

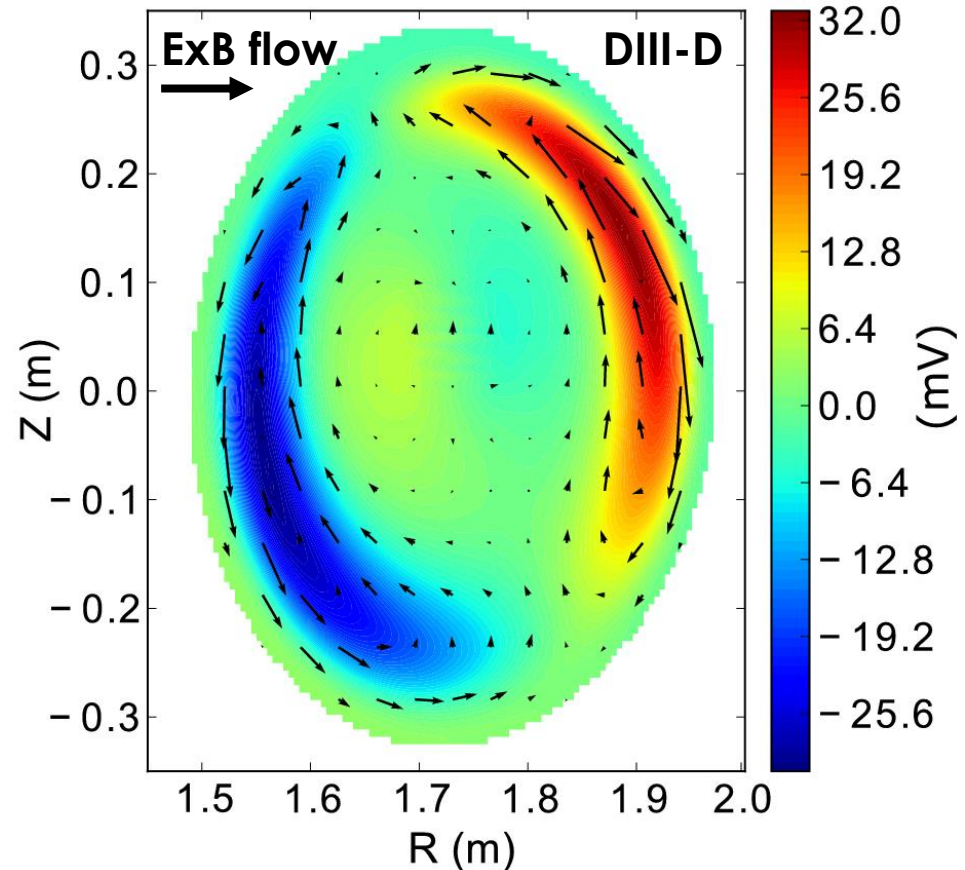
Role of MHD Dynamo in the Formation of 3D Equilibria in Fusion Plasmas

by
P. Piovesan

Presented at the
**26th IAEA Fusion Energy
Conference,
Kyoto, Japan**

October 17-22, 2016

MHD dynamo EMF of helical core

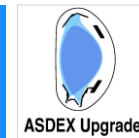


Work supported in part by the US DOE under DE-FC02-04ER54698.



Authors

P Piovesan, D Bonfiglio, S Cappello, L Chacón, C Chrystal, DF Escande, P Franz, CT Holcomb, V Igochine, TC Luce, L Marrelli, MD Nornberg, C Paz-Soldan, L Piron, I Predebon, JS Sarff, NZ Taylor, D Terranova, F Turco, RS Wilcox, A Wingen, P Zanca, B Zaniol, the RFX-mod Team, the MST Team, the DIII-D Team, the ASDEX Upgrade Team, the EUROfusion MST1 Team



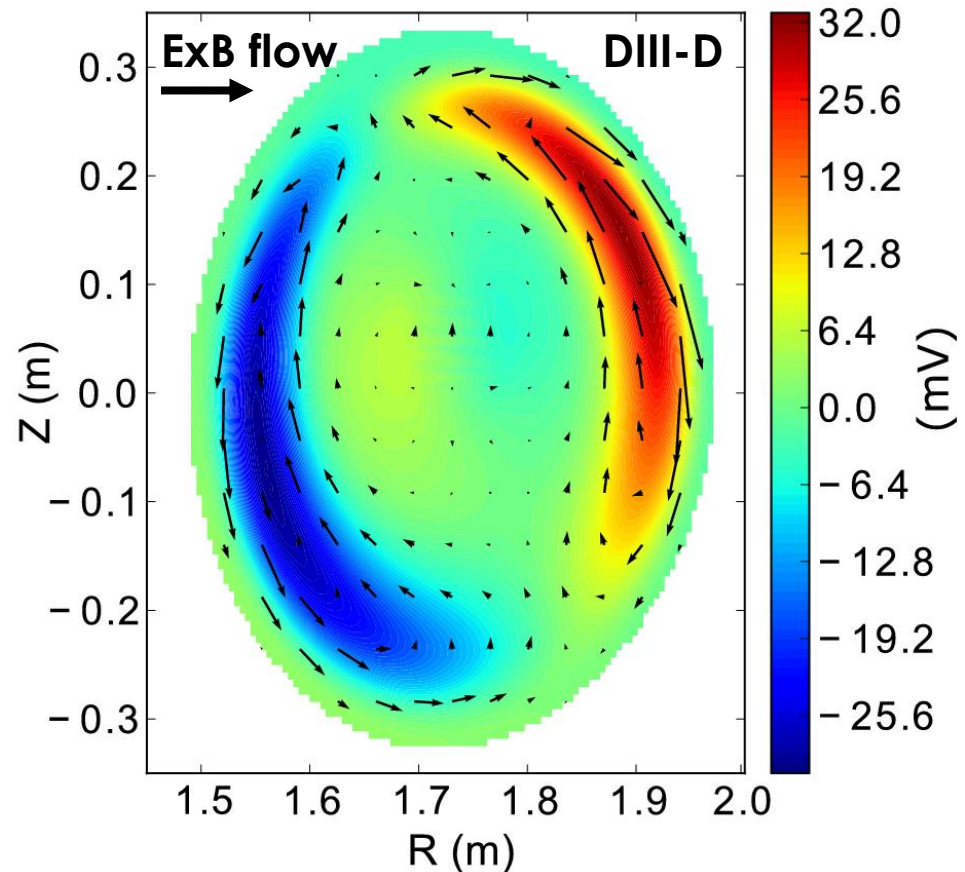
This work has been carried out within the framework of the EUROfusion Consortium and has received funding from the Euratom research and training programme 2014-2018 under grant agreement No 633053. The views and opinions expressed herein do not necessarily reflect those of the European Commission.



MHD Modes Can Broaden the Current Profile Through a Continuous MHD Dynamo

- **MHD modes often saturate into stationary 3D equilibria**
 - affecting current profile, stability, transport, ...

MHD dynamo EMF of helical core

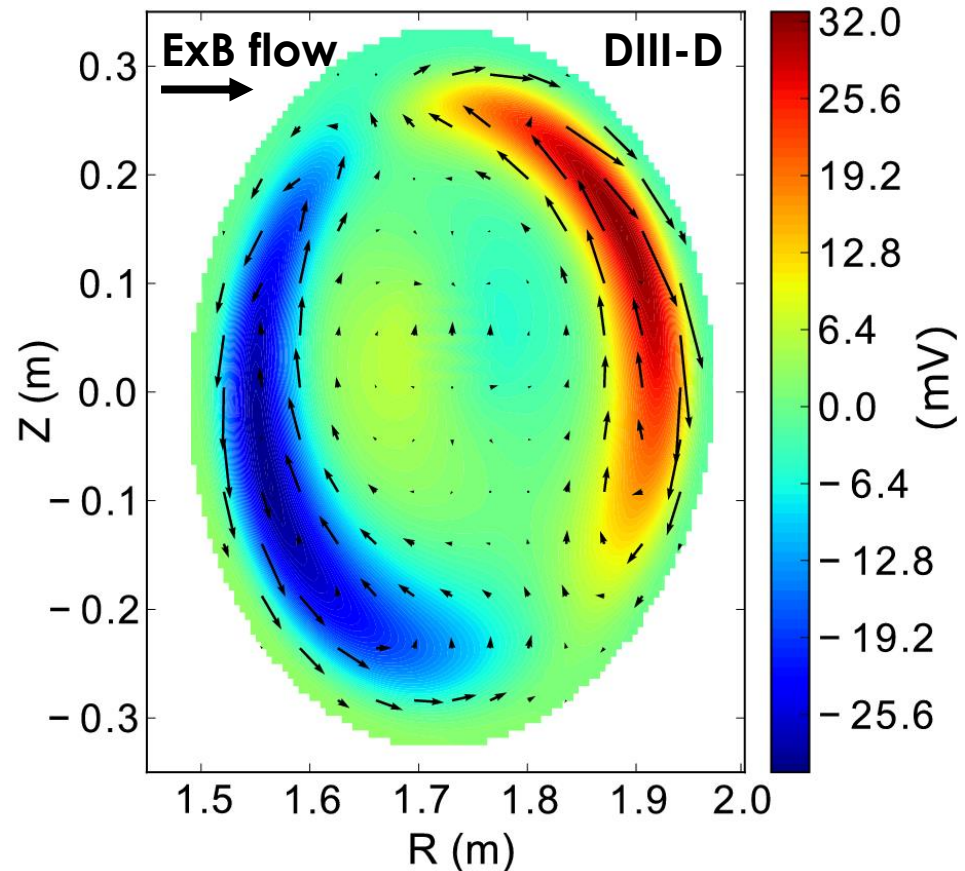


MHD Modes Can Broaden the Current Profile Through a Continuous MHD Dynamo

- **MHD modes often saturate into stationary 3D equilibria**
 - affecting current profile, stability, transport, ...
- **Resistive MHD predicts that helical states induce a continuous dynamo EMF:**

$$E_{loop} + \tilde{v} \times \tilde{b} = \eta j$$

MHD dynamo EMF of helical core



MHD Modes Can Broaden the Current Profile Through a Continuous MHD Dynamo

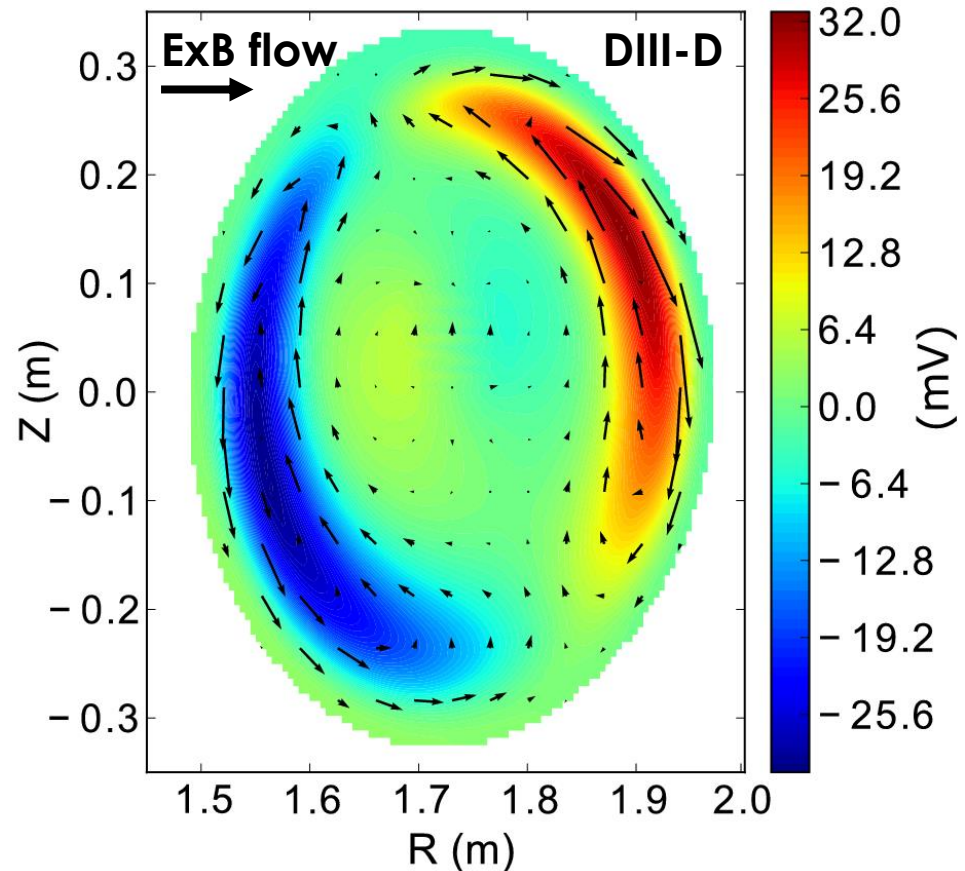
- **MHD modes often saturate into stationary 3D equilibria**
 - affecting current profile, stability, transport, ...

- **Resistive MHD predicts that helical states induce a continuous dynamo EMF:**

$$E_{loop} + \tilde{\mathbf{v}} \times \tilde{\mathbf{b}} = \eta \mathbf{j}$$

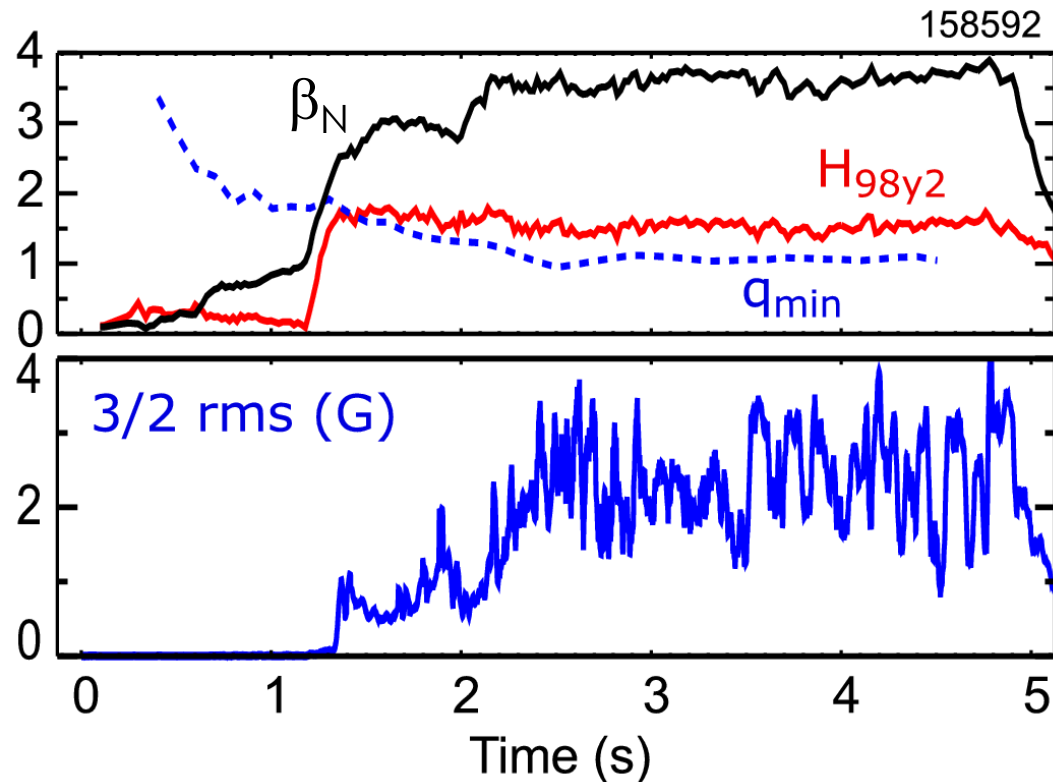
- **Can explain anomalous current broadening in:**
 - high- β hybrid tokamak
 - reversed-field pinch

MHD dynamo EMF of helical core



The Current Profile of Hybrid Tokamak Plasmas is Broadened by Benign MHD

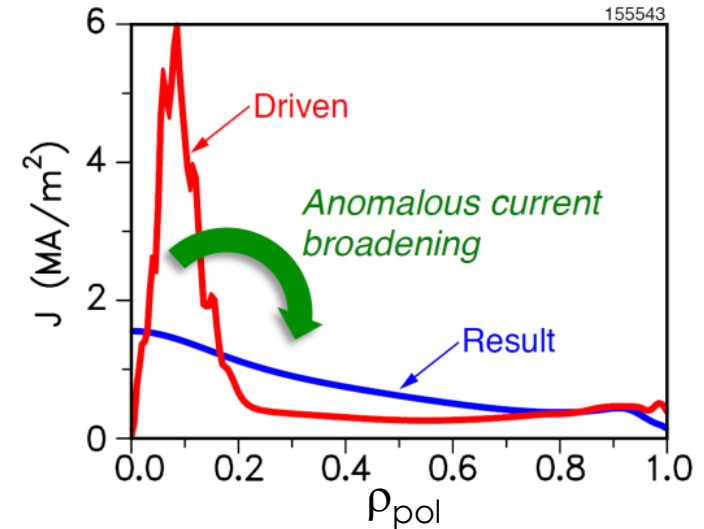
- Hybrid tokamak: H-mode with improved confinement & high- β
- Benign MHD broadens the current profile and keeps $q_{\min} > 1$
- As a result, no sawteeth, deleterious 2/1 mode more stable



Anomalous Current Broadening Enables Steady State Hybrid Operation

- The hybrid scenario has moderate bootstrap current fraction, $f_{BS} \approx 0.5$
- Can be compensated by efficient EC current driven near the center and redistributed by MHD

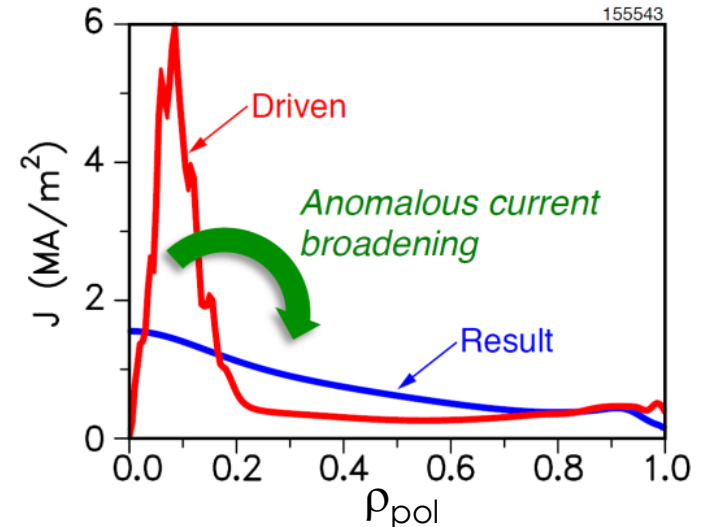
F Turco PoP 2015, CC Petty NF 2016
CC Petty EX/4-1, this conference



Anomalous Current Broadening Enables Steady State Hybrid Operation

- The hybrid scenario has moderate bootstrap current fraction, $f_{BS} \approx 0.5$
- Can be compensated by efficient EC current driven near the center and redistributed by MHD

F Turco PoP 2015, CC Petty NF 2016
CC Petty EX/4-1, this conference



A validated model of current redistribution is needed for extrapolations to future machines

- For example, current redistribution in DIII-D mainly occurs during transient NTM-ELM coupling events [CC Petty PRL 09]
- Will it work in a continuous way in ELM-suppressed plasmas?

GOAL: Test the MHD dynamo model of current redistribution in fusion plasmas

- **A helical core equilibrium forms in hybrid tokamak plasmas perturbed by external $n=1$ fields**
- **Helical core used to probe current broadening**
- **Effect consistent with the MHD dynamo model**
- **Strong similarity with helical RFP dynamo**
- **Conclusions and future work**

Outline

GOAL: Test the MHD dynamo model of current redistribution in fusion plasmas

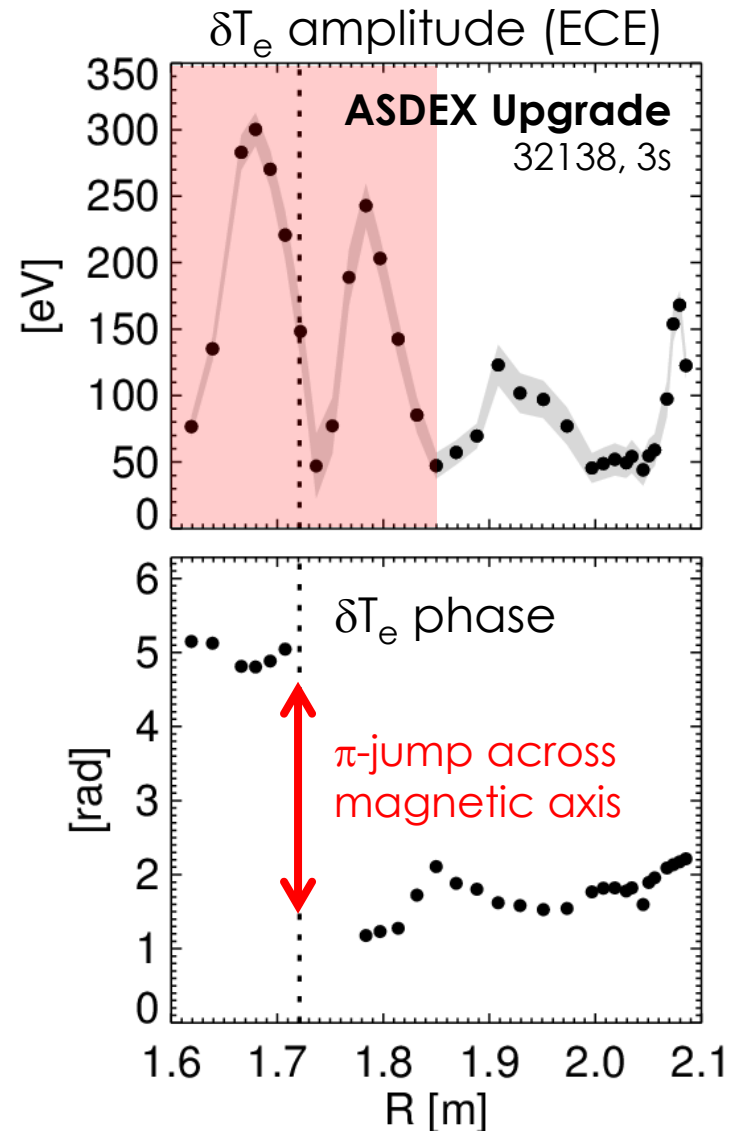
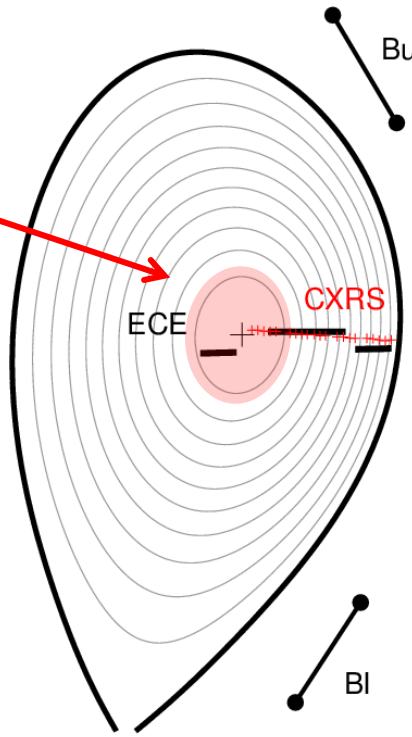
- **A helical core equilibrium forms in hybrid tokamak plasmas perturbed by external $n=1$ fields**
- Helical core used to probe current broadening
- Effect consistent with the MHD dynamo model
- Strong similarity with helical RFP dynamo
- Conclusions and future work

A Helical Core Forms in Hybrid Plasmas Perturbed by External n=1 Fields

- Helical core due to the response of a marginally-stable kink to the externally applied n=1 field
- 1/1 harmonic large due to $q_{\min} \geq 1$

P Piovesan EPS 2016
V Igochine EX/P6-24

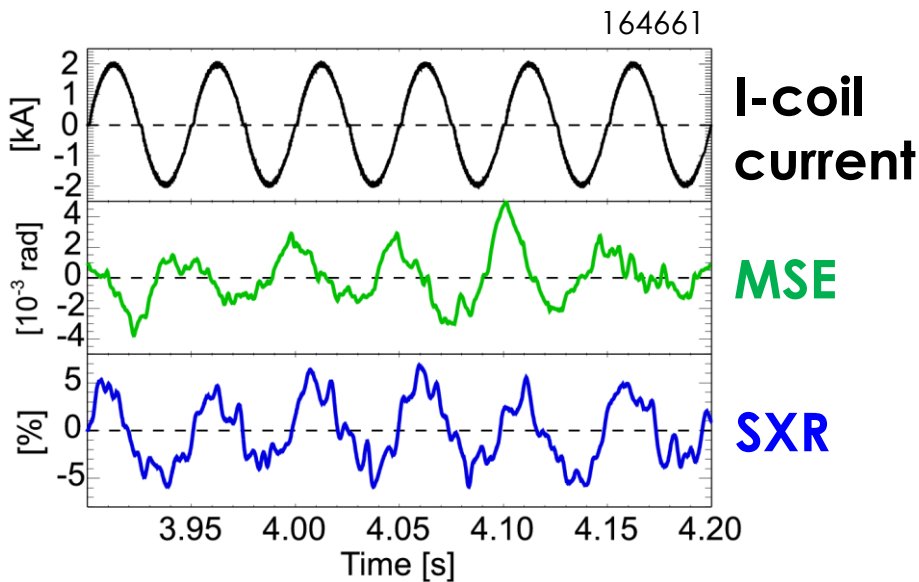
Helical core
detected by ECE
and CXRS as n=1
field rotates at 5Hz



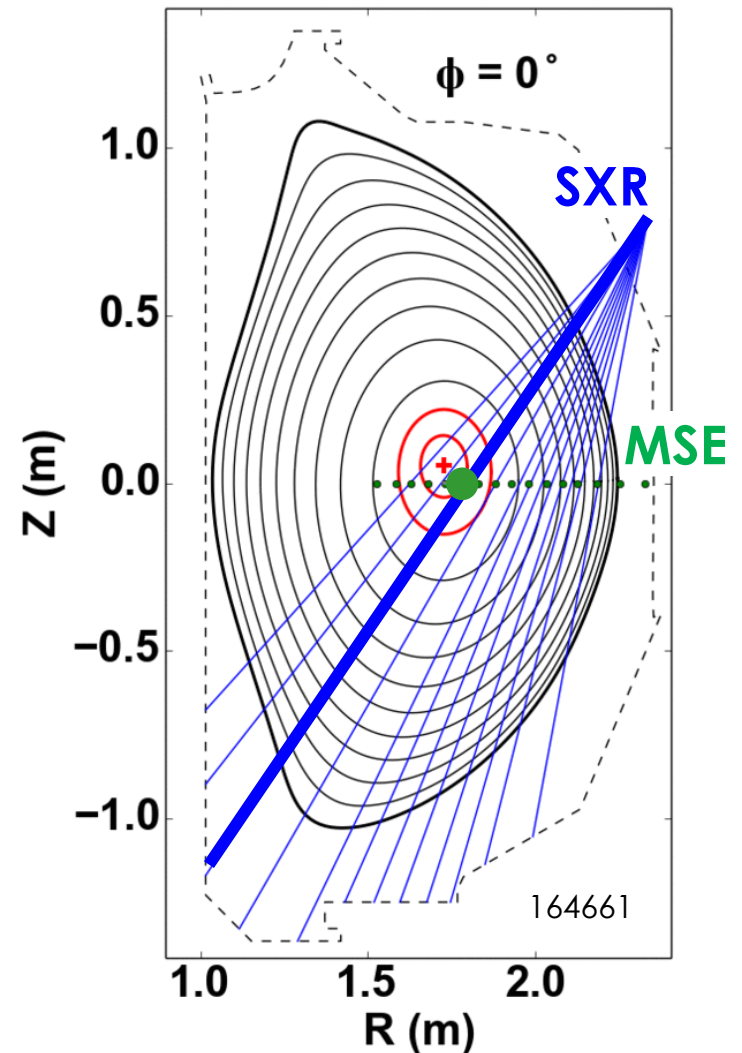
Helical Core Reconstructed by the VMEC/V3FIT 3D Equilibrium Code in DIII-D

- Helical core reproduced in DIII-D
- Reconstructed by VMEC/V3FIT, constrained by **SXR** and **MSE** measurements of the internal helical distortion

A Wingen EPS 2016



Applied $n=1$ field rotates at 20Hz



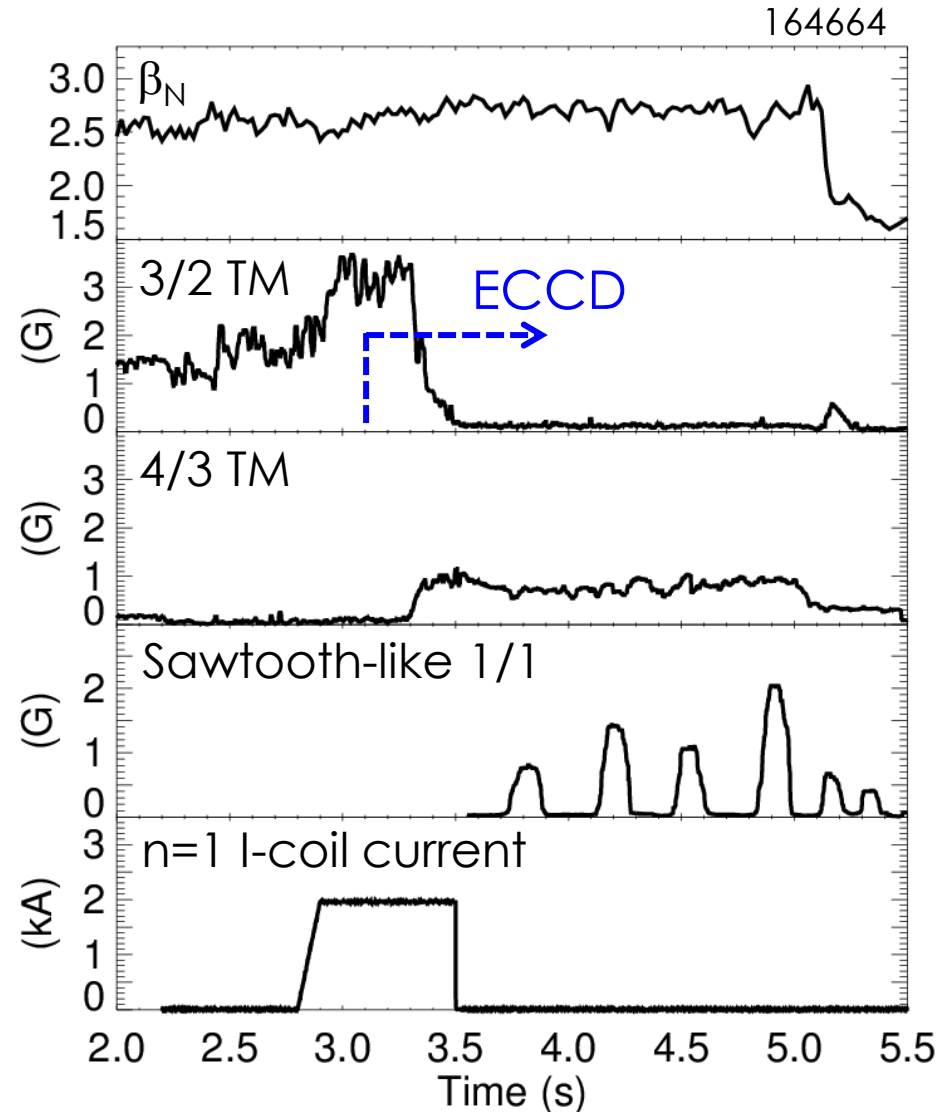
Outline

GOAL: Test the MHD dynamo model of current redistribution in fusion plasmas

- A helical core equilibrium forms in hybrid tokamak plasmas perturbed by external $n=1$ fields
- **Helical core used to probe current broadening**
- Effect consistent with the MHD dynamo model
- Strong similarity with helical RFP dynamo
- Conclusions and future work

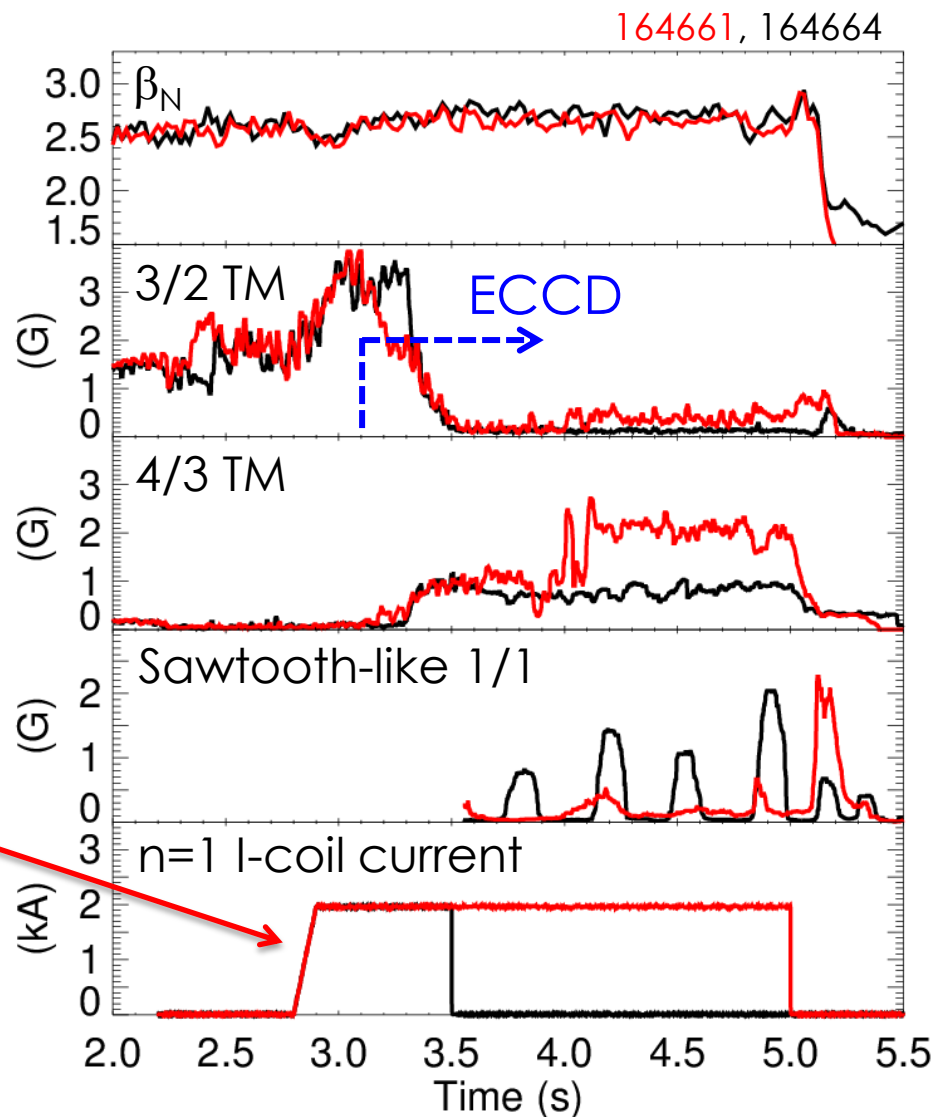
The Helical Core Sustains Hybrid Conditions

- **3/2 tearing mode suppressed by ECCD**
- **Sawteeth come back**
 - Hybrid conditions are lost due to absence of current broadening by the 3/2 TM [MR Wade NF 2005]



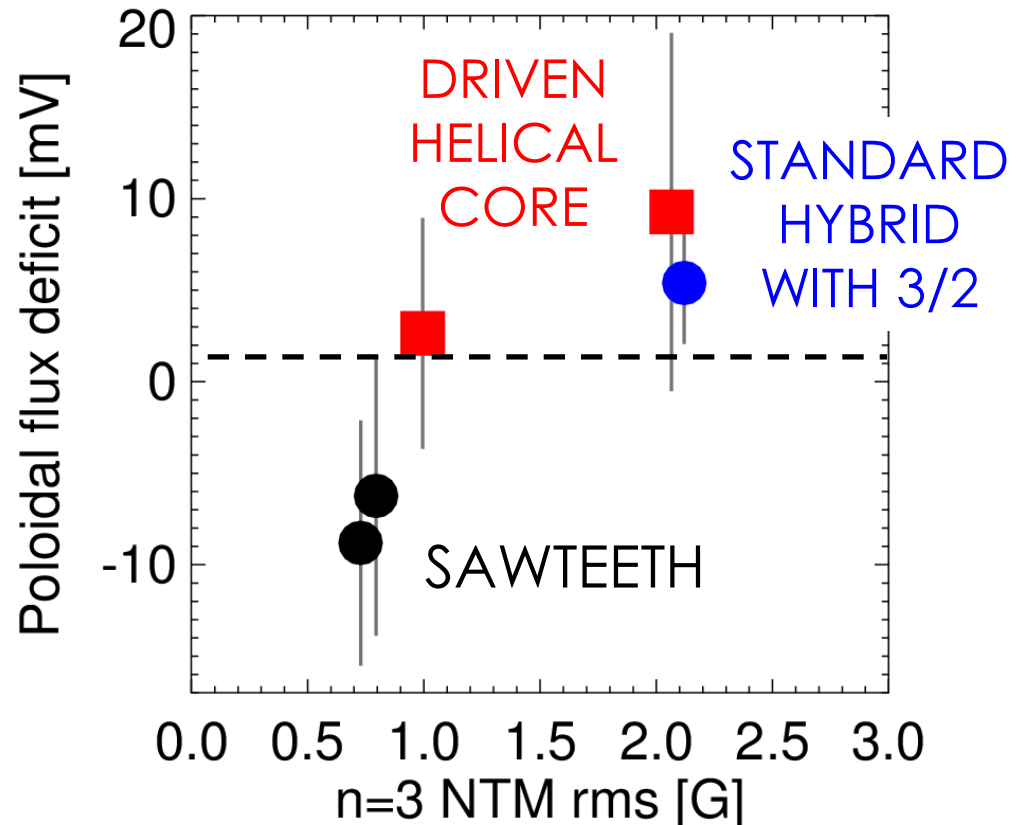
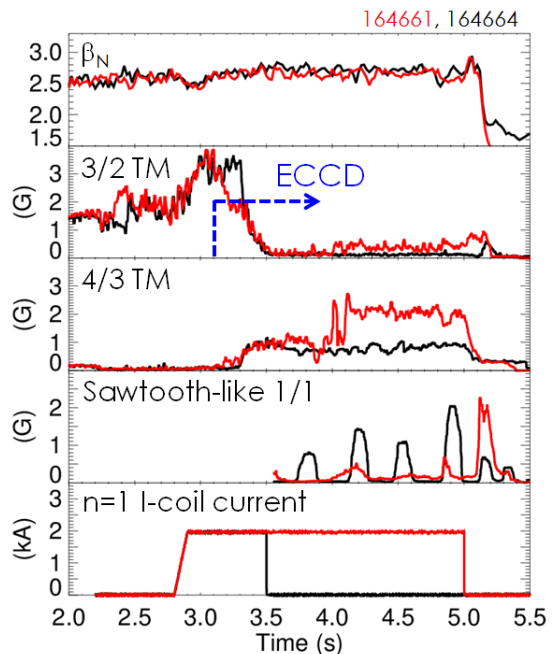
The Helical Core Sustains Hybrid Conditions

- **3/2 tearing mode suppressed by ECCD**
- **Sawteeth come back**
 - Hybrid conditions are lost due to absence of current broadening by the 3/2 TM [MR Wade NF 2005]
- **No sawteeth when the helical core is induced by an external n=1 field**



The Helical Core Causes a Measurable Level of Central Current Broadening

- **Poloidal flux dissipated at a faster rate than it is supplied by coils**
 - The **poloidal flux deficit** is estimated from the time evolution of the reconstructed equilibrium [TC Luce NF 2014] and is proportional to the amount of current broadening in hybrid plasmas [NZ Taylor APS 2016]



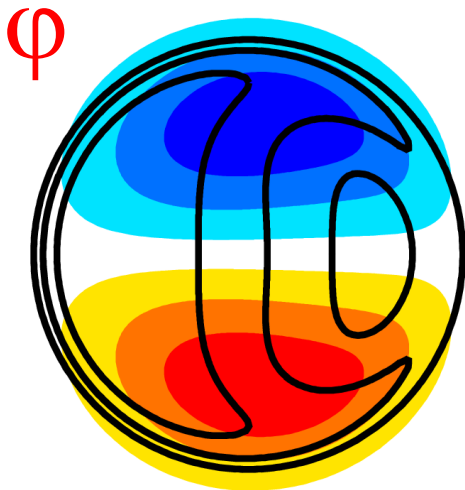
Outline

GOAL: Test the MHD dynamo model of current redistribution in fusion plasmas

- A helical core equilibrium forms in hybrid tokamak plasmas perturbed by external $n=1$ fields
- Helical core used to probe current broadening
- **Effect consistent with the MHD dynamo model**
- Strong similarity with helical RFP dynamo
- Conclusions and future work

Nonlinear MHD Simulations Predict that a Saturated 1/1 Kink Produces a Continuous Dynamo EMF

- An **electrostatic EMF** forms in any helical equilibrium to balance the helical modulation of parallel current, j_{\parallel}



Nonlinear MHD Simulations Predict that a Saturated 1/1 Kink Produces a Continuous Dynamo EMF

- An **electrostatic EMF** forms in any helical equilibrium to balance the helical modulation of parallel current, j_{\parallel}



General result that holds both for **kink** and **tearing** modes.

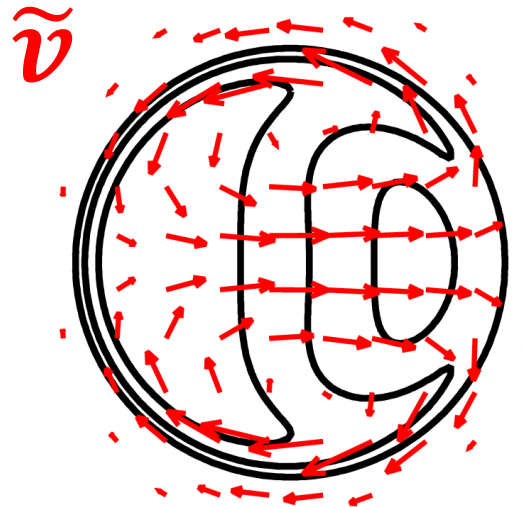
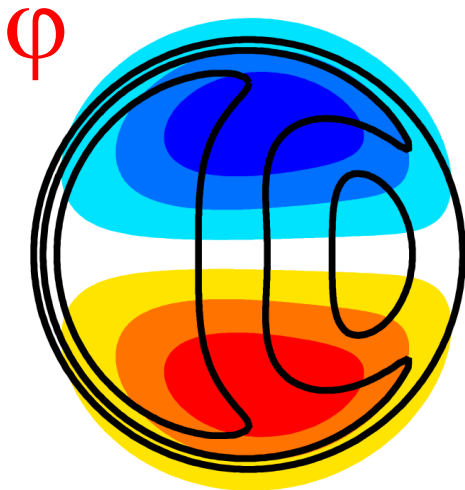
Discovered for **helical RFP** by D Bonfiglio PRL 2005 (SpeCyl).

Recently developed for **hybrid tokamak** plasmas by S Jardin PRL 2015 (M3D-C¹).



Nonlinear MHD Simulations Predict that a Saturated 1/1 Kink Produces a Continuous Dynamo EMF

- An **electrostatic EMF** forms in any helical equilibrium to balance the helical modulation of parallel current, j_{\parallel}
- The associated \mathbf{ExB} **helical flow** $\tilde{\mathbf{v}}$ is a double convective cell



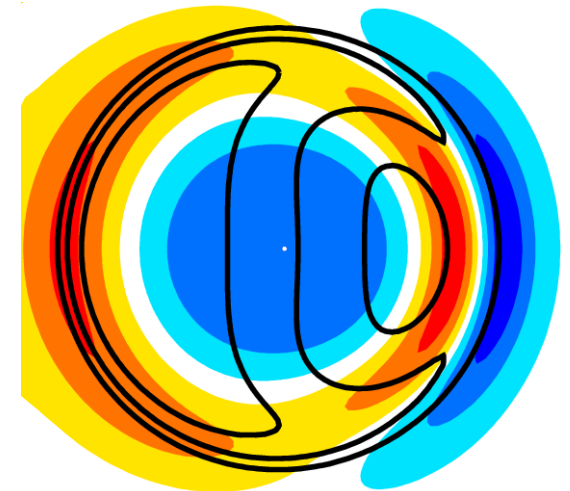
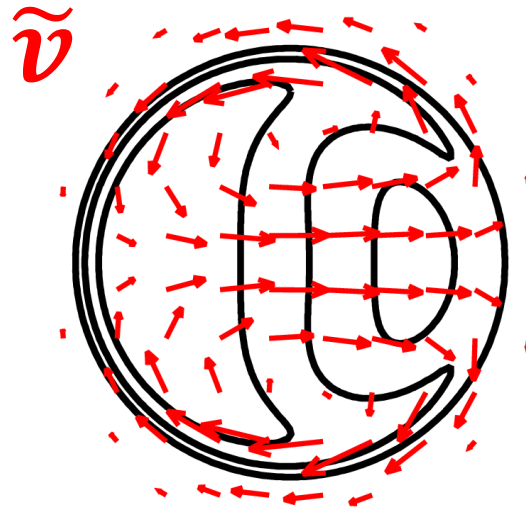
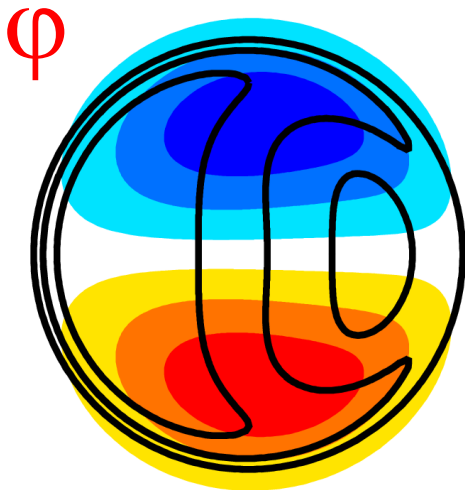
Nonlinear MHD Simulations Predict that a Saturated 1/1 Kink Produces a Continuous Dynamo EMF

- An **electrostatic EMF** forms in any helical equilibrium to balance the helical modulation of parallel current, j_{\parallel}

- The associated $\mathbf{E} \times \mathbf{B}$ **helical flow** $\tilde{\mathbf{v}}$ is a double convective cell

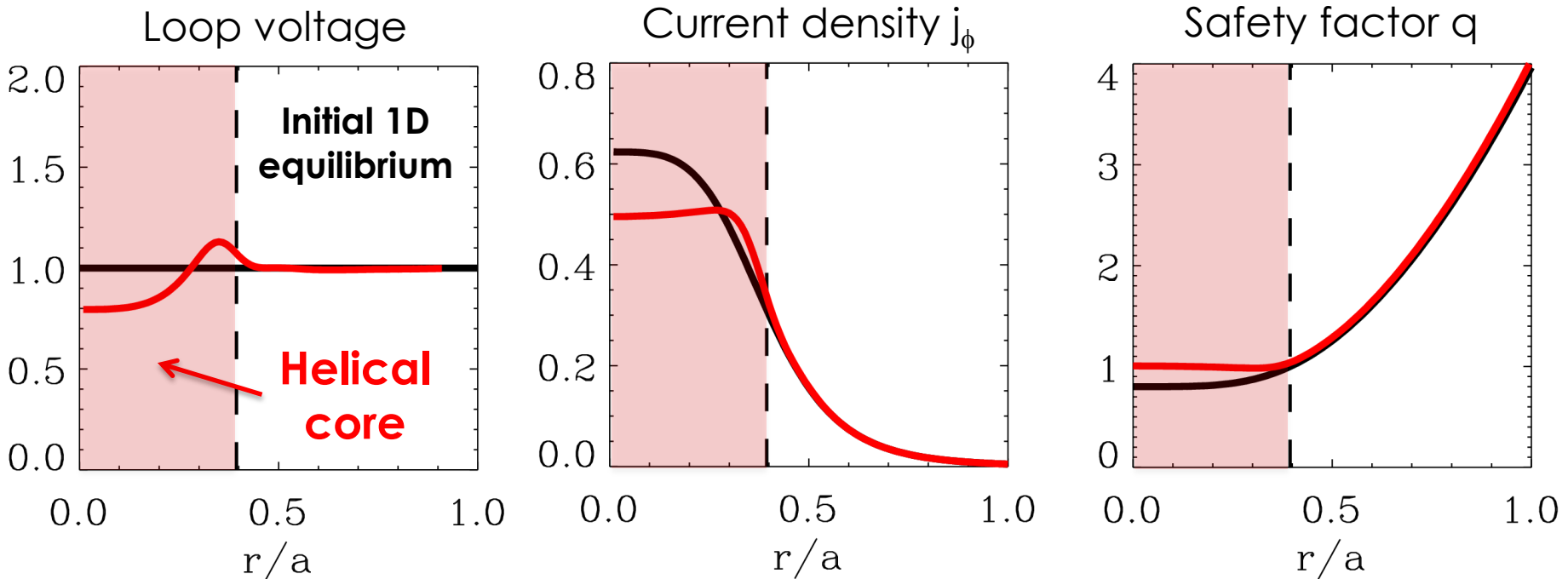
- and produces an **electric field** in parallel Ohm's law, negative in the core

$$\mathbf{E}_{loop\parallel} + \tilde{\mathbf{v}} \times \tilde{\mathbf{b}}_{\parallel} = \eta \mathbf{j}_{\parallel}$$



The Dynamo EMF Due to the 1/1 Kink Redistributes Central Current and Keeps $q_{\min} \geq 1$

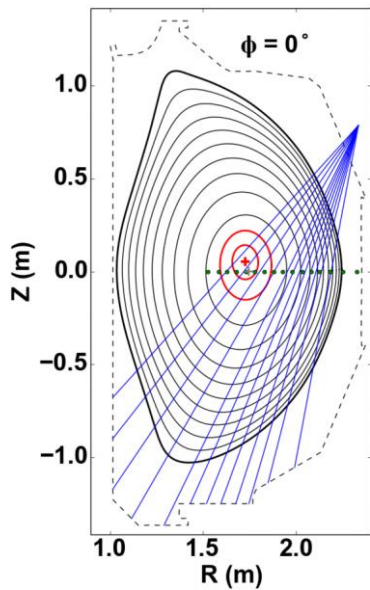
- The mean-field MHD dynamo EMF opposes the applied loop voltage in the core and redistributes central current
- q_{\min} raises near/above 1 as the 1/1 kink nonlinearly saturates in the simulation



More in D Bonfiglio TH/P3-35, this conference

The MHD Dynamo EMF Predicted for the Helical Core is Consistent with the Measured Poloidal Flux Deficit

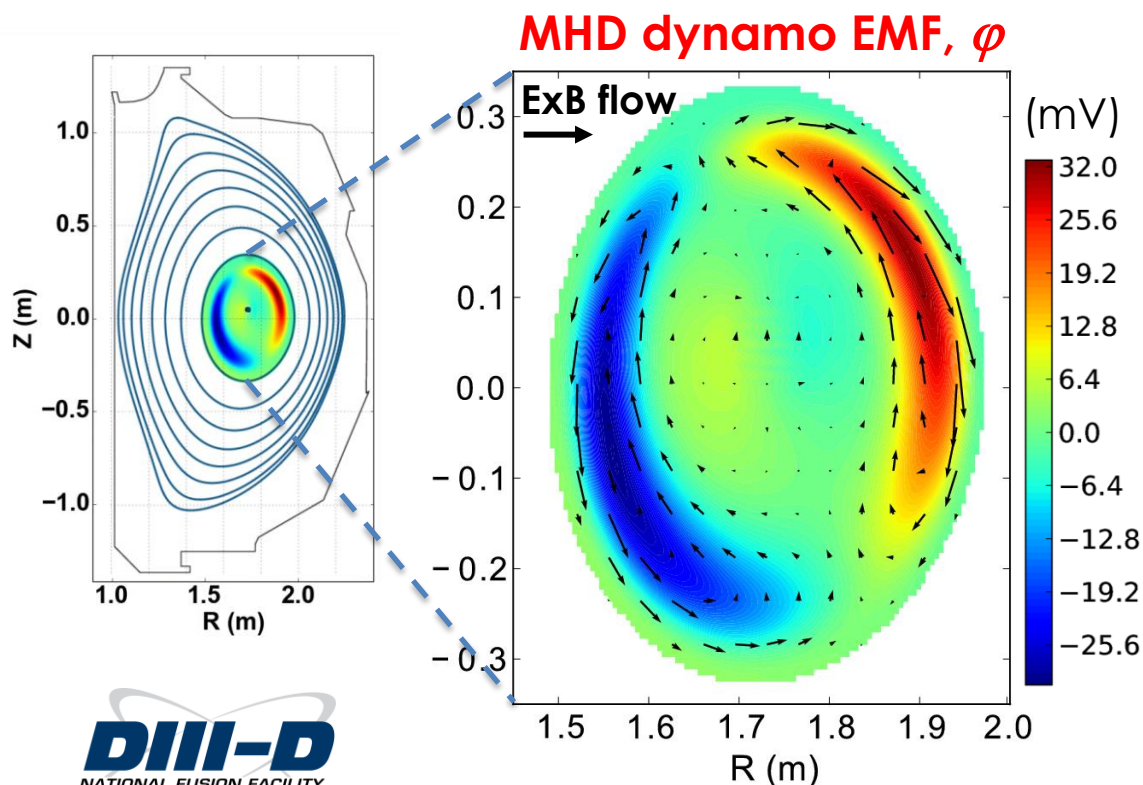
- The MHD dynamo EMF can be calculated for the experimental 3D equilibrium, by balancing Ohm's law over 3D flux surfaces:



The MHD Dynamo EMF Predicted for the Helical Core is Consistent with the Measured Poloidal Flux Deficit

- The MHD dynamo EMF can be calculated for the experimental 3D equilibrium, by balancing Ohm's law over 3D flux surfaces:

$$\frac{\mathbf{B} \cdot (\mathbf{E}_{loop} - \eta_{neo} \mathbf{j}_{Ohm})}{\mathbf{B} \cdot \nabla \theta} = \partial_{\theta} \varphi + q \partial_{\zeta} \varphi \quad \mathbf{j}_{Ohm} = \mathbf{j}_{VMEC} - \mathbf{j}_{CD} - \mathbf{j}_{BS}$$

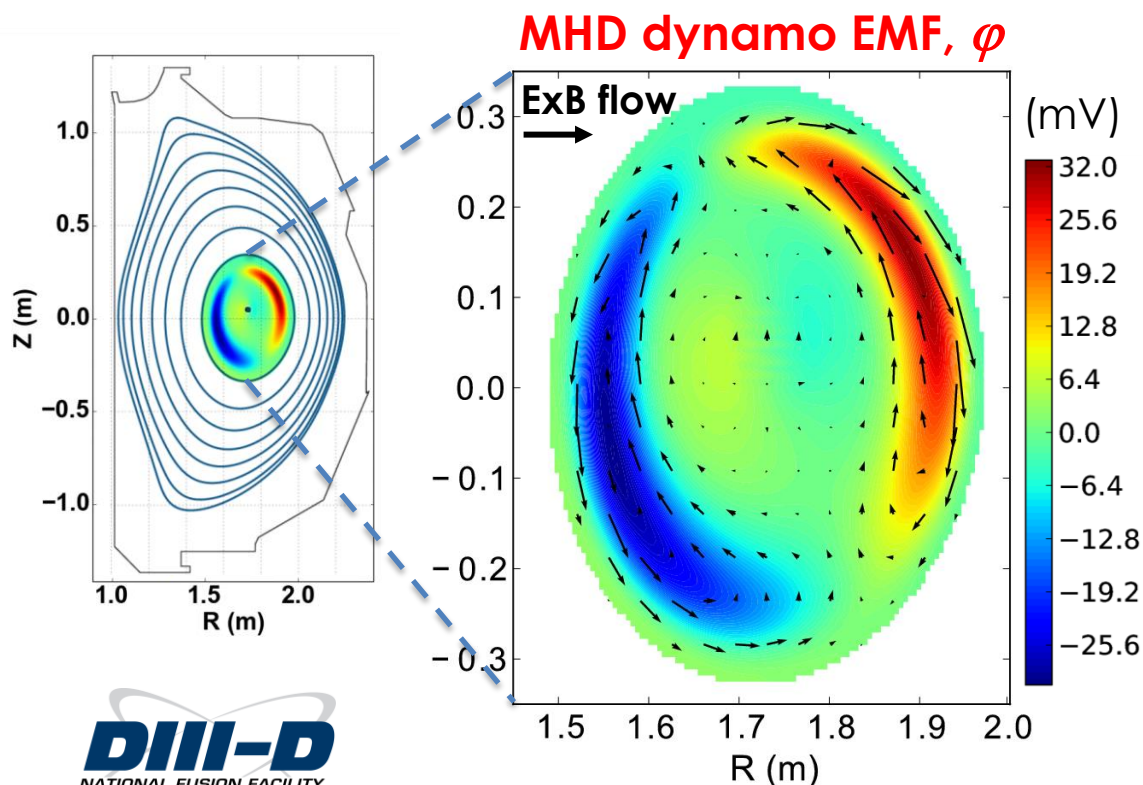


- φ , unknown variable

The MHD Dynamo EMF Predicted for the Helical Core is Consistent with the Measured Poloidal Flux Deficit

- The MHD dynamo EMF can be calculated for the experimental 3D equilibrium, by balancing Ohm's law over 3D flux surfaces:

$$\frac{\mathbf{B} \cdot (\mathbf{E}_{loop} - \eta_{neo} \mathbf{j}_{ohm})}{\mathbf{B} \cdot \nabla \theta} = \partial_{\theta} \varphi + q \partial_{\zeta} \varphi \quad \mathbf{j}_{ohm} = \mathbf{j}_{VMEC} - \mathbf{j}_{CD} - \mathbf{j}_{BS}$$



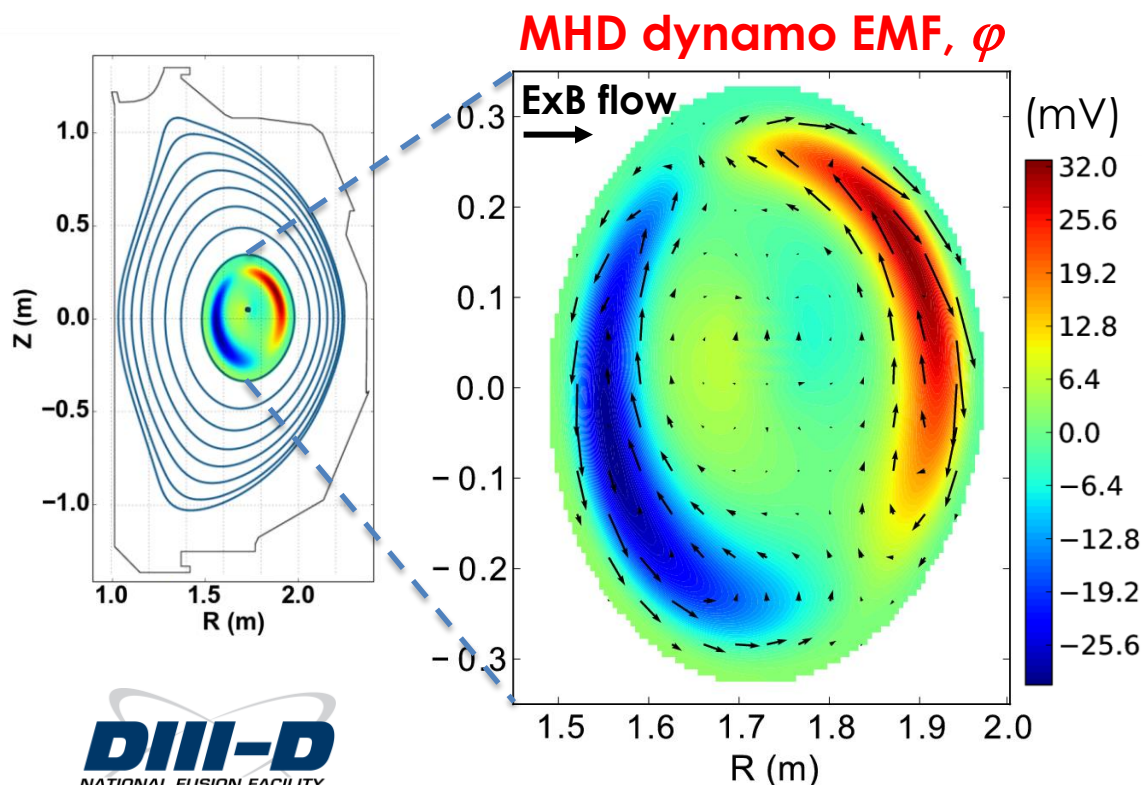
- φ , unknown variable
- 3D quantities: VMEC

The MHD Dynamo EMF Predicted for the Helical Core is Consistent with the Measured Poloidal Flux Deficit

- The MHD dynamo EMF can be calculated for the experimental 3D equilibrium, by balancing Ohm's law over 3D flux surfaces:

$$\frac{\mathbf{B} \cdot (\mathbf{E}_{loop} - \eta_{neo} \mathbf{j}_{ohm})}{\mathbf{B} \cdot \nabla \theta} = \partial_{\theta} \varphi + q \partial_{\zeta} \varphi$$

$$\mathbf{j}_{ohm} = \mathbf{j}_{VMEC} - \mathbf{j}_{CD} - \mathbf{j}_{BS}$$



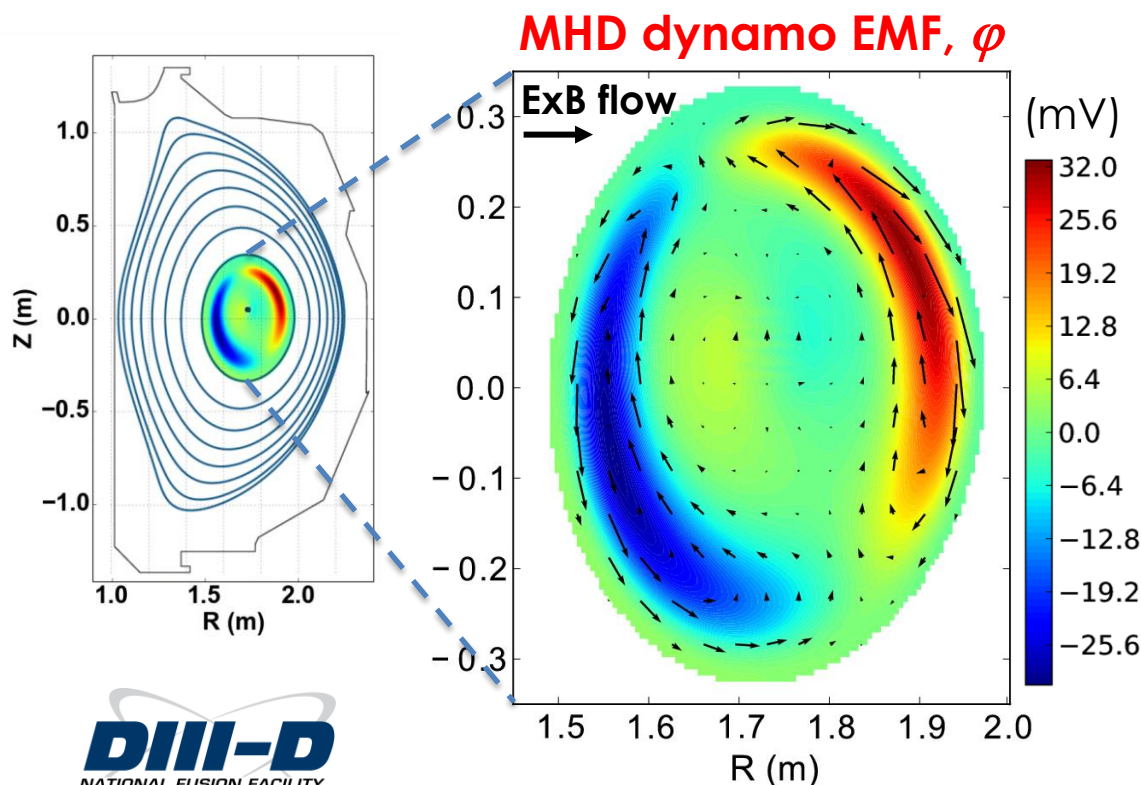
- φ , unknown variable
- 3D quantities: VMEC
- 2D quantities: from ONETWO transport simulation mapped onto 3D equilibrium

The MHD Dynamo EMF Predicted for the Helical Core is Consistent with the Measured Poloidal Flux Deficit

- The MHD dynamo EMF can be calculated for the experimental 3D equilibrium, by balancing Ohm's law over 3D flux surfaces:

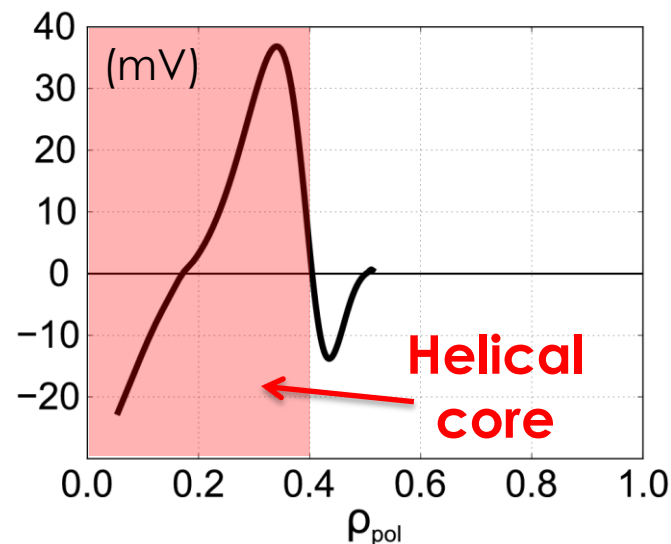
$$\frac{\mathbf{B} \cdot (\mathbf{E}_{loop} - \eta_{neo} \mathbf{j}_{ohm})}{\mathbf{B} \cdot \nabla \theta} = \partial_{\theta} \varphi + q \partial_{\zeta} \varphi$$

$$\mathbf{j}_{ohm} = \mathbf{j}_{VMEC} - \mathbf{j}_{CD} - \mathbf{j}_{BS}$$



$$V_{loop} = \langle -\nabla \varphi \cdot \hat{\mathbf{B}} \rangle 2\pi R_0$$

consistent with measured poloidal flux deficit ~ 10 mV



Outline

GOAL: Test the MHD dynamo model of current redistribution in fusion plasmas

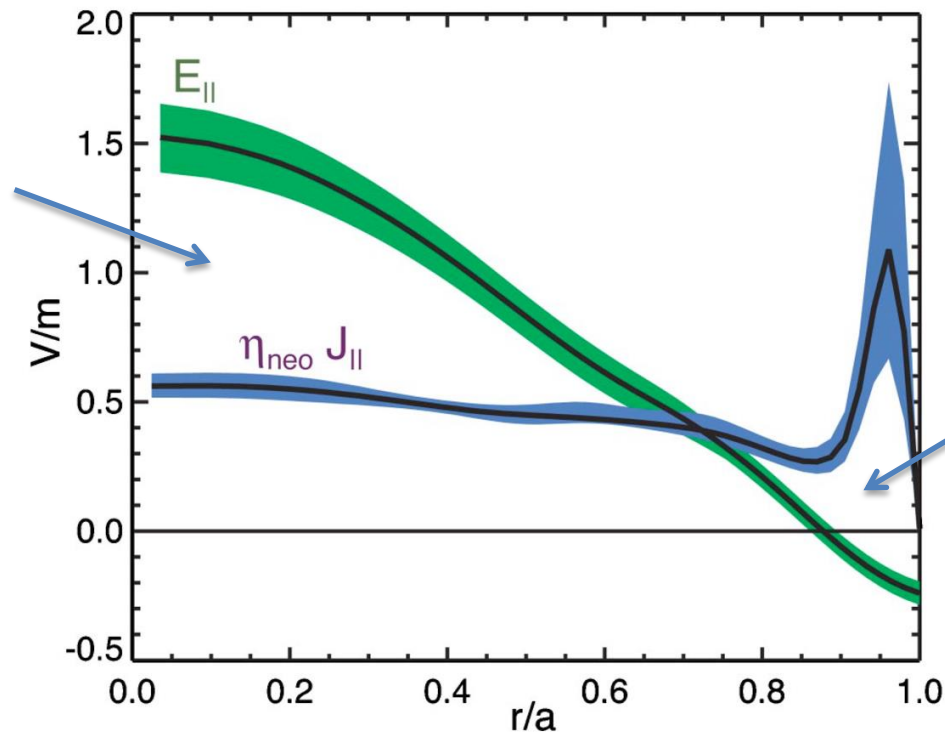
- A helical core equilibrium forms in hybrid tokamak plasmas perturbed by external $n=1$ fields
- Helical core used to probe current broadening
- Effect consistent with the MHD dynamo model
- **Strong similarity with helical RFP dynamo**
- Conclusions and future work

An MHD Dynamo EMF Maintains the Parallel Current Profile in the Reversed-Field Pinch

- **Confirmed by direct measurements of the MHD dynamo terms**

S Ortolani, DD Schnack, Magnetohydrodyn. of Plasma Relaxation 1993,
H Ji PRL 1994, PW Fontana PRL 2000, M Zuin PPCF 2009, DA Ennis PoP 2010, ...

Dynamo limits
the peaking of
central Ohmic
current



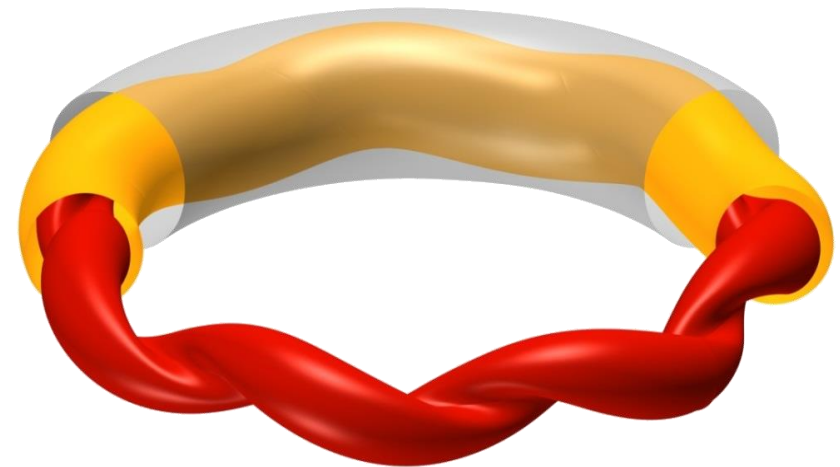
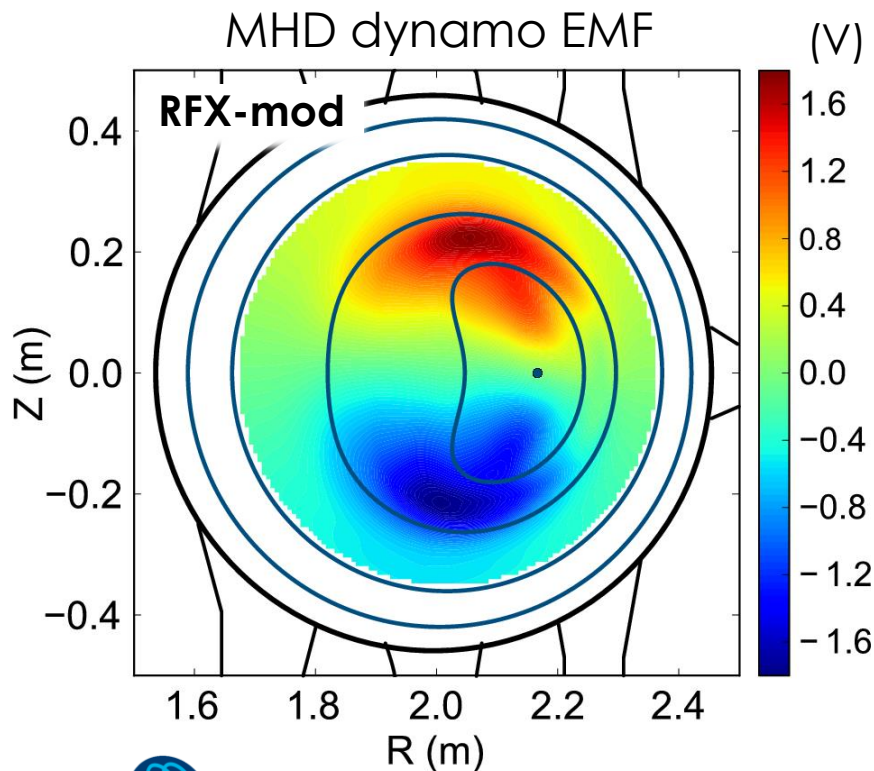
... and drives
edge poloidal
current

From JK Anderson PoP 2005



In Helical RFP Plasmas the MHD Dynamo is Mainly Provided by the Dominant Helicity

- Helical RFP states form spontaneously at high-current $>1\text{MA}$
- Helical equilibrium from VMEC/V3FIT constrained by internal SXR measurements used to predict dynamo EMF in RFX-mod

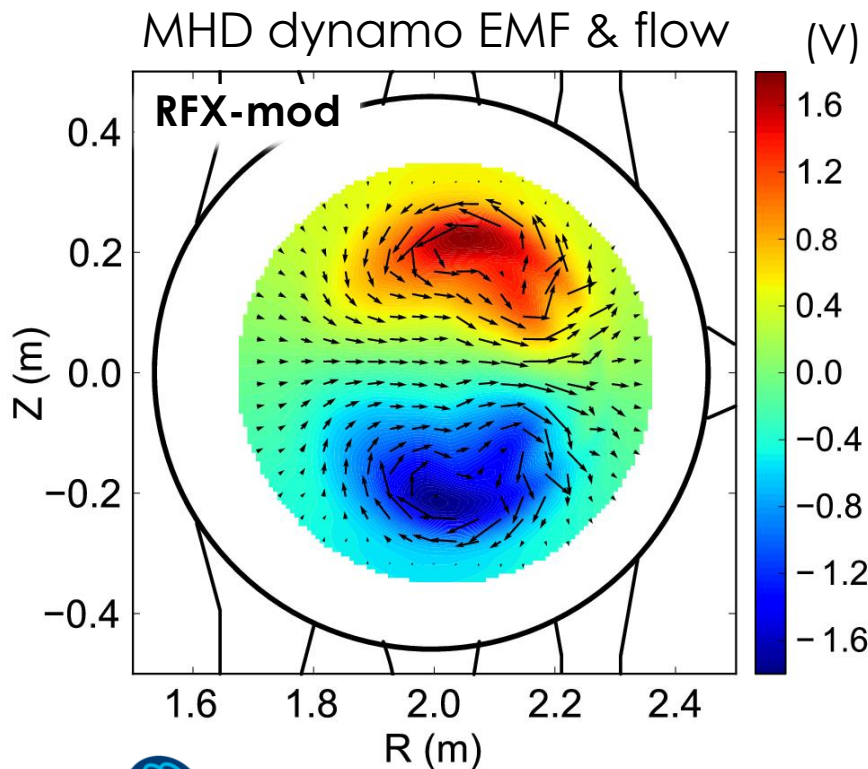


R Lorenzini Nature Phys 2009
D Terranova NF 2013
M Zuin OV/P-2, this conference

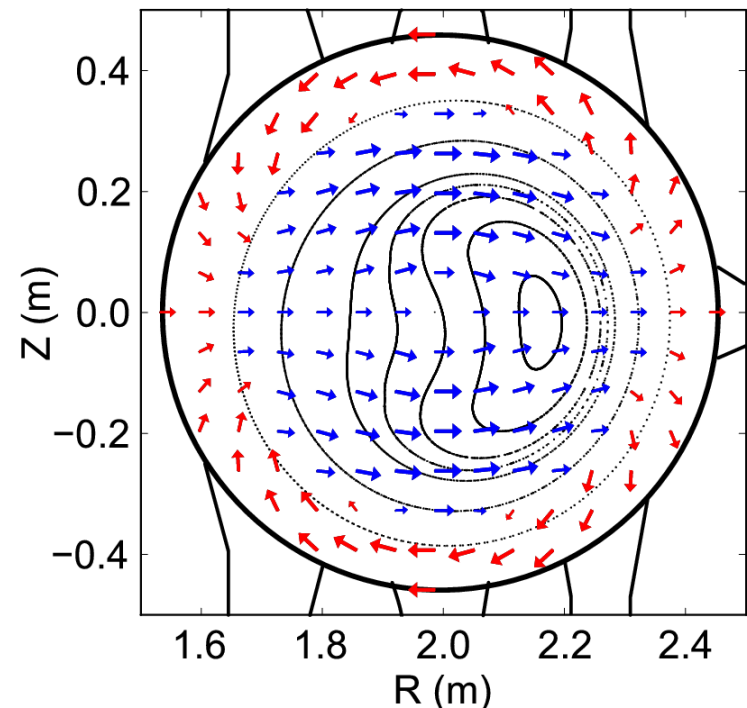


The Predicted MHD Dynamo Flow is Consistent with the Helical Flow Measured in Experiment

- **Both have a double convective cell structure**
 - Measured helical flow $\approx 1\text{km/s}$ much larger than in tokamaks, because it must produce a much larger $V_{\text{loop}} \approx \text{a few Volts}$



Helical flow (Doppler spectroscopy)

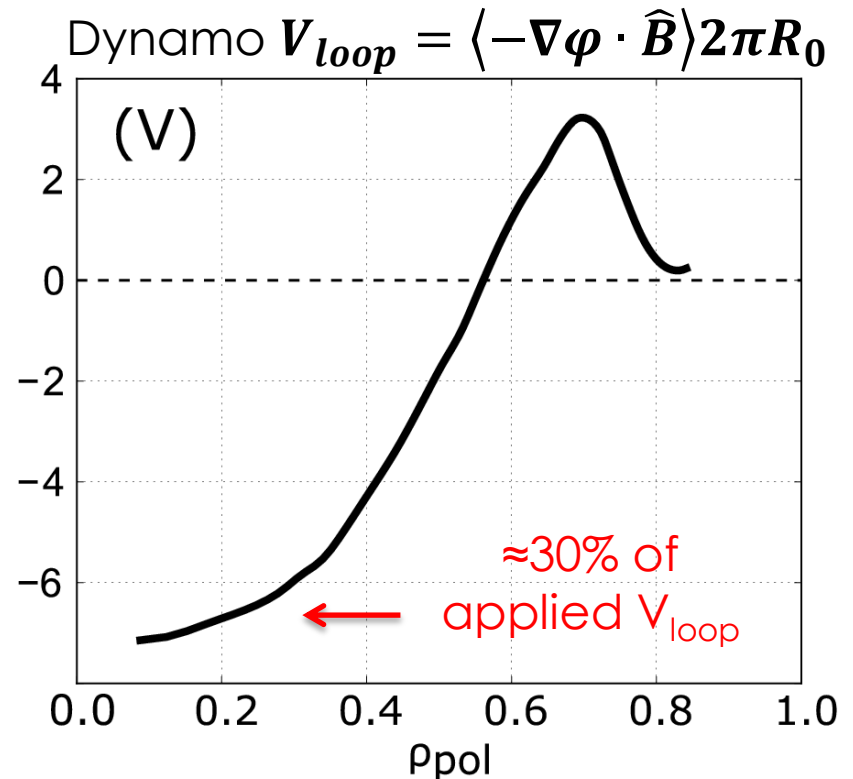
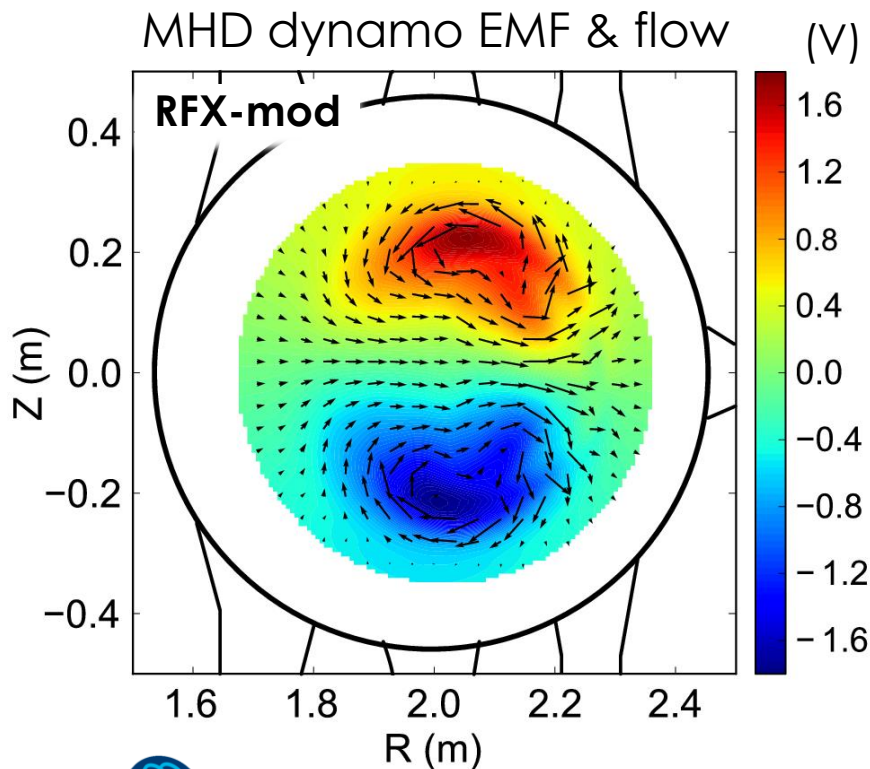


P Piovesan PRL 2004 (MST)
F Bonomo NF 2011 (RFX-mod)



The Predicted MHD Dynamo Flow is Consistent with the Helical Flow Measured in Experiment

- **Both have a double convective cell structure**
 - Measured helical flow $\approx 1\text{km/s}$ much larger than in tokamaks, because it must produce a much larger $V_{loop} \approx \text{a few Volts}$



Outline

GOAL: Test the MHD dynamo model of current redistribution in fusion plasmas

- A helical core equilibrium forms in hybrid tokamak plasmas perturbed by external $n=1$ fields
- Helical core used to probe current broadening
- Results are consistent with the MHD dynamo model
- Strong similarity with helical RFP dynamo
- **Conclusions and future work**

Conclusions

- The **MHD dynamo model** can explain current redistribution by MHD modes in RFP and tokamak plasmas
 - The dynamo EMF can be **directly calculated from 3D equilibria** reconstructed from experimental data
- The dynamo EMF can be produced in a **continuous** way with no need of transient events
 - Expected to work in hybrid plasmas with suppressed ELMs

Conclusions ... and Future Work

- The **MHD dynamo model** can explain current redistribution by MHD modes in RFP and tokamak plasmas
 - The dynamo EMF can be **directly calculated from 3D equilibria** reconstructed from experimental data
- The dynamo EMF can be produced in a **continuous** way with no need of transient events
 - Expected to work in hybrid plasmas with suppressed ELMs
- **Quantitative predictions for hybrid tokamak with a 3/2 tearing and scaling to future machines need more work**
 - Validated nonlinear MHD simulations of tearing modes
 - Add effects beyond 1-fluid MHD, e.g. fast ion transport by MHD
 - More experiments, 3D measurements, rigorous assessment of model/experiment uncertainties

Disclaimer

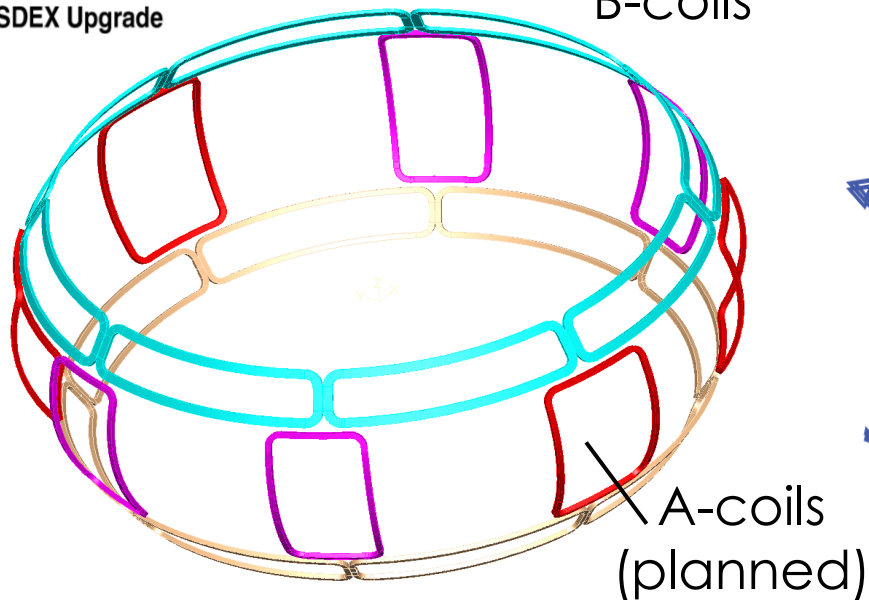
This report was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Government nor any agency thereof, nor any of their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or usefulness of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. Reference herein to any specific commercial product, process, or service by trade name, trademark, manufacturer, or otherwise, does not necessarily constitute or imply its endorsement, recommendation, or favoring by the United States Government or any agency thereof. The views and opinions of authors expressed herein do not necessarily state or reflect those of the United States Government or any agency thereof.

Back-up slides

Non-axisymmetric coils in ASDEX Upgrade and DIII-D

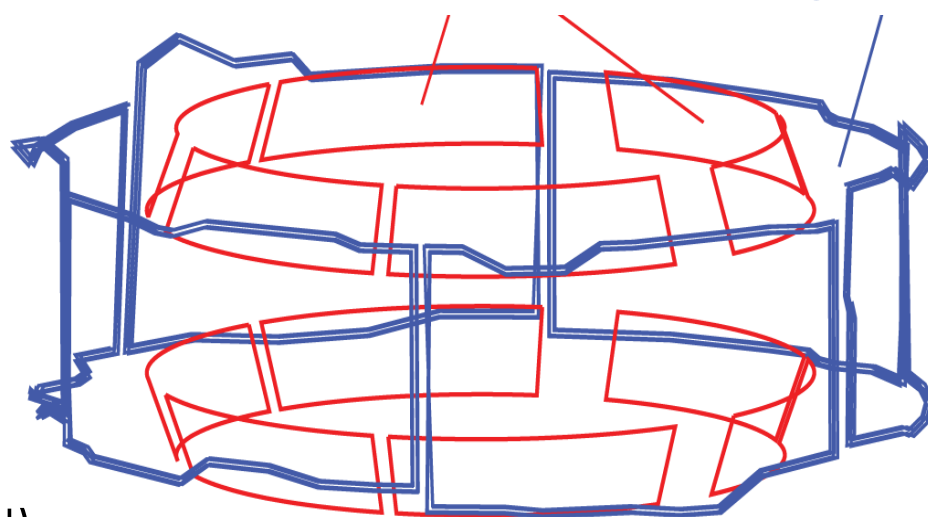


8x2
B-coils



6x2
I-coils

C-coils



Flux states method to evaluate a poloidal flux deficit

- $\Psi_c I_p = W_{\text{coil}} =$ **Energy provided by poloidal coils** coupling with the plasma. **Mutual inductance between coils and the plasma**
 - J_ϕ obtained from EFIT (magnetics + MSE), ψ_c from currents in the 18 poloidal field coils and E-coil

$$W_c = \frac{\mu_0}{4\pi} \iint dV dV' J_c J_\phi / r$$

$$W_c = \int dV J_\phi A_{\phi,c} = \int dR dz J_\phi \psi_c$$

$$\Psi_c = \frac{1}{I_p} \int dR dz J_\phi \psi_c$$

- $\Psi_{\text{kin}} I_p = W_{\text{kin}} =$ **Work done by the electric field within the plasma.** Amount of **poloidal magnetic energy being converted to kinetic energy** in the plasma

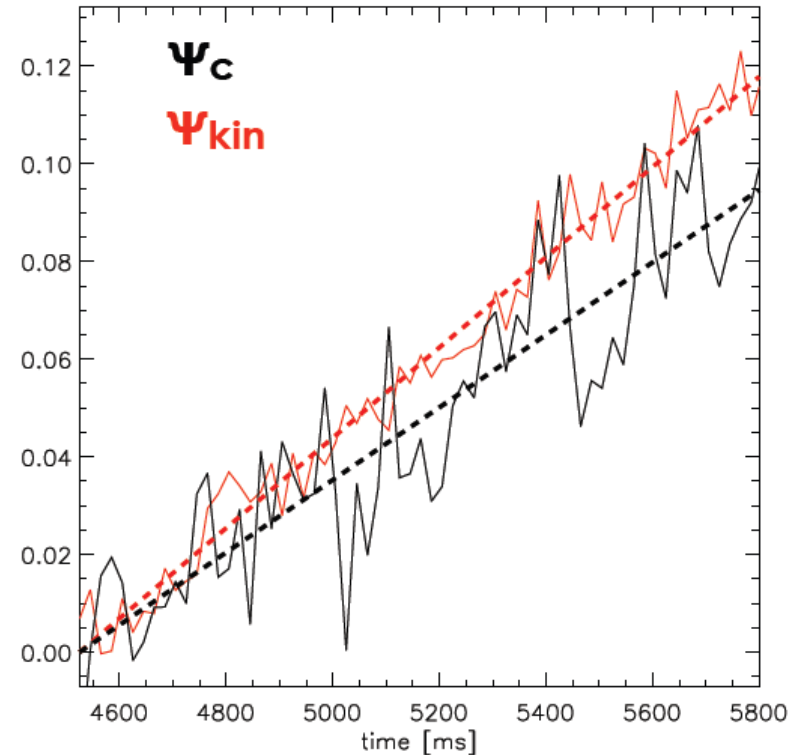
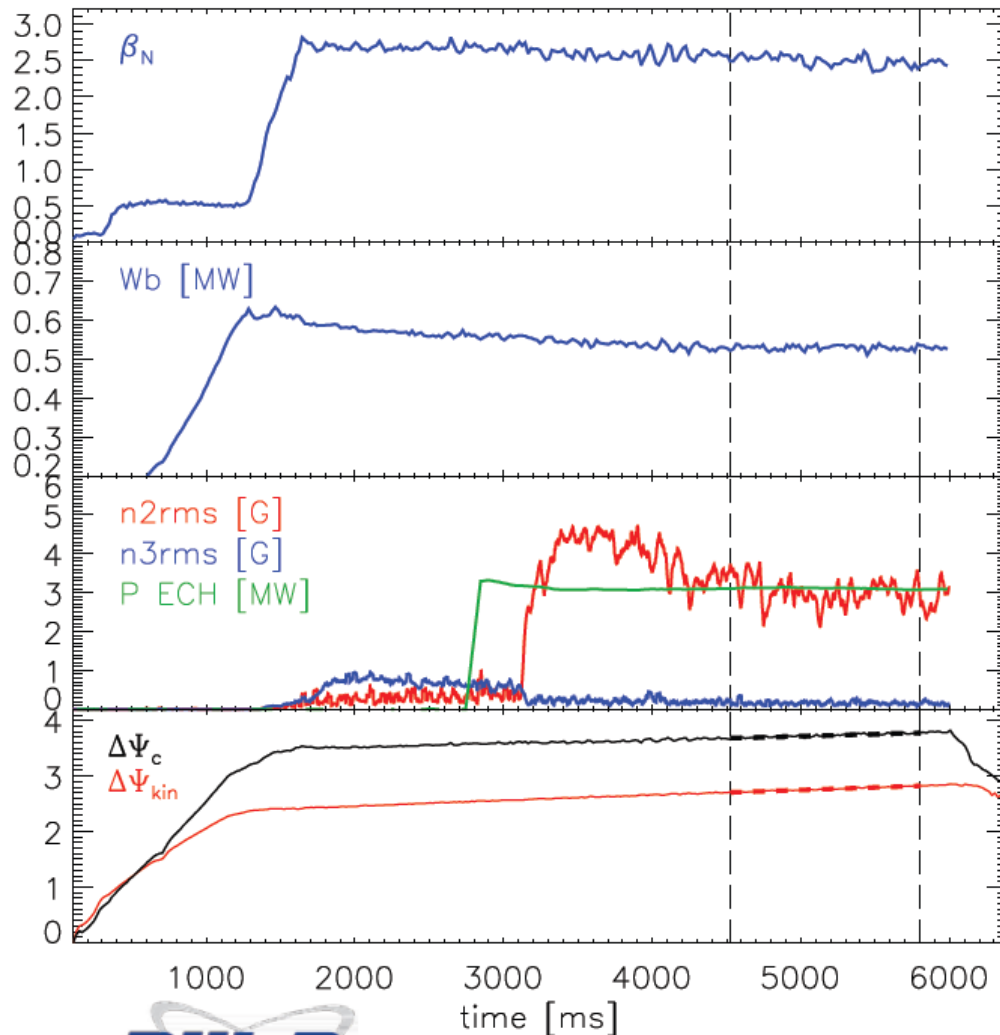
$$\frac{dW_{\text{kin}}}{dt} = - \int dV J \cdot E$$

$$\Psi_{\text{kin}} = \frac{1}{I_p} \int dR dz J_\phi \psi$$

More details in T.C. Luce *et al*, *Inductive flux usage and its optimization in tokamak operation*, Nucl. Fusion **54** 093005 (2014)

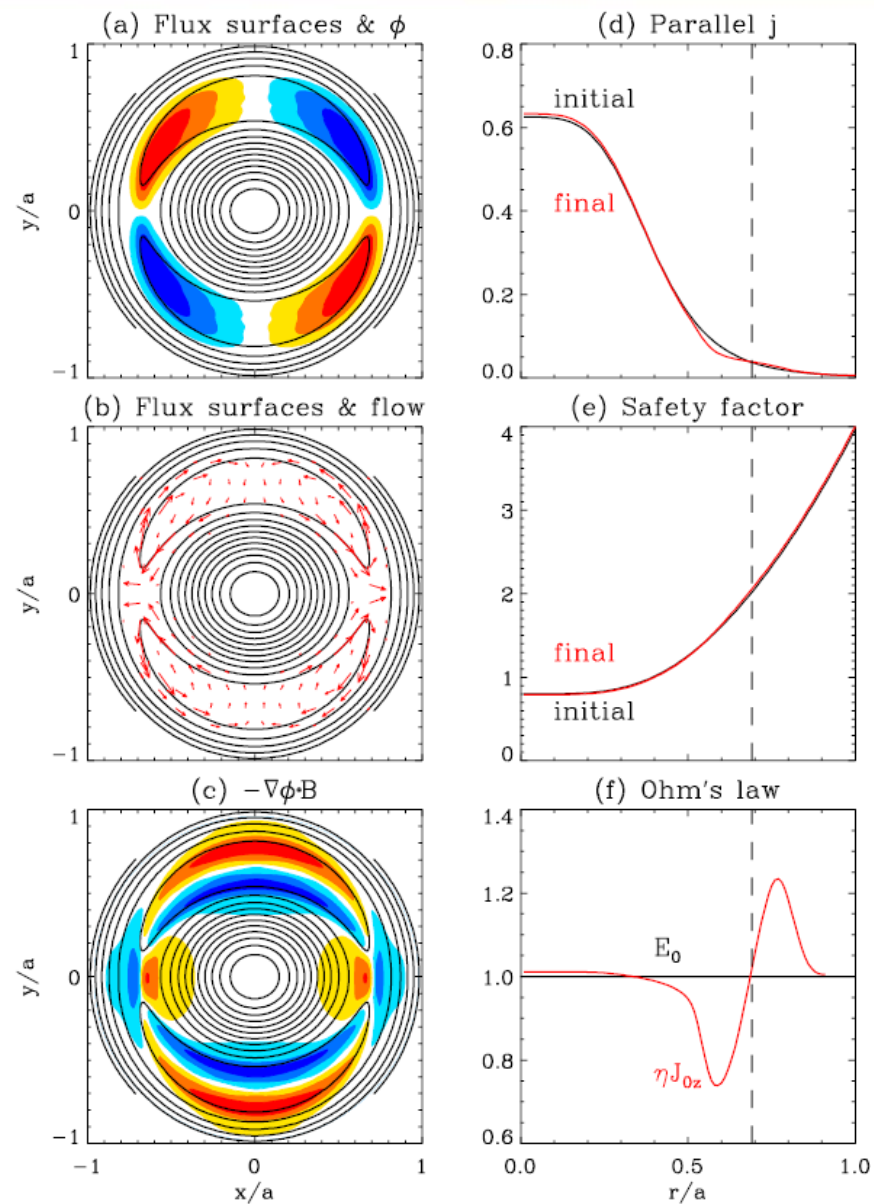
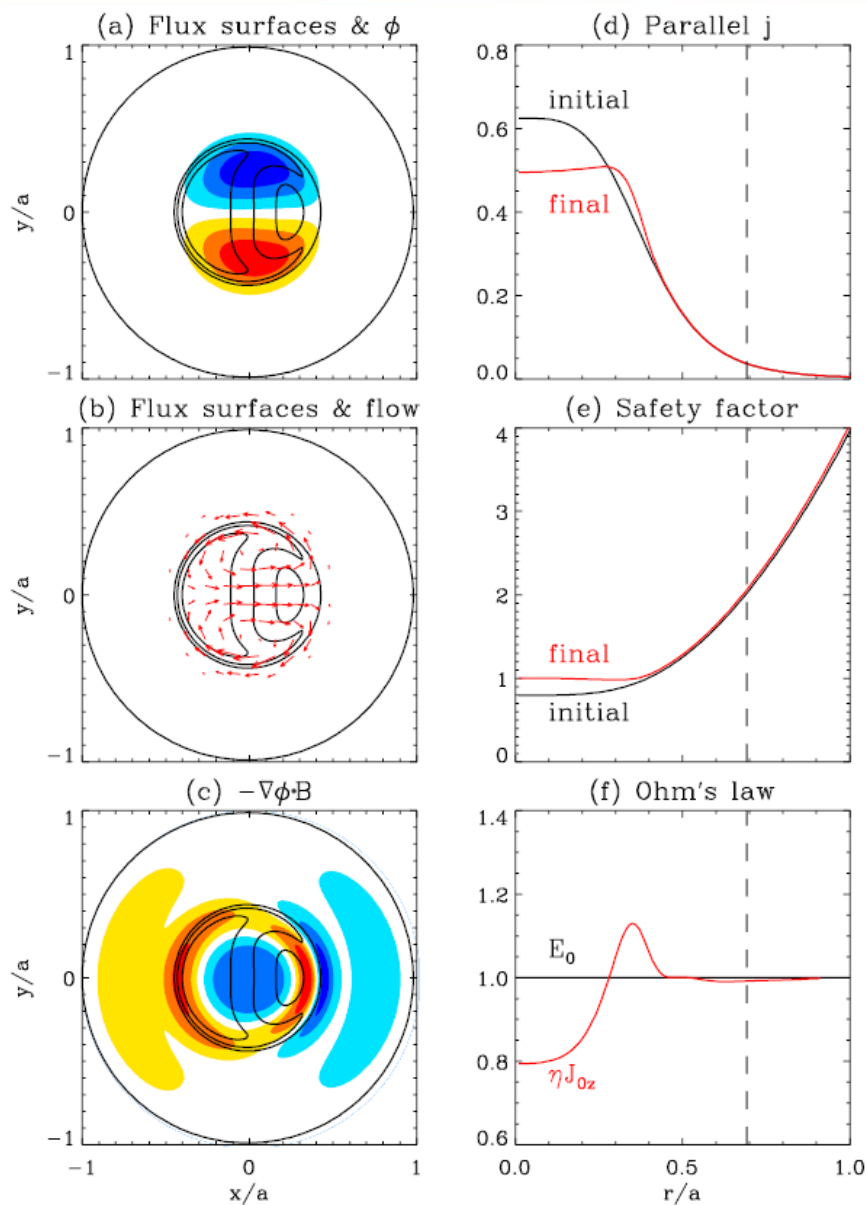
Poloidal flux deficit in a standard hybrid tokamak. Courtesy of NZ Taylor (GA)

162894

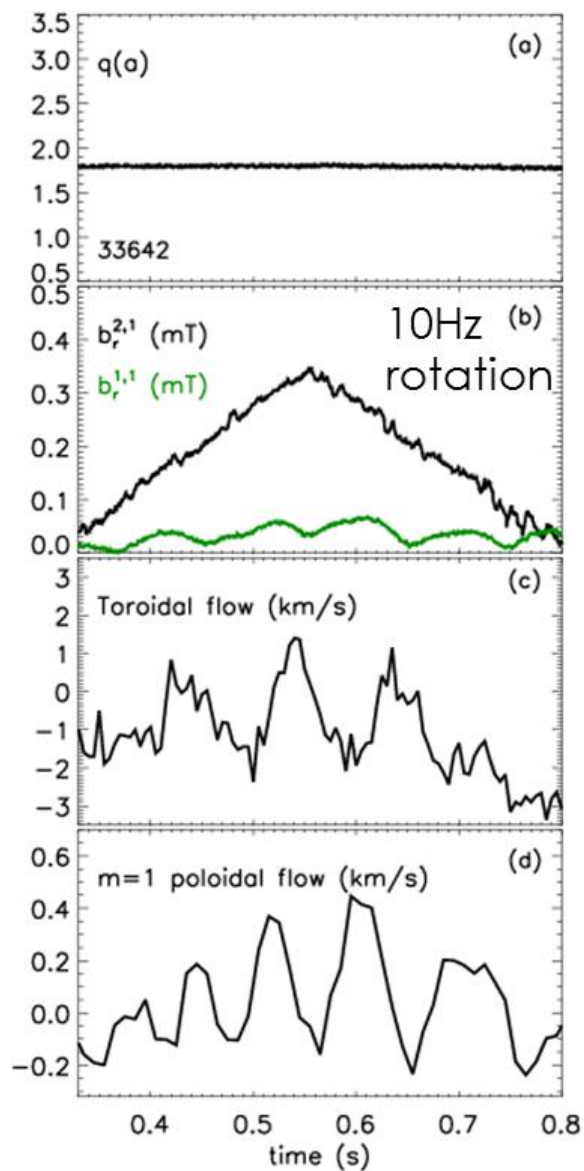


- Hybrid discharge where 3/2 NTM is destabilized
- Counter-ECCD applied to $q=3/2$ surface

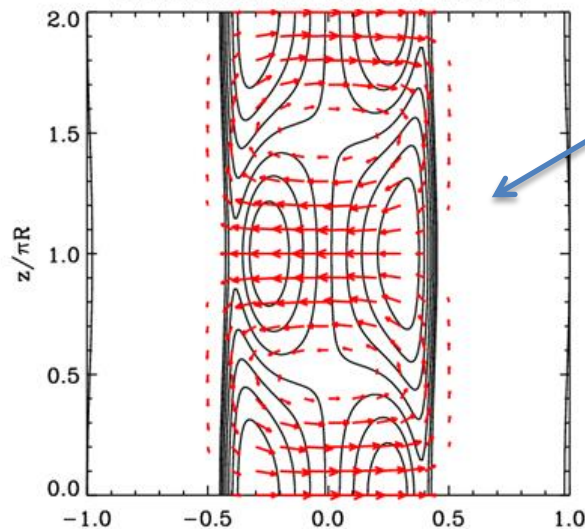
MHD dynamo by 1/1 kink vs 2/1 tearing (SpeCyl code)



Helical flow of 1/1 kink in RFX-mod Ohmic tokamak

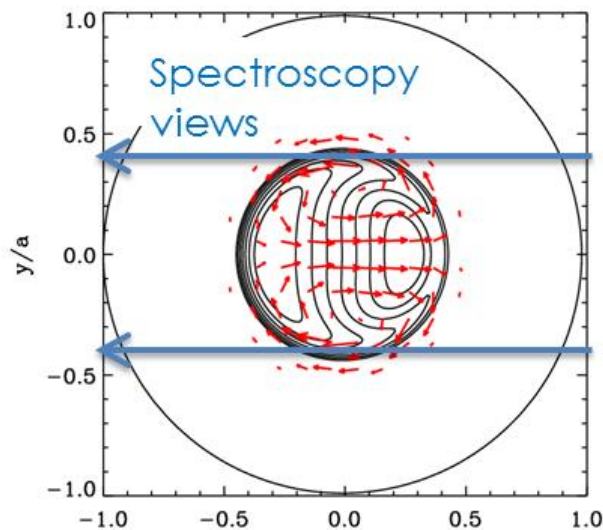


Toroidal cross-section



SpeCyl
nonlinear
MHD
simulation

Poloidal cross-section



L. Piron et al,
submitted to NF

In RFP nonlinear MHD simulations, dynamo is dominated by electrostatic fields even during fast relaxation events

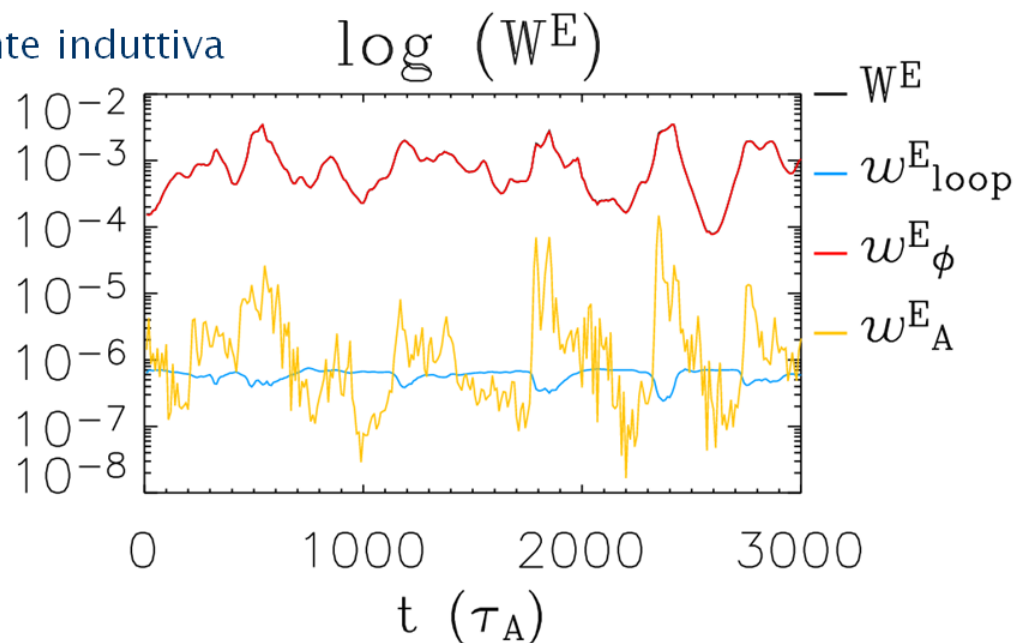
$$E = E_{loop} - \nabla \phi - \frac{\partial A}{\partial t} \rightarrow \text{Componente induttiva}$$

$$W^E = \frac{1}{2} \int_V E^2 = w_{loop}^E + w_{\phi}^E + w_A^E$$

$$w_{loop}^E = \frac{1}{2} \int_V E \cdot E_{loop}$$

$$w_{\phi}^E = \frac{1}{2} \int_V E \cdot (-\nabla \phi)$$

$$w_A^E = \frac{1}{2} \int_V E \cdot \left(-\frac{\partial A}{\partial t} \right)$$



From D Bonfiglio PRL 2005