

OV/4-4

3D effects of edge magnetic field configuration on divertor/SOL transport and optimization possibilities for a future reactor

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Introduction: background, motivation & goal

- **Tokamak devices:** 2D axi-symmetric configuration + symmetry breaking perturbation
→ 3D configuration

For edge transport control (Tore Supra, TEXTOR-DED), ELM mitigation/suppression (DIII-D, JET, AUG, MAST, NSTX..., ITER)

- **Helical devices:** Divertor optimization in 3D magnetic configuration is inevitable

- Recent progress of 3D numerical transport codes, systematic experiments, accumulation of experimental data

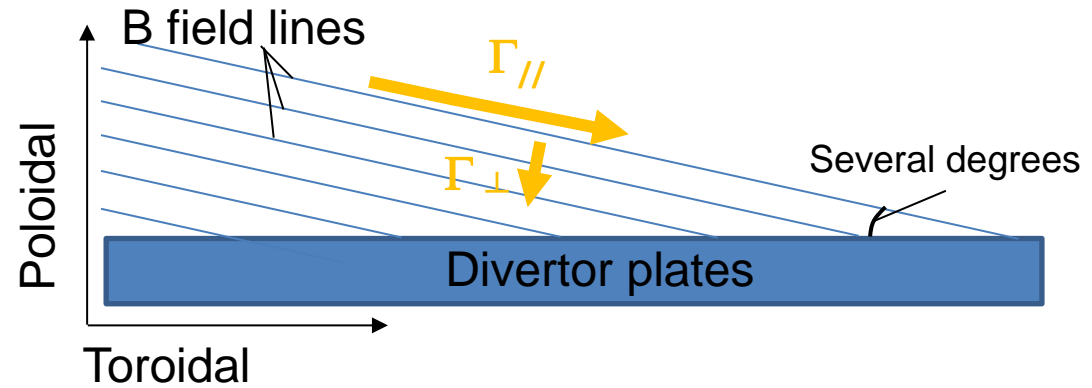
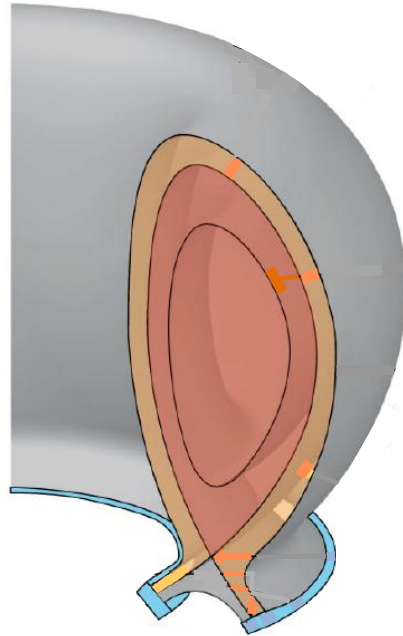
- **Identification of 3D effects** on SOL/divertor transport, **physical interpretation** & **key parameters** that control the effects, will be useful for divertor optimization in future reactors, taking full advantage of 3D configurations.



**Multi-machine comparison between tokamak and helical devices.
Impacts of 3D configuration on the divertor functions.**

- 1. Introduction**
- 2. Definition of 3D effects**
- 3. Impact on divertor density regime**
- 4. Impact on impurity transport**
- 5. Impact on detachment control**
- 6. Summary**

Transport in 2D axi-symmetric configuration



$$\frac{B_{\theta}}{B_t} \sim 0.1 \quad \frac{\Gamma_{//}}{\Gamma_{\perp}} = 10^5 \sim 10^8$$

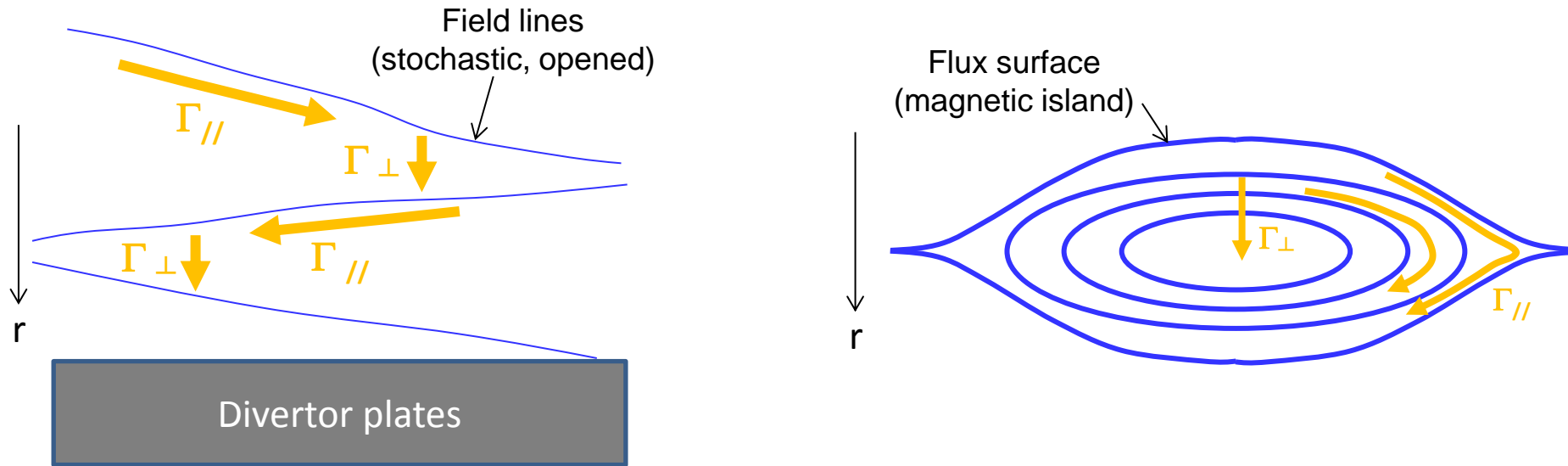


$$\boxed{\Gamma_{//} \left(\frac{B_{\theta}}{B_t} \right)^2 \gg \Gamma_{\perp}}$$

//-transport is dominant to deliver plasma quantity from upstream to divertor plates.

Magnetic field structure & “3D effects”

3D effects: **Competition between // and \perp transport**, which originates from structural/topological change of magnetic field lines, such as **openness of stochastic field lines**, or formation of **magnetic island**.



$$\frac{B_r}{B_t} = 10^{-4} \sim 10^{-3} \quad \frac{\Gamma_{//}}{\Gamma_{\perp}} = 10^5 \sim 10^8$$



3D effect emerges

$$\Gamma_{//} \left(\frac{B_r}{B_t} \right)^2 \sim \Gamma_{\perp}$$

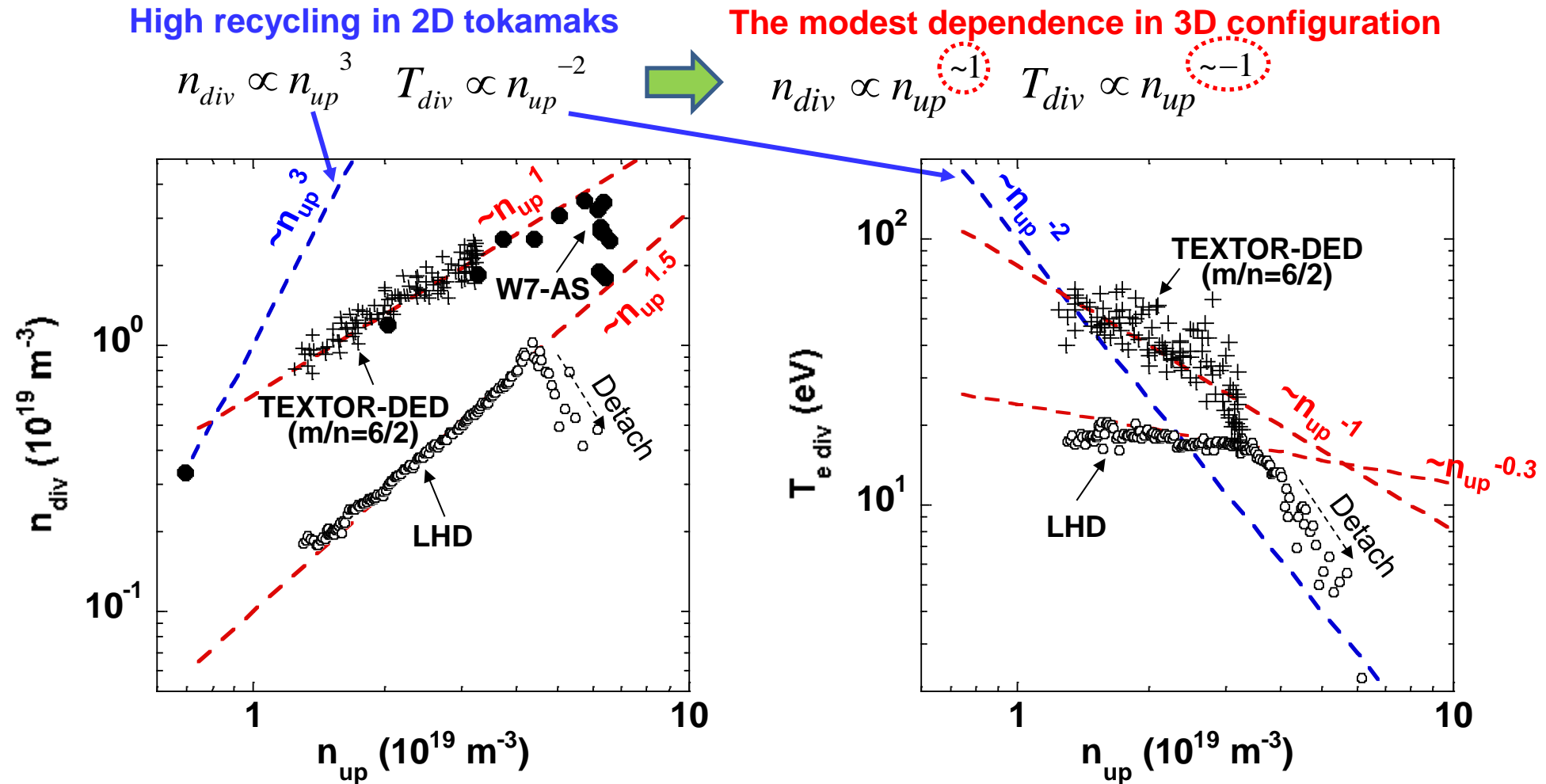
B field is prescribed for present analysis.

→ Self-consistent inclusion of dynamics of B field change due to plasma response is future work.

Impact on divertor density regime

Absence of high recycling regime prior to detachment in the 3D configurations

In helical devices as well as tokamaks with RMP, the modest density dependence is often observed.



Y. Feng et al., PPCF **44** (2002) 611.

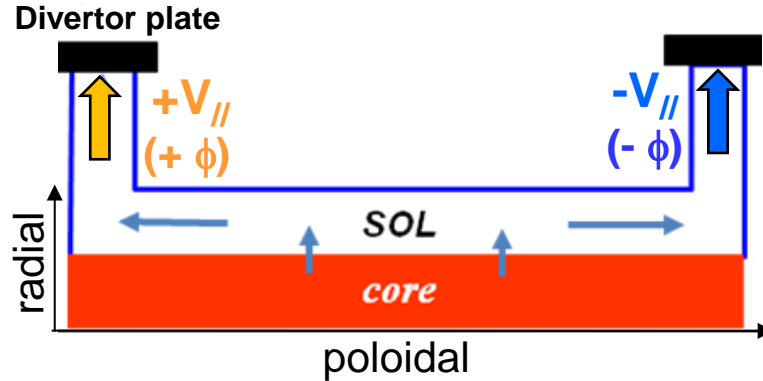
M. Clever et al., Nucl. Fusion **52** (2012) 054005.

S. Masuzaki et al., JNM **313-316** (2003) 852. Petersburg, Russia, 13-18 October 2014

Effects of enhanced \perp interaction of momentum transport on divertor regime

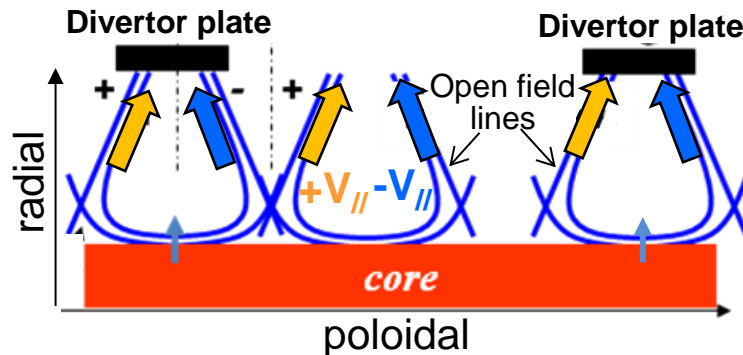
2D axi-symmetric divertor

Pressure conservation along flux tube



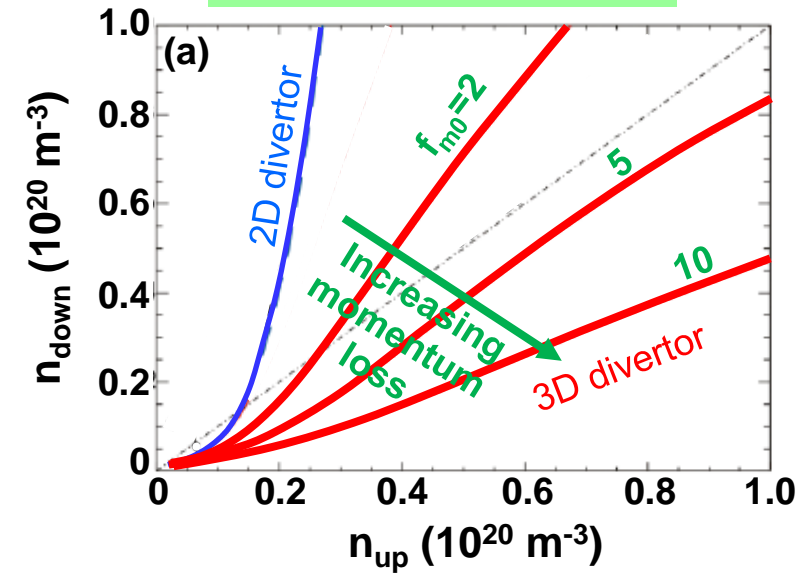
3D configuration (e.g. stochastic layer, ID)

//-Momentum loss due to counter flows

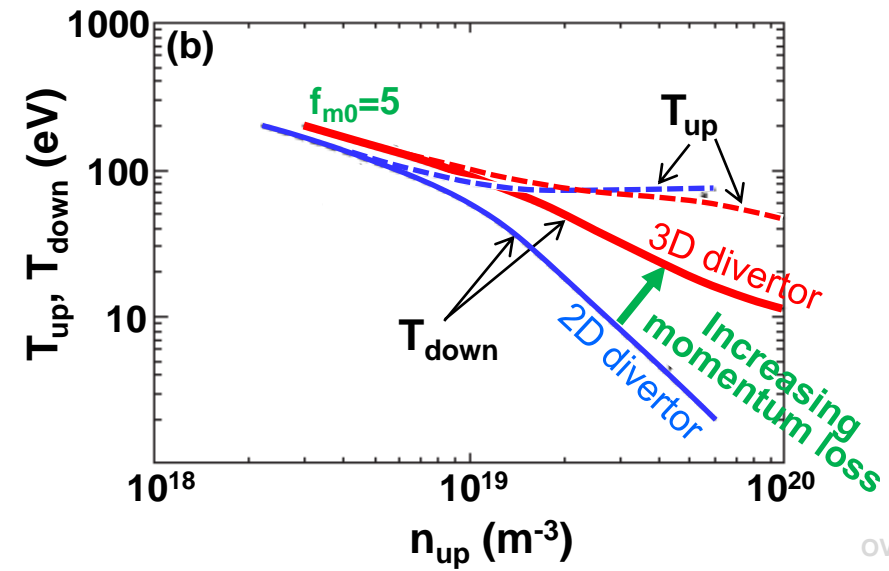


➤ \perp loss of //-Momentum $\frac{\tau_{m//}}{\tau_{m\perp}} = \frac{D_{\perp} L_{//}}{V_{//} \lambda_m^2} \propto f_{m0}$

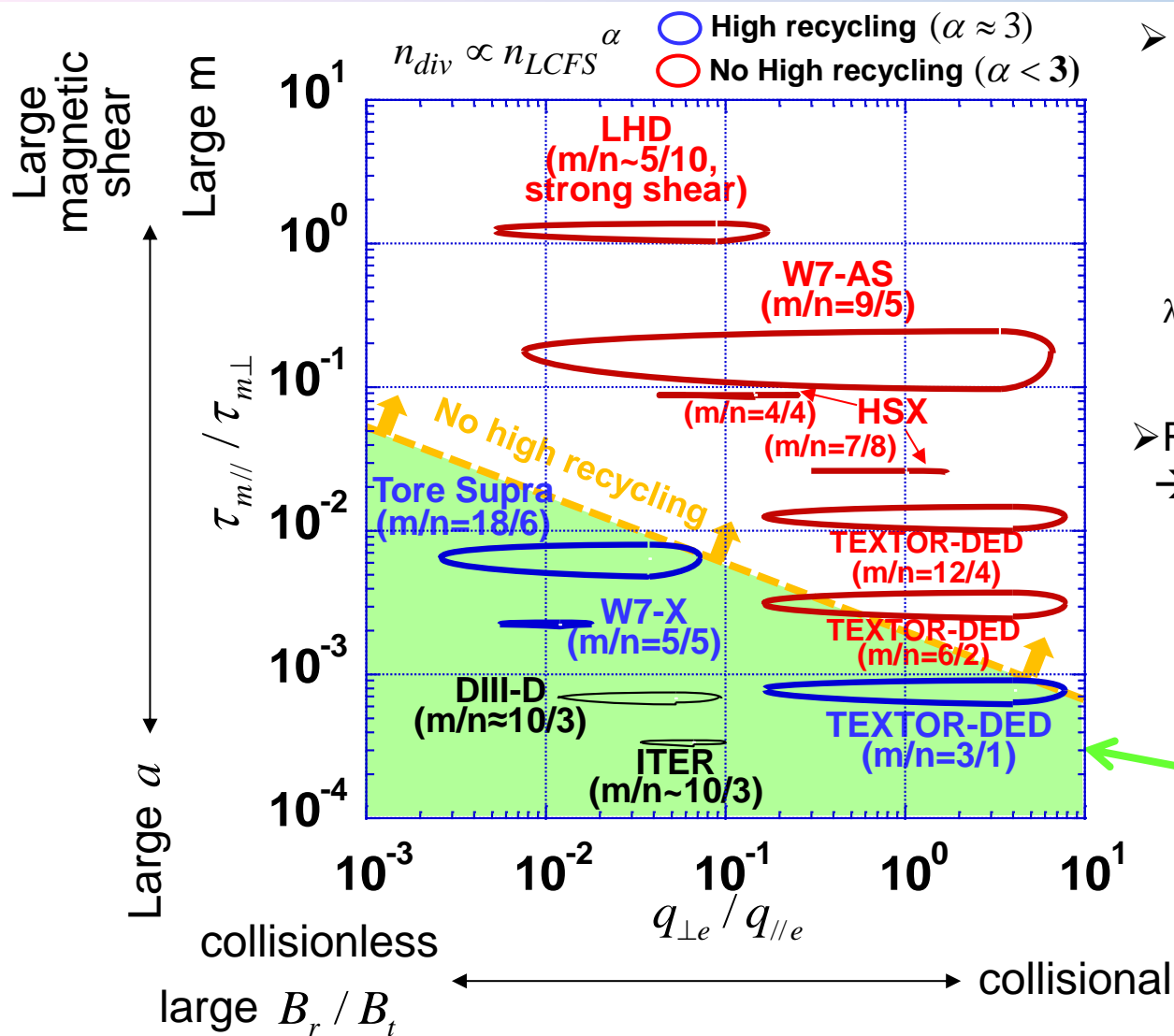
n_{down} is suppressed



//-Temperature drop $T_{\text{up}}/T_{\text{down}}$ becomes small



Multi-machine comparison for divertor density regime



- \perp loss of // -Momentum
→ // -pressure drop $p_{LCFS} > p_{div}$

$$\frac{\tau_{m\parallel}}{\tau_{m\perp}} = \frac{D_{\perp} L_{\parallel}}{V_{\parallel} \lambda_m^2}$$

λ_m : \perp characteristic scale length for momentum loss (e.g. $\sim 2\pi a/m$)

- Replacement of // -energy flux with \perp -flux
→ reduction of // -conduction energy

$$\frac{q_{\perp e}}{q_{\parallel e}} = \frac{n \chi_{\perp e}}{(B_r / B_t)^2 \kappa_{0e} T_e^{2.5}}$$

Operation domain for high recycling regime

$$\left(\frac{\tau_{m\parallel}}{\tau_{m\perp}} \right)^2 \left(\frac{q_{\perp e}}{q_{\parallel e}} \right) < 4 \times 10^{-6}$$

Possible Impacts on divertor functions due to the absence of high recycling regime:
Pumping efficiency ↓, physical sputtering ↑, detach. onset density ↑ (!?)

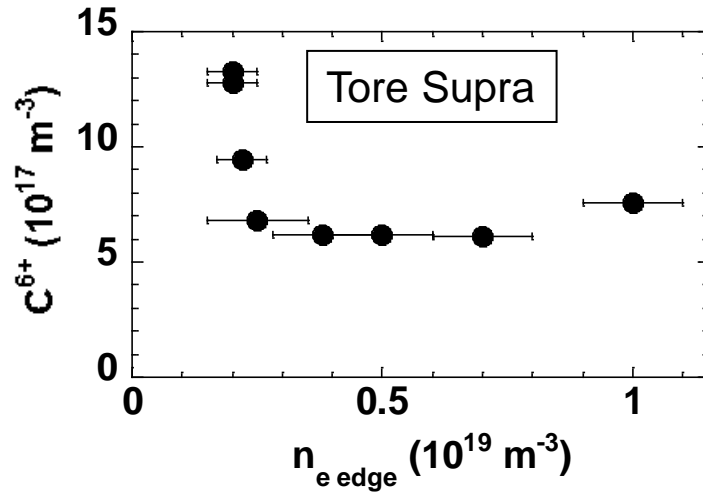
Can be avoided in detached phase

Preferable for core performance

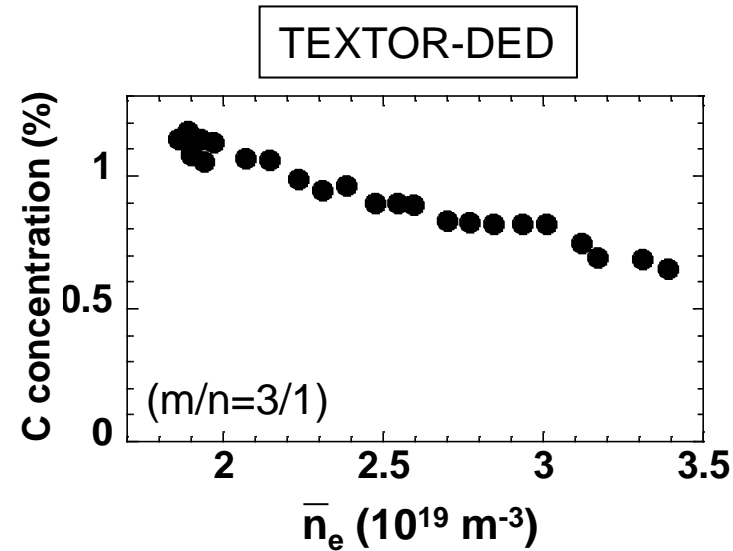
Impact on Impurity screening

Impurity screening has been observed in many devices with edge stochastic layer

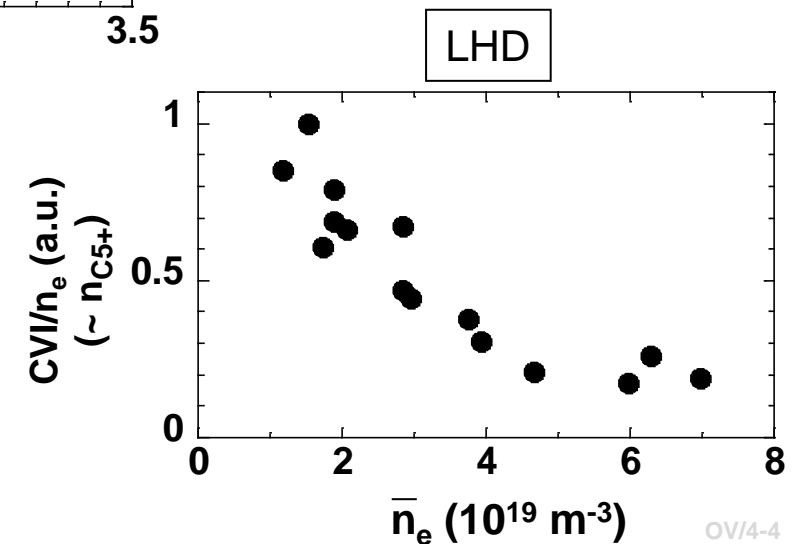
Experiments with density scan shows better screening at high density (low T_e)



Y. Corre et al., Nucl. Fusion **47** (2007) 119.



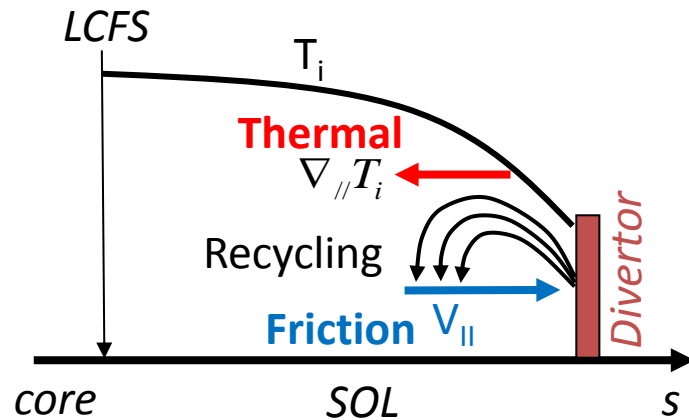
M. Lehnen et al., PPCF **47** (2005) B237.



Interpretation of the impurity screenings: key parameters

//-transport model for impurity momentum

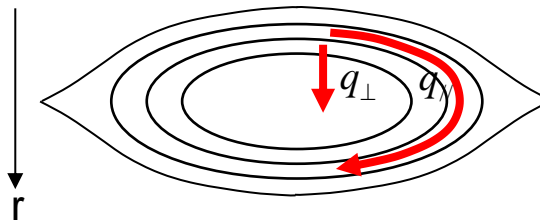
$$m_z \frac{\partial V_{z//}}{\partial t} = \underbrace{m_z \frac{V_{i//} - V_{z//}}{\tau_s}}_{\text{Friction force}} + \underbrace{2.6Z^2 \nabla_{//} T_i}_{\text{Ion thermal force}} + \dots$$



- With B_r , outward plasma flow ($V_{//}$) is enhanced
 - particle confinement time ↓
 - recycling ↑ → **friction force** ↑

$$\frac{D_{st}}{D_{\perp}} = \frac{(B_r / B_t)^2 V_{//} L_{//}}{D_{\perp}}$$

- Replacement of //-energy flux ($q_{//}$) with \perp -flux (q_{\perp}) for ion
 - $q_{//i} = \kappa_{0i} T_i^{2.5} \nabla_{//} T_i$ ↓ → **thermal force** ↓



$$\frac{q_{\perp i}}{q_{//i}} = \frac{n \chi_{\perp i}}{(B_r / B_t)^2 \kappa_{0i} T_i^{2.5}}$$

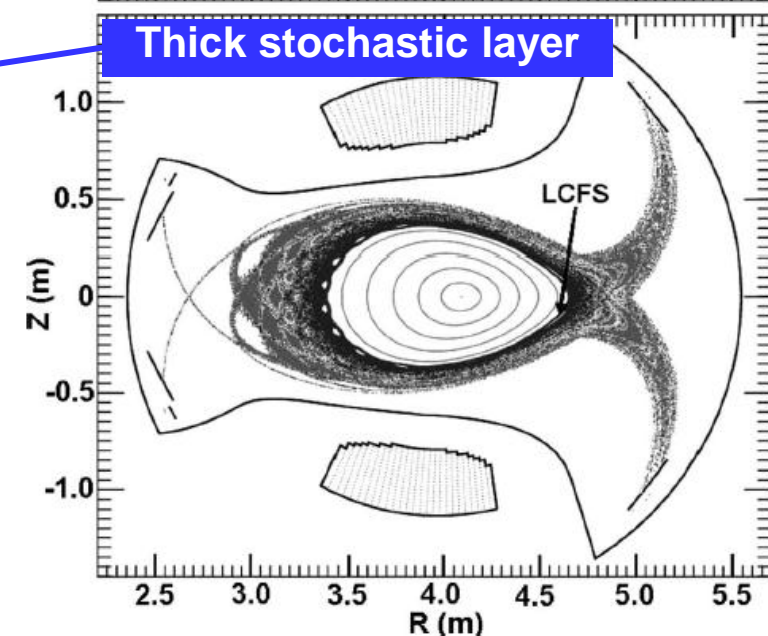
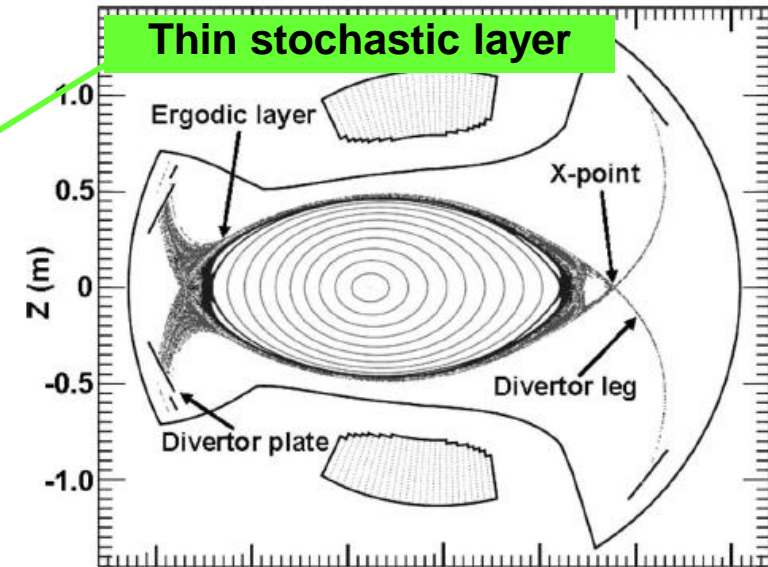
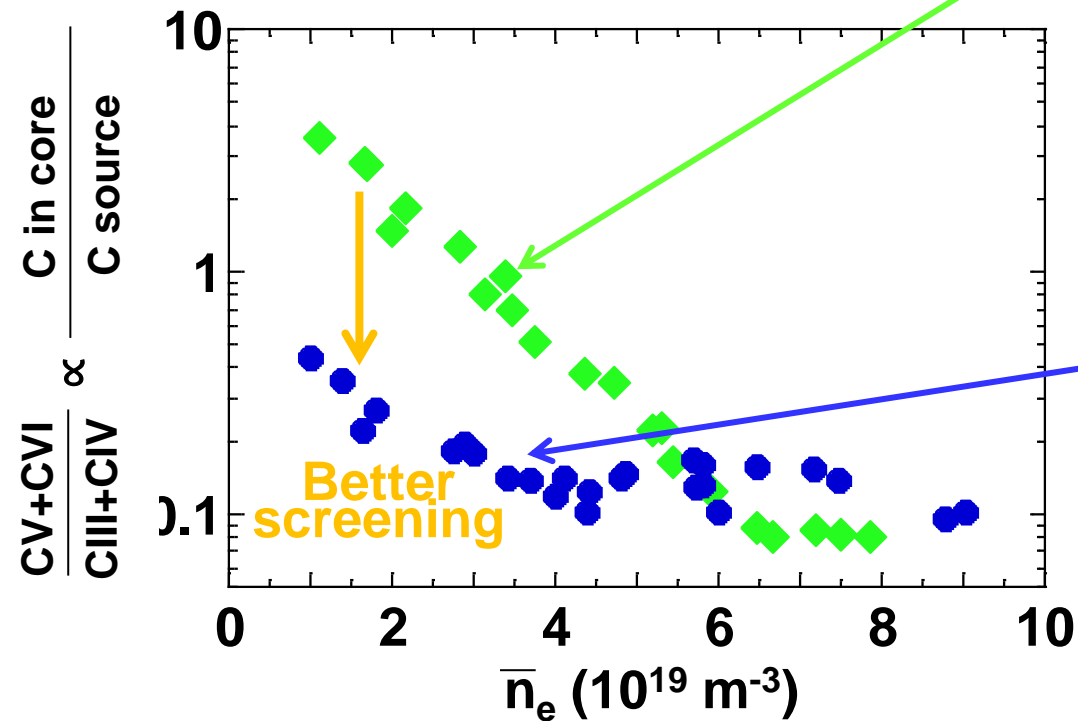
Y. Feng et al., NF **46** (2006) 807.

SOL thickness dependence of impurity screening: thicker stochastic SOL \rightarrow better screening already at low density

➤ Thicker stochastic layer/SOL (λ_{st-SOL}) relative to neutral impurity penetration (λ_{imp})

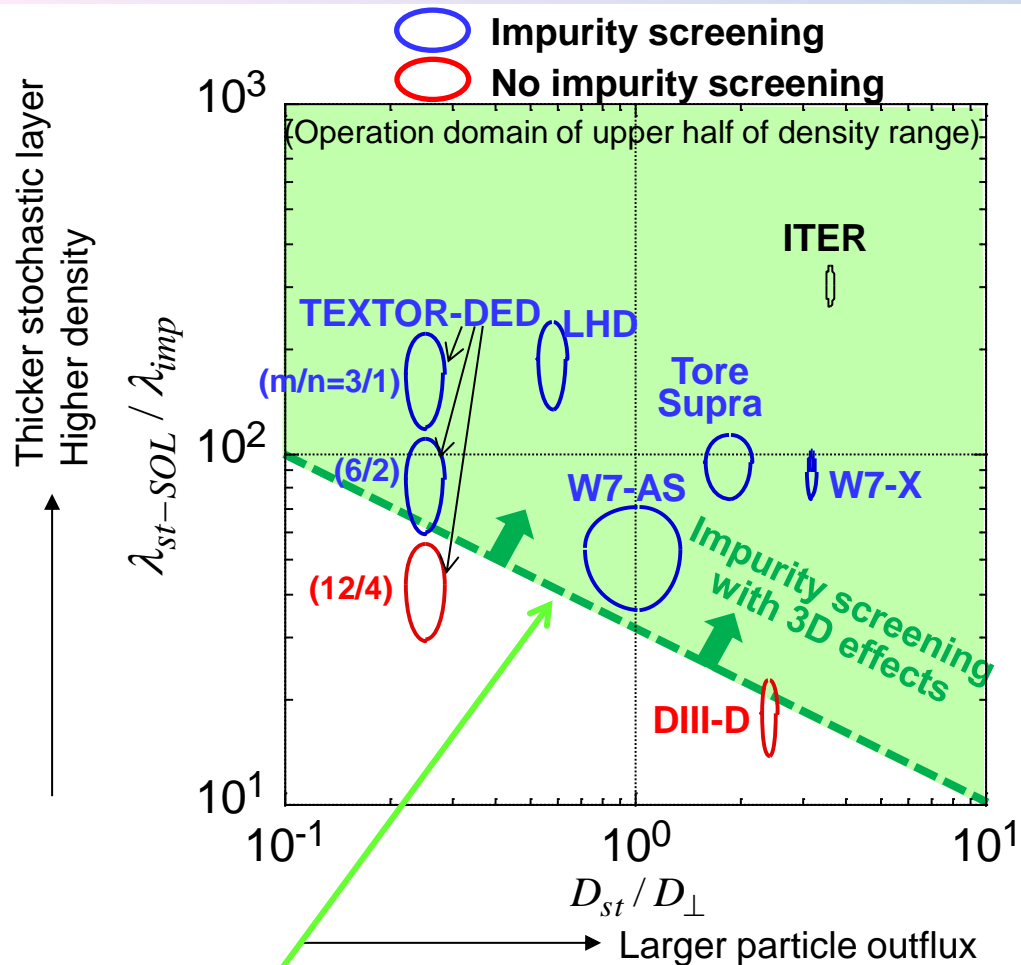
$\lambda_{st-SOL} / \lambda_{imp} \uparrow \rightarrow$ better screening

LHD



Multi-machine comparison for impurity screening at high density range

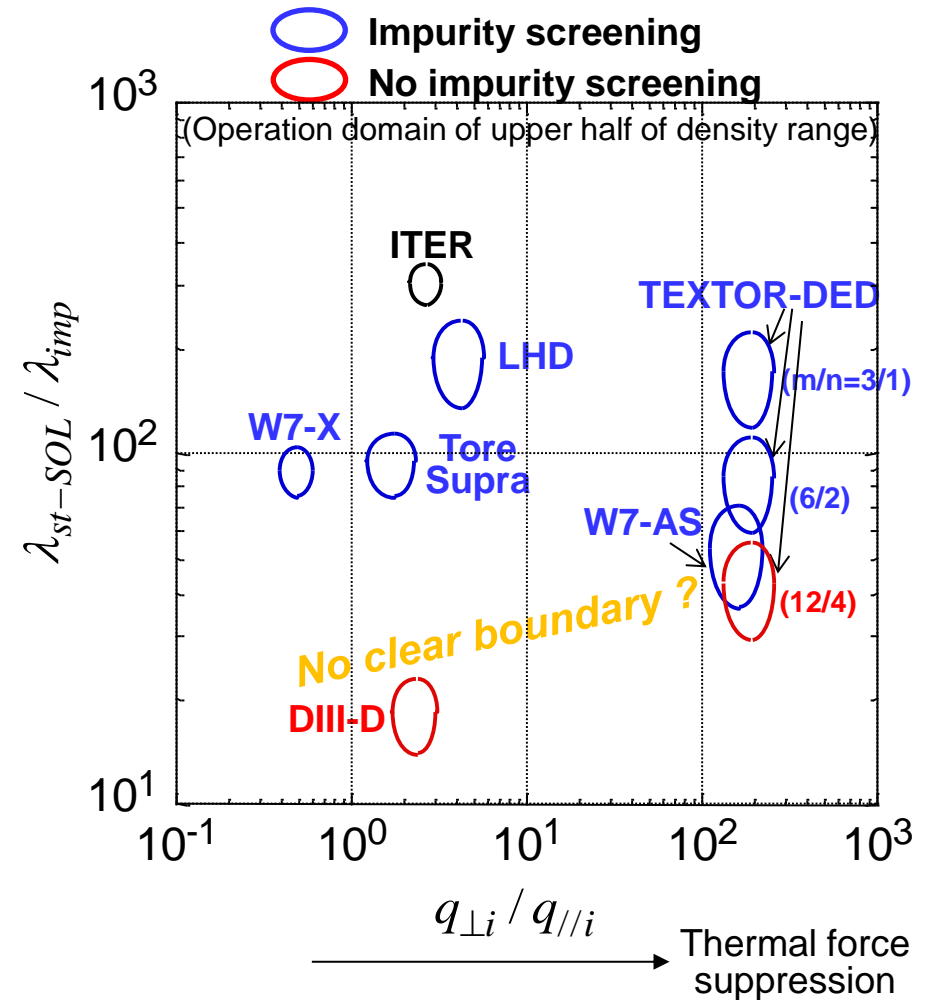
Operation domain of upper half of density range \leftarrow impurity screening is usually observed at high density



Operation domain for Impurity screening

$$\left(\frac{\lambda_{st-SOL}}{\lambda_{imp}} \right)^2 \left(\frac{D_{st}}{D_{\perp}} \right) > 10^3$$

- Thicker stochastic layer/SOL & enhanced particle transport seem to provide screening effects.



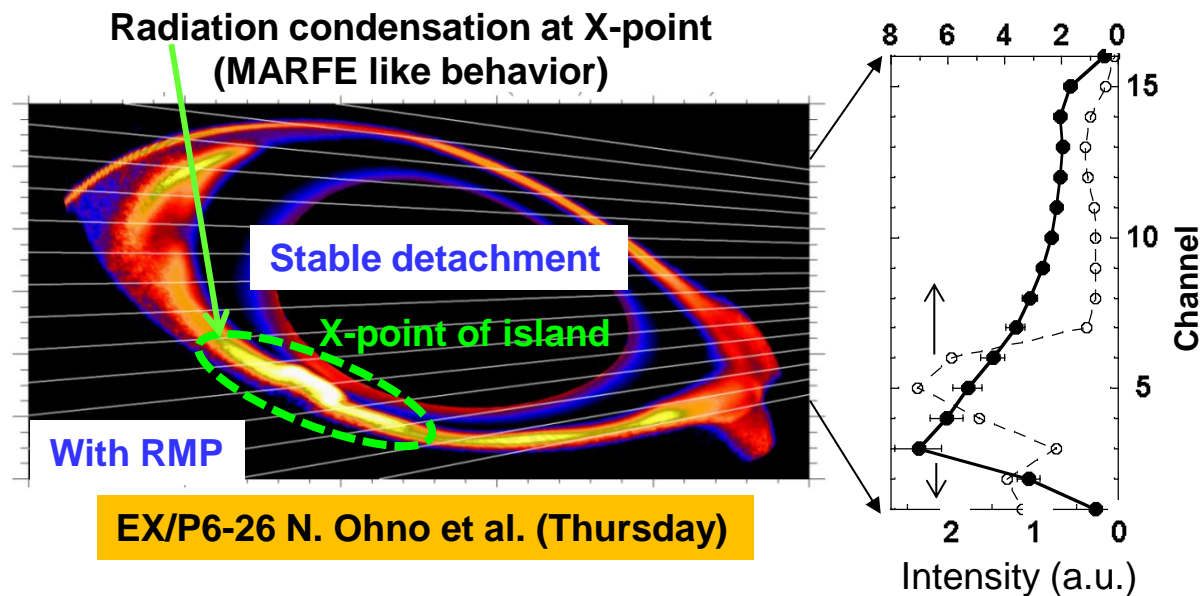
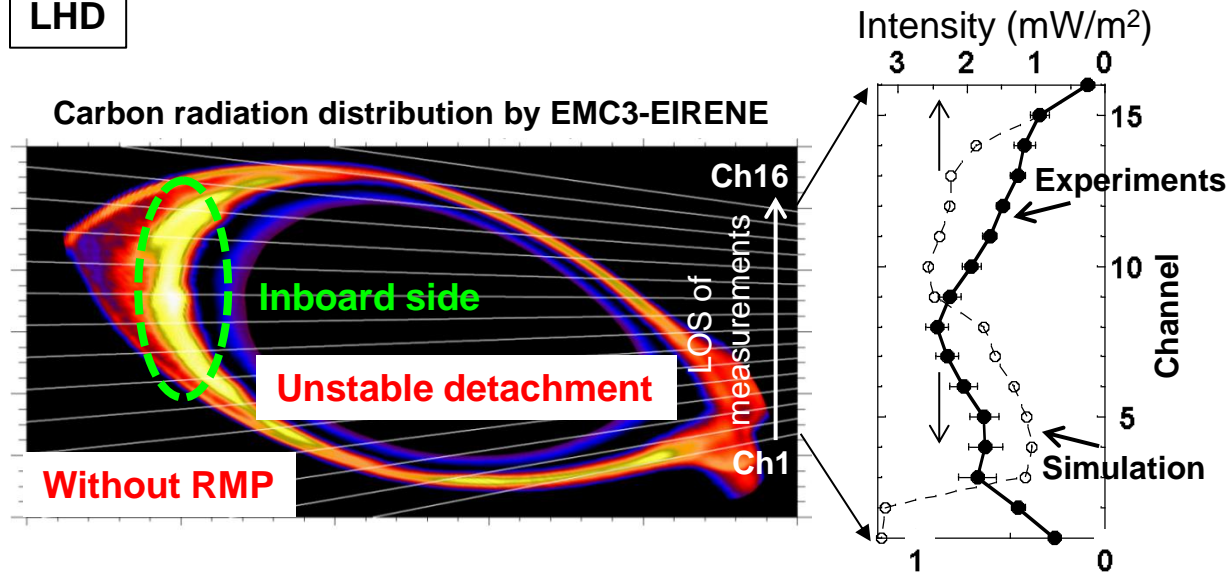
- No clear boundary identified in the parameterization with thermal force suppression.

Further study: Quantification of screening, impurity injection energy, source location, drift, E field, turbulent transport

Impact on detachment control

Detachment stabilization with RMP application (LHD, W7-AS)

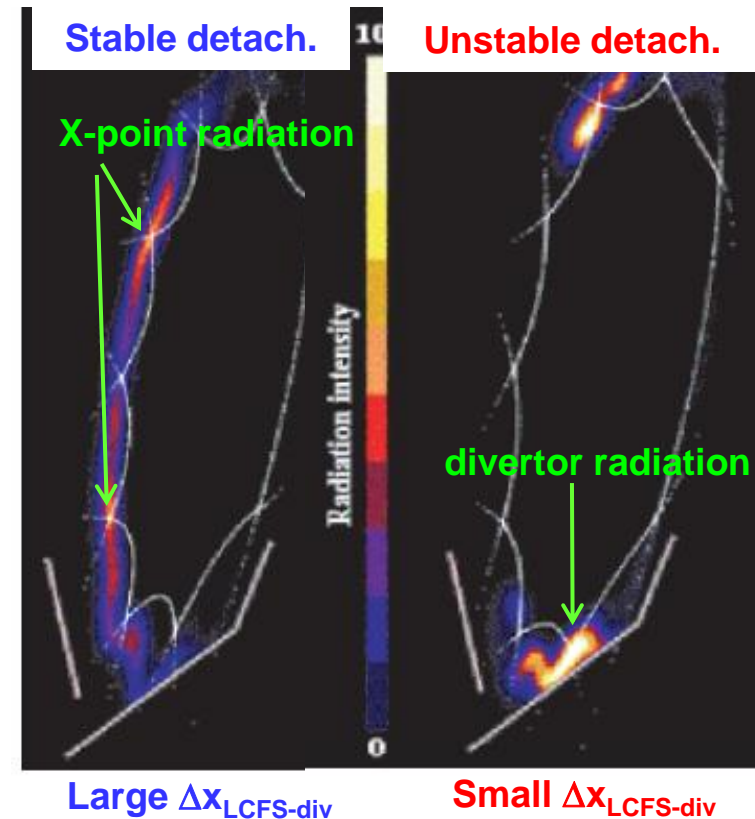
LHD



EX/P6-26 N. Ohno et al. (Thursday)

W7-AS

Carbon radiation distribution by EMC3-EIRENE

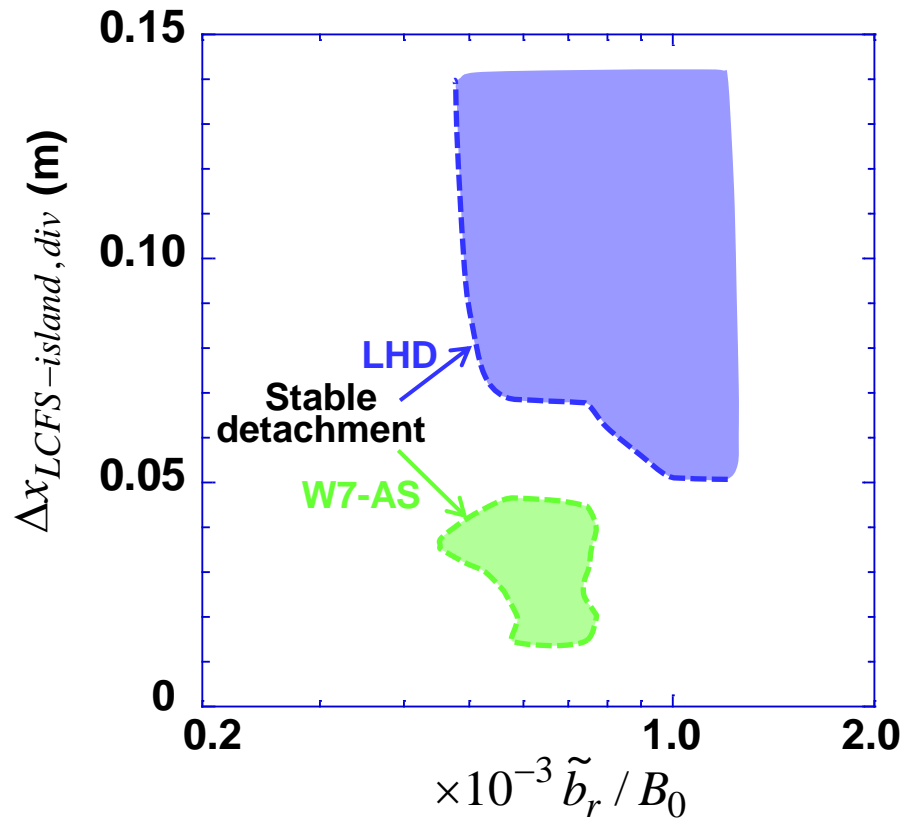


Y. Feng et al., NF 45 (2005) 89.

- Modification of 3D edge radiation structure with RMP application → stable detachment
- Separation between radiation region & confinement region is important factor for stable detachment

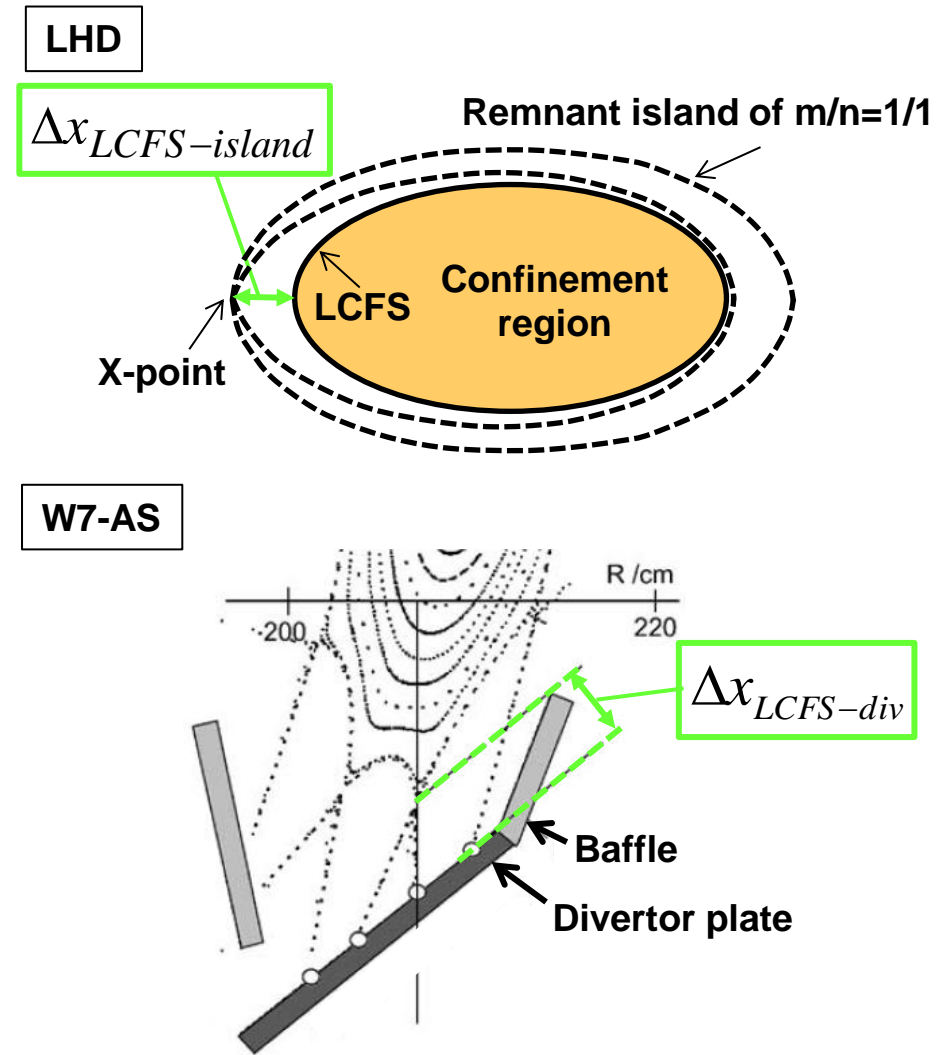
Operation domain of stable detachment in LHD & W7-AS

Key geometric parameters: $\Delta x_{LCFS-island,div}$, \tilde{b}_r / B_0



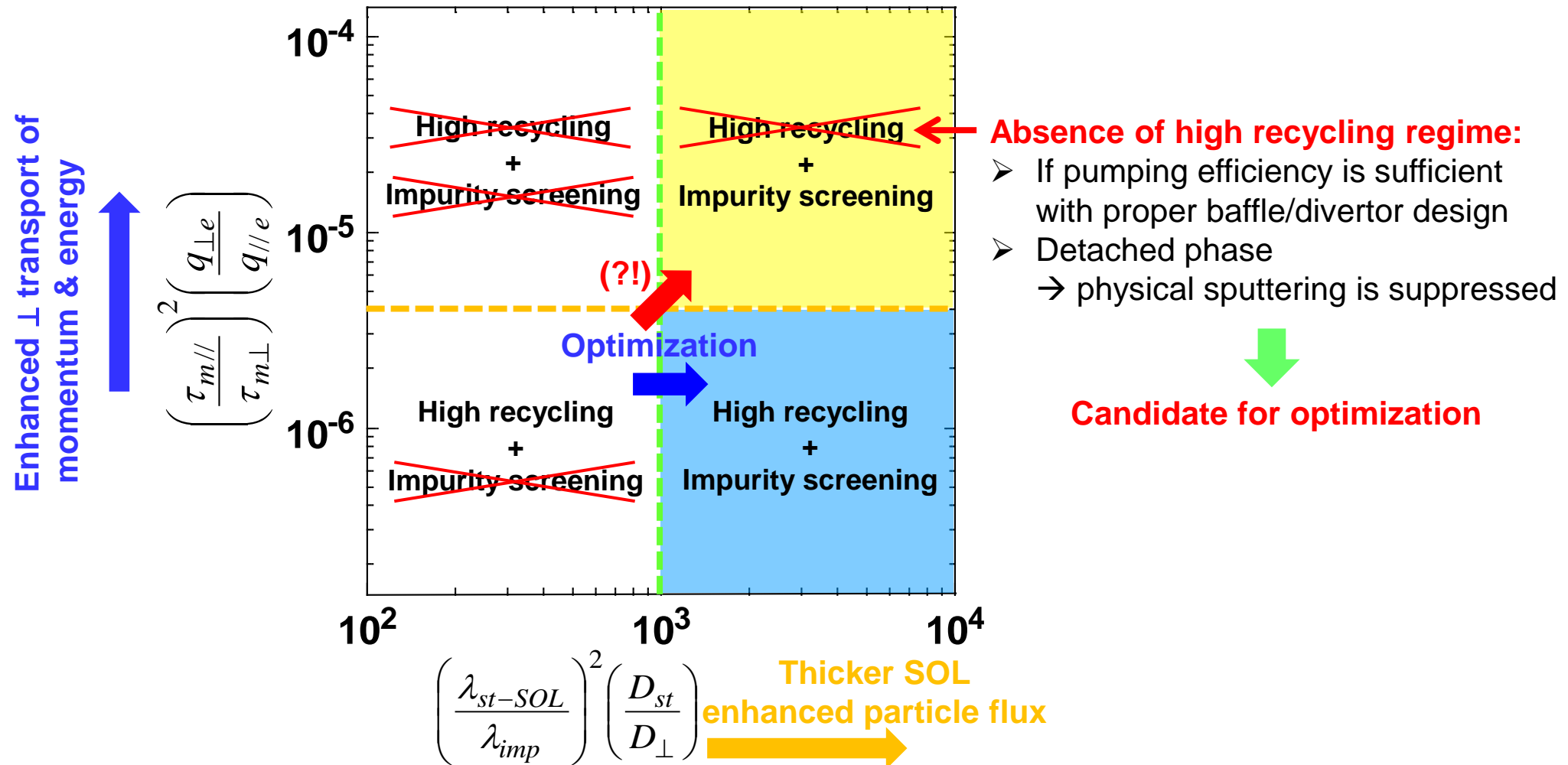
Conditions for stable detachment

- 1) RMP field must be strong enough (intrinsic ones insufficient, additional MP needed).
- 2) the geometric SOL width ($\Delta x_{LCFS-island,div}$) must be sufficiently large.



Recently the effects of RMP on detachment are being investigated in NSTX, DIII-D, too.

Optimization possibility for a future reactor



Summary

➤ **The 3D effects** : competition between transports parallel (//) and perpendicular (⊥) to magnetic field, in open stochastic field lines or magnetic islands.

➤ **The competition process affects energy, particle and momentum transport in divertor/SOL region**

♦ **Density regime** → absence of high recycling regime

Momentum loss via ⊥-transport, enhanced ⊥-energy transport, //-convection energy flux

Operation domain for high recycling:

$$\left(\frac{\tau_{m//}}{\tau_{m\perp}} \right)^2 \left(\frac{q_{\perp e}}{q_{//e}} \right) < 4 \times 10^{-6}$$

♦ **Impurity screening**

Friction force enhancement, ion thermal force suppression, thicker SOL with stochastization & ID

Operation domain for impurity screening:

$$\left(\frac{\lambda_{st-SOL}}{\lambda_{imp}} \right)^2 \left(\frac{D_{st}}{D_{\perp}} \right) > 10^3$$

→ Need further study on quantification of screening efficiency, source characteristics, E-field, turbulence

♦ **Detachment stability**

Radiation modification by RMP, sufficiently large \tilde{b}_r / B_0 , separation between radiation region & confinement region $\Delta x_{LCFS-island, div}$

→ Detachment stabilization

Further study: Edge E-field/turbulence, plasma response to MP, Divertor heat load (strike-line splitting), ELM mitigation/suppression, control of high-Z impurity

Systematic understandings of the impact of 3D divertor configurations will open new perspective on divertor optimization for future reactors.

Absence of high recycling regime prior to detachment in the 3D configurations

In helical devices as well as tokamaks with RMP, the modest density dependence is often observed.

High recycling in 2D tokamaks

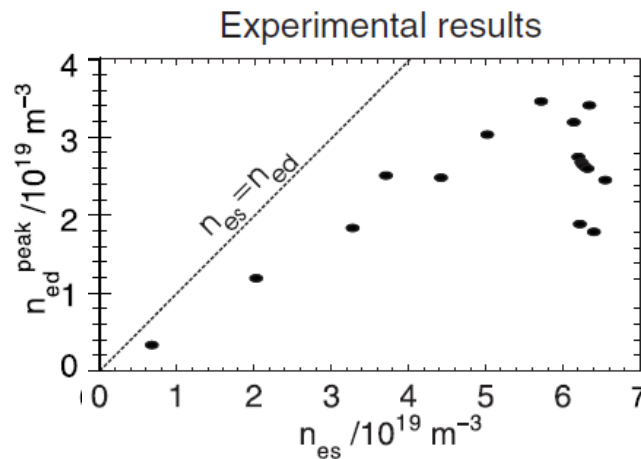
$$n_{div} \propto n_{up}^3 \quad T_{div} \propto n_{up}^{-2}$$



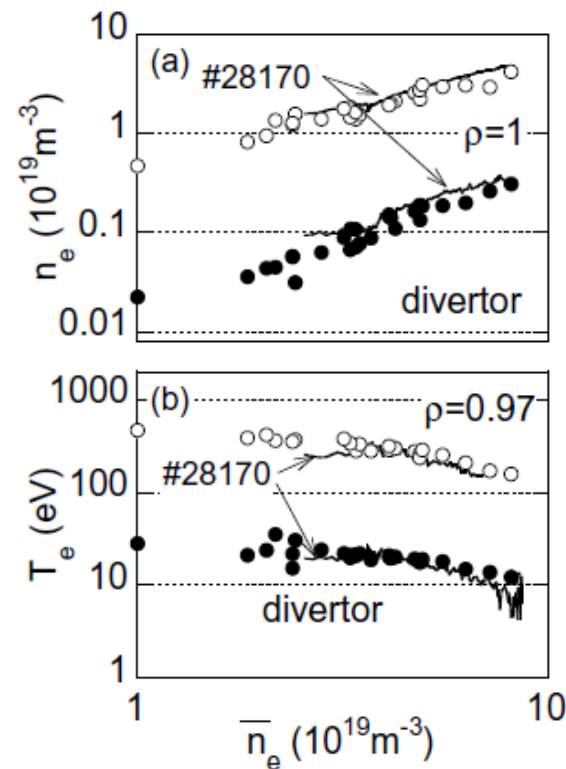
The modest dependence in 3D configuration

$$n_{div} \propto n_{up}^{\sim 1} \quad T_{div} \propto n_{up}^{\sim -1}$$

W7-AS

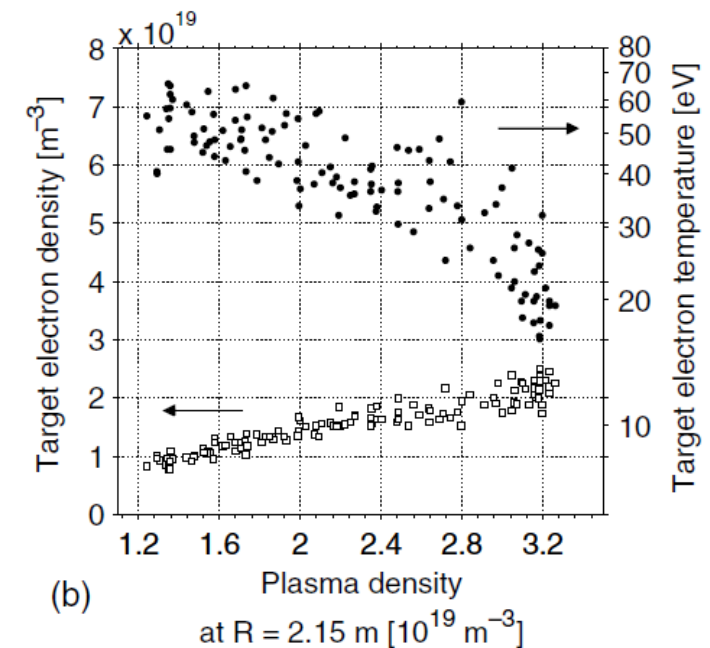


LHD



S. Masuzaki et al., JNM **313-316** (2003) 852.

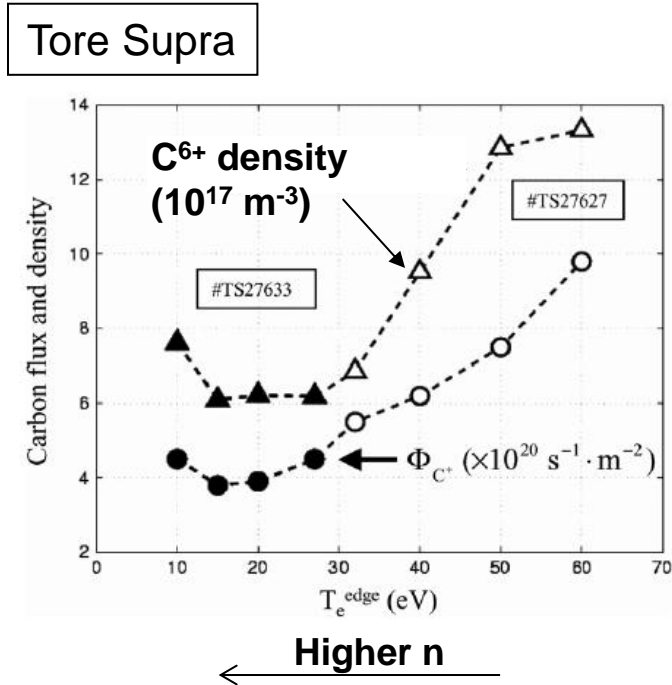
TEXTOR-DED (m/n=6/2)



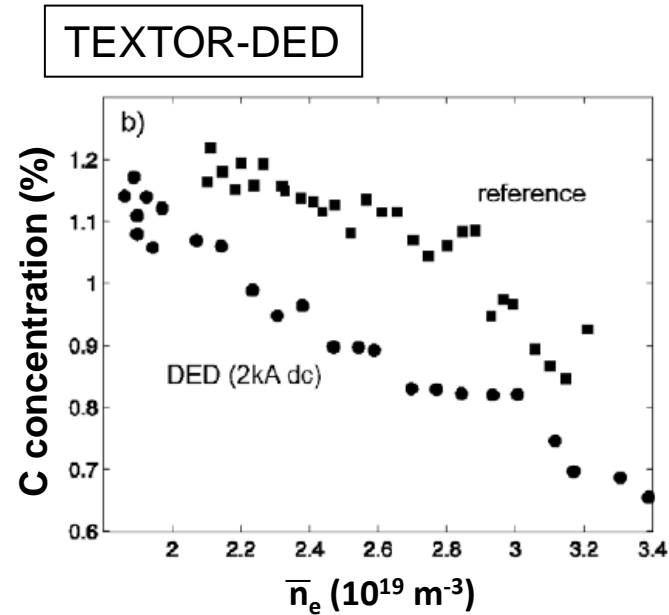
M. Clever et al., Nucl. Fusion **52** (2012) 054005.

Impurity screening has been observed in many devices with edge stochastic layer

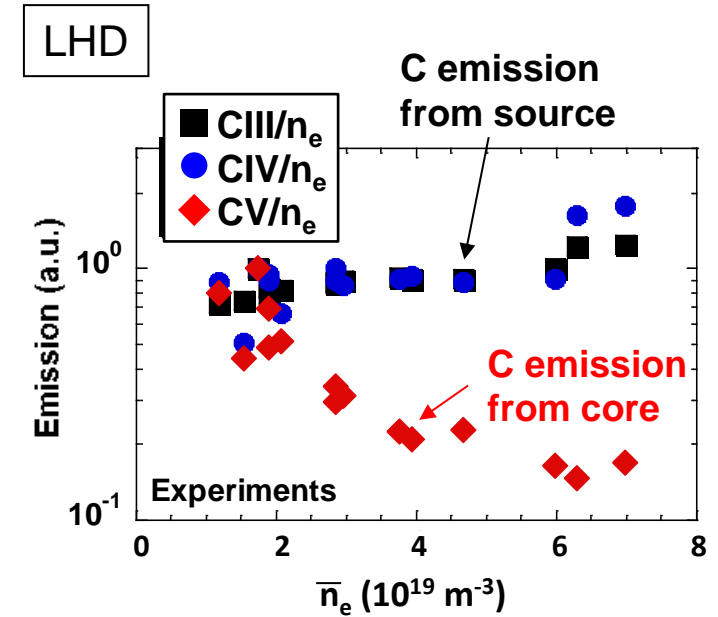
Experiments with density scan shows better screening at high density (low T_e)



Y. Corre et al., Nucl. Fusion **47** (2007) 119.

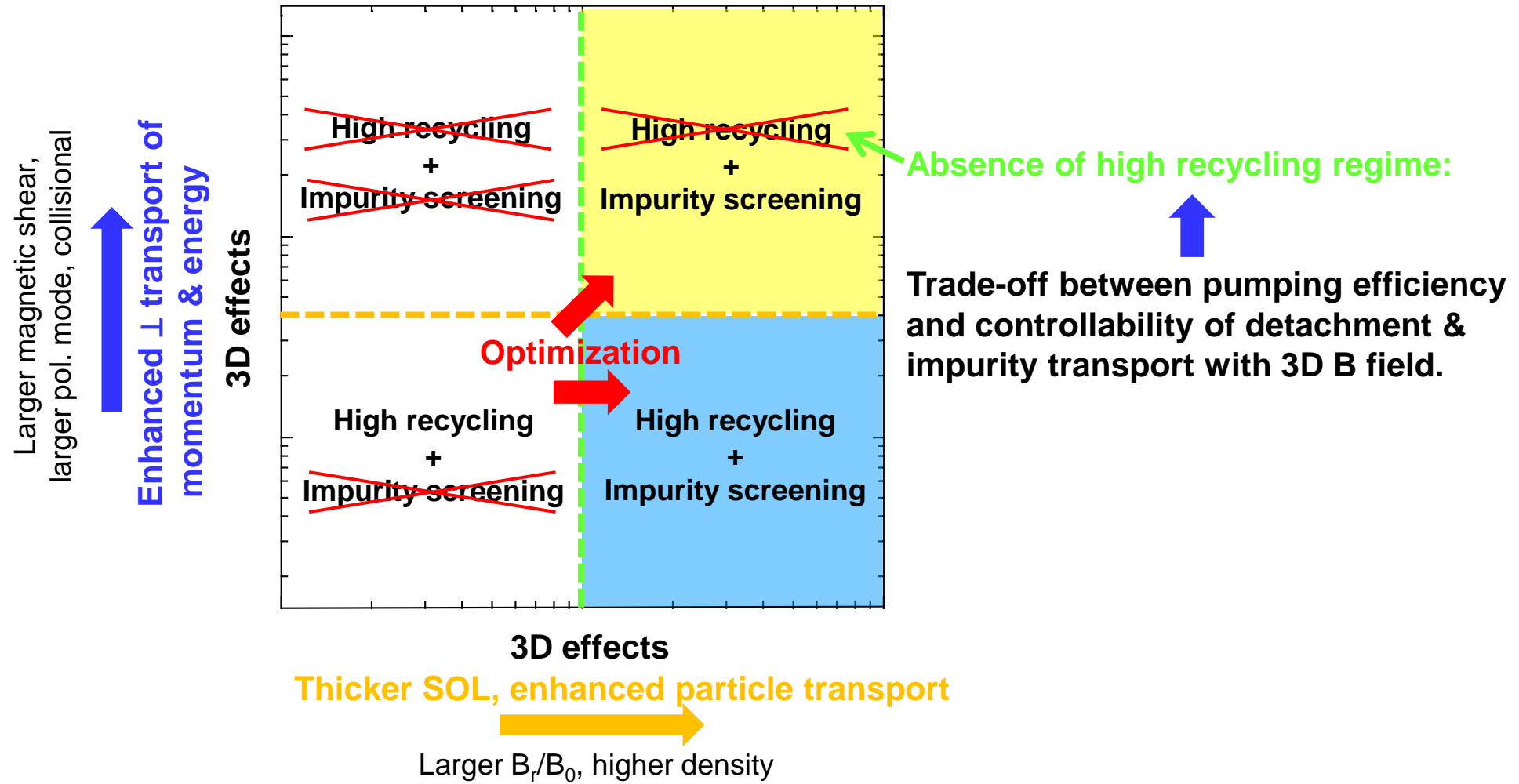


M. Lehnen et al., PPCF **47** (2005) B237.

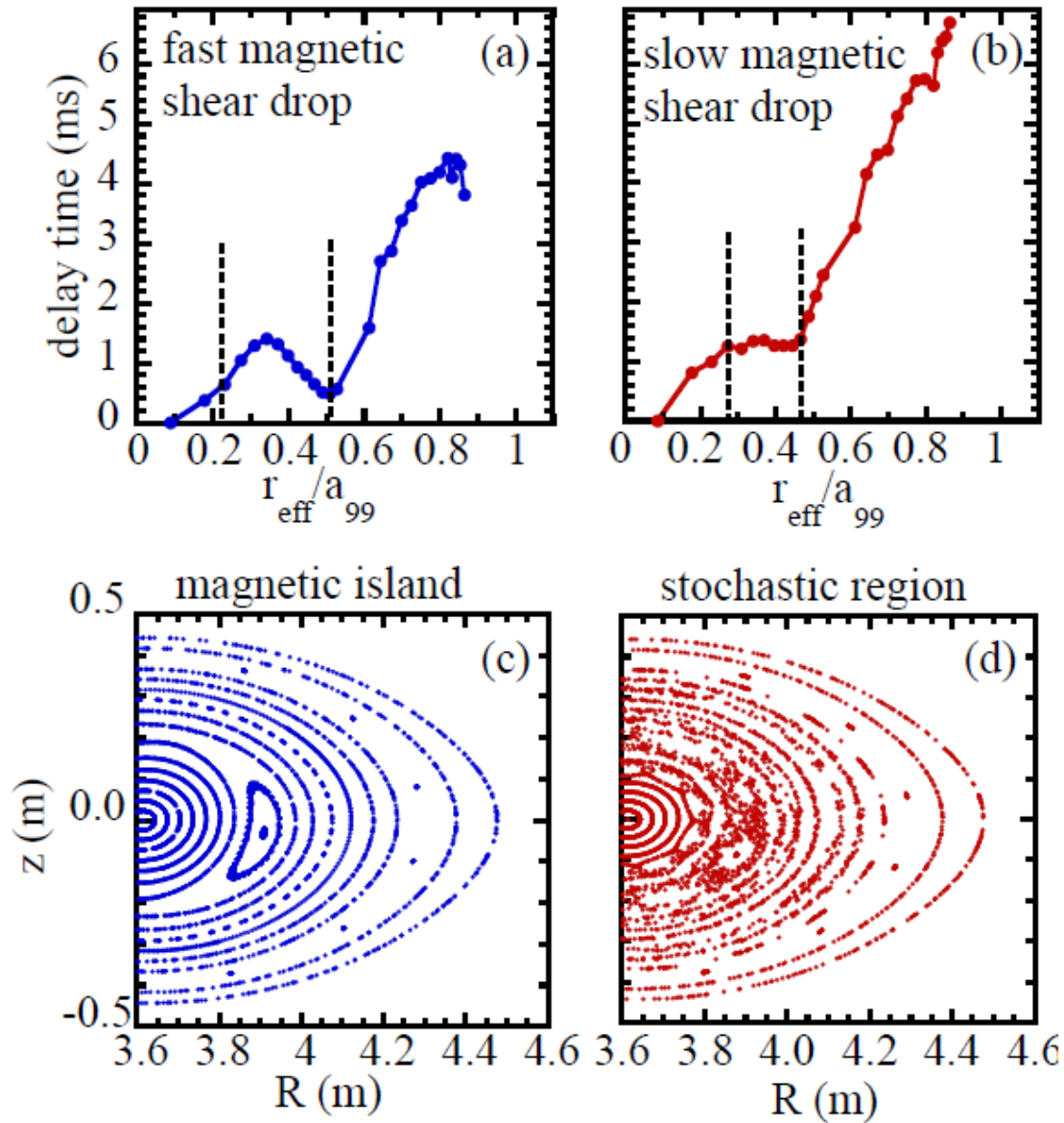


M. Kobayashi et al., Nucl. Fusion **53** (2013) 033011.

Which direction to go for 3D divertor optimization of a future reactor?



Magnetic field structure (stochastic or island) can be investigated with heat propagation experiments



The time delay of heat propagation changes depending on the magnetic topology:

Magnetic island → time delay at O-point

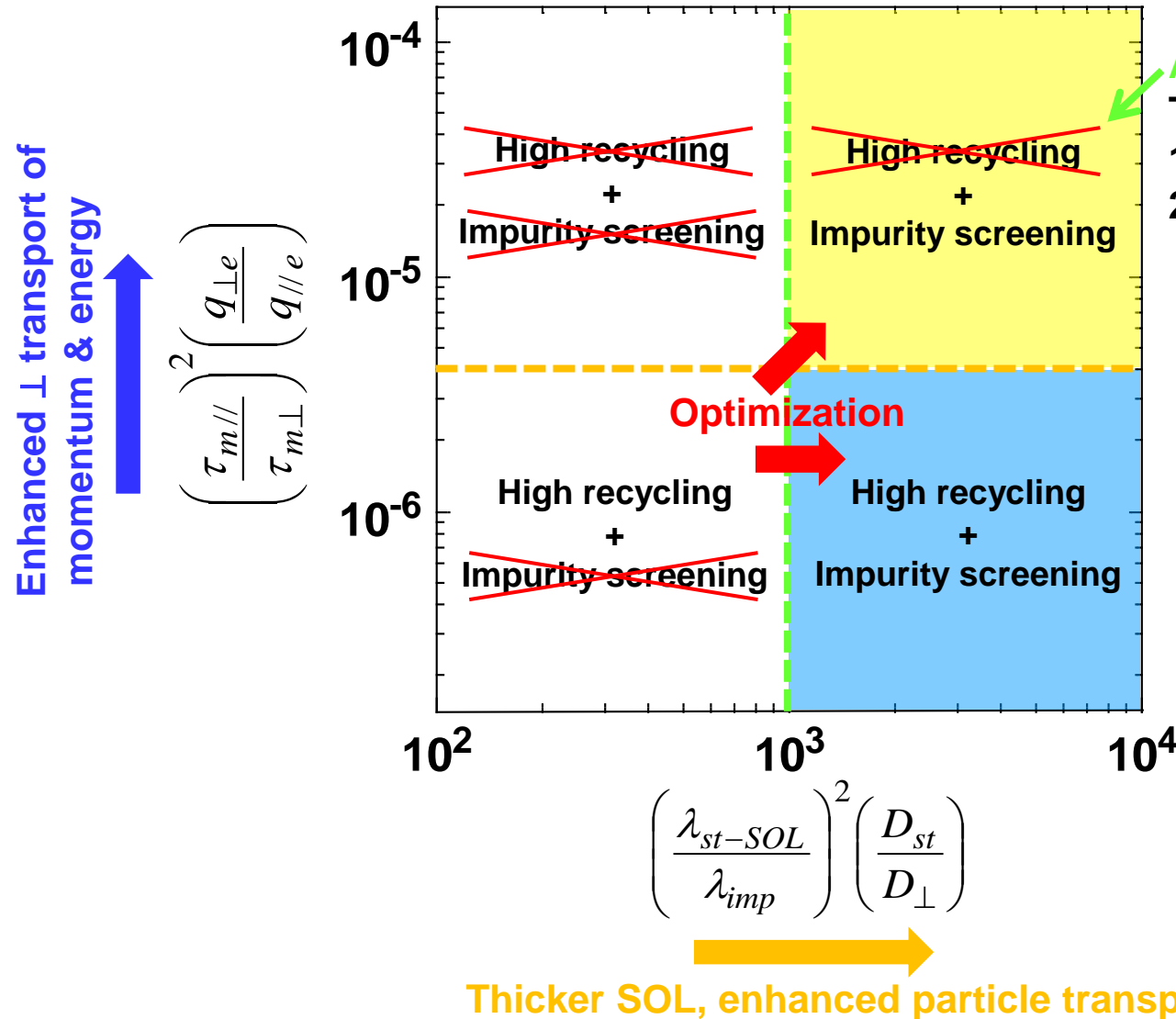
Stochastic field → no time delay

OV/2-3 K.Ida et al. (Monday)

EX/1-3 T. Evans et al. (Tuesday)

K. Ida et al., New Journal of Physics **15** (2013) 013061.

Indication for 3D divertor optimization of a future reactor



Absence of high recycling regime:

This might not be drawback

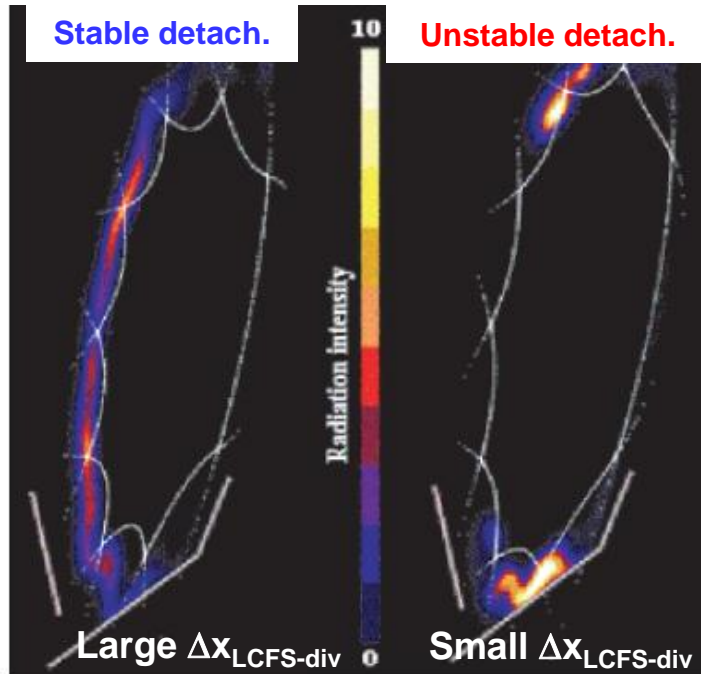
1. After **detached phase** is achieved.
2. Since relatively **high n_{up}** can be achieved due to slow T_{div} decrease
 \rightarrow **better impurity screening** via thermal force suppression
 \rightarrow better core plasma performance

\downarrow

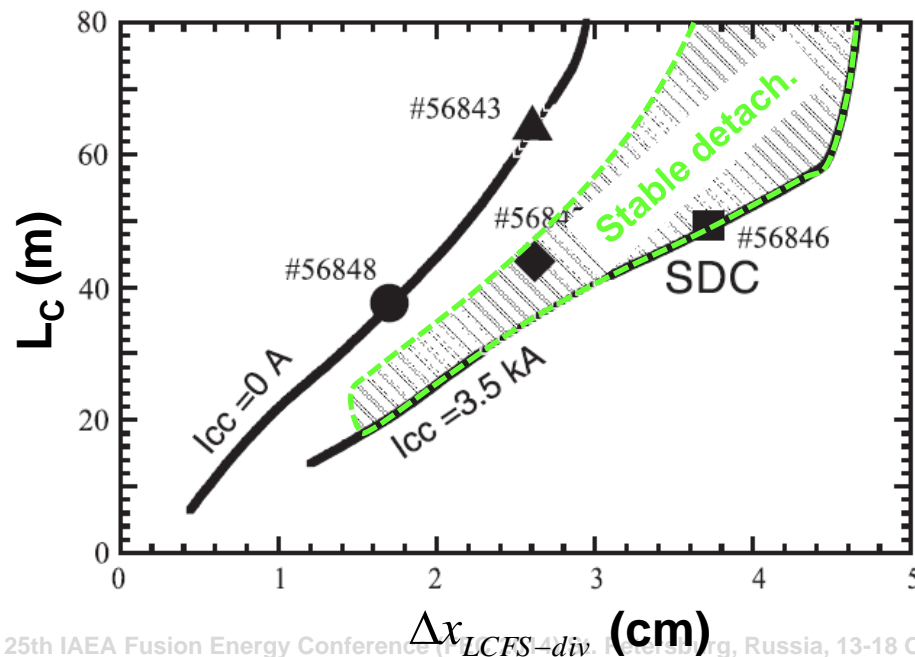
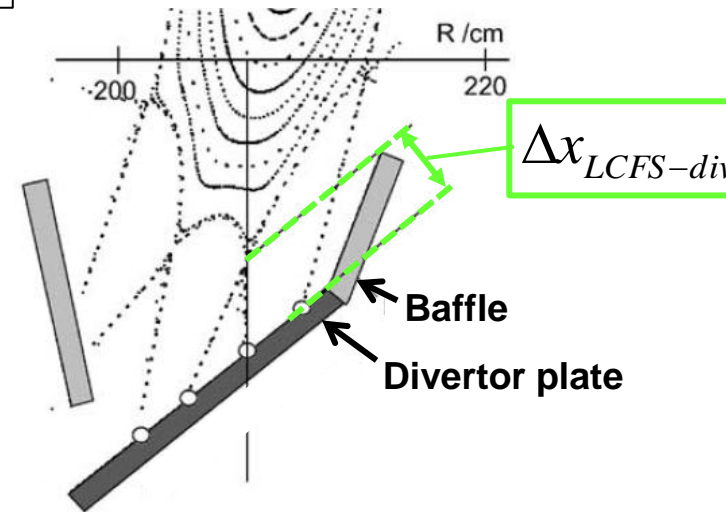
A divertor should be optimized considering balance with pumping efficiency.

Possibility of 3D edge radiation structure control and radiation stabilization

W7-AS



W7-AS



- Large $\Delta x_{LCFS-div}$ and short L_C
- Radiation region moves to inboard side
 - Hot island
 - Better neutral screening
 - Stable detachment

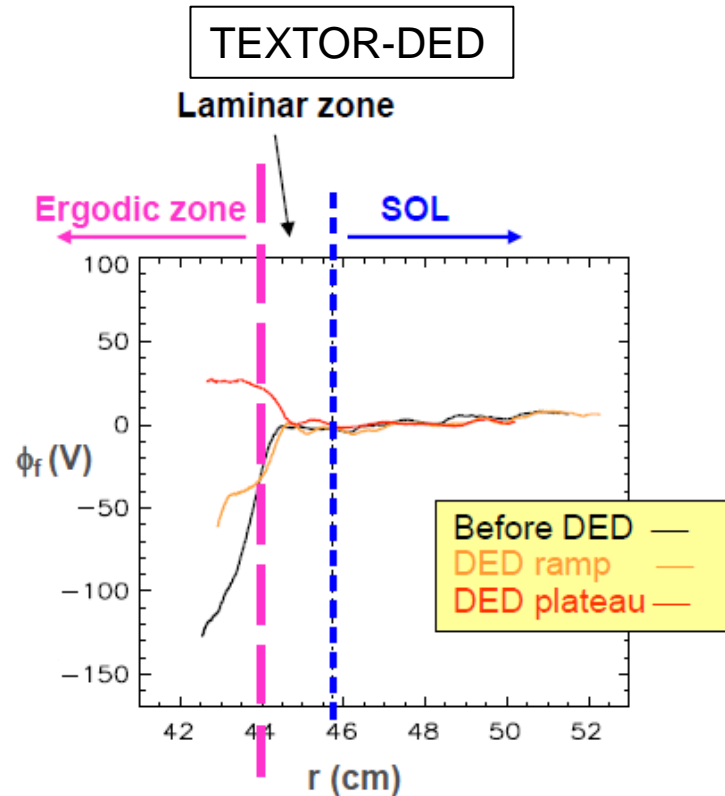
Decoupling between core and recycling region in terms of neutral fueling plays a key role.

[22] P. Grigull et al. JNM **313-316** (2003) 1287.

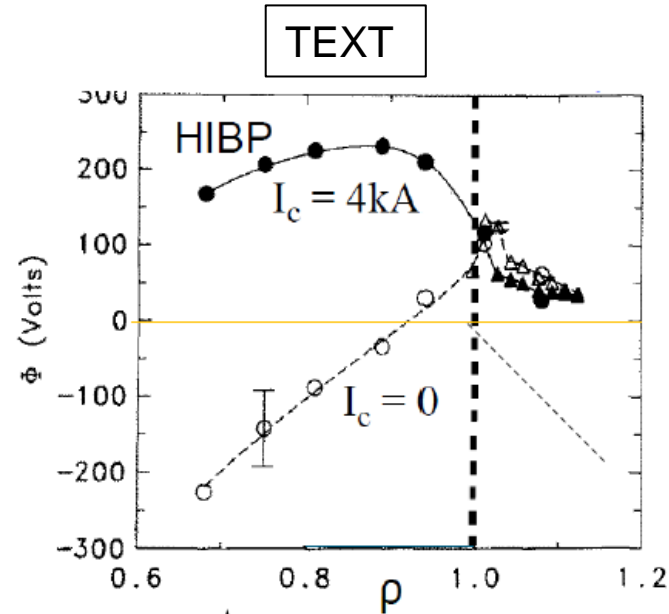
[23] Y. Feng et al., NF **45** (2005) 89.

Impact of RMP on edge electric field

In many devices, the change of edge electric field (potential profile) has been observed
→ Effects on edge turbulence, drift → impurity transport

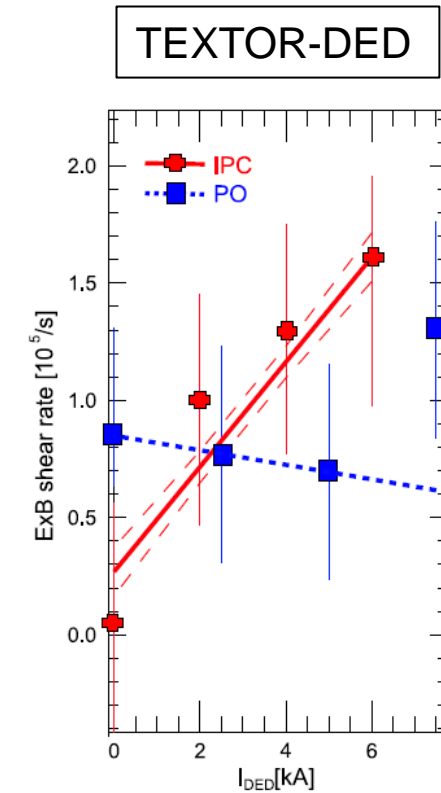


Y. Xu, EPS(2008), O4-6



V_f profile affected by RMP in TEXT
(Limiter)

A. Wootton, JNM 176-177 (1987)



O. Schmitz et al., JNM 390-391 (2009) 330.

These results suggest that the stochastic layer induced by RMP application can impact on edge turbulence transport, and hence on plasma-wall interaction, impurity transport and also plasma confinement.