Overview of Fusion Reactor Materials Study at SWIP

Liu Xiang

Southwestern Institute of Physics, Chengdu, China

Co-authors: J.M. Chen, P.F. Zheng, P.H. Wang, J.H. Wu, Y.Y. Lian, Y.J. Feng, K,M, Feng, Z.Y. Xu, X.R. Duan and Y. Liu

SWIP Southwestern Institute of Physics

Outline

- 1. Motivations
- 2. Plasma facing materials/components.
- W alloys
- W joining with heat sink or structural materials
- 3. Structural materials.
- Ferrite/martensite steels
- Vanadium alloys
- 4. Functional materials.
- Tritium breeder Neutron multiplier
- 5. Summary

Motivations



Plasma facing materials

W alloys



Plasma facing materials

Fast CVD-W coating (up to 0.5 mm/h)



Microstructure characterization



| 0.3-0.5 mm/h |
|--------------|
| 1-3mm |
| 99.9999% |
| >180 W.m/K |
| >99% |
| 430 |
| W/Cu >50Mpa |
| |

Transient event simulations of W materials

disruption-like thermal loads (single shot)



CVD-W seems more sensitivity to the cracking suppression at elevated temperature



Numerical simulations



Numerical simulations



Southwestern Institute of Physics

Plasma facing components

--W/CuCrZr mockups--





Inductive melting + Forging + Cold rolling



Cold rolling

No. melting reprocess O content **Re-melting** Inductive 75 ppm melting 1times 2 Inductive Add C 126 ppm melting powder Inductive Re-melting 2 3 40 ppm melting times 4 Resistance Add deoxidizer heating

Processes

0 1 2 3 4 Time /h Already developed technique:

400 °C

Brazing

Fast cooling

480 °C

950 °C

1000

800

600

400

200

° ℃

emperature

 Traditional furnace + fast cooling + aging
 Fast brazing using Electron-Beam
 Cupper coating + HIPing with fast cooling



Cu-Mn non-crystalline filler

--Design based on molecular cluster theory--



HHF tests--Plasma facing components



- Castellated mockup (30×60×30 mm)
- with 5 mm thickness of W tile Thermal fatigue tests: (increase water cooling to 10m/s)
 1) Screening test:1-9 MW/m2
 2) 1000 cycles at 8 MW/m2
 Surface temperature variation < 10% No visible damage





EMS 60

<u>X. Liu et al,</u> ICFRM-16, Oral

Filler for He cooling divertor targets

Ti-base and Fe-base amorphous brazing alloys



Structural materials-RAFM steel

Composition and fabrication technique optimization--up to 1 ton ingots

•Cooperated with domestic institutes and factories



N as the controllable element, at the upper limit

| Alloy element | Cr | С | W | Та | Mn | V | ✓ N → | х А |
|-----------------|---------|------------|---------|-----------|------------------|---------|------------|--------|
| Content control | 8.5±0.3 | 0.11±0.015 | 1.5±0.2 | 0.10±0.03 | 0.5±0.2 | 0.3±0.1 | 0.02-0.035 | ; |
| Impurity | S | Р | Ti | В | Nb | 0 | Ni | Мо |
| Content control | <0.005 | <0.005 | <0.01 | <0.005 | <0.01 | <0.005 | <0.01 | <0.01 |
| Impurity | Cu | AI | Si | Со | As | Sn | Sb | Zr |
| Content control | <0.01 | <0.03 | <0.05 | <0.01 | As+Sn+Sb+Zr<0.05 | | | |



Properties database of CLF-1 steel (1)



Southwestern Institute of Physics

Properties database of CLF-1 steel (2)

10

Thermal creep properties

- Temperature: 500°C, 550°C, 600°C ;
- Stress level : 250- 300MPa (500°C), 180-260MPa (550°C), 100-160MPa (600°C)

Thermal fatigue properties

- Temperature: room temperature, 300°C, 500°C ;
- total strain of 0.2%~1%。
- Stress rate of 0.1 %/s



The CLF-1 steel shows adequate creep rupture level with low minimum creep rate long rupture time. Some of the tests have been carried out for more than 11000 h and are still in progress. Cyclic softening was observed at all test temperatures under strain controlled fatigue test. The effect of test temperature on fatigue property of CLF-1 steel is very small.

Thermal stability of CLF-1 steel





- No obvious degradation in tensile properties, only a DBTT shift.
 - thermal ageing did not affect the grain size but strongly affected the precipitation behaviors
- M₂₃C₆ type carbides coarsened and agglomerated
- No obvious increase of the MX-type precipitates was observed.
- Higher nitrogen content cause finer MX-type precipitates and the coarsening and agglomeration of $M_{23}C_6$ carbides is much lower.



Neutron irradiation data

--1 dpa data will be available by the end of this year--



High Flux Engineering Test Reactor

Neutron irradiation of CLF-1, CLAM steels

Power: 125MW Maximum Flux: 6.2×10¹⁴n/cm².s

- Post-irradiation examination
- 1) Mechanical properties
- -Tensile properties
- -Charpy impact properties
- 2) Microstructure analysis

More detail:

<u>P.H. Wang, Poster</u> <u>MPT/P8-7</u>





Material preparation for TBM fabrications



•According to the structure design of CN HCCB TBM, different joining technologies will be used for the fabrication of the mock-ups, such as:

•Vacuum diffusion welding (VDW): First wall; Sub-module.

•electron beam welding(EB): Sub-module; The welding between the grids and the plates and pipes of the back system.

•Tungsten Inert Gas (TIG): The welding between the grids and the plates and pipes of the back system. Grid and the first wall.

More detail: <u>K.M. Feng, FIP/3-5Ra</u>

Southwestern Institute of Physics



Structural materials-V alloys

--Heat treatments of V-4Cr-4Ti--



4+0

2.05 🛏

2.05

5=1888

<u>**R14**</u> 12+001

16 ±005

Work cooperated with M

t = 0.25 + 0.01

Repeated cold working followed by aging (SA+CW+A+CW+A) is effective to strengthen the V-4Cr-4Ti alloy at both room temperature and high temperatures, while the ductility loss is comparatively small.

Key point: to stabilize the line defects

Structural materials-V alloys --Dispersion strengthened--

Co-combined particles dispersion strengthened V alloy

Mechanical alloyed V-alloys are expected to work at higher temperatures. Research of such V-alloys is a main work in recent years in the word.



Structural materials-V alloys

--Dispersion strengthened--



Functional materials-Tritium multiplier

Tritium Breeder Material--Li₄SiO₄ pebbles

- Tritium breeding material of the CN HCCB breeder blanket concept is a ceramic breeder in the form of pebble beds
- <u>Reference option:</u> Lithium Orthosilicate (Li4SiO4; OSi); 1.0 mm pebbles produced by melt-spraying process ;SWIP&KUST
- <u>Back-up option:</u> Lithium Metatitanate (Li2TiO3; MTi); 1.0 mm pebbles; SWIP
- For TBM \rightarrow Li-6 enrichment of 80 at%







Schematic drawing of melt-spray for OSi



Fabrication facility



Thermal properties of Li4SiO4 pellets and pebble beds





SWIP Southwestern Institute of Physics

Thermal properties of Be pebbles



Summary

1, Fusion materials study at SWIP is focusing on the applications of HL-2A(M), ITER-TBM and CFETR or DEMO-China.

2, For PFMs/PFCs, several kinds of tungsten based materials are developed, such as oxides and carbides dispersion strengthened W alloy, and a fast CVD-W coating. They shows higher cracking thresholds at transient heat loading. CVD-W indicates a better crack suppression effect at elevated temperature.

3, One kind of RAMF steel CLF-1 is developed for the use of CN-ITER-HCCB TBM. The property data base is being established, including creep tests by more than 11000 h and neutron irradiation data at 0.3-1 dap (by the end of this year). Meanwhile Its qualification is under way according to ITER requirements.

4, An engineering scale V-4Cr-4Ti alloy (30kg) was prepared. Further strengthening by combined Y, Ti and SiC particles was carried out and V-4Cr-4Ti-1.5Y-0.3Ti₃SiC₂ shows better strengthened effects.

5, Beryllium and Li_4SiO_4 pebbles as neutron and trillium multipliers have been developed and characterized.



Southwestern Institute of Physics

Jhanks for your attentions !

SWIP Southwestern Institute of Physics