

Integrated core-SOL-divertor modelling for ITER including Impurity: effect of tungsten on fusion performance in H-mode and hybrid scenario

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Integrated core-SOL-divertor modelling by **COREDIV + ASTRA/JETTO CODES:**

- Development of core H-mode and hybrid scenario with ASTRA and JETTO (GLF23)
- Effect of toroidal rotation in hybrid scenario
- Self-consistent modelling of multi-fluid 1D transport in the core and 2D transport in the SOL
- Divertor plate material W
- Neon gas puff
- □ Sensitivity studies regarding W transport in core and SOL and W production in the divertor
 - o Effect of W pinch
 - o Effect of SOL diffusion coefficient
 - o Effect of sepatarix density
 - o Effect of prompt re-deposition at the divertor

CONCLUSIONS:



□ Impurity pinch: manifests through the influence of helium on fusion power level and on the ITER operational window (power to the SOL is reduced with increased pinch)

Radial diffusion in SOL: determins screening efficiency of the SOL and is very important for operational window (operation above the L-H power threshold and low power to divertor at $D > 0.5 \text{ m}^2/\text{s}$)

Separatrix density: very important for controlling tungsten production and screening (operation above the L-H power threshold and low power to divertor only for $n_e^{sep} > 3.0 \times 10^{19} \text{m}^{-3}$)

Prompt redeposition at the divertor plate: helps to stay above L-H power threshold, but higher gas puff required to reduce the heat flux in the divertor to acceptable level

Reduction of fusion gain with Ne puff is compensated by **toroidal rotation shear (ExB shear) stabilisation** in hybrid scenario