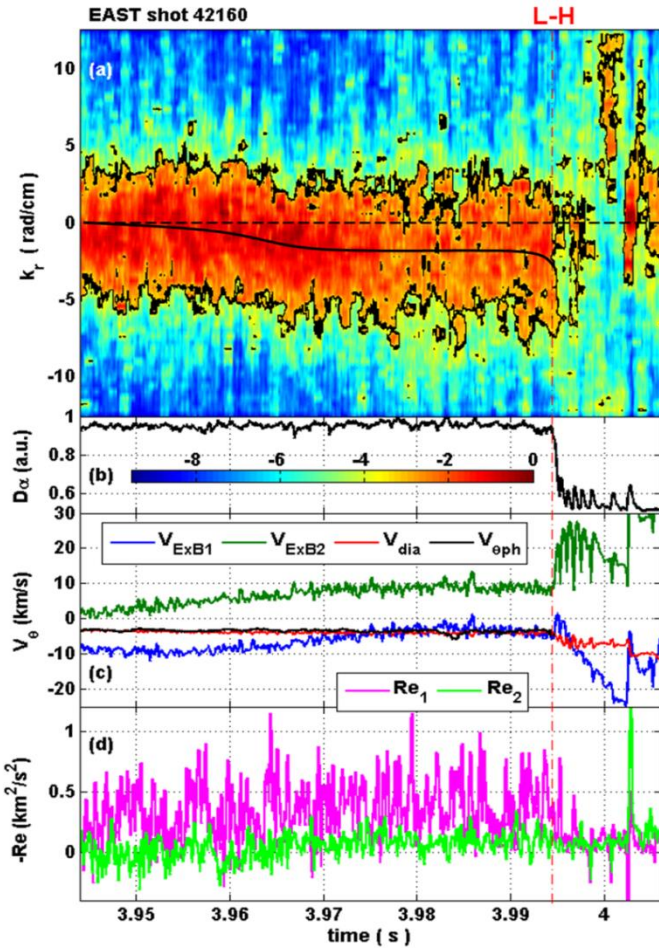


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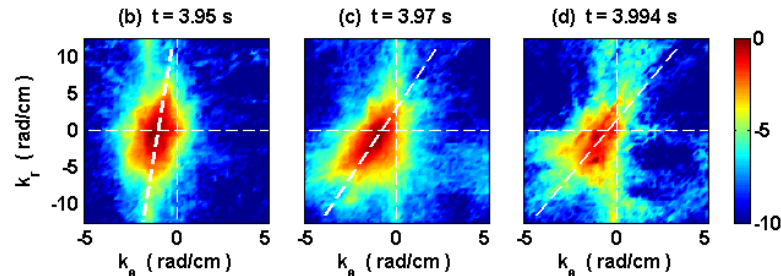
- ❑ **Polarimeter Interferometer (POINT):** n_e , j_ϕ , q , B_p profiles
- ❑ **Core & edge TS:** T_e , n_e
- ❑ **AXUV & Bolometer:** radiation
- ❑ **CXRS & XCS:** T_i , rotation
- ❑ **SXPHA & ECE:** T_e
- ❑ **Reflectometry:** pedestal n_e
- ❑ **He-BES:** edge n_e , T_e
- ❑ **Recip.-LPs:** SOL n_e , T_e , flow
- ❑ **Bremsstrahlung:** Z_{eff}
- ❑ **FIDA:** $V_{\text{fast-particle}}$
- ❑ **High speed CCD**
- ❑ **IR camera:** heat flux
- ❑ **Div-LPs:** div. particle/heat flux
- ❑ **Total:** >70 diagnostics



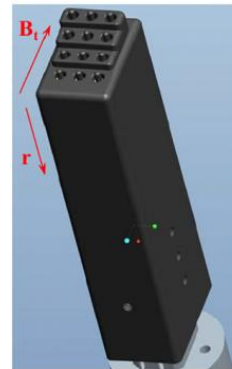
- **Using POINT measurement as constraint, accurate q profile were derived**
- **Powerful tools for developing scenarios**



Shift accelerates at the transition, accompanied by turbulence suppression.



Eddy tilt increase as approaching the L-H transition.



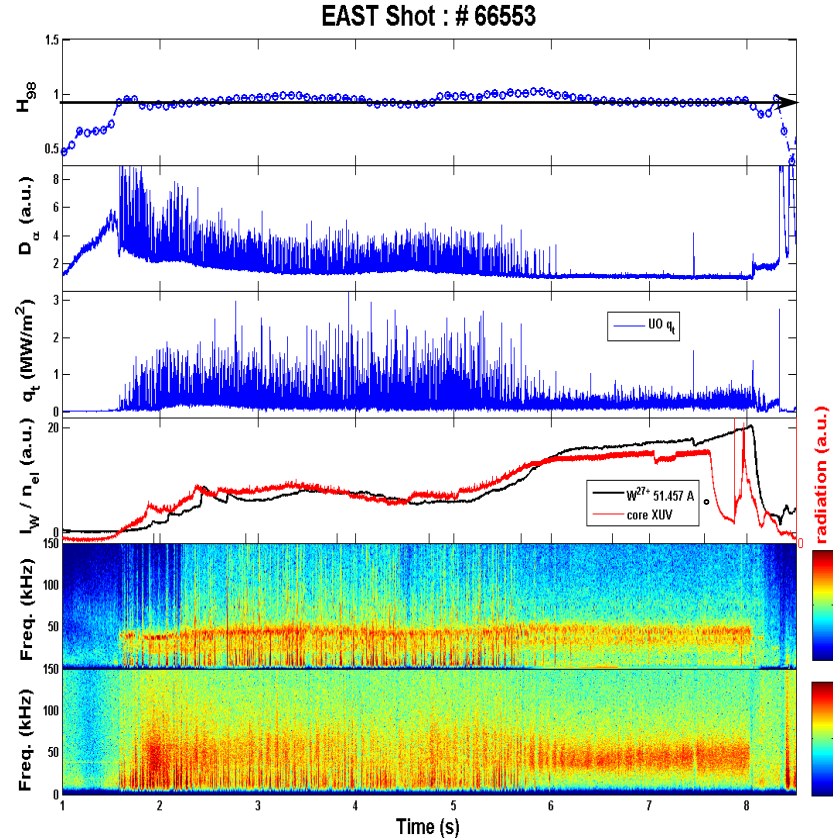
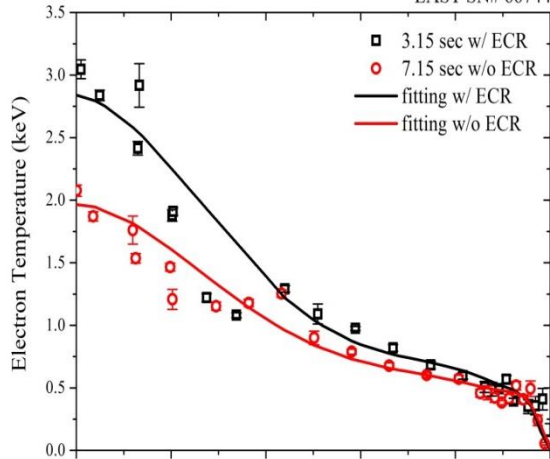
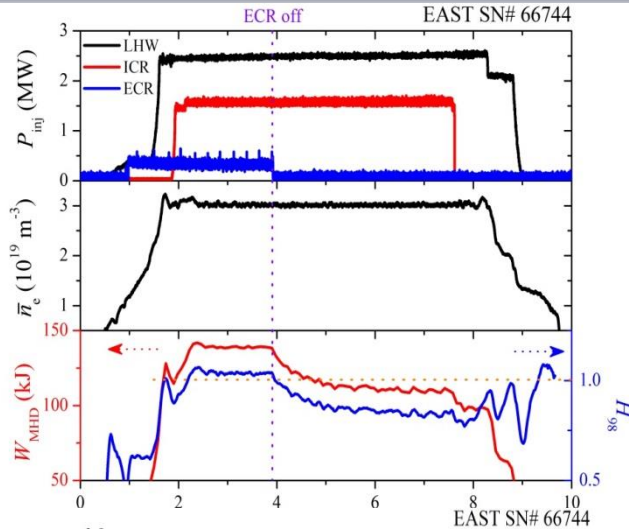
Reciprocating Langmuir probe array

First direct observation of L-H transition mediated by turbulence k_r spectral shift and eddy tilting.

Core confinement improvement and ELM effect for long pulse H-mode operation



^ SIPP



ELM-free H-mode (6.06-8s) { High confinement ($H_{98} \sim 0.93$)
Impurity accumulation and high core radiation

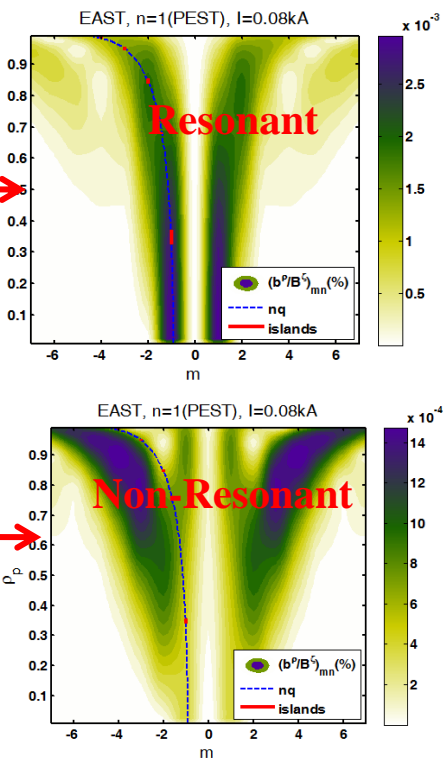
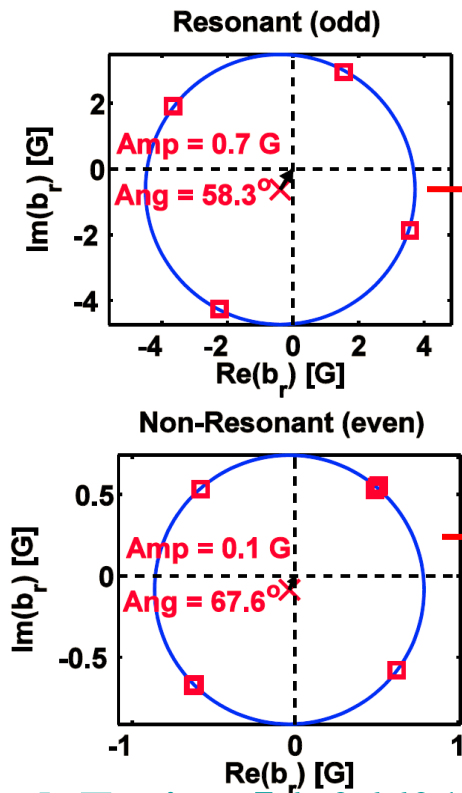
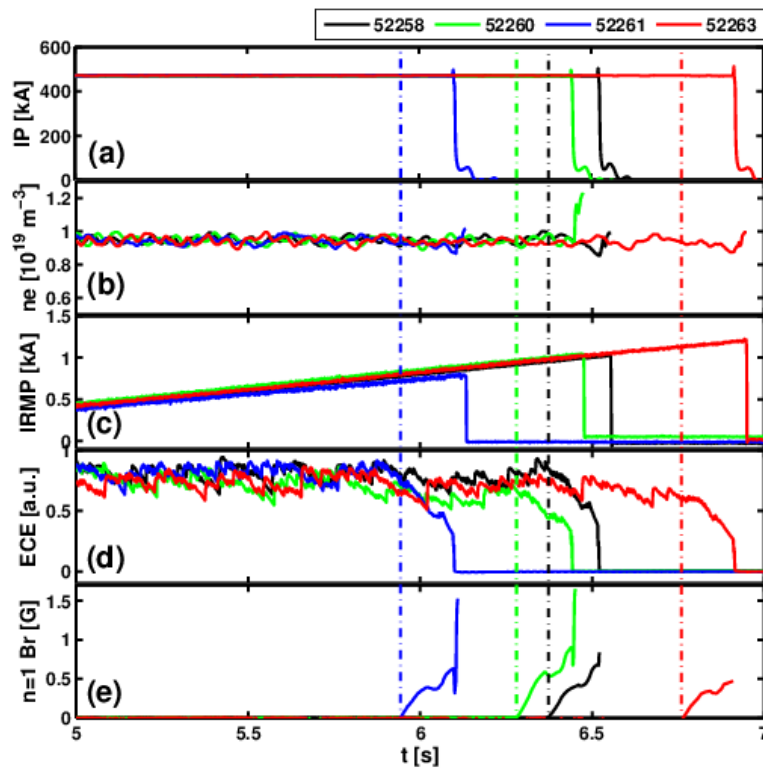
Small-ELM H-mode (1.6-6.06s) { High confinement ($H_{98} \sim 0.98$)
Lower impurity/radiation level
Suitable for long-pulse H-mode33 operation

Adding additional central heating power i.e. ECR, is critical for keeping high performance

Low $n=1$ intrinsic error field measured in EAST



ASIPP



Compass scan [Wang H.H *et al*, Nucl. Fusion 56, 066011(2016)]

- The measured $n=1$ intrinsic error field is of the order $B_{2/1}/B_0 \sim 10^{-5}$.
- The amplitude depends on the RMP configuration used, which agrees with linear plasma response modeling by MARS-F.
 - ✓ Non-Resonant: $B_{2/1}/B_0 \sim 0.6 \times 10^{-5}$ (better coupling)
 - ✓ Resonant: $B_{2/1}/B_0 \sim 4.4 \times 10^{-5}$