Burning Plasma Relevant Control Development: Advanced Magnetic Divertor Configurations, Divertor Detachment and Burn Control

by

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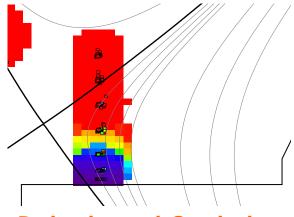
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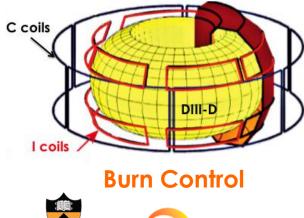




Snowflake Control

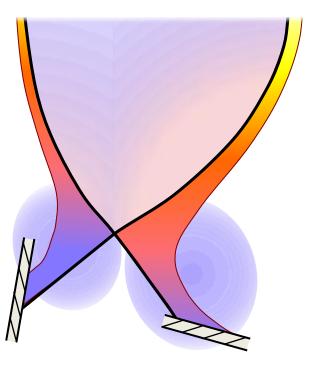


Detachment Control



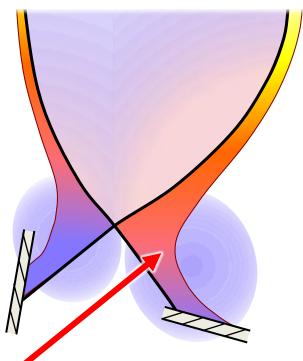


- There are **technological limits on heat flux removal**, and the problem gets more challenging for future devices
- High fidelity control gives opportunity to solve some of these challenges





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- Reduce peak heat flux
- Possibly reactor application



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1. Snowflake Divertor:

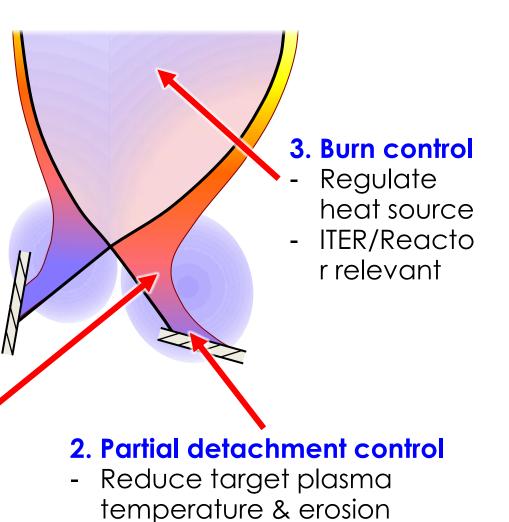
- Reduce peak heat flux
- Possibly reactor application

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2. Partial detachment control

- Reduce target plasma temperature & erosion
- ITER relevant

- There are **technological limits on heat flux removal**, and the problem gets more challenging for future devices
- High fidelity control gives opportunity to solve some of these challenges



1. Snowflake Divertor:

- Reduce peak heat flux
- Possibly reactor application



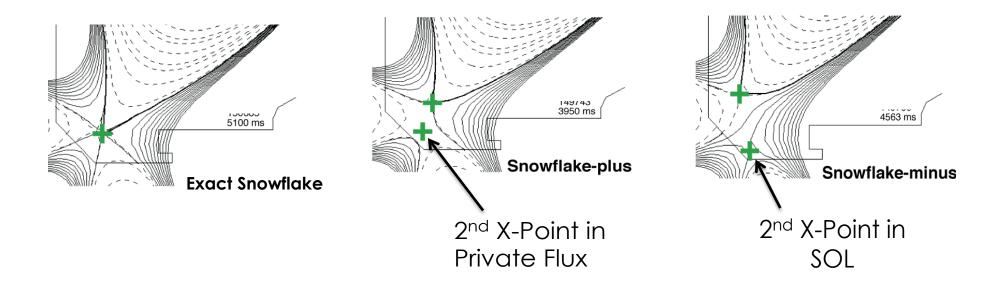
- ITER relevant

Heat Flux Reduction via

- 1. Snowflake Divertor Control
- 2. Detachment Control
- 3. Burn Control with 3D Coils



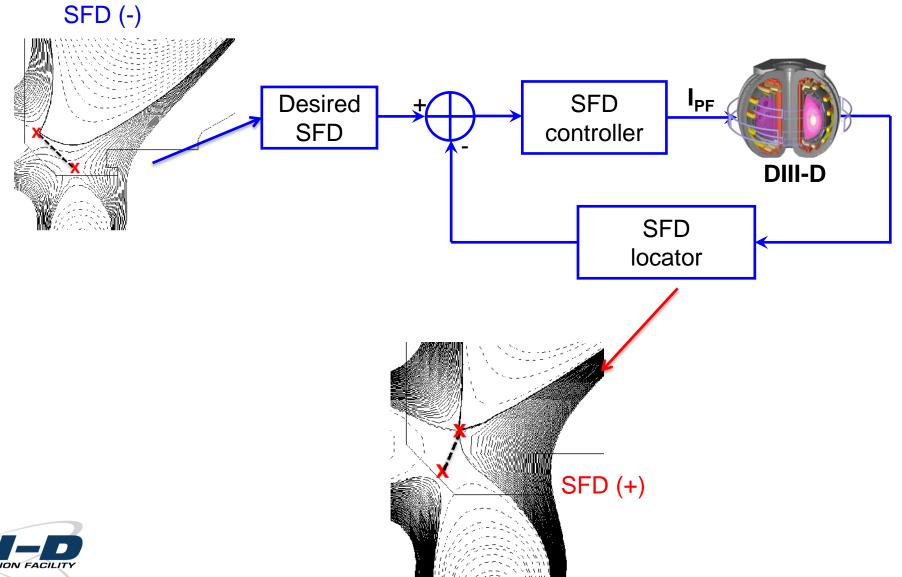
Snowflake Divertor (SFD) Has Advantages Compared to the Standard X-point Divertor



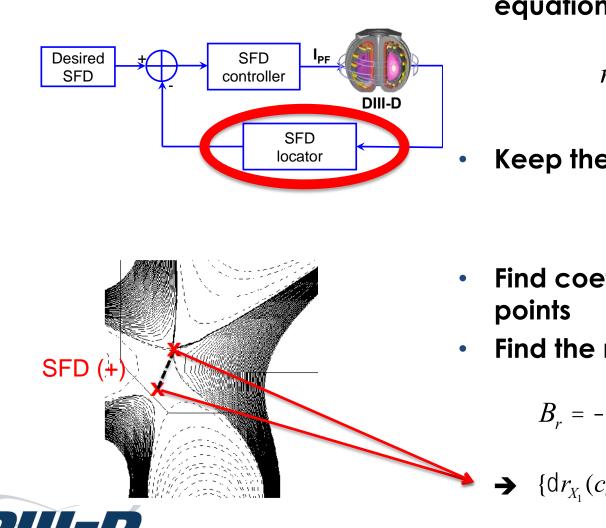
- Snowflake divertor(SFD): second-order null (2 X-points)
- Geometric changes compared to standard divertor can lead to:
 - High poloidal flux expansion, large plasma-wetted area \rightarrow reduce peak q_{div}
 - Four strike points \rightarrow share P_{div}



Snowflake Control System



Snowflake Locator: Finding the Two X-points



 Locally expand the Grad-Shafranov equation in toroidal coordinates:

$$r\frac{\partial}{\partial r}\left(\frac{1}{r}\frac{\partial\Psi}{\partial r}\right) + \frac{\partial^{2}\Psi}{\partial z^{2}} = 0$$

Keep the 3rd order terms

$$\Upsilon_{exp} = \Upsilon(c_{exp}, dr, dz)$$

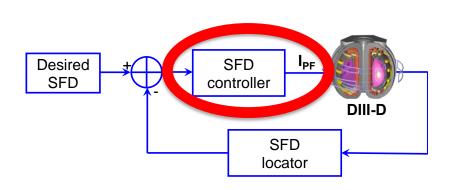
- Find coefficients, c_{exp}, from sample points
- Find the null points (X-points) <250us

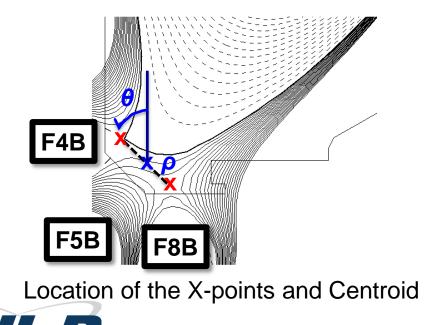
$$B_r = -\frac{1}{r} \frac{\P Y_{exp}}{\P dz} = 0 = B_z = \frac{1}{r} \frac{\P Y_{exp}}{\P dx} = 0$$

→ { $dr_{X_1}(c_{exp}), dz_{X_1}(c_{exp}), dr_{X_2}(c_{exp}), dz_{X_2}(c_{exp})$ }

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Snowflake Control: Controlling the PF Coil Currents





- Snowflake parameters:θ, ρ, r_c, z_c
- Calculate A matrix which shows how PF coils affect X-points (2 ms)

$$\begin{bmatrix} \delta \theta \\ \delta \rho \\ \delta r_{c} \\ \delta z_{c} \end{bmatrix} = A \begin{bmatrix} \delta I_{F4B} \\ \delta I_{F5B} \\ \delta I_{F8B} \end{bmatrix}$$

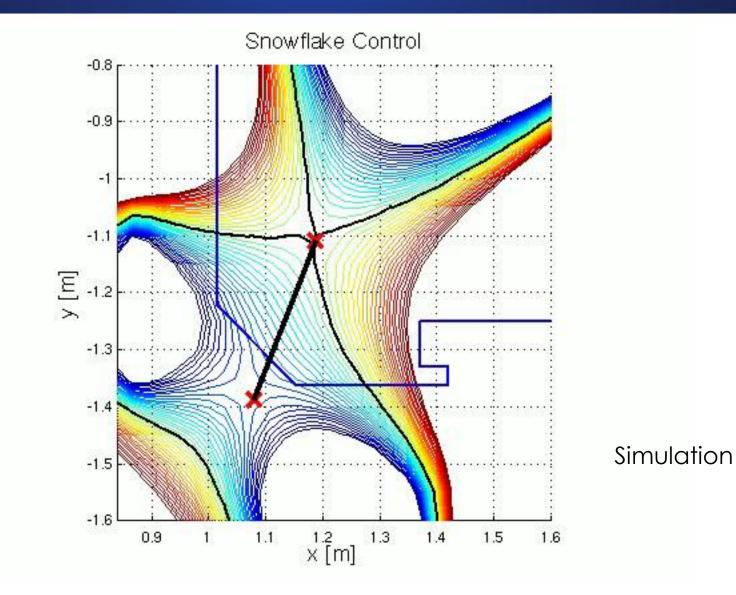
• 3 closest PF coils are used for controlling the formation

$$\begin{bmatrix} \delta I_{F4B} \\ \delta I_{F5B} \\ \delta I_{F8B} \end{bmatrix} = (A^T A)^{-1} A^T W \begin{vmatrix} \delta \theta \\ \delta \rho \\ \delta r_c \\ \delta z_c \end{vmatrix}$$

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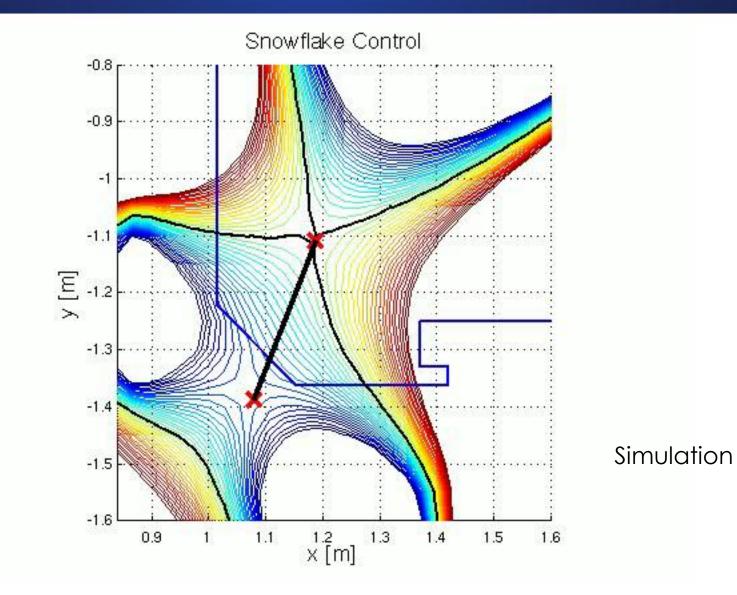
Snowflake Control: Obtaining Exact Snowflake (p Scan)





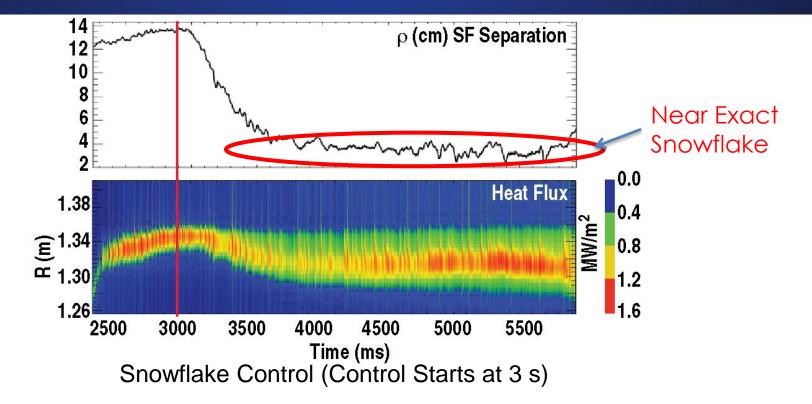
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Snowflake Control: Obtaining Exact Snowflake (p Scan)





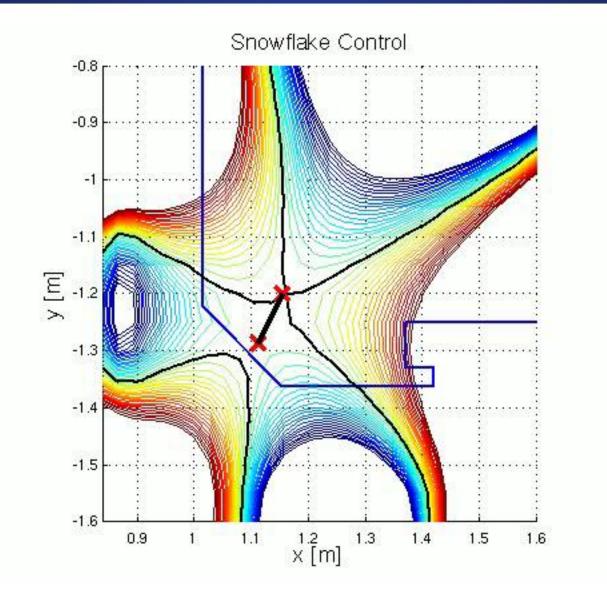
Snowflake Control: Obtaining Snowflake (Exact, + and -)



- Obtained long stable S-F close to exact S-F
 - No adverse confinement degradation
 - Pedestal profile for S-F has little change compared to regular divertor
 - Observed broadening of heat flux profiles with snowflake



Snowflake Control: Scanning the Angle

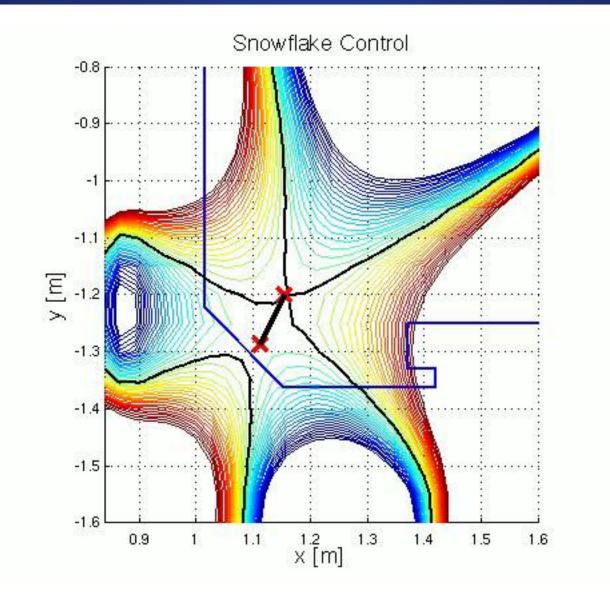




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Simulation

Snowflake Control: Scanning the Angle

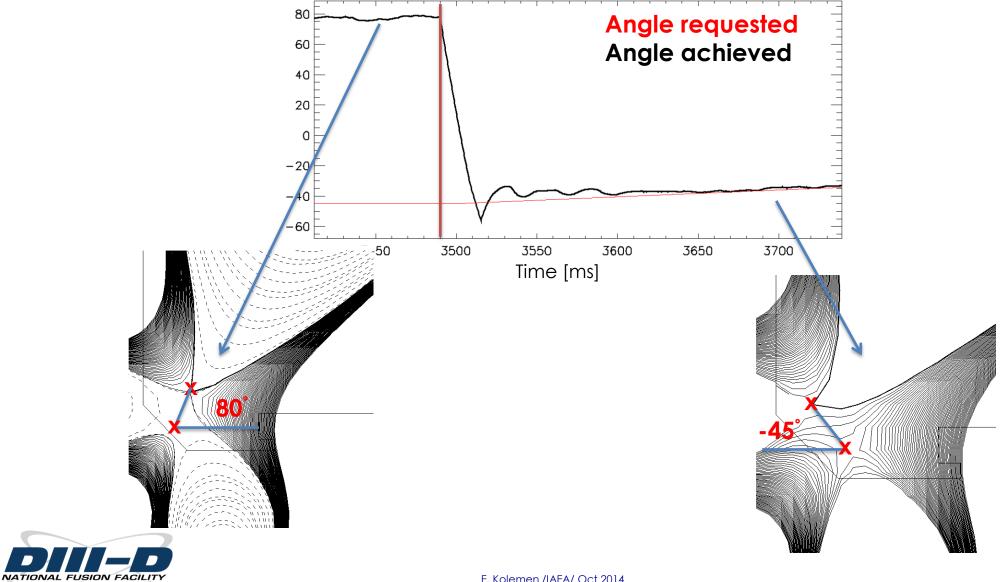




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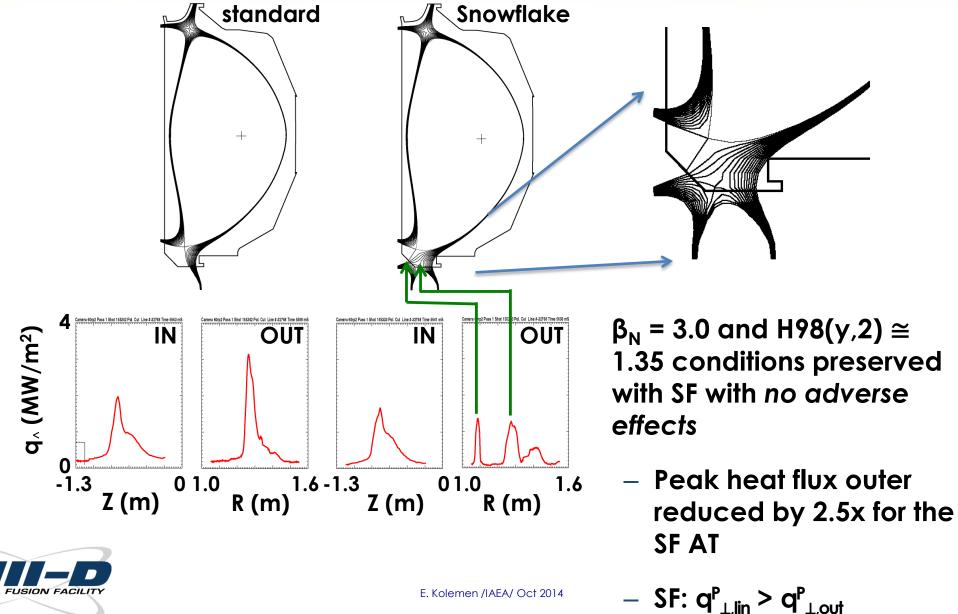
Simulation

Snowflake Control: Angle Control (+80° to -45°)



16

Snowflake with 2.5x Reduced Heat Flux Compatible with High Performance Plasmas

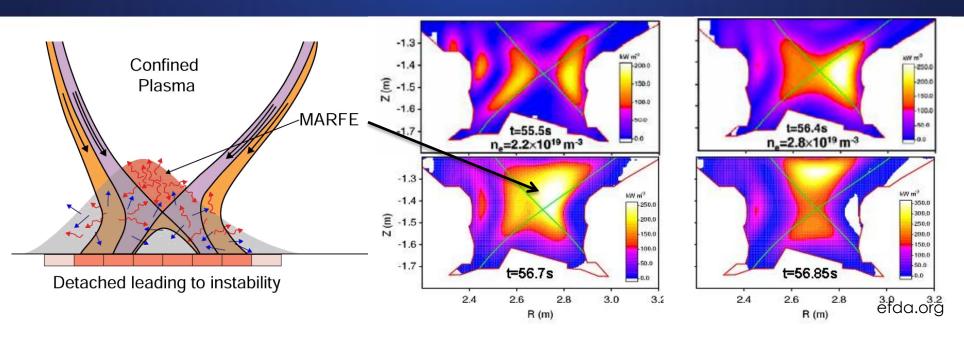


Heat Flux Reduction via

- 1. Snowflake Divertor Control
- 2. Detachment Control
- 3. Burn Control with 3D Coils



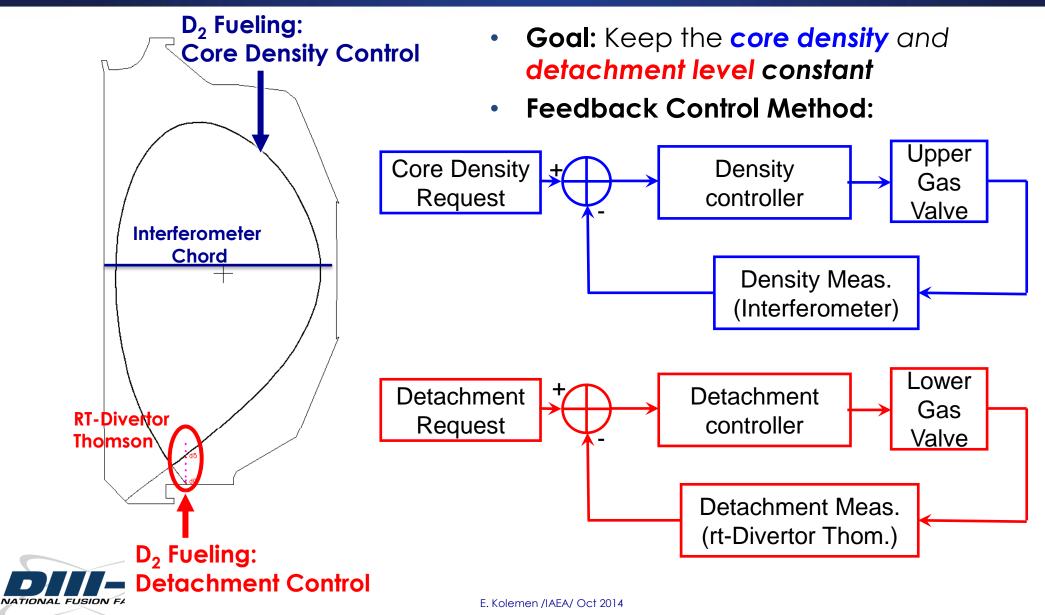
Partial Detachment Control Needed for ITER



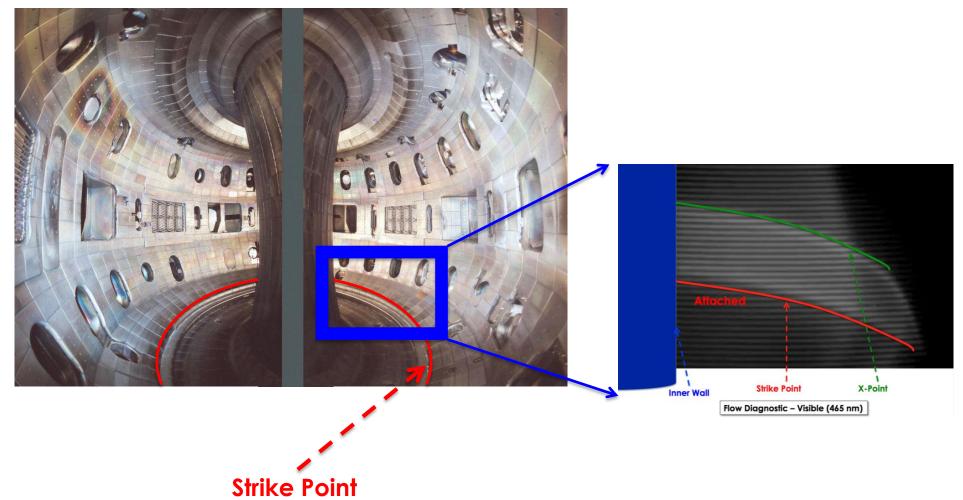
- Not enough detachment \rightarrow T_e and heat flux too high \rightarrow Erosion
- Too much detachment
 Instabilities (MARFE) and core degradation
- MARFE Instability:
 - Full detachment → large cold areas
 - Neutrals/Impurities influx \rightarrow high radiation from the core
 - Thermal instability of the whole plasma



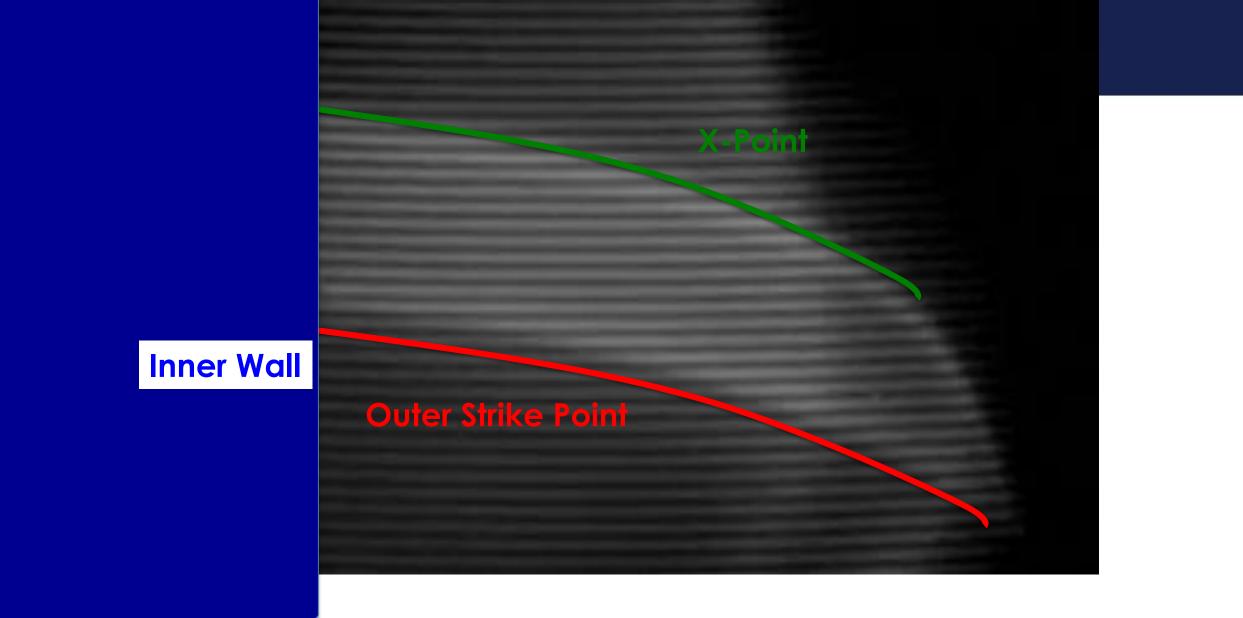
Effective Detachment Control at Constant Core Density Requires Two Feedback Channels



Detachment Control in Action

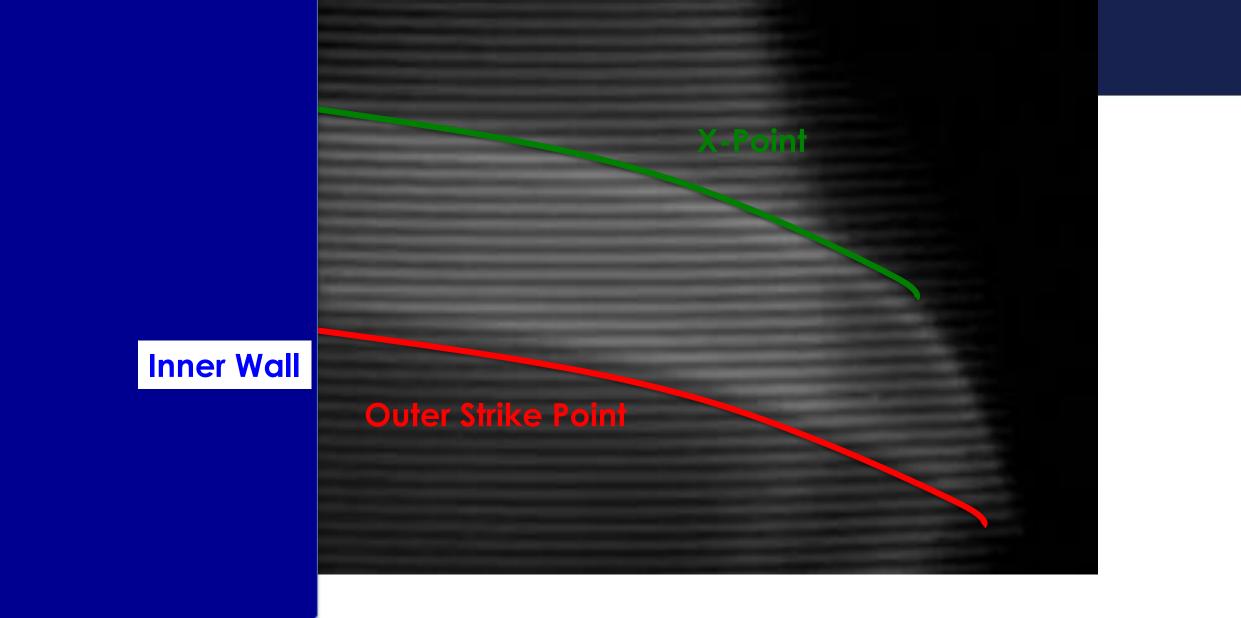








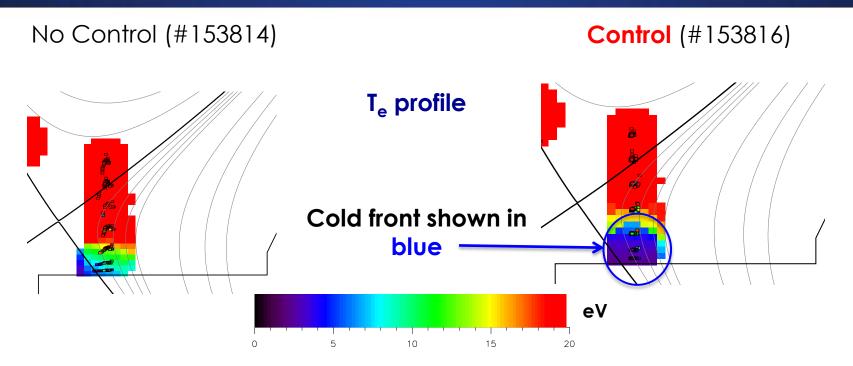
CIII Emission – Visible (465 nm)





CIII Emission – Visible (465 nm)

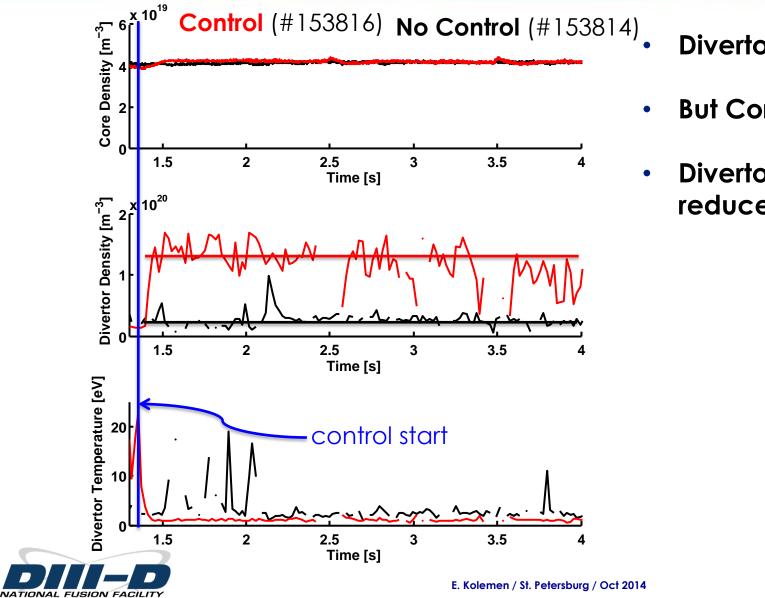
Partial Detachment Control: Forms a Cold Front in L-mode



- Control achieves partial detachment
- Keep the cold front midway between the X-point and strike point

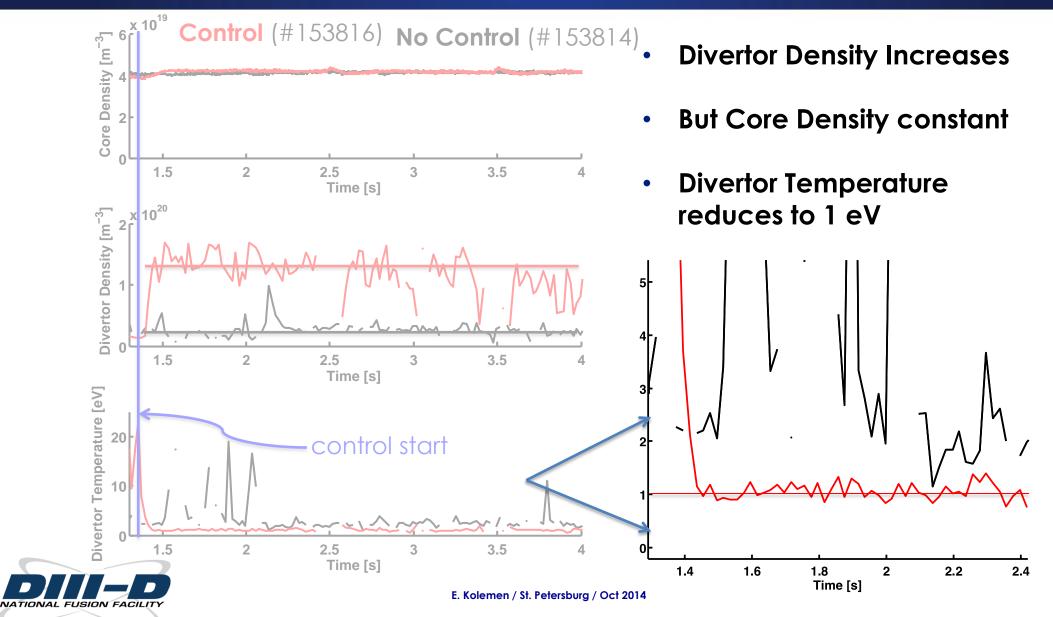


Control Stabilized Divertor Temperature (Detachment) but Keeps Core Density Constant



- **Divertor Density Increases**
- But Core Density constant
- Divertor Temperature reduces to 1 eV

Control Stabilized Divertor Temperature (Detachment) but Keeps Core Density Constant



Heat Flux Regulation via

- 1. Snowflake Divertor Control
- 2. Detachment Control
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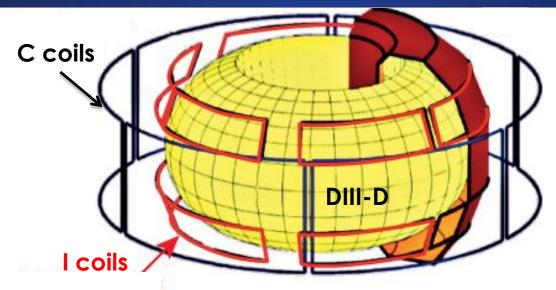


Burn Control: We Need Methods for Faster Control of Fusion Burn Rate

- Burn: D + T → He + n + 17.6 MeV
- ITER concerned with <u>power surges</u> during burning phase and burn entry/exit conditions
- Normal methods (heating, density) are slow
 - Auxiliary heating control: more heating power capability cost
 - **Density** control is limited:
 - Upper density set by Greenwald limit
 - Lower density set by detached divertor
 - Impurity injection: significant time delays for penetration?



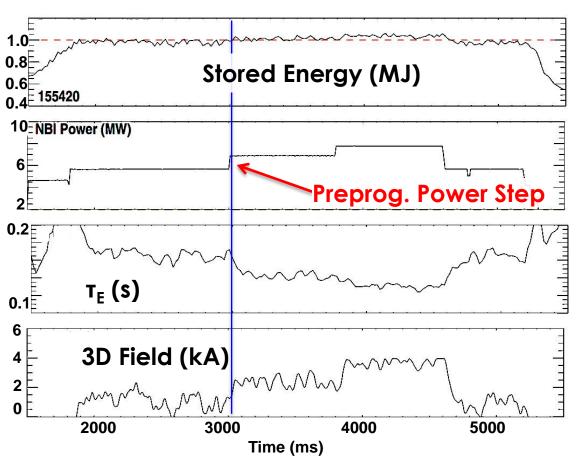
Burn Control by Non-Axisymmetric (3D) Coils (Hawryluk, PPC/P2-33)



- 10% change in energy confinement near ignition
 - → factor of 2 reduction in fusion power
- 3D magnetic field (n=3) reduces confinement in many plasma conditions by increasing edge stochasticity

 \rightarrow 3D coils actuator to control confinement time & fusion power



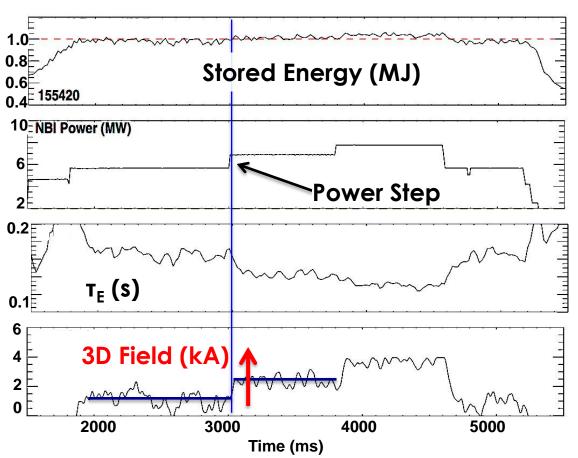


 Simulate the surge with Neutral Beams (NBI)

Add NBI steps (0.8 and 1.6 MW) to see the effect on control

- 1. Adjust 3D coil current
- 2. 3D coils in turn control the confinement time
- 3. This keeps fusion power constant

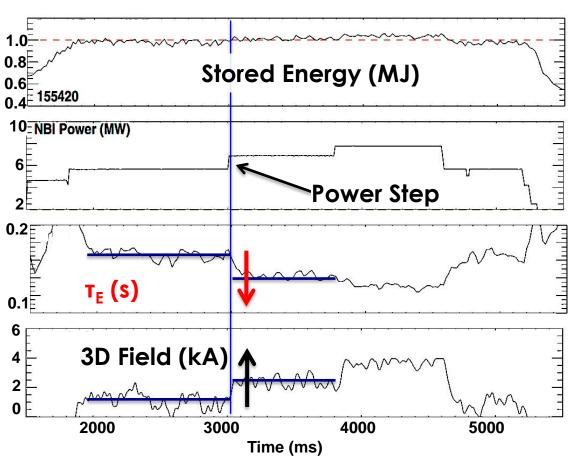




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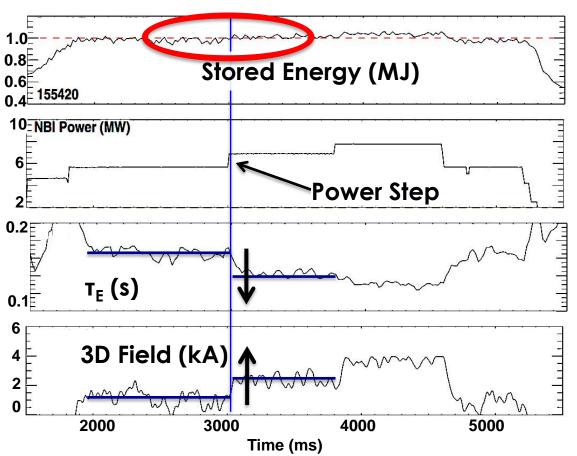




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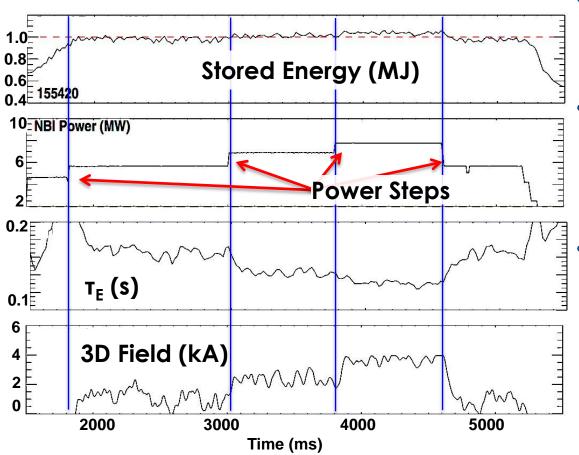




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- Simulate the surge with Neutral Beams (NBI)
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Control keeps the Burn (Stored Energy) constant:

- 1. Adjust 3D coil current
- 2. 3D coils in turn control the confinement time
- 3. This keeps fusion power constant



Conclusion: New Control Solutions Enable Advances in Heat Flux Management

 Advanced Magnetic Divertor Control reduces the peak heat flux without affecting the core properties

 Double feedback Partial Detachment Control keeps the detachment front stable between the X-point and strike point, while keeping the core properties constant

• Burn Control is feasible by using Non-Axisymmetric (3D) Coils

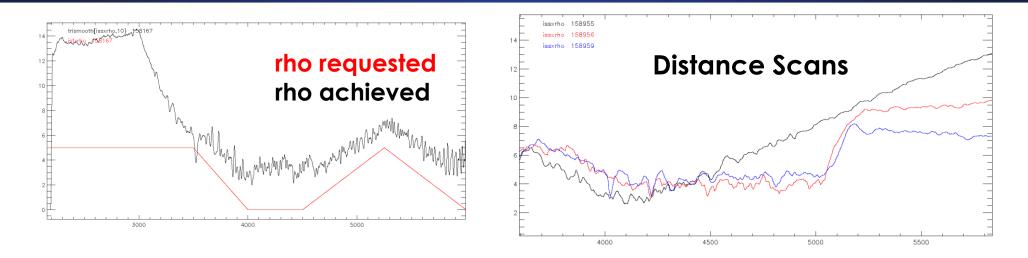


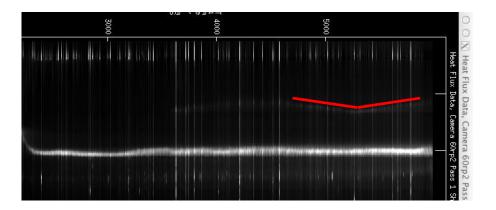




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Snowflake Control: p Control



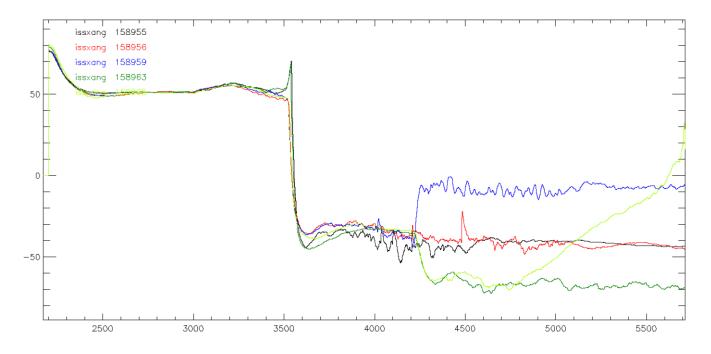


- Constantpof 3, 5, 7 cm
- pscan from 3 to 15 cm.



Snowflake Control: Angle Control and Scans

Angle Control/Scans



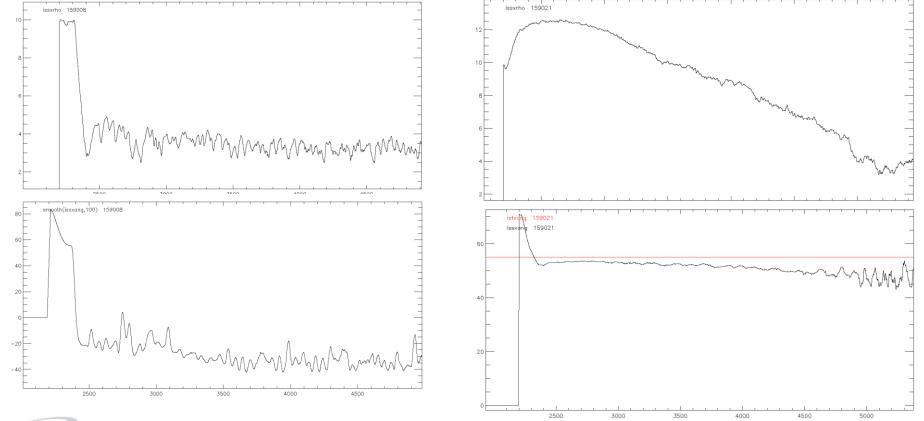
- Constant angle for -75, -45, +10, +50
- Scan from -75 to +25.



Snowflake Control: Combined rho and Angle Manipulation

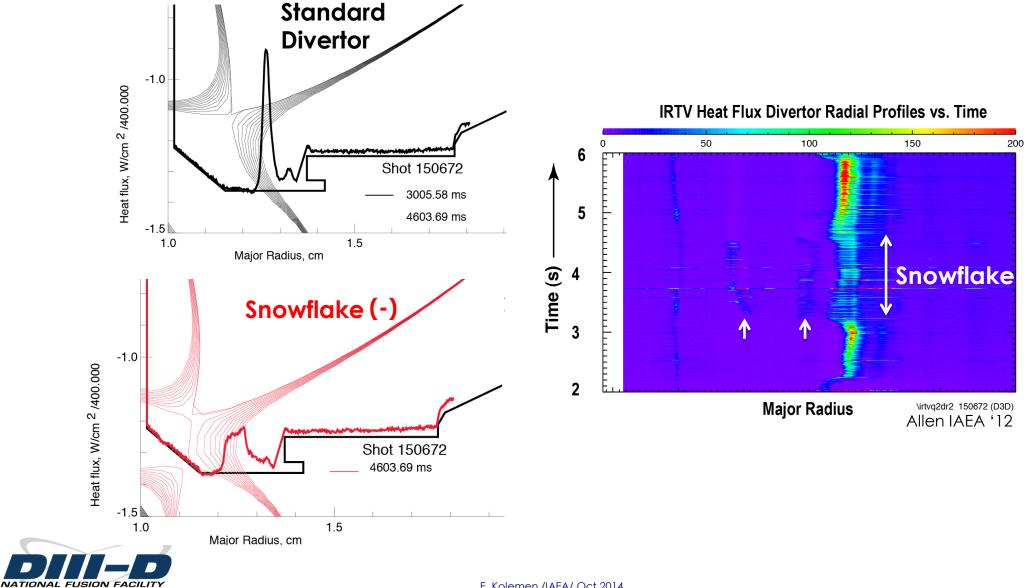
Snowflake (-)/exact

Snowflake (+)

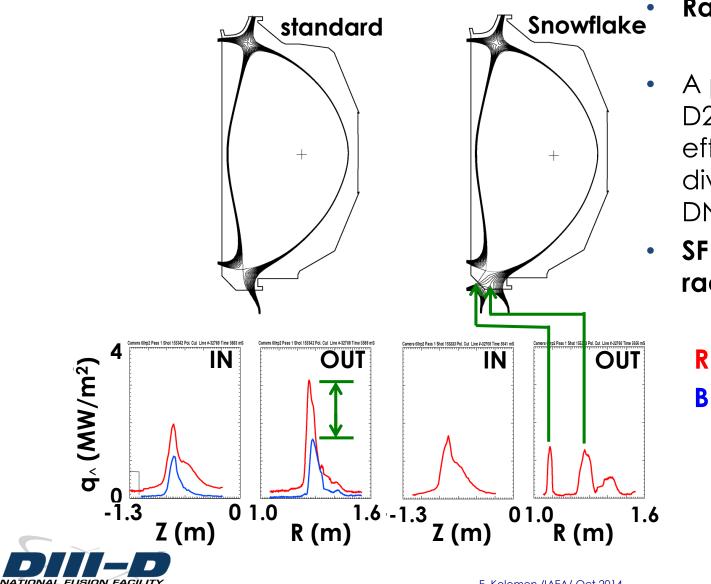




Divertor Peak Heat Flux Reduced by 2.5x in SF (-) Due To **Changes in Divertor Geometry**



AT DN radiating divertor similar reducing heat to SFD without radiating divertor

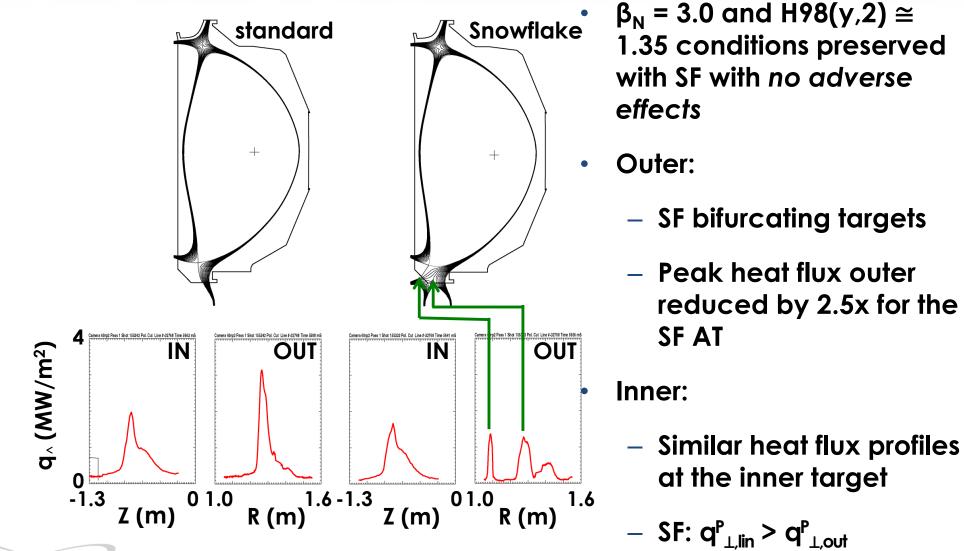


Radiating divertor case:

- A perturbing mix of
 D2+neon had a greater
 effect on q_⊥ in the outer
 divertor for the standard
 DN
- SF heat flux similar to radiating divertor

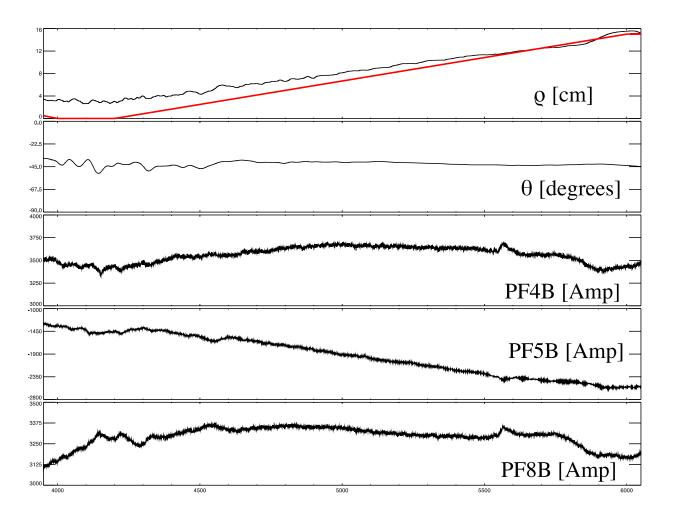
RED – Before gas puffing BLUE – Radiating divertor

Snowflake with 2.5x Reduced Heat Flux Compatible with High Performance Plasmas



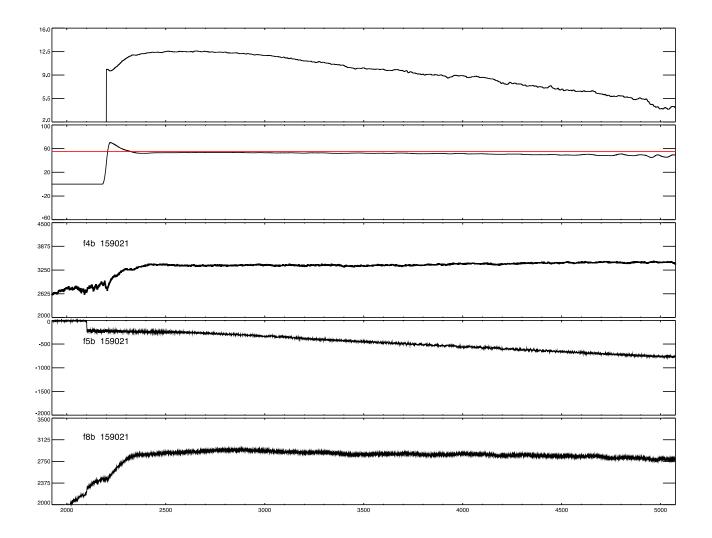


Snowflake Control: rho scan in Snowflake (-) (-45 deg)





Snowflake Control: rho scan in Snowflake (+) (+55 deg)





44

Snowflake Control: Snowflake from t=2 sec

