TH/2-3

## **Gyrokinetic simulation of blob** transport and divertor heat-load

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**SciDAC-3 Center for Edge Physics Study** 



#### **Outline**

- Introduction of the problem
- Introduction to XGC1
- Gyrokinetic edge blobs
- Divertor heat-load footprint and  $I_P$  scaling from XGC1
- Status of the XGC1 development
- Conclusion and discussion

#### Core Gyrokinetic Turbulence Code

GEM (U of Colorado) models PBM and KBM in DIII-D H-mode pedestal (Wan PRL 2012) and nonlinear ELM (Wan PoP 2013)



#### Edge Gyrokinetic Turbulence Code XGC1 - full x-point, neutrals





#### Divertor heat-load width is a serious issue for ITER and future tokamak reactors

- If extrapolated from the present-day trend ( $\propto 1/I_P$ ),
  - ♦ Divertor heat-load width in ITER would be  $λ_q$ ≈1mm when mapped back to the outboard midplane, and
  - The localized heat-load would far exceed the material tolerance limit.
- Unanswered critical questions:
  ♦ Will the 1/I<sub>P</sub> trend hold for ITER?
  ♦ How can we control λ<sub>q</sub>?
- Physics understanding is needed for reliable and predictive answers.
- Scrape-off plasma is in nonequilibrium kinetic state
  - Kinetic neoclassical + turbulence simulation is needed
  - ♦ Difficult to simulate!



#### **Total-f Gyrokineic code XGC1 in diverted geometry**

#### XGC1: X-point included Gyrokinetic Code 1

- Edge plasma is in a non-thermal equilibrium state and requires a non-perturbative kinetic simulation
  - Heat and momentum (and particle) flux from the core
  - Losses to material wall with neutral recycling, radiative loss, wall-sheath
  - Magnetic separatrix geometry: Orbit loss and X-point transport
  - Steep pedestal, with the gradient-width being ~ ion banana width
  - Blobs:  $(\delta n_{max} \delta n_{min})/\langle n \rangle = \mathcal{O}(1)$
  - Non-Maxwellian, requiring nonlinear Fokker-Planck collisions.

#### Nonlocal self-organization and overlapping multi-scale physics

- Neoclassical, turbulence, (logical) sheath, and neutral particles with atomics physics (and wall) self-organize together non-locally
- Core-edge self-organization: artificial core-edge boundary is undesirable.

#### **XGC1** is designed to study such plasmas

- -- Requires extreme scale computing (2014 total award ~300M hrs)
- -- Efficient scalability to extreme scale (maximal Titan/Mira/Edison)

# XGC1 determines the E<sub>r</sub> profile automatically, from the multiscale physics of orbit loss, neoclassical, turbulence, neutral particles and (logical) wall-sheath.

Equilibrium  $E_r$  evolution and feedback is important in the edge, while being more passive in the core.

→  $E_r$  in edge needs to be "determined," instead of being calculated from given plasma n, T, V profiles using force balance, as usually done in core.



DIII-D H-mode plasma with ITG turbulence

## $\lambda_q$ versus $I_p$

- $\lambda_q$ : Divertor heat-load width mapped back to outboard midplane
- Three calculated  $\lambda_q$  points approximately line up with the  $1/I_p$  curve
- The  $I_P$ = 0.68 & 1.26 MA cases are manufactured from the 0.97MA case by multiplying a uniform constant to  $B_P$ , while keeping the plasma profiles and the flux surface shape unchanged.
- Agrees with the neoclassical scaling found for DIII-D, NSTX and C-Mod from XGC0 in 2010 [2010 DOE JRT Report]
- Agrees with the simple heuristic neoclassical argument by R. Goldston [Nucl.Fusion, 2012]

lp (MA)	λq (mm)
0.68	7.4
0.97	5.1
1.26	4



## As soon as the drift-kinetic electrons were added to the gyrokinetic ions, the edge blobs appeared.

- DIII-D H-mode 96333
- The simulation ended at ~ 1ms.
- No core-edge boundary used
- Birth and life of the edge blobs being studied.
  - ♦ Birth of blobs through ExB shearing can be seen.



## **Synthetic diagnostics**



- Synthetic blob detection/analysis software has been developed (J. Lang and M. Churchill)
  - Blobs are found to carry not only the mass, energy, and momentum but also the vorticity that could affect the L-H and H-L transitions.
- Data from an extreme scale XGC1 simulation is too big for I/O.
  - We are placing the synthetic diagnostics in the code (HPC compute memory) for in situ analysis.
  - Poloidal blob speed from XGC1 is similar to experimental observation in H-mode (Boedo et al., Phys. Plasmas 2003)

#### Poloidal potential variation in the scrape-off layer is also calculated in XGC1 (with nonlinear collisions and neutrals)



## **Divertor heat-load width in attached plasma**

- Heat-load footprint has been measured from the three XGC1 simulation points
  - DIII-D H-mode shot #096333
- Electrostatic blobby turbulence, neoclassical physics and nonlinear collisions are included self-consistently.





 Calculated heat-load width and I<sub>P</sub> scaling are similar to experiment

◦ XGC1:  $\lambda_q$  (midplane) ∝1/I<sub>P</sub>

 Simulation results should be compared with blue experimental dots (2.0 < B<sub>T</sub> < 2.2 T)</li>

1 ms simulation time for approximate steady state? Non-thermal kinetic equilibration process is much faster than the fluid equilibration process based on thermal equilibrium diffusion coefficients.

## $\lambda_q$ is dominated by ions in DIII-D



- $\lambda_q = 5.1 \text{ mm at}$ I<sub>P</sub>=0.97MA
  - Neutral particles play an insignificant role in this attached plasma
- λ<sub>q</sub> is closer to ion orbit spreading width (~3mm, represented by the red flat top) than the radial blob size (>1cm)

Heat-load spreading by blobs (represented by  $\lambda_{qe}$  ~2mm in the figure) is masked by the ion orbital spreading.

## **Physical Interpretation of the DIII-D results**

 Fast parallel particle motion allow only partial spreading of the heat-load width by blobs before hitting divertor plates

 $-\lambda_{qe} \sim 2 \text{ mm}$ 

- Ion orbit excursion  $\Delta_i$ dominates over the  $\delta ExB$ convective spreading by blobs
- In ITER, Δ<sub>i</sub> ≤1mm, but the meso-scale blob size ∝(ρ<sub>i</sub>a)<sup>1/2</sup> may remain similar



→ Dominance of  $\Delta_i$  could be lost → breaking of the 1/I<sub>P</sub> scaling?

#### An ITER simulation to be done soon to answer this important question

## NSTX - Collisional effects on $\lambda_{\alpha}$

- If the neoclassical orbit width is important for prediction of  $\lambda_q,$  shouldn't the collisions broaden  $\lambda_q?$
- NSTX without collisions ( $\lambda_q \propto 1/I_P^{0.8}$ )
- Collisions are found to broaden  $\lambda_q$  significantly ( $\lambda_q \propto 1/I_P^{1.45}$ )



## **Status of the XGC1 development**

- XGC1 is acquiring E&M capability, including reduced MHD modes
  - Heat-load from gyrokinetic
    ELMs is to be included
  - E&M ballooning mode effect on heat-load is to be included
- Kinetic shear-Alfven modes and the ITG-KBM transition have been verified





#### No DIII-D L-mode shortfall in XGC1 with full edge model

2014 INCITE, using 1/3 Mira capacity. 32-way OpenMP threading.



## **Conclusion and Discussion**

- In the present-day tokamak devices, the previous neoclassical XGC0 and gyrokinetic XGC1 study show nearly  $\lambda_q \propto 1/I_P$ 
  - It appears that neoclassical orbit effects are large and dominant relative to the blob spreading of  $\lambda_{\rm q}.$
- However, in ITER where the neoclassical ion orbit excursion is ≤1mm, and the 1/I<sub>P</sub> trend may fail due to the blob size ∝ (ρ<sub>i</sub>a)<sup>1/2</sup> effects
  - These important XGC1 simulations are to be done soon in an ITER model plasma
- Electromagnetic capability is coming online in XGC1
  - Nonlinear evolution of ELM (nonlinear saturation of PBM) impact on divertor heat-load
  - Other electromagnetic turbulence effects
- XGC1 with full edge model does not show DIII-D L-mode shortfall

#### Relationship between midplane $\nabla p$ -width and $\lambda_q$ ?

