

Development Status of FeCrAl-ODS Cladded Accident Tolerant Fuel for LWRs

K. Sakamoto: Nippon Nuclear Fuel Development Co., Ltd.

J. Matsunaga: Global Nuclear Fuel-Japan Co., Ltd.

M. Sasaki: Hitachi GE Vernova Nuclear Energy, Ltd.

N. Oono-Hori: Tohoku University

S. Yamashita: Japan Atomic Energy Agency

Contents

- Background
- Concept of FeCrAl-ODS
- Current status:
 - Manufacturing technology development
 - Experimental study
 - Analytical study
- Challenges and data gaps

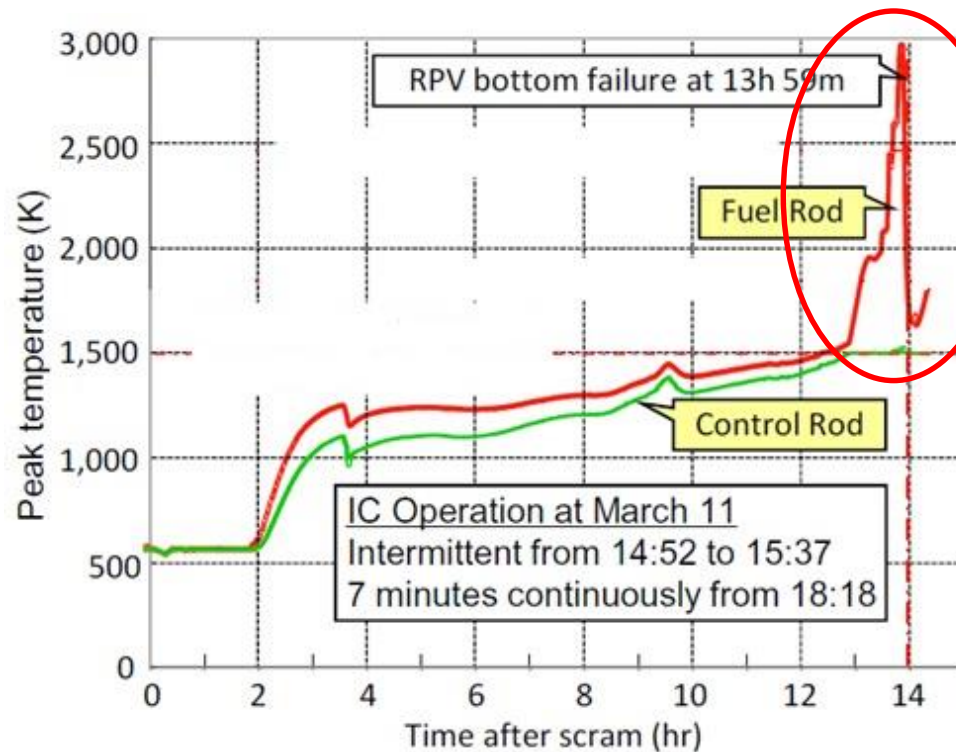
3

Background (2)

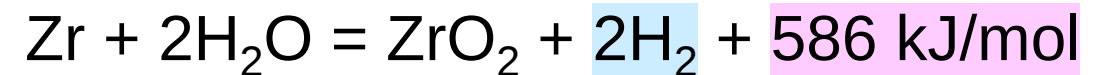
- Requirements for ATF of LWRs^{*1}

- #1: High resistance to steam oxidation

*1: Light Water Reactors



Catastrophic temperature increase by zirconium oxidation



Hydrogen explosion

Temp. escalation

Accident analysis of Fukushima-Daiichi unit 1 by SAMPSON code

Reference: M. Naito, et al. , Proc. Int. Mtg. on SA Assessment and Management, Paper 6005, San Diego, USA, November 11-15, 2012

- Requirements for ATF of LWRs
 - #2: Applicability to existing (current) LWRs

ATF needs to fit the current fuel cycle system

- ☐ Manufacturing
- ☐ Normal operation, AOO^{*1}, DBA^{*2}
- ☐ Storage
- ☐ Recycling
- ☐ Disposal

*1: Anticipated Operational Occurrence

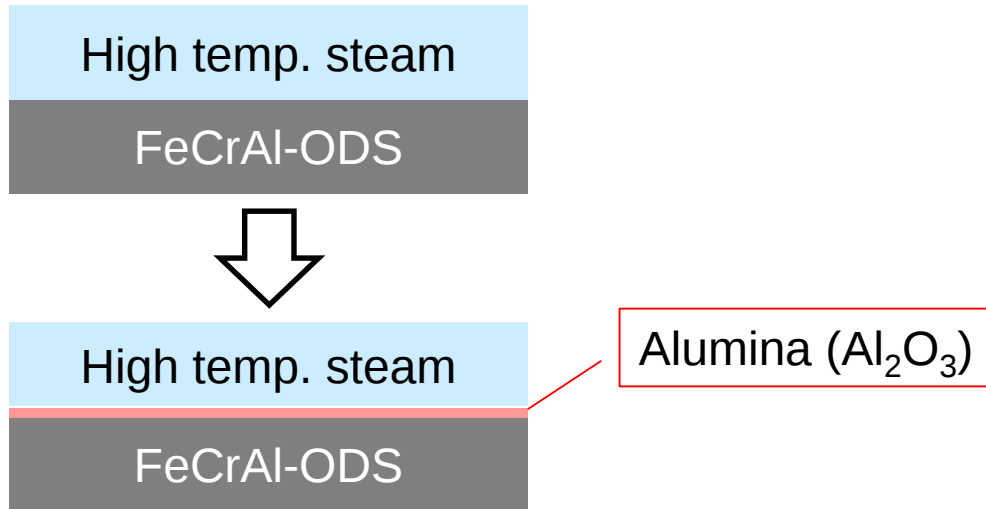
*2: Design Based Accident

Contents

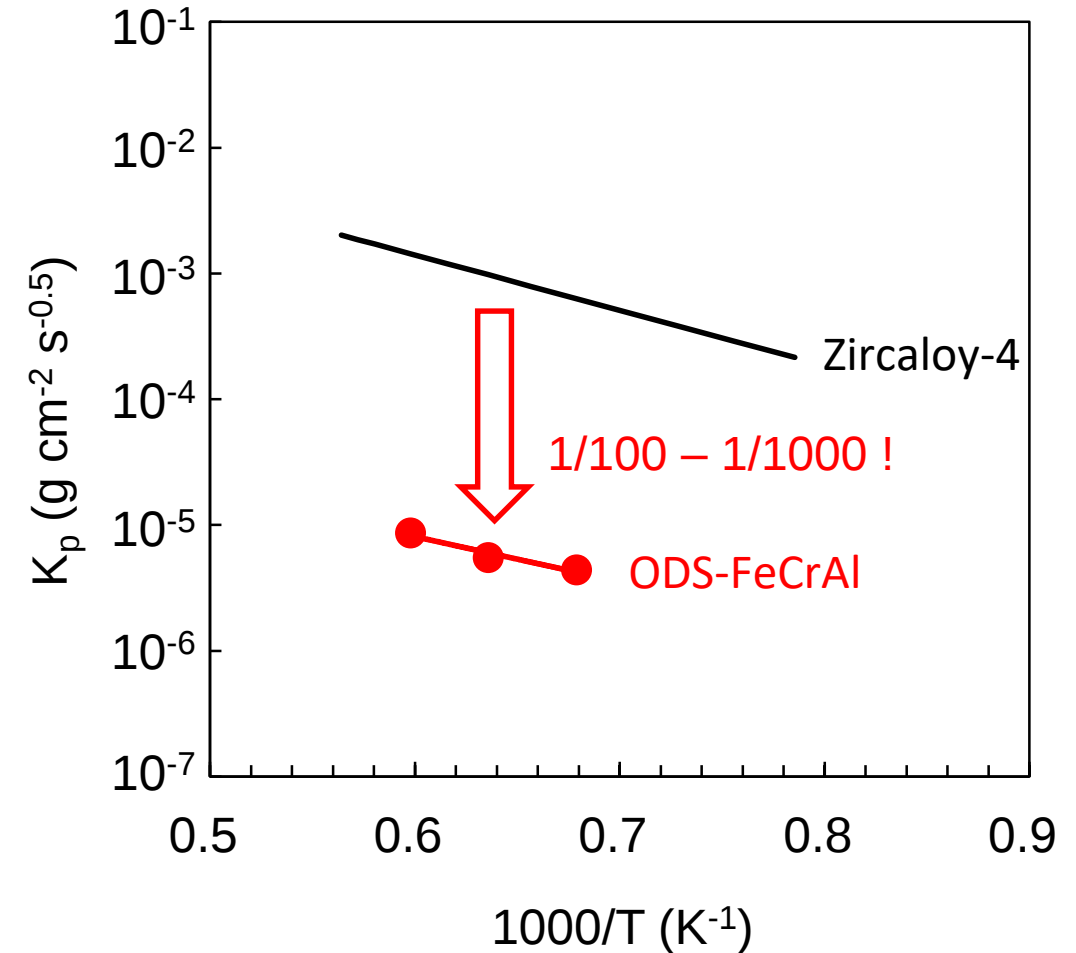
- Background
- Concept of FeCrAl-ODS
- Current status:
 - Manufacturing technology development
 - Experimental study
 - Analytical study
- Challenges and data gaps

Concept of FeCrAl-ODS (1)

● Suppression of steam oxidation



Alumina layer suppresses steam oxidation by $10^2 - 10^3$
→ Suppression of heat and hydrogen generation

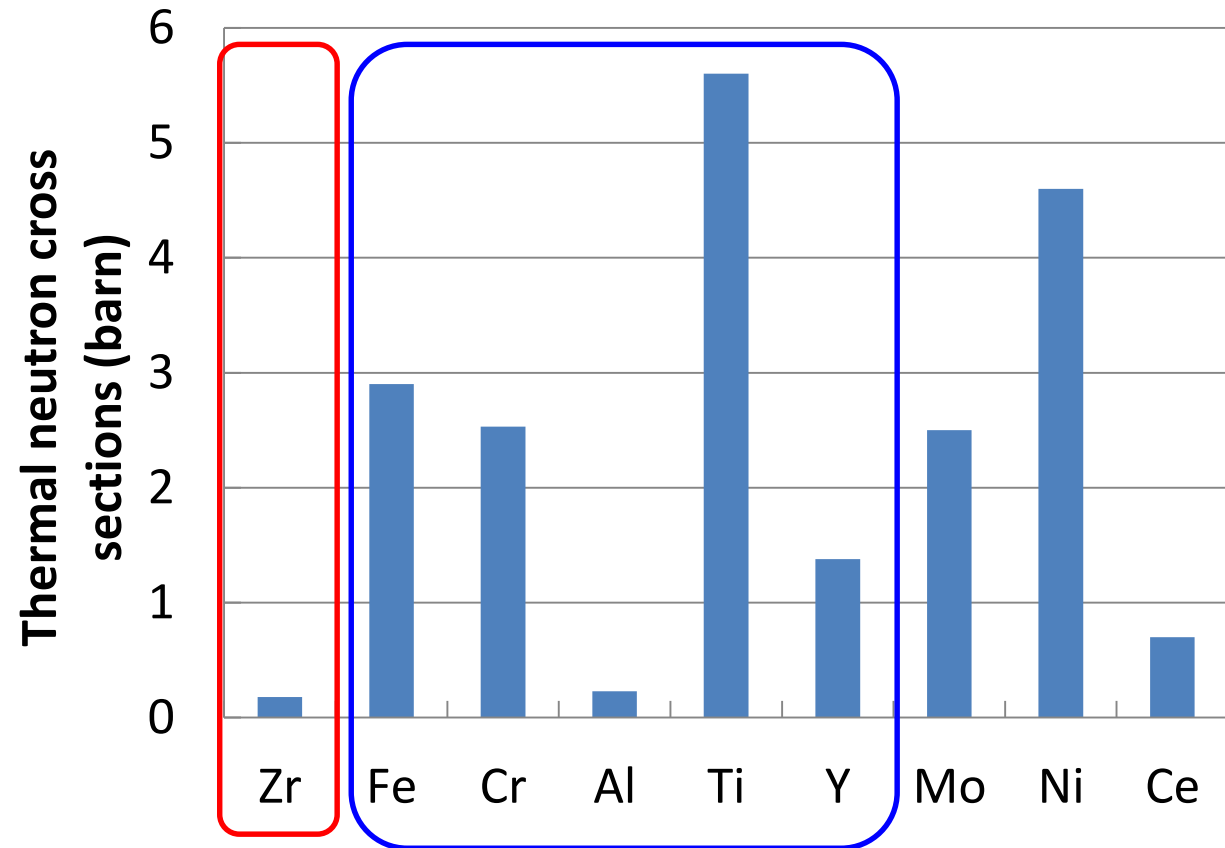


Oxidation rate of Zry-4 and FeCrAl-ODS

Reference: K. Sakamoto, et al. , Proc. Top Fuel 2018, 30 Sep. – 04 Oct. 2018, Prague, Czech Republic

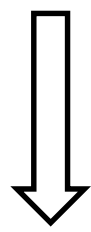
Concept of FeCrAl-ODS (2)

- Compensation of higher reactivity penalty (1)



Zircaloy:
Zr >98 mass%

FeCrAl-ODS:
 $\text{Fe-12Cr-6Al-0.5Ti-0.4Zr-0.5Y}_2\text{O}_3$



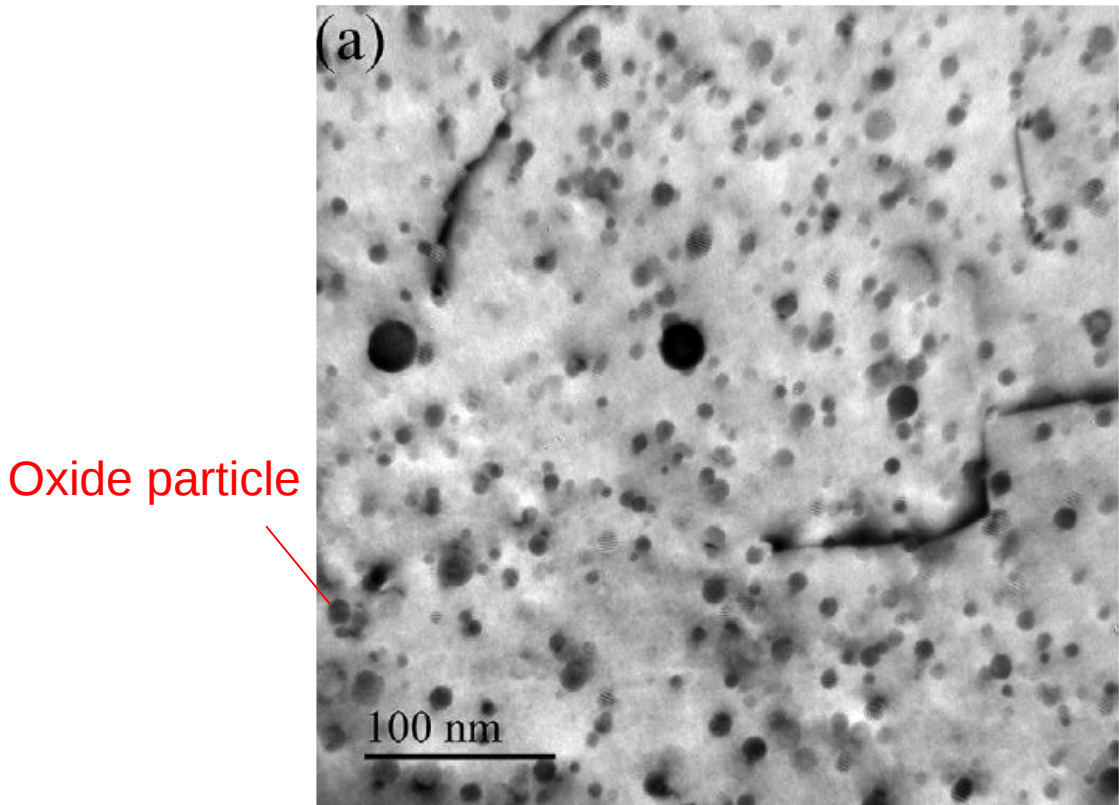
Reduction of cladding material is essential to use in LWRs

Comparison of thermal neutron cross sections

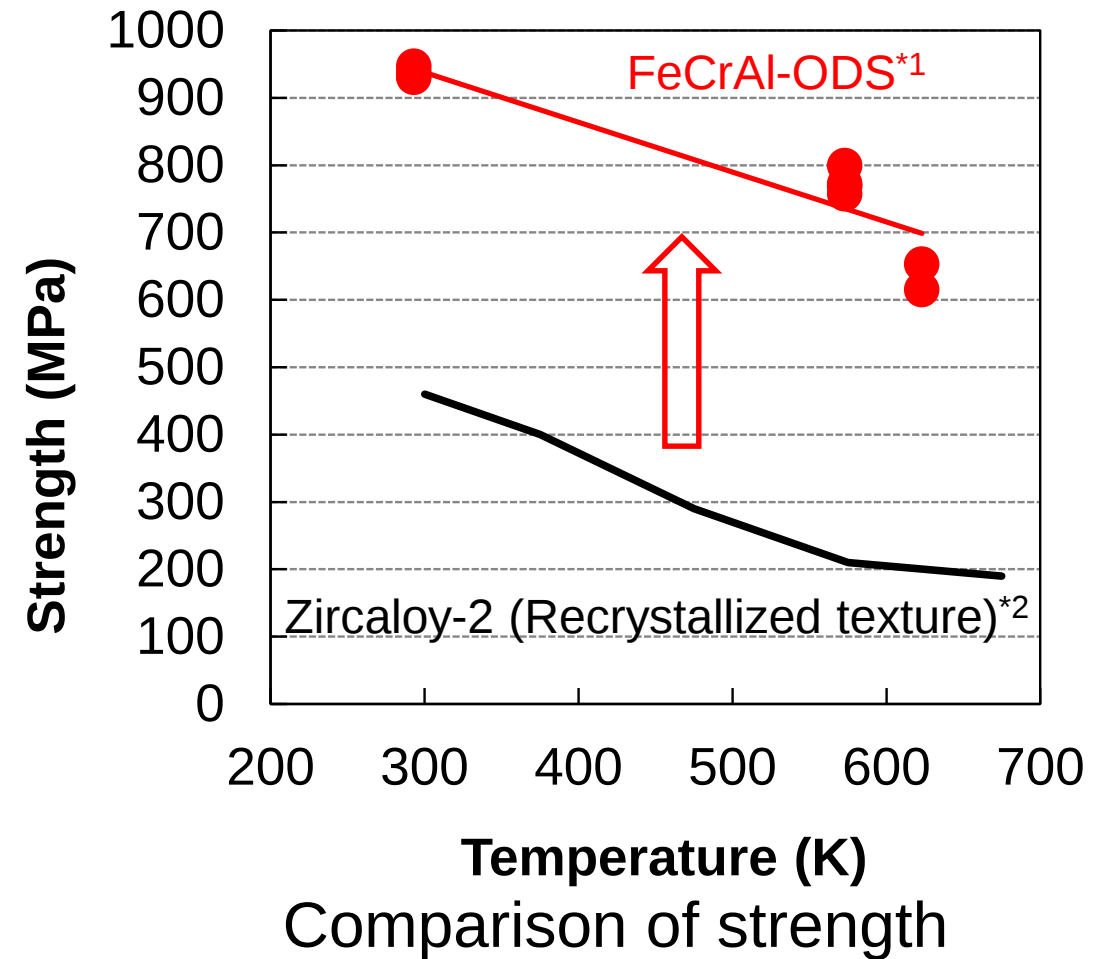
Reference: K. Kimura , Hoshasen data book (in Japanese), Chijin Syokan, 1958.

Concept of FeCrAl-ODS (3)

- Compensation of higher reactivity penalty (2)



TEM observation of oxide particles



Reference: J. Gao et al., J. Nucl. Mater., 524 (2019) 1-8

Reference:

*1: K. Sakamoto, et al., Proc. Top Fuel 2018, 30 Sep. – 04 Oct. 2018, Prague, Czech Republic

*2: Onchi et al., J. Nucl. Mater., 88 (1980) 226-235

Contents

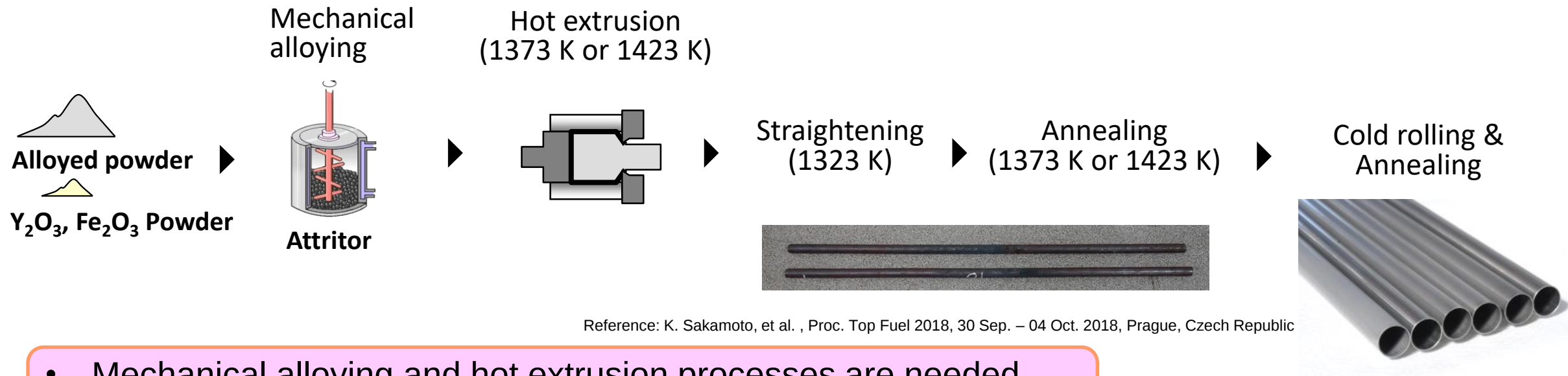
- Background
- Concept of FeCrAl-ODS
- Current status:
 - Manufacturing technology development
 - Experimental study
 - Analytical study
- Challenges and data gaps

Manufacturing technology development (1)

- Chemical composition (in mass %)

Material	Cr	Al	Ti	Zr	Y ₂ O ₃
FeCrAl-ODS tube	12	6	0.5	0.4	0.5

- Procedure (Tube fabrication)



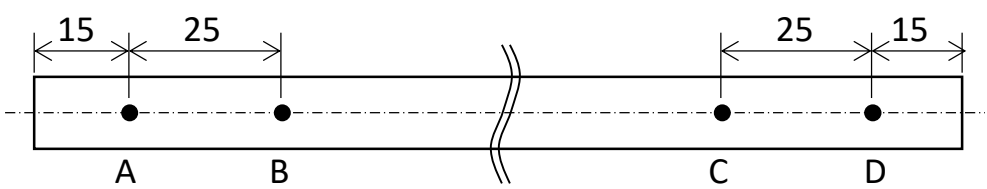
Reference: K. Sakamoto, et al. , Proc. Top Fuel 2018, 30 Sep. – 04 Oct. 2018, Prague, Czech Republic

- Mechanical alloying and hot extrusion processes are needed
- Current cold rolling & annealing system can be adopted

Manufacturing technology development (2)

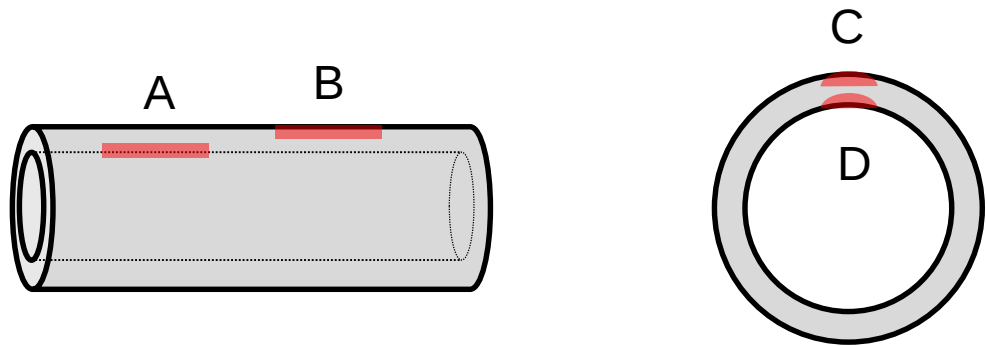
● Flaw detection

Fuel type: 10x10-type, Wall-thickness: 0.3 mm
Units: mm



Location	Surface	Direction	Length (μm)	Width (μm)	Depth (μm)
A	Inner	Circumferential	768	49	28
B	Outer	Circumferential	753	49	29
C	Outer	Longitudinal	767	48	30
D	Inner	Longitudinal	763	47	31

~10% of wall-thickness

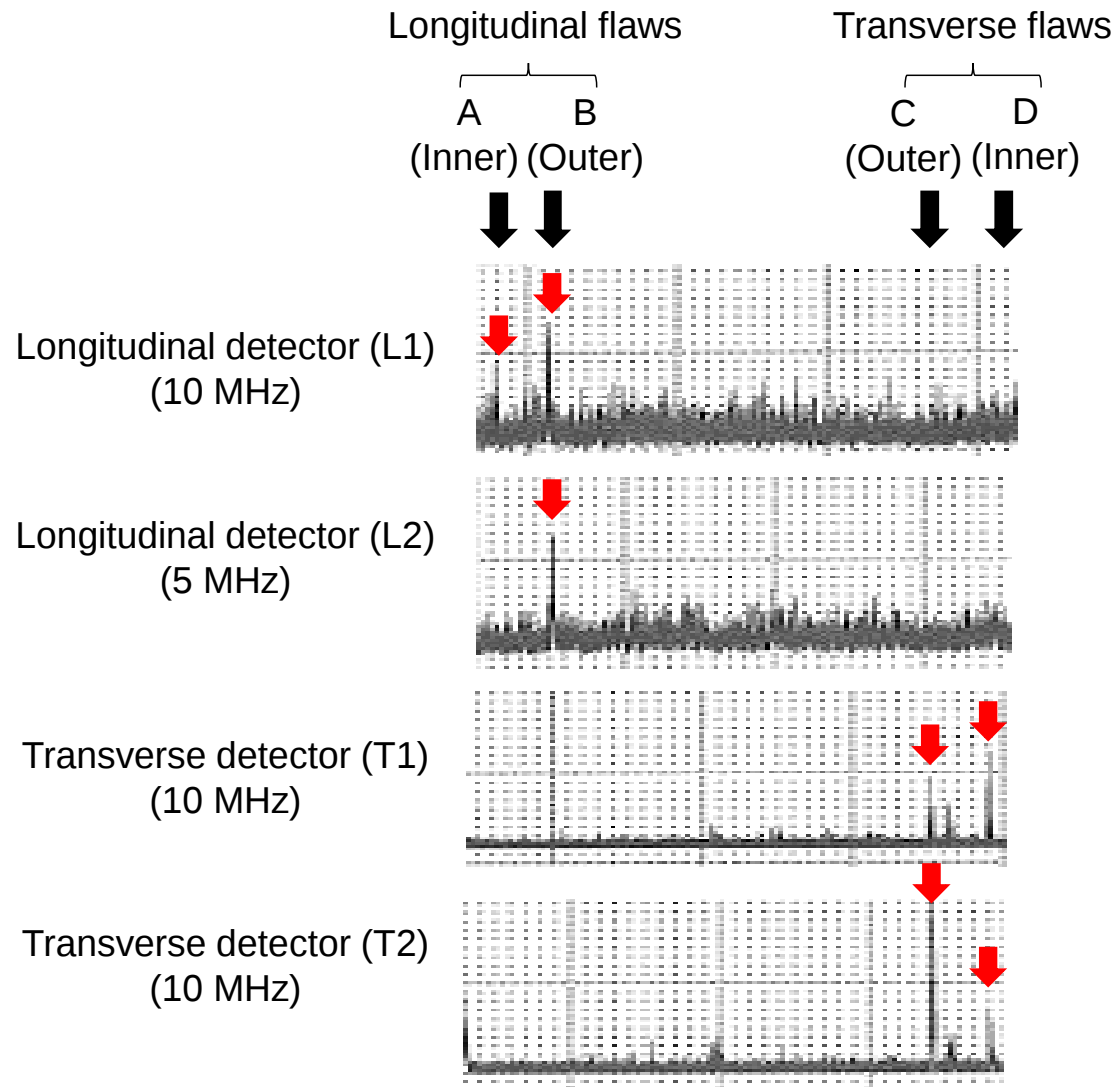


● Ultrasonic detection methods

- Inspection in water
- 4 detectors (2 for transverse and 2 for longitudinal (axial) flaws)
- Ultrasonic angle beam method at 10 MHz or 5 MHz

Reference: K. Sakamoto, et al., Proceedings of TopFuel2021, Santander, Spain, October 24-28, 2021,

Manufacturing technology development (3)



- All four artificial flaws with $\sim 30 \mu\text{m}$ depth ($\sim 10\%$ of wall-thickness) were successfully detected
- S/N ratio in detection of longitudinal flaws was worse than that of transverse flaws ($\rightarrow$ anisotropic shape of crystal grains in FeCrAl-ODS tube)
- A change of frequency was required for L2 from 10 MHz to 5 MHz

- Ultrasonic flaw detection method would be applicable to the inspection of FeCrAl-ODS tubes
- Further optimization is clearly needed for the practical application

Reference: K. Sakamoto, et al., Proceedings of TopFuel2022, Raleigh, NC, October 9-13, 2022, 161-169

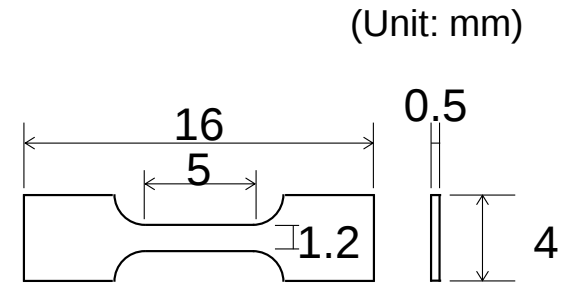
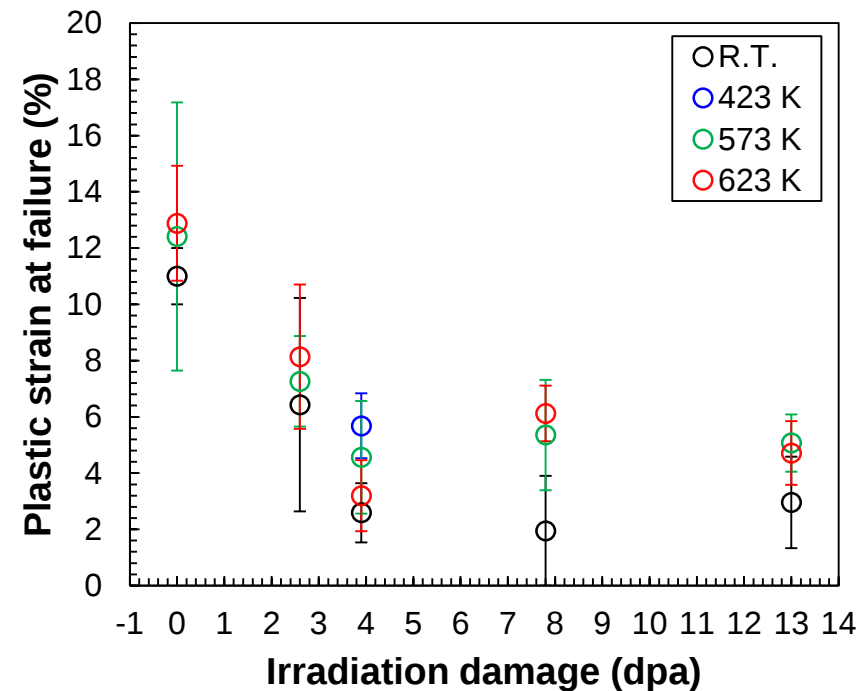
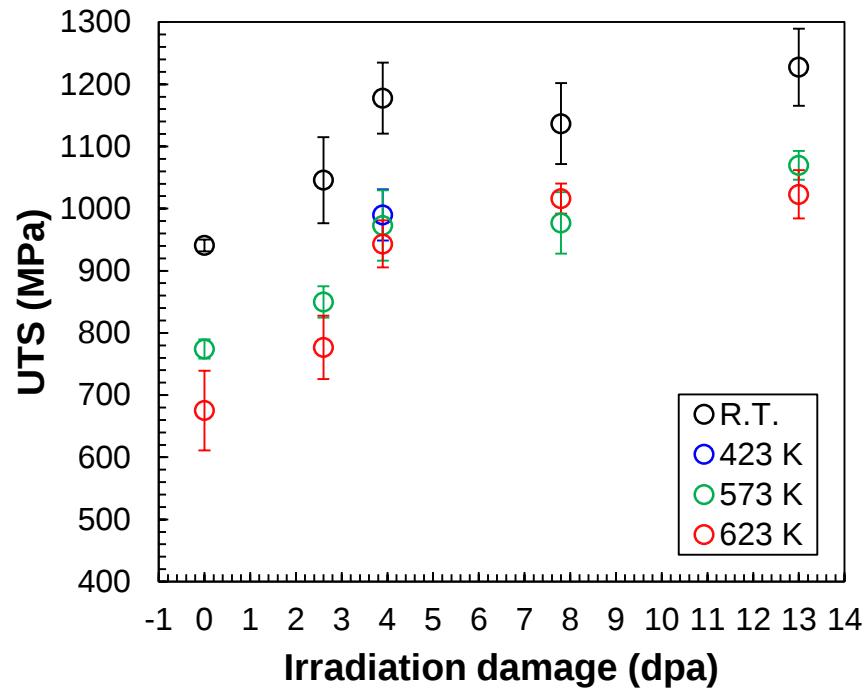
Contents

- Background
- Concept of FeCrAl-ODS
- Current status:
 - Manufacturing technology development
 - Experimental study
 - Analytical study
- Challenges and data gaps

Experimental study (1)

- Effect of neutron irradiation on mechanical properties

- SS-J2 type specimens
- Irradiated in ORNL HFIR at 573 K up to 13 dpa



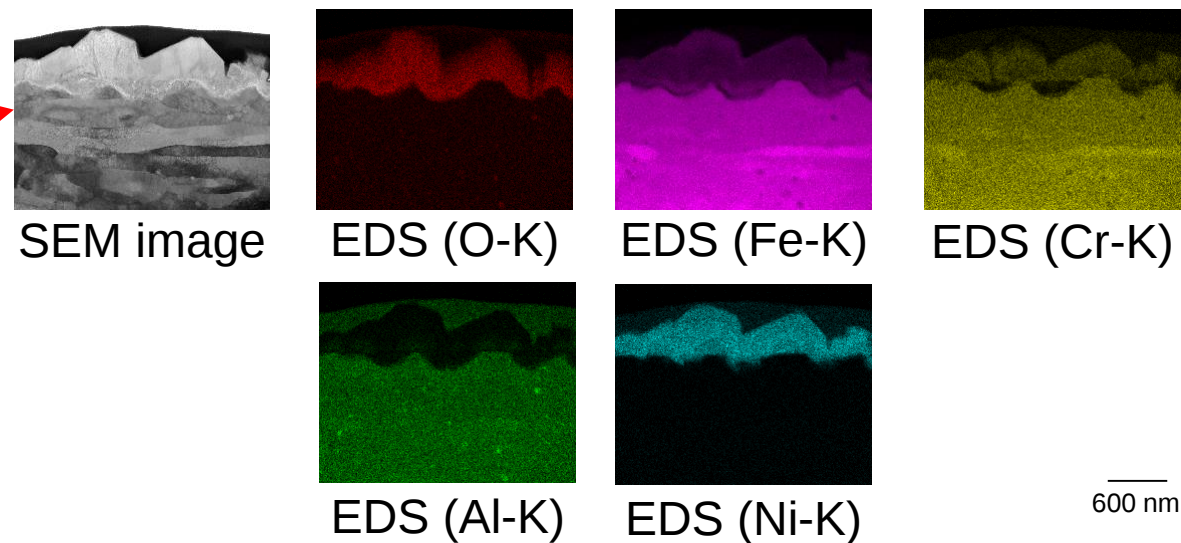
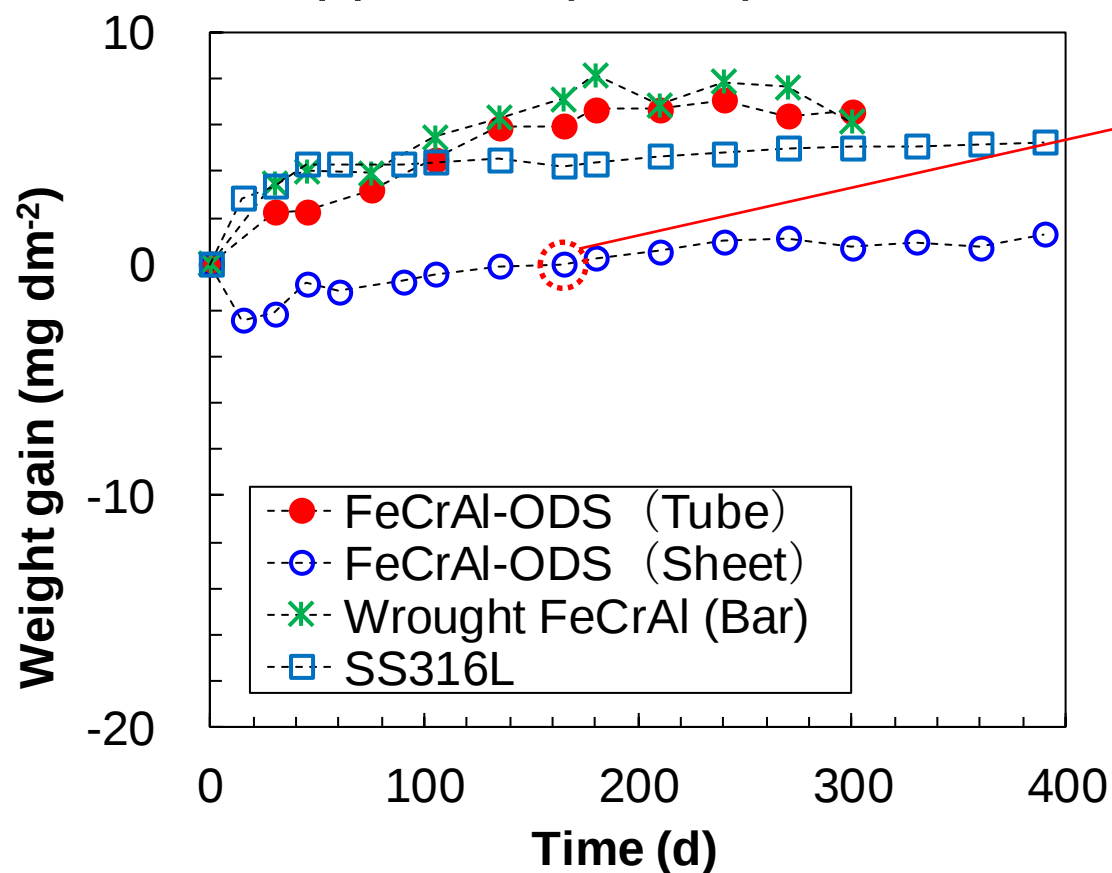
Reference: K. Sakamoto et al., Proceedings of the 2023 Water Reactor Fuel Performance Meeting, WRFPM2023, July 17-21, Xi'an, China, pp. 20-28, 2024

- Strength increases and ductility decreases with irradiation damage up to 3.9 - 7.8 dpa and is almost at saturation value at higher irradiation damage levels

Experimental study (2)

● Corrosion property

- Corroded in autoclave with circulation pure water at 563 K, ~ 8MPa and 6 ppmDO (563 K)



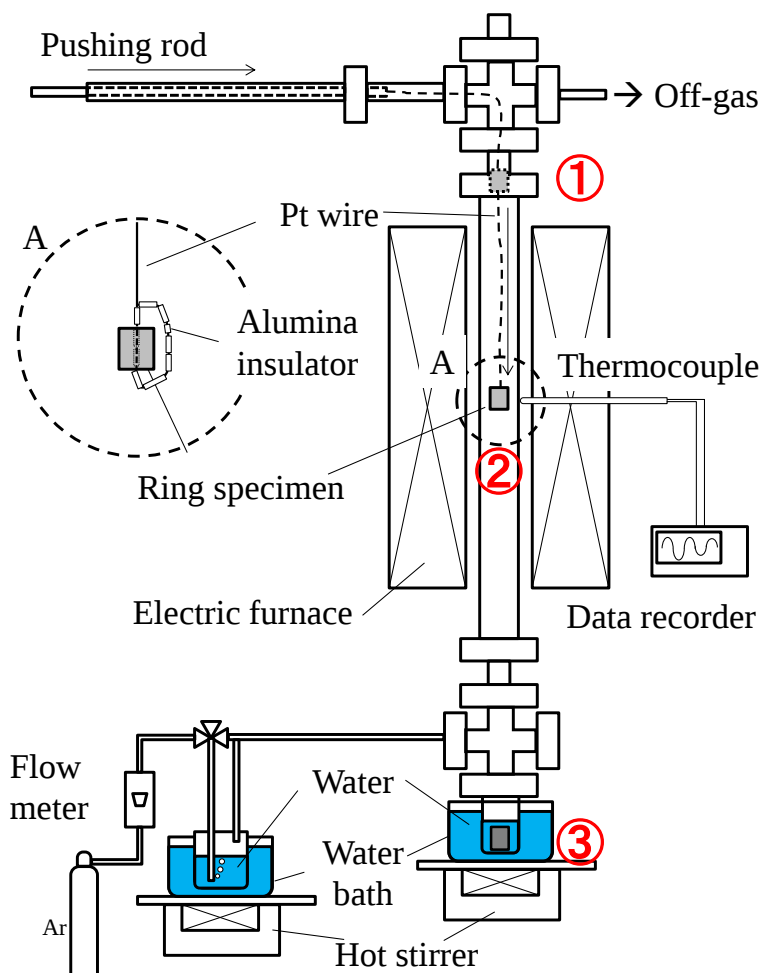
Thickness reduction due to dissolution ~ 0.3 μm

- Stable corrosion resistance
- Very limited dissolution to coolant water

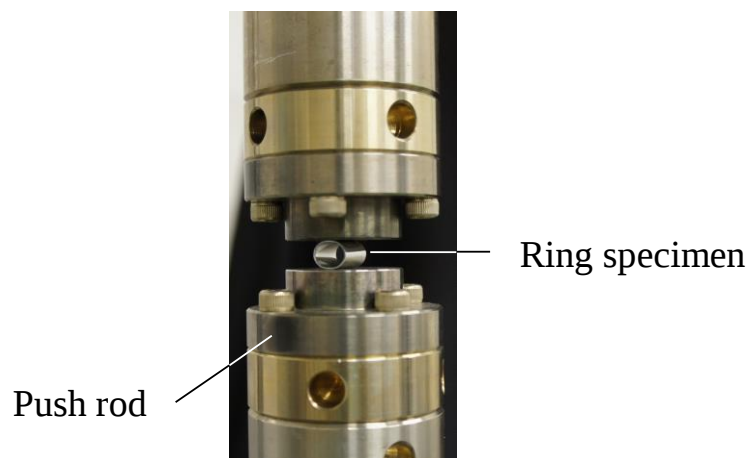
Reference: K. Sakamoto, et al. , Proc. Top Fuel 2018, 30 Sep. – 04 Oct. 2018, Prague, Czech Republic

Experimental study (3)

● Resistance to high temperature oxidation followed by water quenching



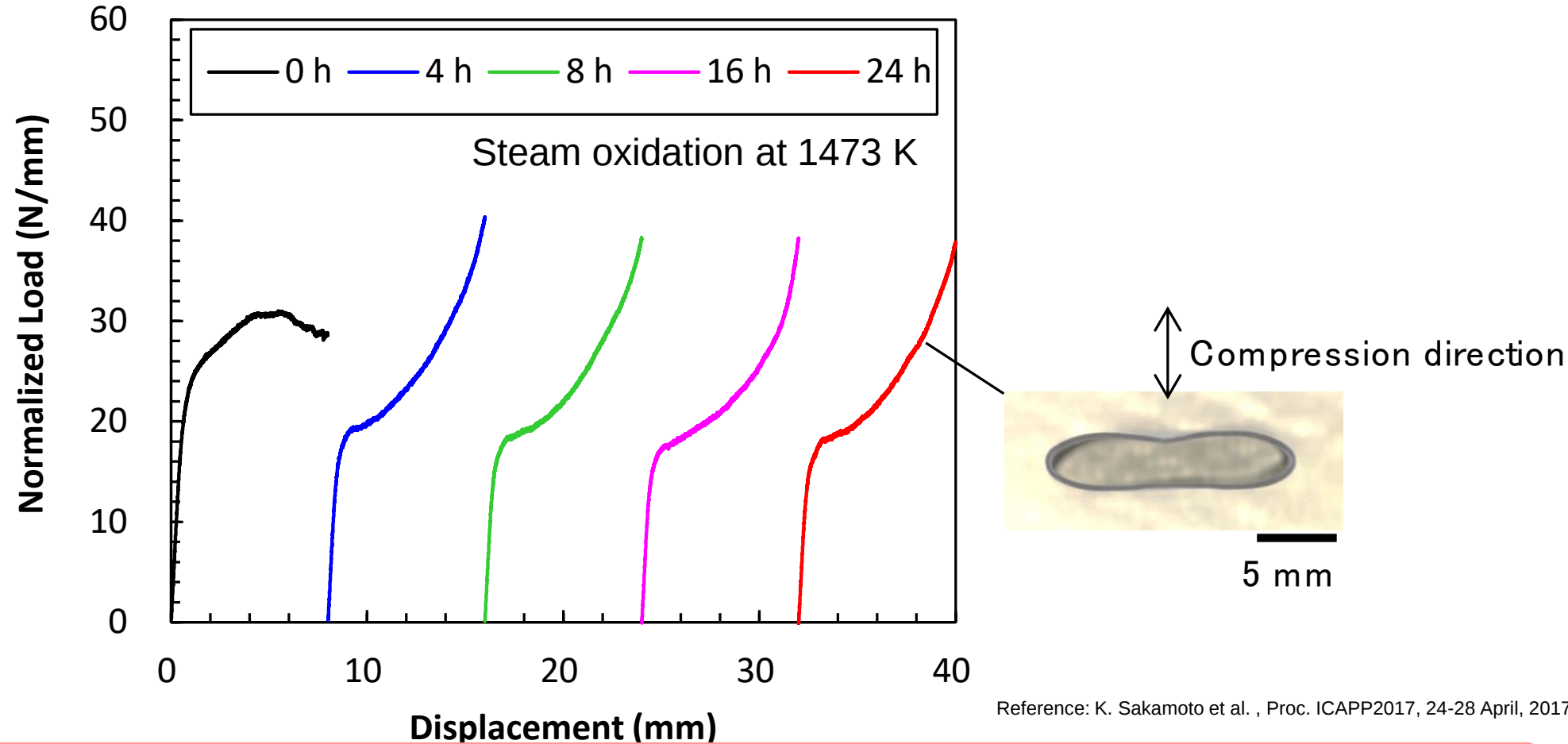
- Steam oxidation at 1473 K
 - Heating of electric furnace (Position ①)
 - Moving down to center zone of electric furnace (Position ②)
 - Dropping down to water (Position ③)
- Ring compression test at RT



Reference: K. Sakamoto et al. , Proc. ICAPP2017, 24-28 April, 2017, Fukui and Kyoto, Japan

Experimental study (4)

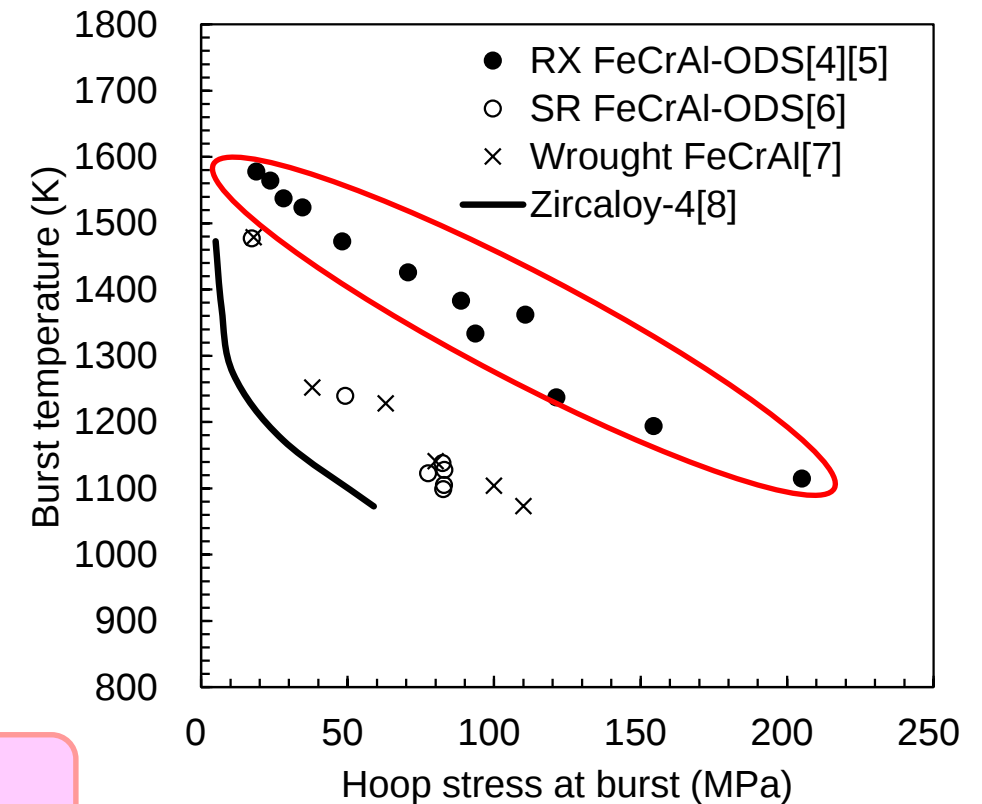
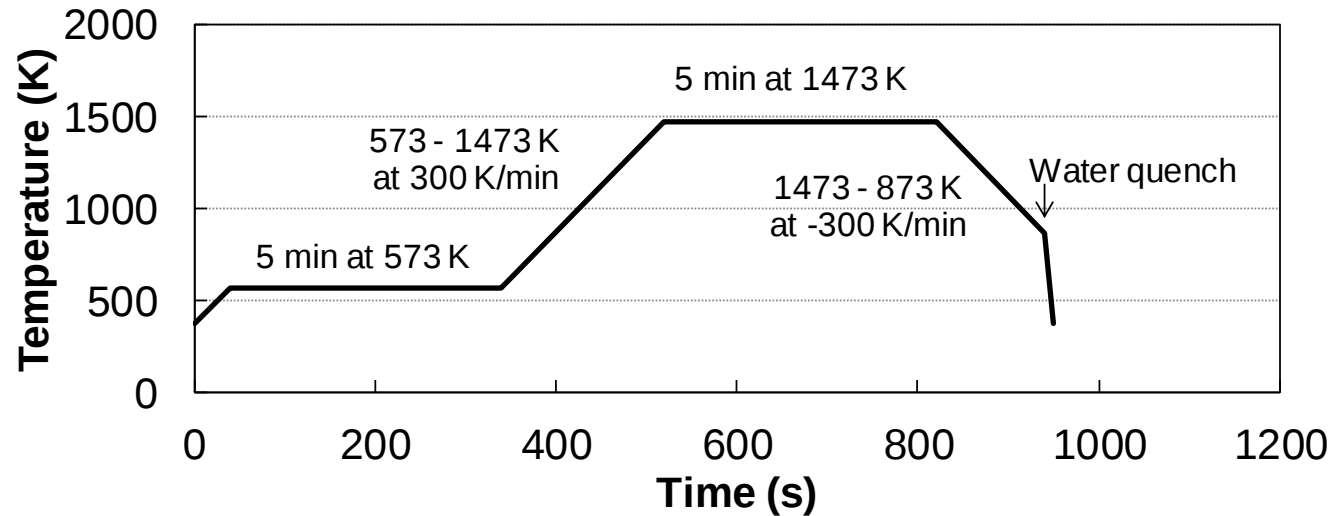
- Resistance to high temperature oxidation followed by water quenching



- Excellent resistance to high temperature steam and water quenching

Experimental study (5)

- Resistance to LOCA events (Integral LOCA tests)
 - Cooperative research activity under U.S. – Japan CNWG
 - Integral LOCA furnace module at ORNL SATS (Severe Accident Test Station)



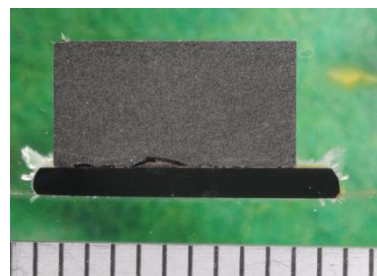
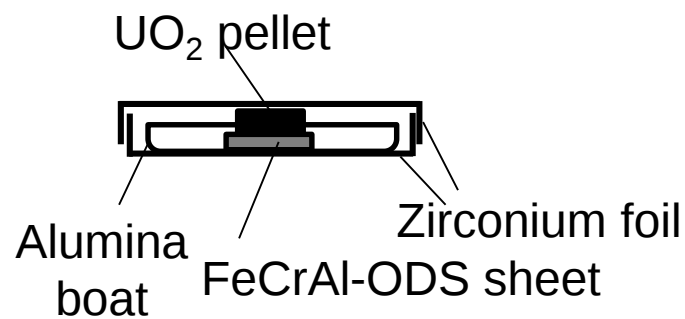
- [4] K. Sakamoto et al., J. Nucl. Mater., 557 (2021) 153276
- [5] K. Sakamoto et al., Proceedings of TopFuel2022, Raleigh, NC, October 9-13, 2022, 161-169
- [6] T. Narukawa et al., J. Nucl. Mater., 582 (2023) 154467
- [7] S.B. Bell et al., J. Nucl. Mater., 557 (2021) 153242
- [8] M. Ishikawa and S. Shiozawa, J. Nucl. Mater., 95, 1-2, (1980) 1-30

- No apparent embrittlement after water quenching
- Higher resistance to fracture (burst) than Zircaloy

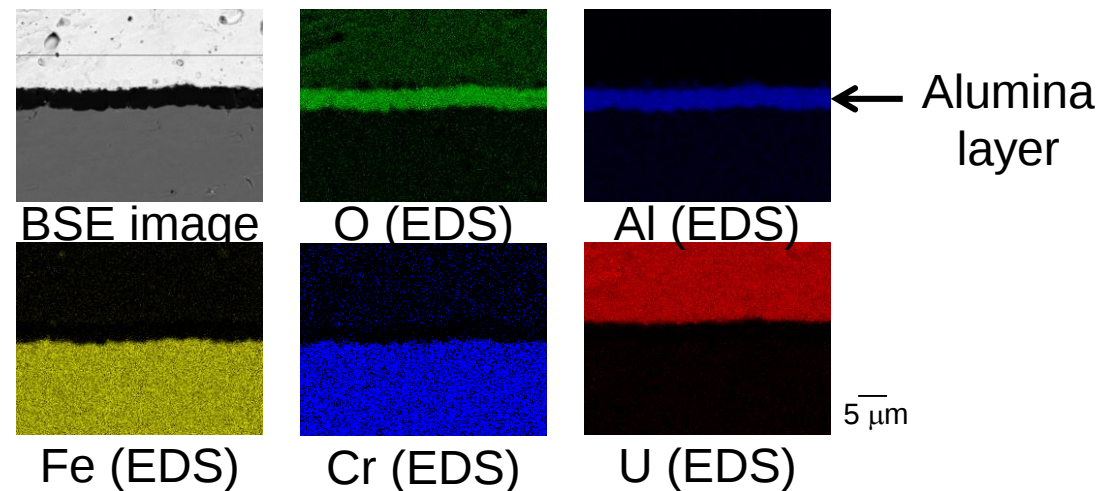
Reference: K. Sakamoto et al., Proceedings of the TopFuel 2024, September 29-October 3, Grenoble, France, pp. 63-69, 2024

Experimental study (6)

- Compatibility with UO_2 fuel
 - Sheet material was attached with UO_2 at 1723 K up to 25 h in inert-gas atmosphere



Optical image
(1723 K x 25 h)



Reference: K. Sakamoto, et al. , Proc. Top Fuel 2018, 30 Sep. – 04 Oct. 2018, Prague, Czech Republic

- Excellent resistance to fuel at high temperatures

Contents

- Background
- Concept of FeCrAl-ODS
- Current status:
 - Manufacturing technology development
 - Experimental study
 - Analytical study
- Challenges and data gaps

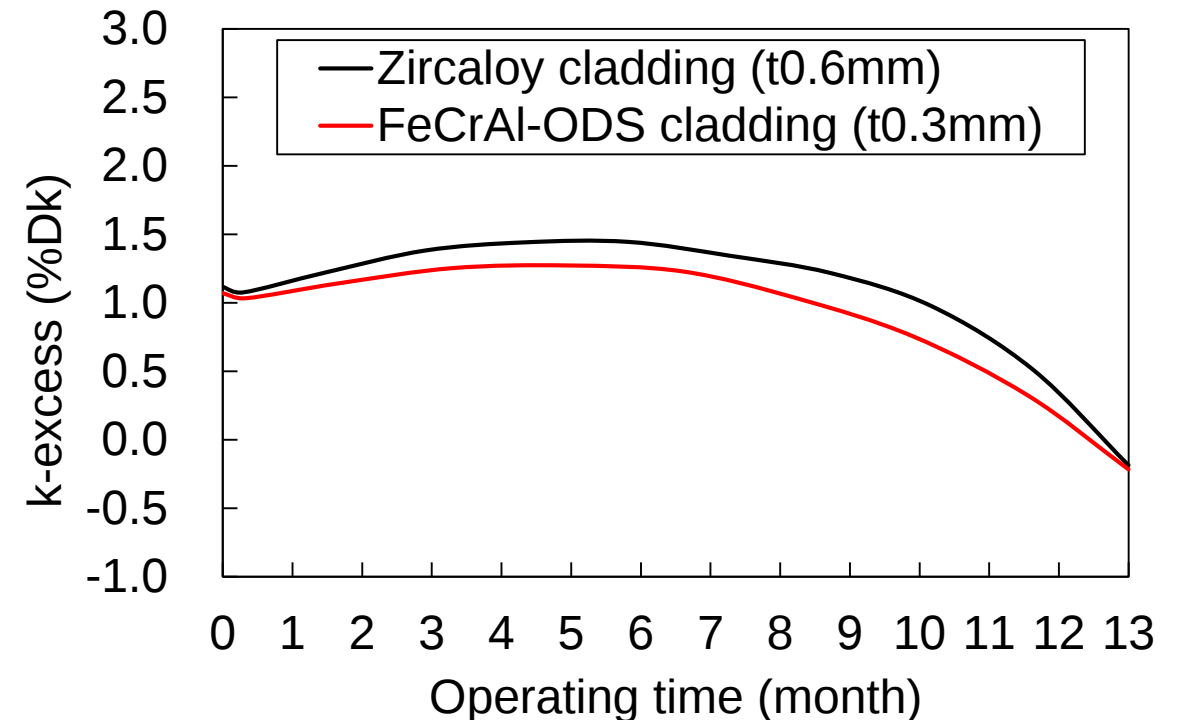
Analytical study (1)

● Analytical evaluation of core characteristics

➤ Analysis condition

Core (ABWR)	Rated thermal power	3926 MW
	Number of bundles	872
	Operating term	13 month
Fuel type	10 x 10	

➤ Core reactivity



- Reactivity penalty (due to Fe and Cr) can be compensated
- No unacceptable core characteristic degradation

* Reference: K. Sakamoto et al., J. Nucl. Mater., 557 (2021) 153276

- Analytical evaluation of fuel behavior in severe accidents
 - ATF performance was evaluated during SA of BWRs with and w/o accident management (AM) by MAAP5.05β

Item	Analysis conditions
Analysis code	MAAP5.05β *
Plant type	ABWR
Fuel type	9×9 (StepIII)
Cladding and channel box materials	Zry, ODS**, and SiC
Accident sequence groups	TQUV (high/low pressure water injections failure), and TB (Station blackout)
Analysis range	Until fuel cladding failure *

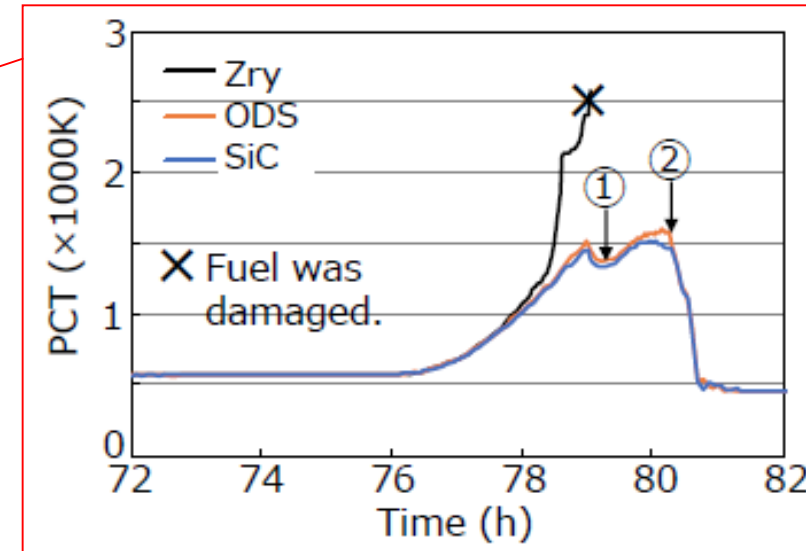
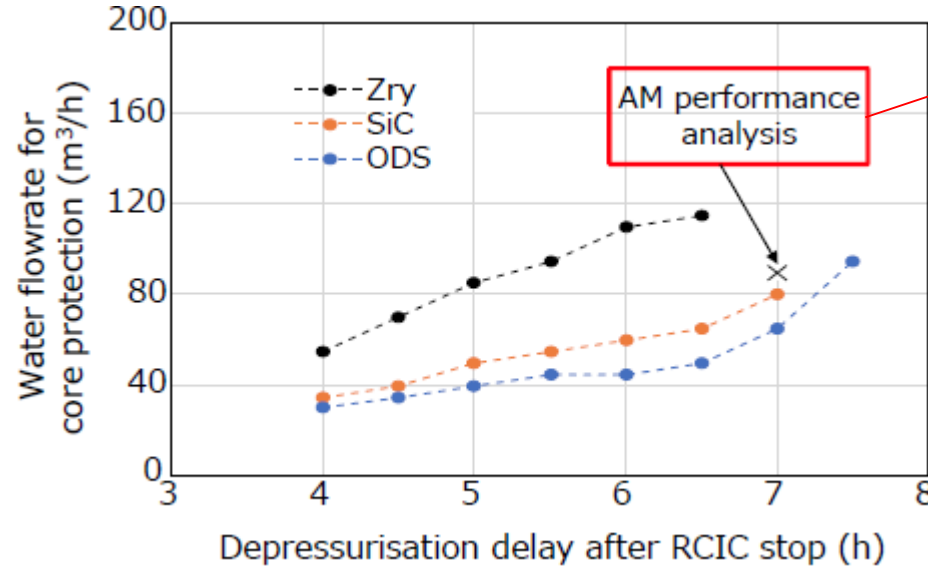
*: MAAP5.05 beta version has the limitation that it can handle ATF behaviour only inside the core.

** : Thin-walled fuel was applied for ODS in consideration of neutron economy.

Reference: T. Ikegawa, et al. , Proc. Top Fuel 2018, 30 Sep. – 04 Oct. 2018, Prague, Czech Republic

Analytical study (3)

- Merit of ATF was evaluated with long term TB scenario
- “Grace time to depressurization” and “Minimum flow rate” to prevent fuel damage were analyzed



PCT (Peak clad temperature)

- ATF can increase “Grace time” and/or reduce AM capacity/resources

Reference: T. Ikegawa, et al. , Proc. Top Fuel 2018, 30 Sep. – 04 Oct. 2018, Prague, Czech Republic

Contents

- Background
- Concept of FeCrAl-ODS
- Current status:
 - Manufacturing technology development
 - Experimental study
 - Analytical study
- Challenges and data gaps

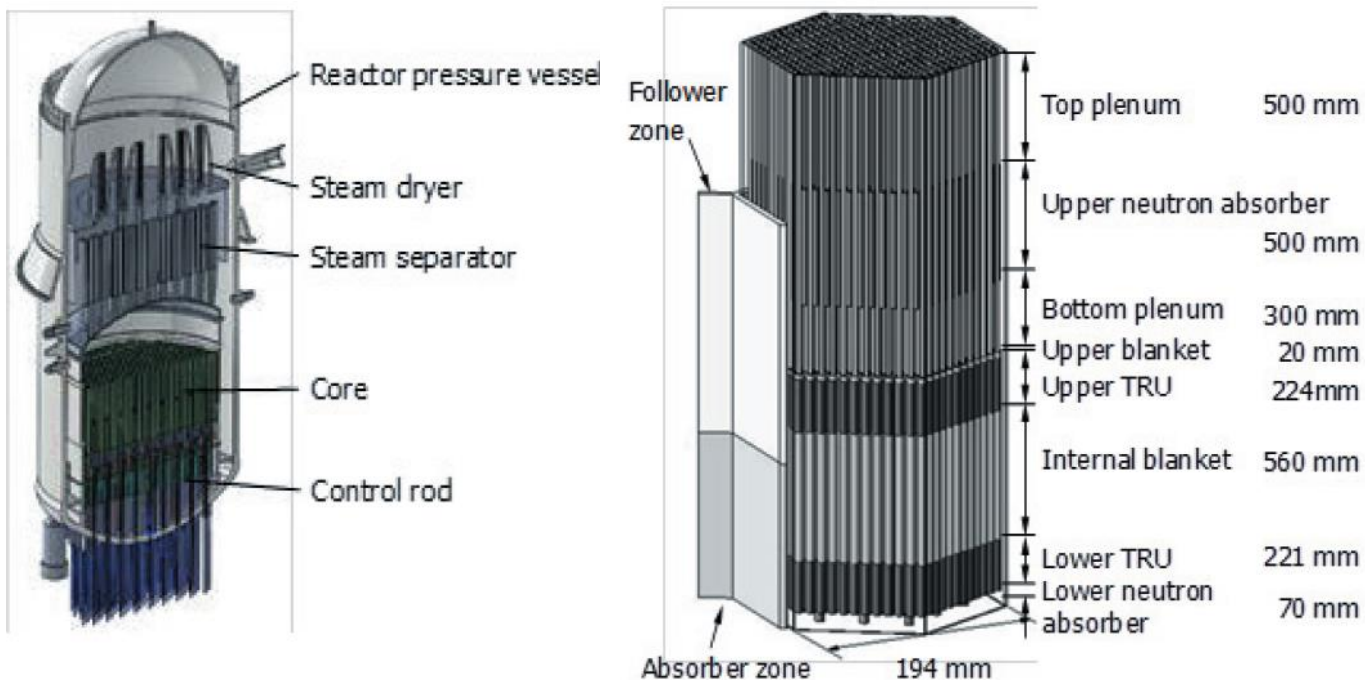
Challenges and data gaps (1)

- R&D status is now TRL 3-4
- Short-term challenges:
 - Measurement of fuel behavior in a test reactor
 - Validation of thermal-mechanical code predictions
- Short-term data gap:
 - Irradiation creep
 - Mechanical property changes with neutron irradiation
 - Heat transfer characteristics
 -

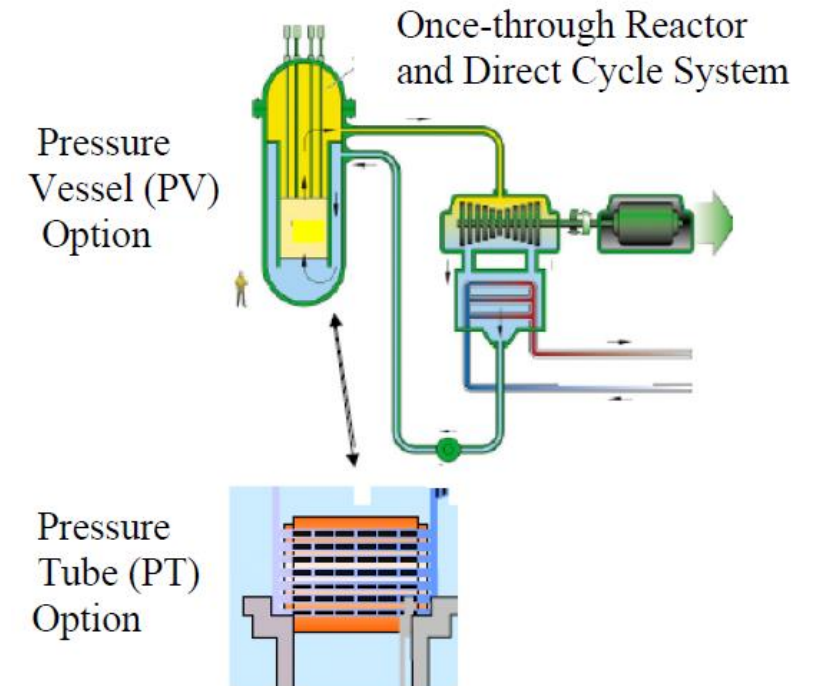
Challenges and data gaps (2)

● Other applications

- FeCrAl-ODS can bring “safety” to the next generation NPPs using water
- Application to FBR-like LWRs are favorable due to much less reactivity penalty



Resource-renewable BWR (RBWR)



SuperCritical-Water-cooled Reactor (SCWR)

Reference: T. Mitsuyasu, Sakamoto et al. , Proc. ICAPP2017, 24-28 April, 2017, Fukui and Kyoto, Japan

Reference: T. Schulenberg et al. , IAEA-CN-164-5S06, 2009

- Part of this study is the result of "Development of Technical Basis for Introducing Advanced Fuels Contributing to Safety Improvement of Current Light Water Reactors" program carried out by JAEA under the project on technical development for improving nuclear safety, supported by Ministry of Economy, Trade and Industry (METI) of Japan (Contract number: JPMT003830).