

The Evaluated Nuclear Reaction Data for fusion at CNDC

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2025.05

Outlines

Introduction

Nuclear Reaction Study

Perspective

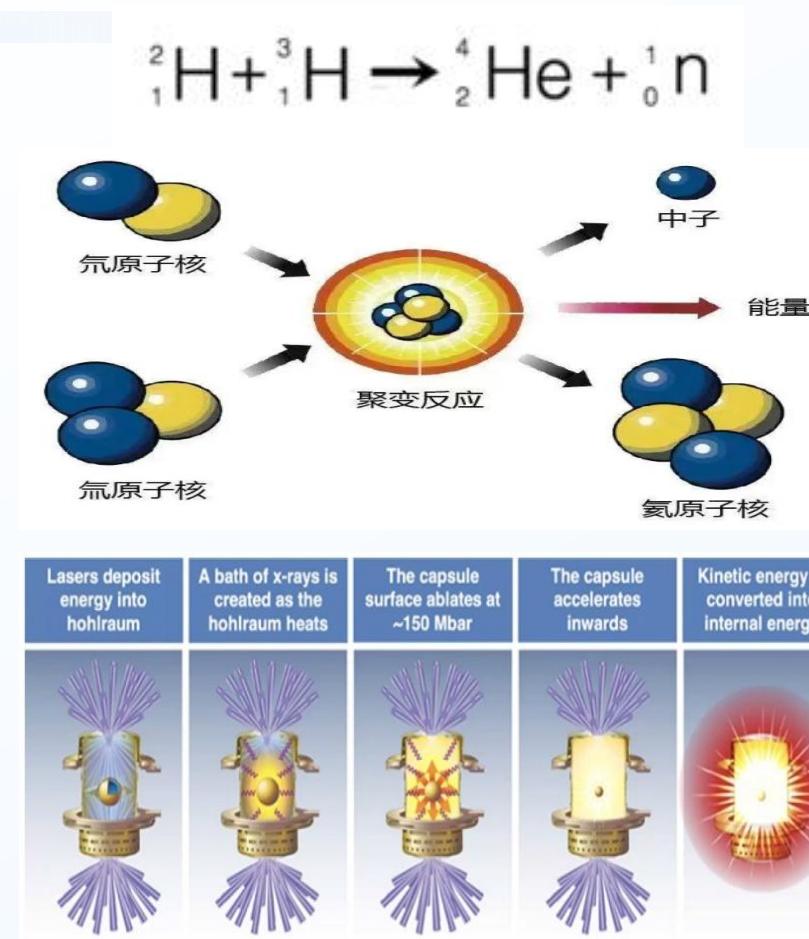
Nuclear Reaction Study

- Platform for Theoretical Models
- Nuclear Data Improvement
- Covariance Evaluations
- AI and Nuclear Data Evaluation

Fusion energy

Fusion energy follows the principle of nuclear fusion, where light atomic nuclei (e.g., deuterium and tritium) merge under extreme heat/pressure to form heavier nuclei (helium).

Neutron, and electrons emission as well as vast energy releasing via mass-to-energy conversion ($E=mc^2$), will provide huge energy source.

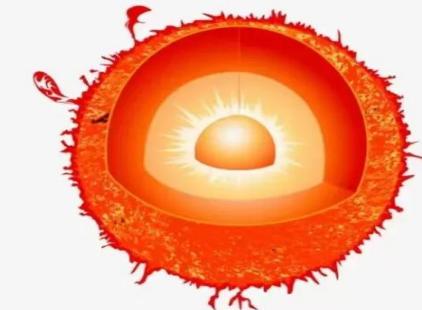


Ultra-high Temperature Plasma Environment

Average energy released per nucleon in fusion **3.6MeV**
 Average energy released per nucleon in fission **0.85MeV**

Alternative to fossil fuels, offering zero-carbon operation and virtually inexhaustible fuel supplies.

Earth's oceans harbor 45 trillion tons of deuterium, enough to sustain global energy demand for tens of billions of years





General Information of CNDC

Nuclear data study started in 60's last century by measurements with the first reactor and cyclotron in CIAE, China Nuclear Data Center (CNDC) was established in 1975 and joined the nuclear data activities of IAEA as the national nuclear data center of China since 1984. As a window, CNDC has been open to the world since 1978. and CNDC has established a good cooperative relationship with the IAEA, OECD/NEA, and major nuclear data centers and institutions in the world.

The main task of CNDC:

- ✓ The management of domestic nuclear data activities.
- ✓ The nuclear data evaluations, libraries and relevant methodology studies.
- ✓ Nuclear data measurements and methodology studies
- ✓ The exchange of nuclear data activities with IAEA, foreign nuclear data centers and agencies.
- ✓ The services for domestic and foreign nuclear data application users.



Mainly tasks of CNDC in 2025:

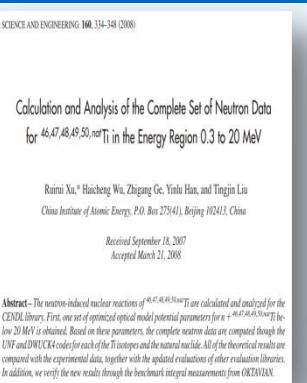
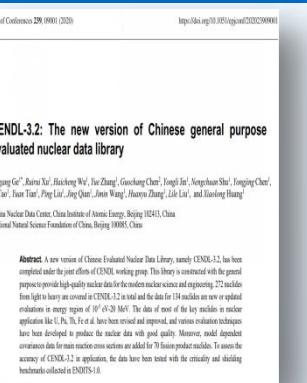
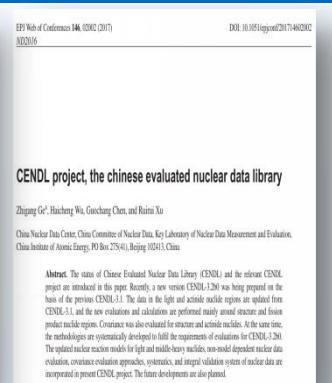
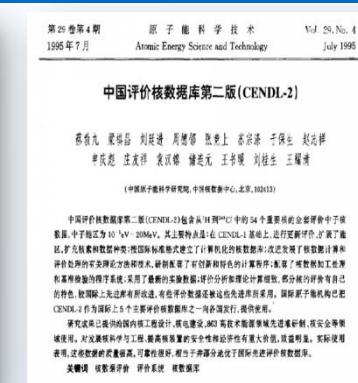
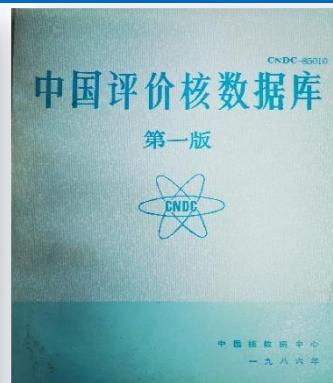
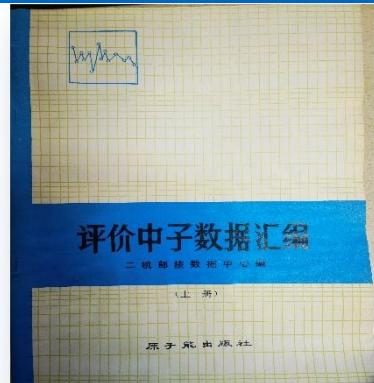
- ✓ New Five Years Plan (2021-2025) for nuclear data (CENDL Project).
- ✓ Data evaluation for next CENDL version and sub-libraries
- ✓ Methodological studies of nuclear data evaluation(incl. theoretical and experimental for fission process...).
- ✓ Nuclear data measurements and related methodological studies.(Mr.Ruan)
- ✓ The compilations for EXFOR.
- ✓ Nuclear data services.



The 1st reactor and cyclotron in China

Introduction

| Nuclei | Edited ~ 1979, 16 | CENDL-1, 1986, 36 | CENDL-2 1991, 66 | CENDL-3.1 2010, 240 | CENDL-3.2 2020, 272 |
|--------------|---|--|---|--|--|
| Few body | $^1, ^2, ^3\text{H}, ^3, ^4\text{He}$ | $^1, ^2, ^3\text{H}, ^3, ^4\text{He}$ | $^1, ^2, ^3\text{H}, ^3, ^4\text{He}$ | $^1, ^2, ^3\text{H}, ^3, ^4\text{He}$ | $\text{nn-1}, ^1, ^2, ^3\text{H}, ^3, ^4\text{He}$ |
| Light | $^6, ^7\text{Li}, ^9\text{Be}$ | $^6, ^7\text{Li}, ^9\text{Be}, ^{10,11}\text{B}, ^{14}\text{N}, ^{16}\text{O}, ^{19}\text{F}$ | $^6, ^7\text{Li}, ^9\text{Be}, ^{10,11}\text{B}, ^{14}\text{N}, ^{16}\text{O}, ^{19}\text{F}$ | $^6, ^7\text{Li}, ^9\text{Be}, ^{10,11}\text{B}, ^{12}\text{C}, ^{14}\text{N}, ^{16}\text{O}, ^{19}\text{F}$ | Same |
| Medium heavy | $^0\text{Pb}, ^0\text{W}, ^0\text{Al}, ^0\text{Fe}$ | 19 nuclei $^{23}\text{Na}, ^0\text{Mg}, ^{27}\text{Al}, ^0\text{Si}, ^{31}\text{P}, ^0\text{S}, ^0\text{K}, ^0\text{Ca}, ^0\text{Ti}, ^0\text{V}, ^{0,50,52,53,54,55}\text{Cr}, ^{55}\text{Mn}, ^{0,54,56,57,58}\text{Fe}, ^{59}\text{Co}, ^0\text{Ni}, ^{0,63,65}\text{Cu}, ^0\text{Zn}, ^0\text{Zr}, ^{93}\text{Nb}, ^0\text{Mo}, ^0\text{Sn}, ^0\text{Hf}, ^{181}\text{Ta}, ^0\text{W}, ^{197}\text{Au}, ^0\text{Pb}$ | 45 nuclei $^{23}\text{Na}, ^0\text{Mg}, ^{27}\text{Al}, ^0\text{Si}, ^{31}\text{P}, ^0\text{S}, ^0\text{K}, ^0\text{Ca}, ^0\text{Ti}, ^0\text{V}, ^{0,50,52,53,54,55}\text{Cr}, ^{55}\text{Mn}, ^{0,54,56,57,58}\text{Fe}, ^{59}\text{Co}, ^0\text{Ni}, ^{0,63,65}\text{Cu}, ^0\text{Zn}, ^0\text{Zr}, ^{93}\text{Nb}, ^0\text{Mo}, ^{0,107,109}\text{Ag}, ^0\text{Cd}, ^0\text{In}, ^0\text{Sn}, ^0\text{Sb}, ^0\text{Lu}, ^0\text{Hf}, ^{181}\text{Ta}, ^0\text{W}, ^{197}\text{Au}, ^0\text{Hg}, ^0\text{Tl}, ^0\text{Pb}$ | 192 nuclei $^{23}\text{Na} \sim ^{208}\text{Pb}$ | 223 nuclei $^{23}\text{Na} \sim ^{208}\text{Pb}$ |
| Actinides | $^{235, 238}\text{U}, ^{239, 240}\text{Pu}$ | $^{235, 238}\text{U}, ^{239, 240}\text{Pu}$ | 8 nuclei $^{235, 238}\text{U}, ^{237}\text{Np}, ^{239, 240}\text{Pu}, ^{241}\text{Am}, ^{249}\text{Bk}, ^{249}\text{Cf}$ | 34 nuclei $^{232}\text{Th} \sim ^{249}\text{Cf}$ | 34 nuclei $^{232}\text{Th} \sim ^{249}\text{C}$ |



THE UPDATED VERSION OF CHINESE EVALUATED NUCLEAR DATA LIBRARY (CENDL-3.1)

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The updated Chinese Evaluated Nuclear Data Library CENDL-3.1 is an important achievement based on the nuclear data measurements in recent years, at China Nuclear Data Center (CND) in cooperation with China Nuclear Data Committee (CNDC). CENDL-3.1 has evaluated data for reactions with incident neutron energy on 20 MeV to 10 GeV. The new version includes the latest data and the improvements made in the evaluation methodology and the predictions for nuclear reaction cross sections. In most important part of this library, the systematic testing and validation have been performed, the comparisons with other nuclear data libraries (ENDF, JEFF, BROND, JEF, etc.) have been done. The testing version of CENDL-3.1 is CENDL-3.1.0, which has been used for the prediction of nuclear reaction cross sections for Chinese domestic users. Following the work of CENDL-3.1.0, an improvement has been done. The CENDL-3.1 is provided for users by ENDF format and is released to the world in the end of 2007. The overview, methodology and some important results of new evaluation, benchmark testing and validation for the library will be introduced in this contribution.

© 2008 Nuclear Data Unit, IAEA, Vienna, Nuclear Model, ENDF

The New Evaluation Project & Data in CENDL-4.0 (2025. 12)



| Nuclear | CENDL-4.0-v1 共 A=410, Z=86 |
|--|--|
| Light Elements (A=17) (Z=10) | ^1n , $^{1,2,3}\text{H}$, $^{3,4}\text{He}$, $^{6,7}\text{Li}$, ^{9}Be , $^{10,11}\text{B}$, $^{12,13}\text{C}$, $^{14,15}\text{N}$, ^{16}O , ^{19}F |
| Fission Products & Medium Elements (A=359) (Z=69) | $^{22,23}\text{Na}$, $^{24,25,26}\text{Mg}$, ^{27}Al , $^{28,29,30}\text{Si}$, ^{31}P , $^{32,33,34,36}\text{S}$, $^{35,37}\text{Cl}$, $^{39,40,41}\text{K}$, $^{40,42,43,44,46,48}\text{Ca}$, $^{46,47,48,49,50}\text{Ti}$, $^{50,51}\text{V}$, $^{50,52,53,54}\text{Cr}$, ^{55}Mn , $^{54,56,57,58}\text{Fe}$, ^{59}Co , $^{58,59,60,61,62,64}\text{Ni}$, $^{63,65}\text{Cu}$, $^{64,65,66,67,68,70}\text{Zn}$, $^{69,71}\text{Ga}$, $^{70,71,72,73,74,75,76,77,78}\text{Ge}$, $^{74,75}\text{As}$, $^{74,76,77,78,79,80,82}\text{Se}$, $^{79,81}\text{Br}$, $^{78,80,81,82,83,84,85,86,87,88}\text{Kr}$, $^{85,86,87}\text{Rb}$, $^{84,86,87,88,89,90}\text{Sr}$, $^{89,90,91}\text{Y}$, $^{90,91,92,93,94,95,96}\text{Zr}$, $^{93,94,95,96}\text{Nb}$, $^{92,93,94,95,96,97,98,99,100}\text{Mo}$, $^{99,104}\text{Tc}$, $^{96,98,99,100,101,102,103,104,105,106}\text{Ru}$, $^{103,105}\text{Rh}$, $^{102,104,105,106,107,108,110}\text{Pd}$, $^{107,109,111}\text{Ag}$, $^{106,108,109,110,111,112,113,114,115,116}\text{Cd}$, $^{113,115}\text{In}$, $^{112,113,114,115,116,117,118,119,120,122,123,124,125,126,128}\text{Sn}$, $^{121,122,123,124,125,126,127}\text{Sb}$, $^{120,122,123,124,125,126,127,128,130,132}\text{Te}$, $^{127,129,130,131,133,135}\text{I}$, $^{123,124,126,128,129,130,131,132,133,134,135,136}\text{Xe}$, $^{133,134,135,136,137}\text{Cs}$, $^{130,132,133,134,135,136,137,138,139,140}\text{Ba}$, $^{138,139,140}\text{La}$, $^{136,138,139,140,141,142,143,144}\text{Ce}$, $^{141,142,143,145}\text{Pr}$, $^{142,143,144,145,146,147,148,149,150}\text{Nd}$, $^{147,148,148m,149,150,151}\text{Pm}$, $^{144,145,146,147,148,149,150,151,152,153,154}\text{Sm}$, $^{151,152,153,154,155,156,157}\text{Eu}$, $^{152,153,154,155,156,157,158,159,160,161}\text{Gd}$, $^{157,158,159,160,161}\text{Tb}$, $^{156,157,158,159,160,161,162,163,164,165}\text{Dy}$, $^{163,165,166}\text{Ho}$, $^{162,164,166,167,168,169,170}\text{Er}$, $^{168,169,170,171,172,173,174,175,176}\text{Tm}$, $^{168,169,170,171,172,173,174,175,176}\text{Yb}$, $^{175,176,177}\text{Lu}$, $^{174,176,177,178,179,180,181}\text{Hf}$, $^{180,181,182}\text{Ta}$, $^{180,182,183,184,186,187,188}\text{W}$, $^{185,187}\text{Re}$, $^{191,193}\text{Ir}$, ^{197}Au , $^{196,198,199,200,201,202,204}\text{Hg}$, $^{203,205}\text{Tl}$, $^{204,206,207,208}\text{Pb}$, ^{209}Bi |
| Actinides (A=34) (Z=7) | ^{232}Th , $^{232,233,234,235,236,237,238,239,240,241}\text{U}$, $^{236,237,238,239}\text{Np}$, $^{236,237,238,239,240,241,242,243,244,245,246}\text{Pu}$, $^{240,241,242,242m,243,244}\text{Am}$, ^{249}Bk , ^{249}Cf |

~ neutron data of 30 nuclei are being improved based on the new experimental data

light elements: ^6Li , $^{9,10,11}\text{B}$, $^{12,13,14,15}\text{C}$, $^{14,15}\text{N}$

Medium elements: ^{27}Al , $^{50,51}\text{V}$, ^{52}Cr , $^{54,56-58}\text{Fe}$, $^{69,71}\text{Ga}$, ^{95}Mo , $^{204,206-208}\text{Pb}$

Actinides: $^{235,238}\text{U}$, $^{239,240,241}\text{Pu}$

Outlines

Introduction

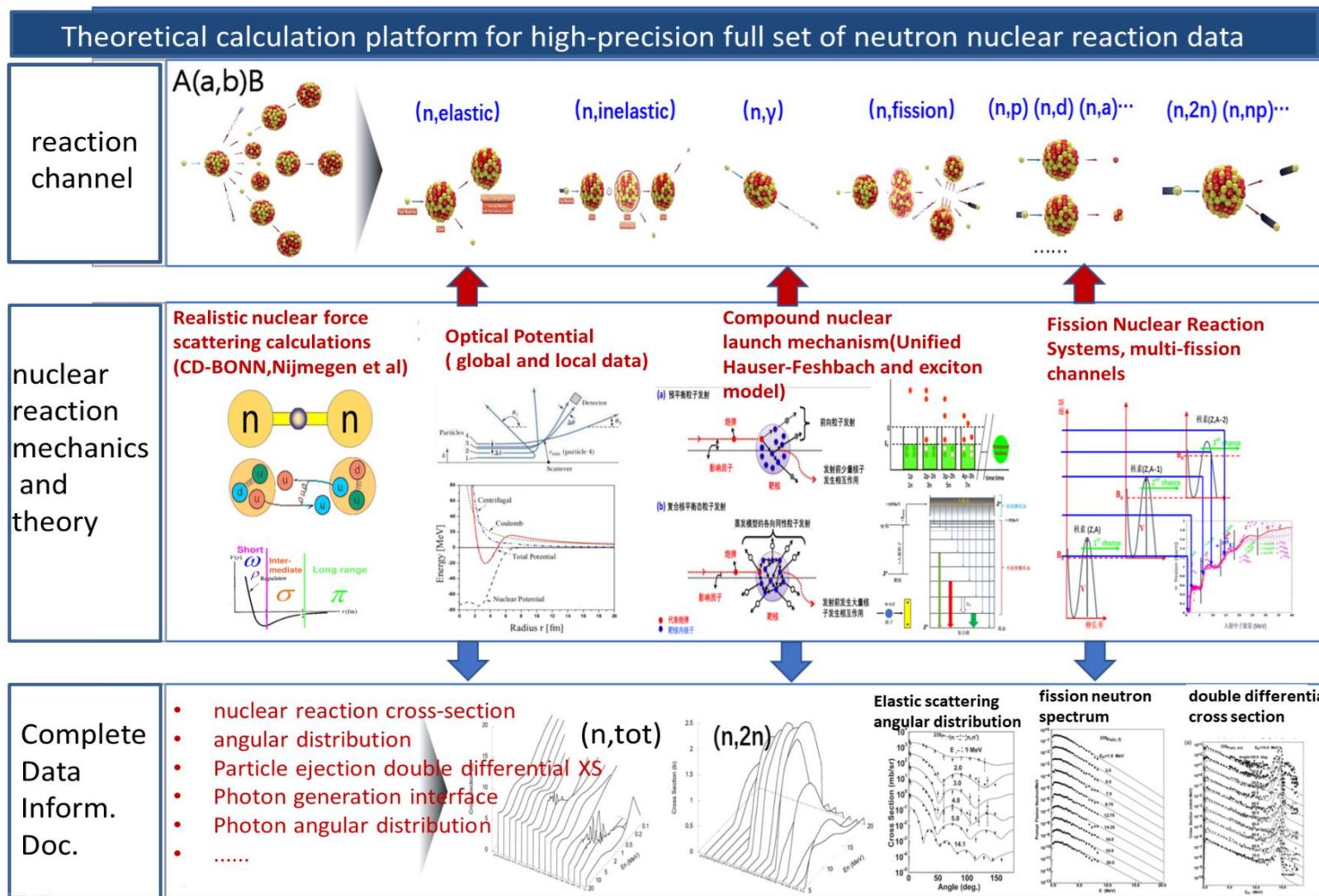
Nuclear Reaction Study

Perspective

Nuclear Reaction Study

- Platform for Theoretical Models
- Nuclear Data Improvement
- Covariance Evaluations
- AI and Nuclear Data Evaluation

Platform of Chinese Theoretical Model for nuclear reaction For Nuclear Data — Neutron and Photon beams



Chinese Nuclear Reaction Codes:

1. UNF: UNiFed function

- Incident: Neutron, proton, photon
- Target: light, medium heavy, actinides
- Energy: ~ 20MeV

J.S. Zhang, Nucl. Sci. Eng. 142, 207, 2002

J.S. Zhang, Nucl. Sci. Eng. 114, 55, 1993

2. MEND: Medium Energy Nuclear Data

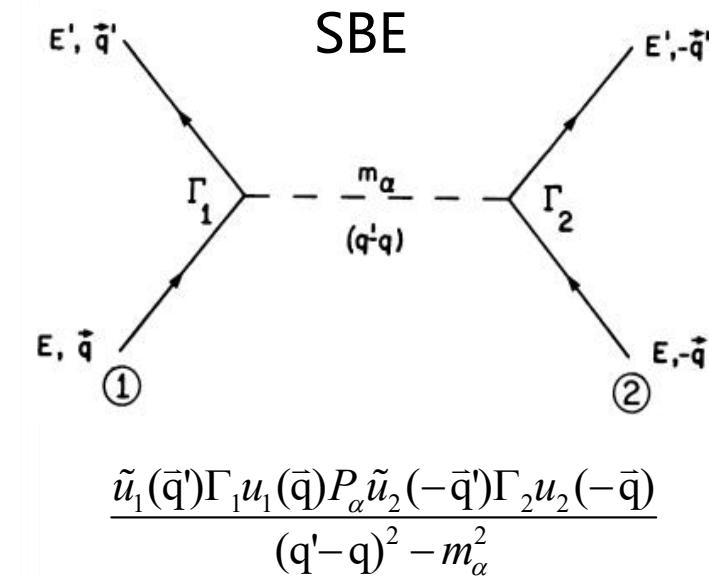
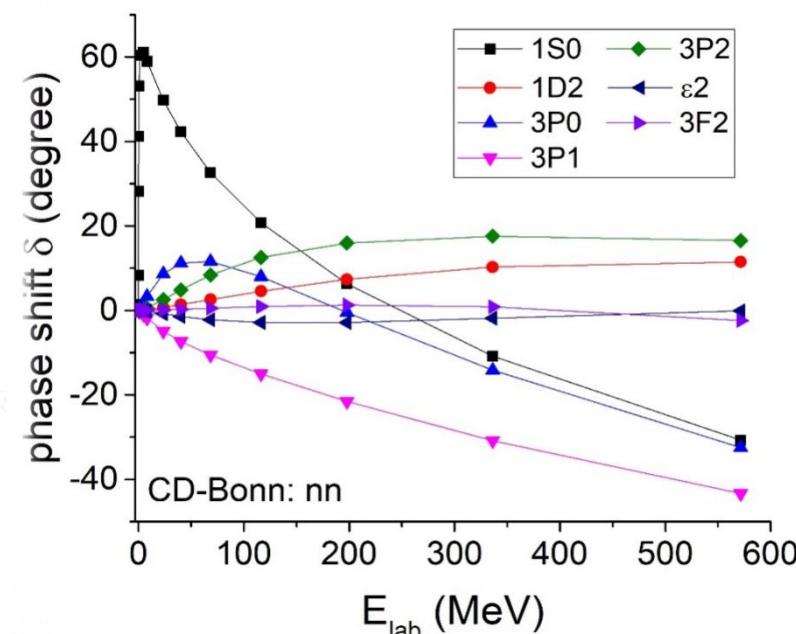
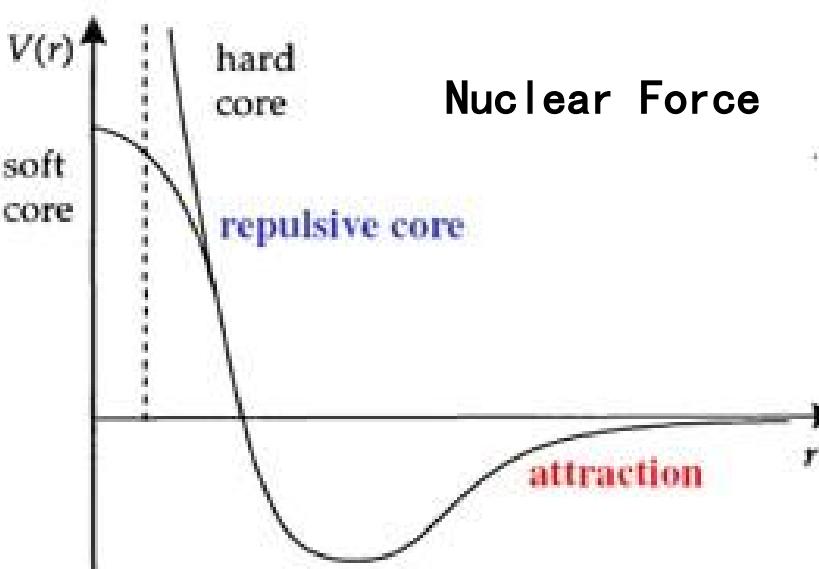
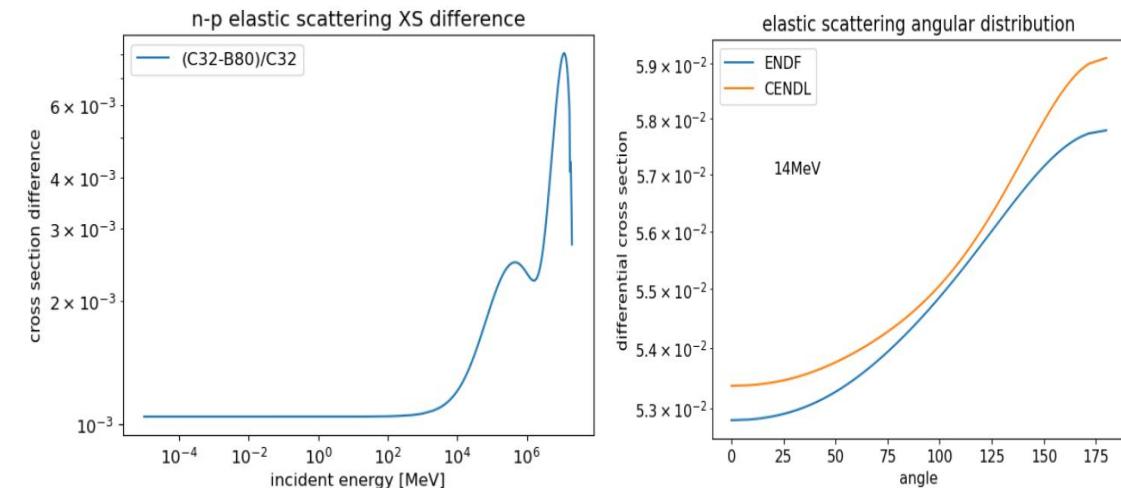
- Incident: neutron, proton, deuteron, triton, alpha, photon
- Target: medium heavy, actinides
- Energy: ~ 200MeV

C.H. Cai, Nucl. Sci. Eng. 153, 93, 2006

C.H. Cai, Chin. J. Comp. Phys. 20, 279, 2003

1. Neutron-proton scattering evaluation

- Based on the knowledge of nuclear force and the partial wave analysis of phase shift, the nucleon-nucleon scattering data is well understood.
- The neutron-proton scattering cross section and its angular distribution are theoretically calculated using the CD-Bonn interaction for CENDL-3.2.
- The calculation is in good agreement with ENDF values, with a deviation of about 0.1% for low-energy and a deviation less than 0.6% when the energy increases.



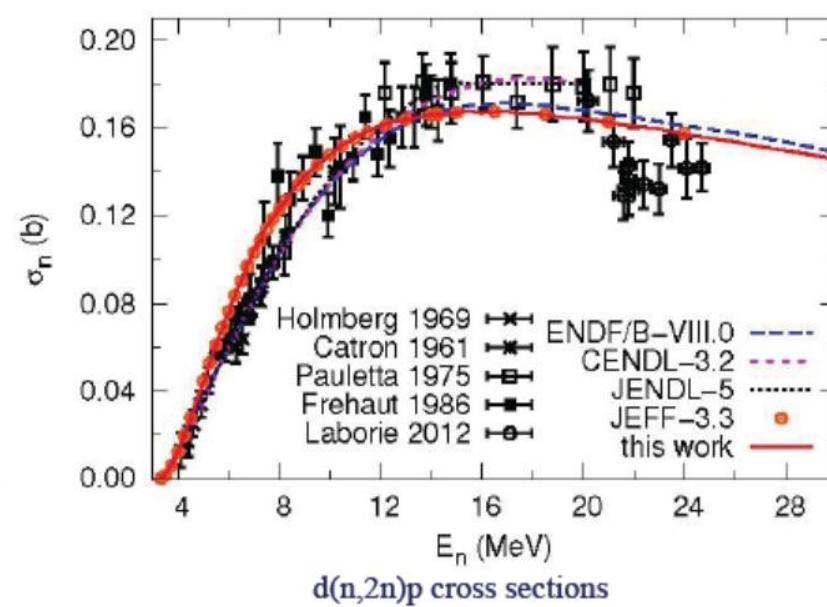
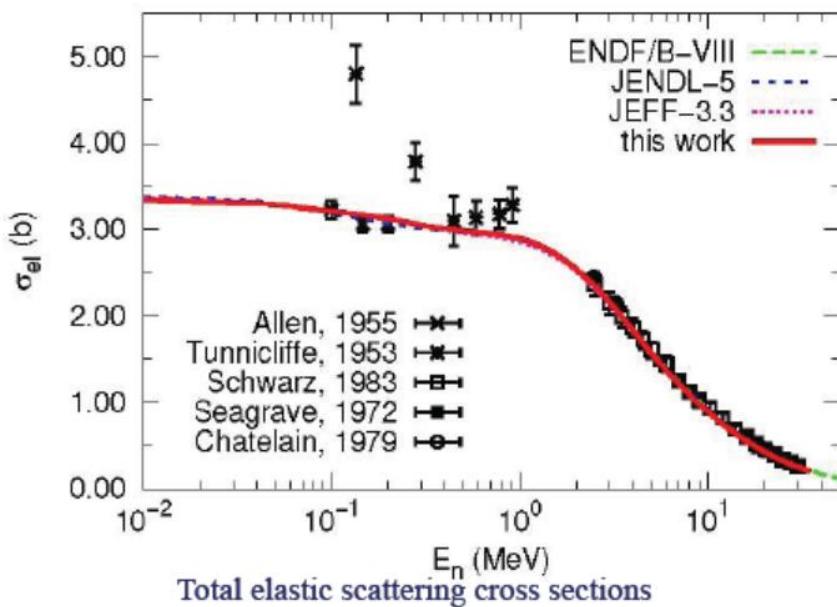
2. Improving nuclear data library of light nucleus

The n+d systems are calculated by solving the following Faddeev-AGS equation:

$$U = Pv_1 + Pv_1G_1U,$$

where U , P , v_1 and G_1 are the transition operator, the permutation operator, the channel interaction, and the channel resolvent, respectively. The Faddeev equation is solved with the wave-packet method described in *O.A. Rubtsova et al., Ann. Phys. 360, 613 (2015)*.

NN interactions used: For elastic scattering and total reaction cross sections: **Nijmegen potential: nijmI**. For breakup double-differential cross sections: **MT I-III s-wave separable potential**



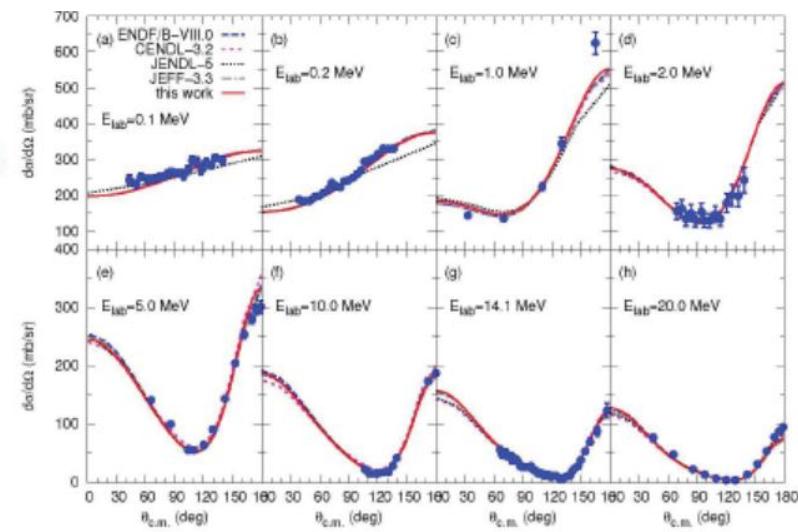
The elastic scattering amplitude:

$$A_{\text{el}}^{\Gamma\alpha\beta}(q_0) \approx \frac{2m}{3q_0} \frac{\langle Z_{1j_0}^{\Gamma\alpha\beta} | u^\Gamma | Z_{1j_0}^{\Gamma\alpha\beta} \rangle}{d_{j_0}}$$

The breakup amplitude:

$$A_{bu}^{\Gamma\alpha\beta}(\theta) = \frac{4\pi m}{3\sqrt{3}} q_0 K^4 e^{\frac{i\pi}{4}} T^{\Gamma\alpha\beta}(p, q), \quad \theta = \tan^{-1}\left(\frac{\sqrt{3}q}{2p}\right)$$

$$T^{\Gamma\alpha\beta}(p, q) \approx e^{i\delta(p_k)} \frac{T_{1j_0,kj}^{\Gamma\alpha\beta}}{p_k q_j q_0} \quad T_{1j_0,kj}^{\Gamma\alpha\beta} \equiv \frac{\langle Z_{1j_0}^{\Gamma\alpha\beta} | u^\Gamma | Z_{kj}^{\Gamma\alpha\beta} \rangle}{\sqrt{d_{j_0} d_k d_j}}, \quad \begin{cases} q_0 \in \mathfrak{D}_{j_0} \\ q \in \mathfrak{D}_j \\ p \in \Delta_k^r \end{cases}$$

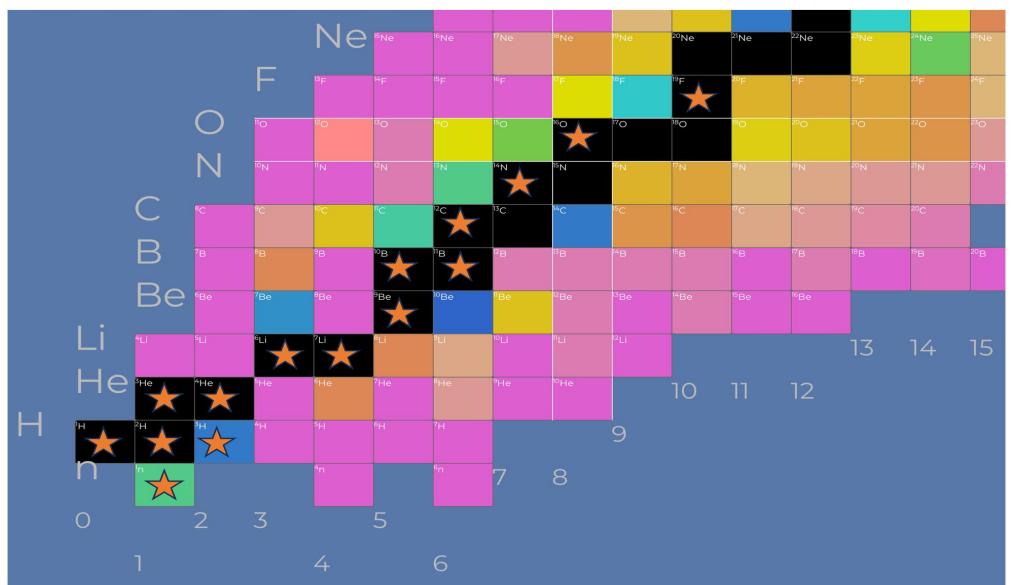


Elastic scattering angular distributions

Atomic Energy Science and Tech.. Vol.56, No5(2022).

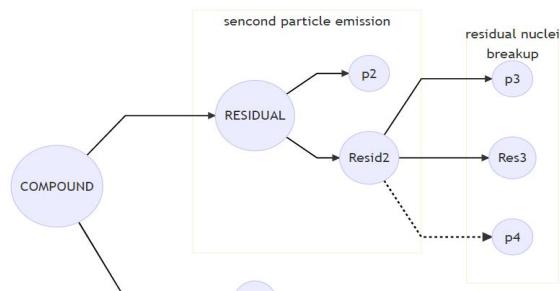
2 Improving nuclear data library of light nucleus

- Neutron induced nuclear reaction dataset for light nucleus are incomplete, only including ${}^3, {}^4\text{He}$, ${}^6, {}^7\text{Li}$, ${}^9\text{Be}$, ${}^{10, 11}\text{B}$, ${}^{12}\text{C}$, ${}^{14}\text{N}$, ${}^{16}\text{O}$, ${}^{19}\text{F}$ in CENDL-3.2.
- Key steps and technologies:
 - Theoretical models beyond R matrix
 - Consistent evaluation both experimental and theoretical results for the fast neutron energy
 - Nuclear data processing research



Applying Statistical Theory of Light Nucleus reactions to predict complete nuclear reaction data, key considerations:

- Must include special but not negligible channels, for example ${}^6\text{Li}(n, \text{ddt})$ with tens of mb cross section
- The number of open levels increase significantly when induced neutron energy as large as 30 MeV
- Providing both cross-sections and energy-angular double differential spectra data
- Understanding reaction mechanism of pre-equilibrium emission and complex particle formation process



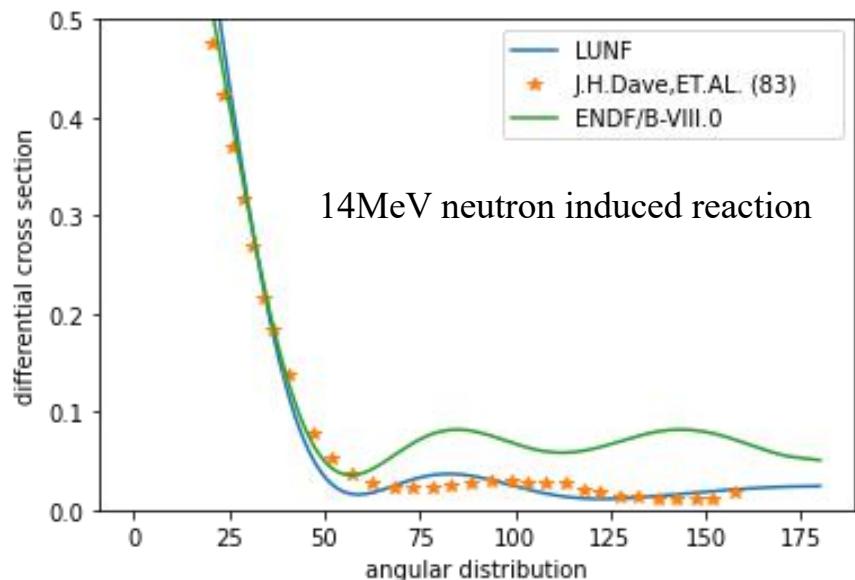
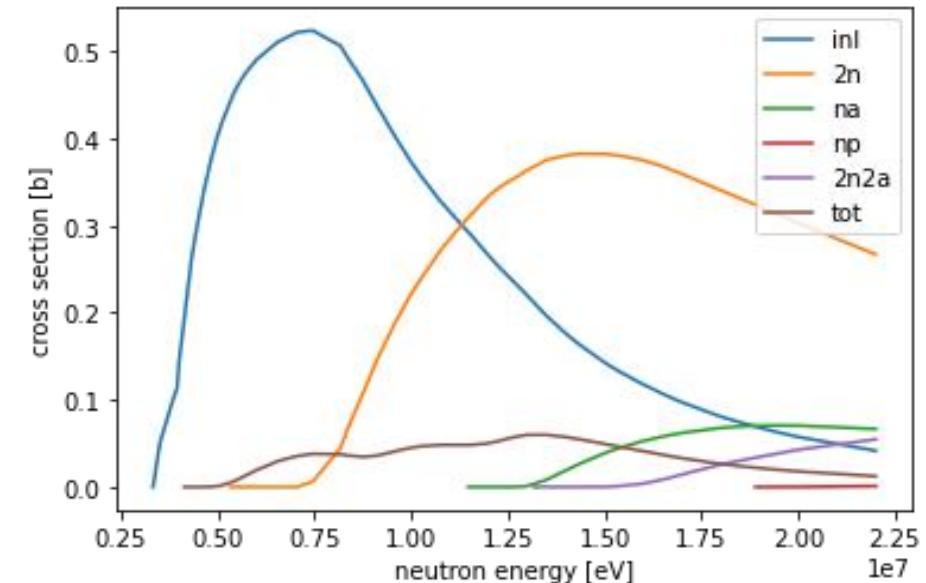
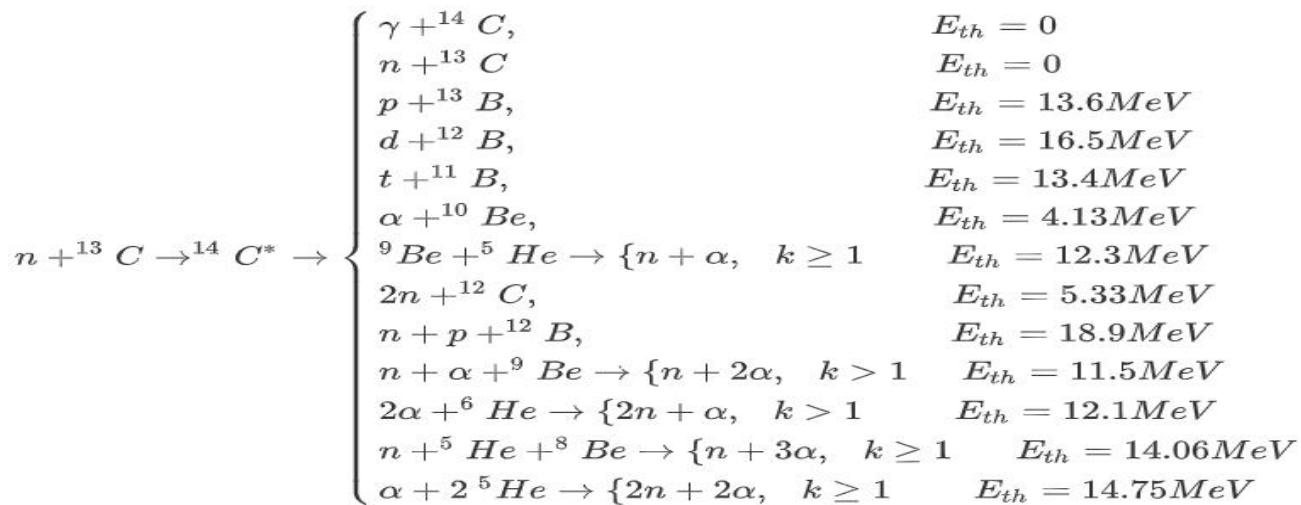
1. Dynamics of Compound nuclei

2. Kinematics of emission

$$\sigma_{m_1, k_1}(E_L) = \sum_{j\pi} \sigma_a^{j\pi}(E_L) \left\{ \sum_{n=3}^{n_{max}} P^{j\pi}(n) \frac{W_{m_1, k_1}^{j\pi}(n, E^*, \varepsilon_{m_1})}{W_T^{j\pi}(n, E^*)} + Q^{j\pi}(n) \frac{W_{m_1, k_1}^{j\pi}(E^*, \varepsilon_{m_1})}{W_T^{j\pi}(E^*)} \right\}$$

Absorb XS
pre-equilibrium emission
equilibrium emission

3. n+¹³C nuclear reaction data



- The cross section and its angular distribution are obtained simultaneously with neutron energy lower than 20 MeV
- The differential cross section well agree with measurements

$$\sigma_{m_1, k_1}(E_L) = \sum_{j\pi} \sigma_a^{j\pi}(E_L) \left\{ \sum_{n=3}^{n_{max}} P^{j\pi}(n) \frac{W_{m_1, k_1}^{j\pi}(n, E^*, \varepsilon_{m_1})}{W_T^{j\pi}(n, E^*)} + Q^{j\pi}(n) \frac{W_{m_1, k_1}^{j\pi}(E^*, \varepsilon_{m_1})}{W_T^{j\pi}(E^*)} \right\}$$

Absorb XS

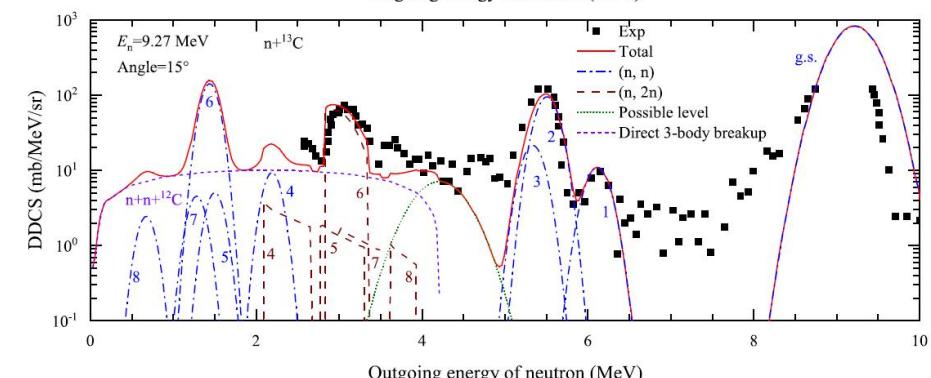
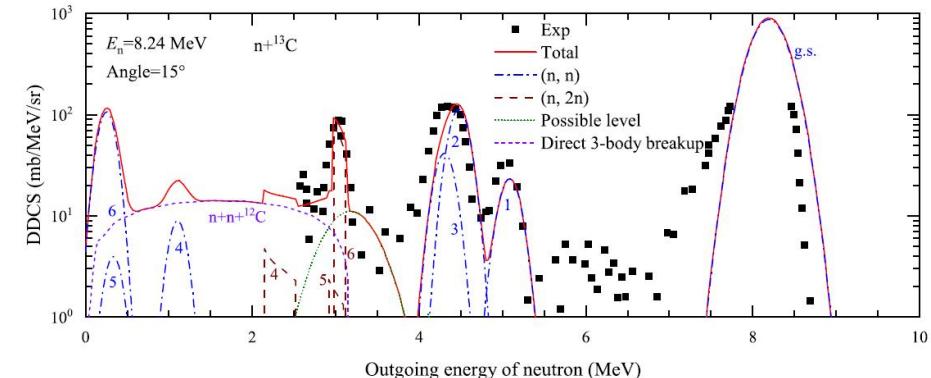
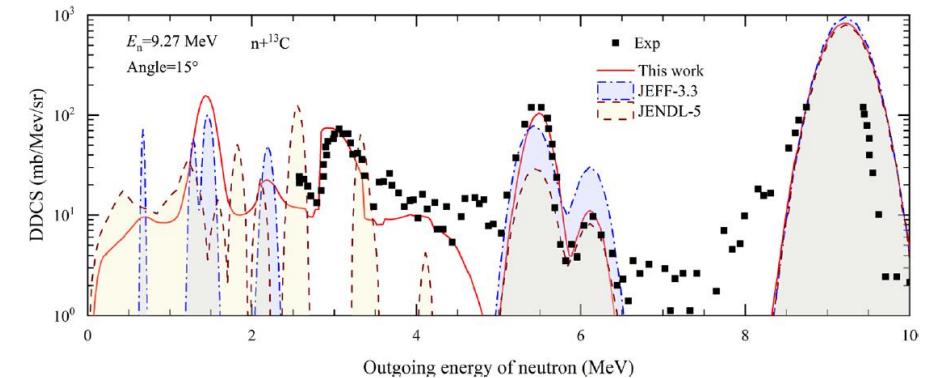
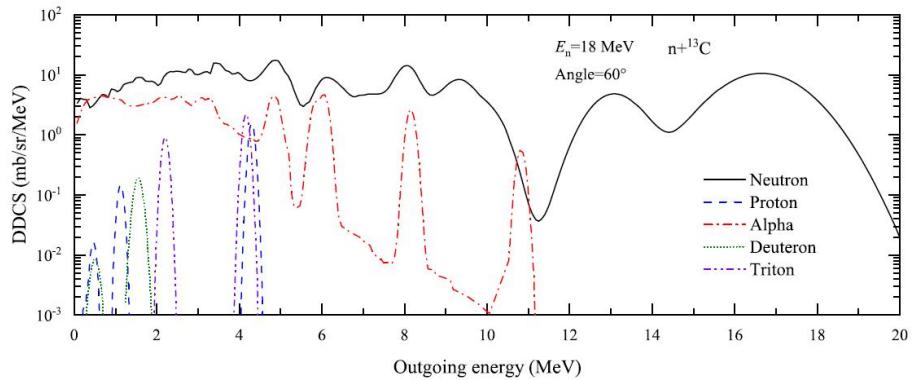
pre-equilibrium emission

equilibrium emission

3. $n+^{13}\text{C}$ nuclear reaction data

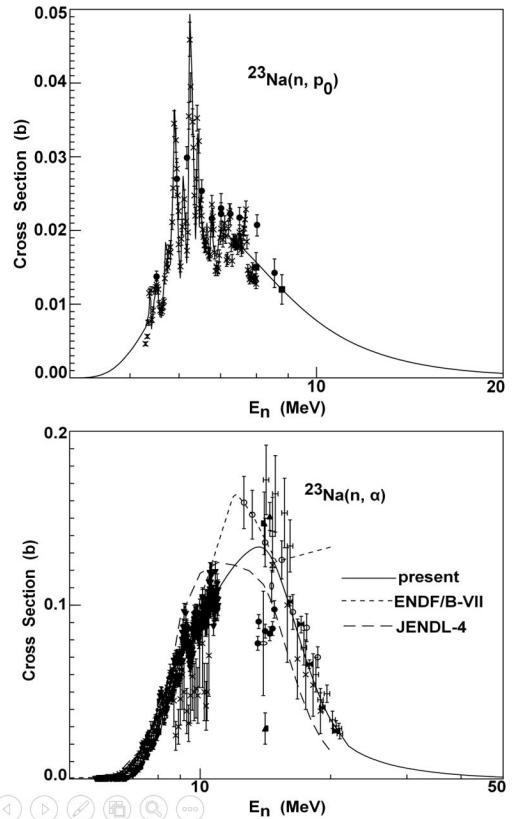
Effects of energy levels on the double-differential cross sections for $n+^{13}\text{C}$ reaction below 20 MeV

- Based on the unified Hauser Feshbach and exciton model STLN theory, the calculated results are in reasonable agreement with experimental measurements.
- Considering the three body breakup process is crucial for inelastic scattering data and can compensate for the shortcomings of the DWUCK.
- Theoretical self-consistency describes the emission cross sections of neutron and charged particles such as protons, deuterons, tritium nuclei, and alpha particles, which can be used to analyze the generation of gases in fusion devices.

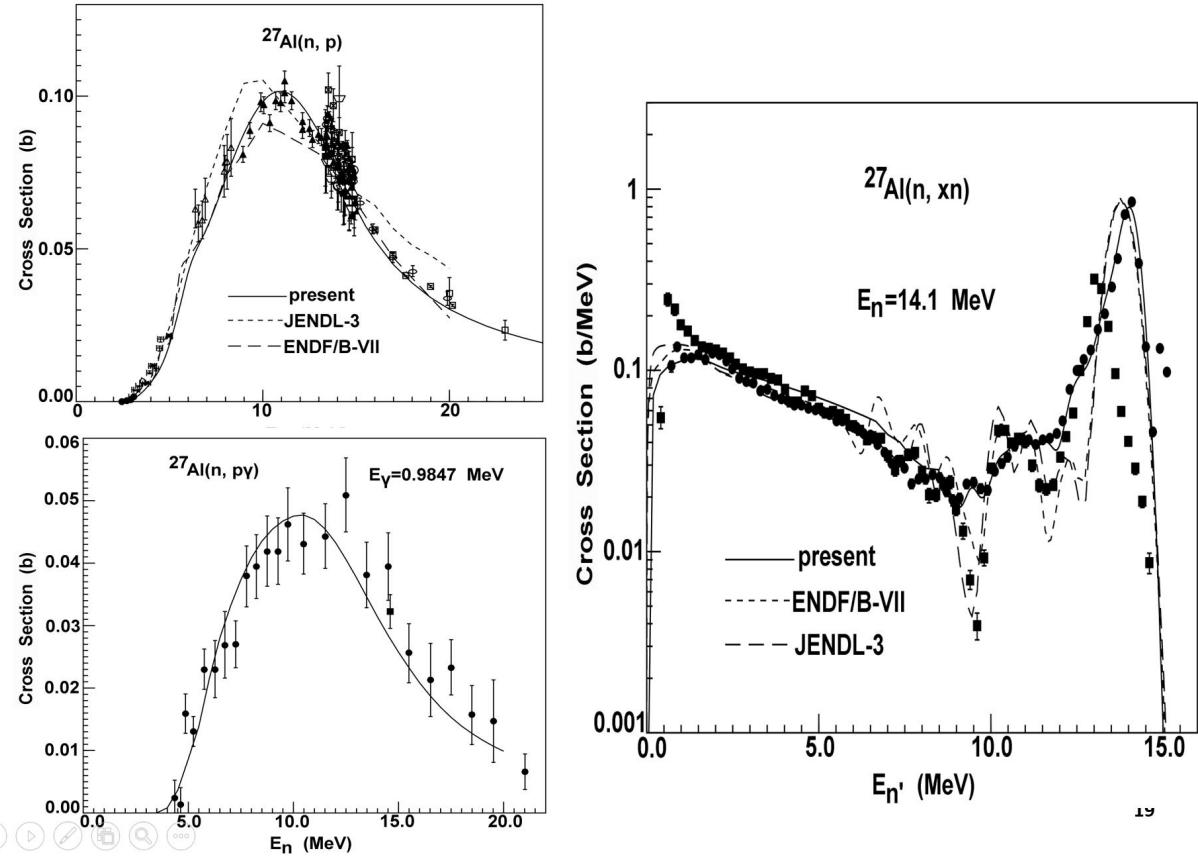


4. n+ ^{23}Na and ^{27}Al

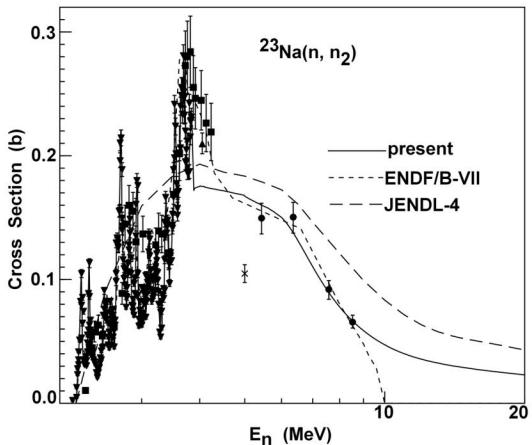
^{23}Na



^{27}Al



Nuclear Reactions

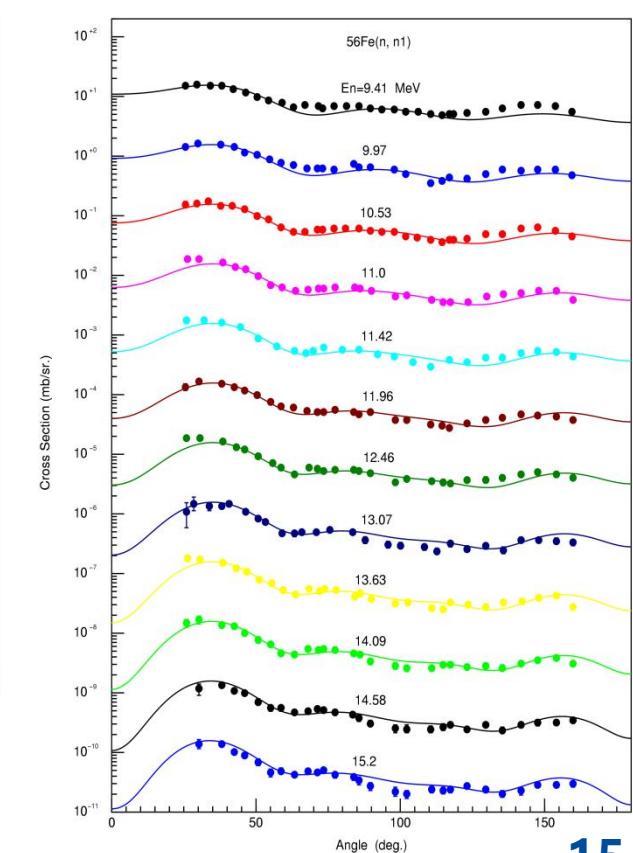
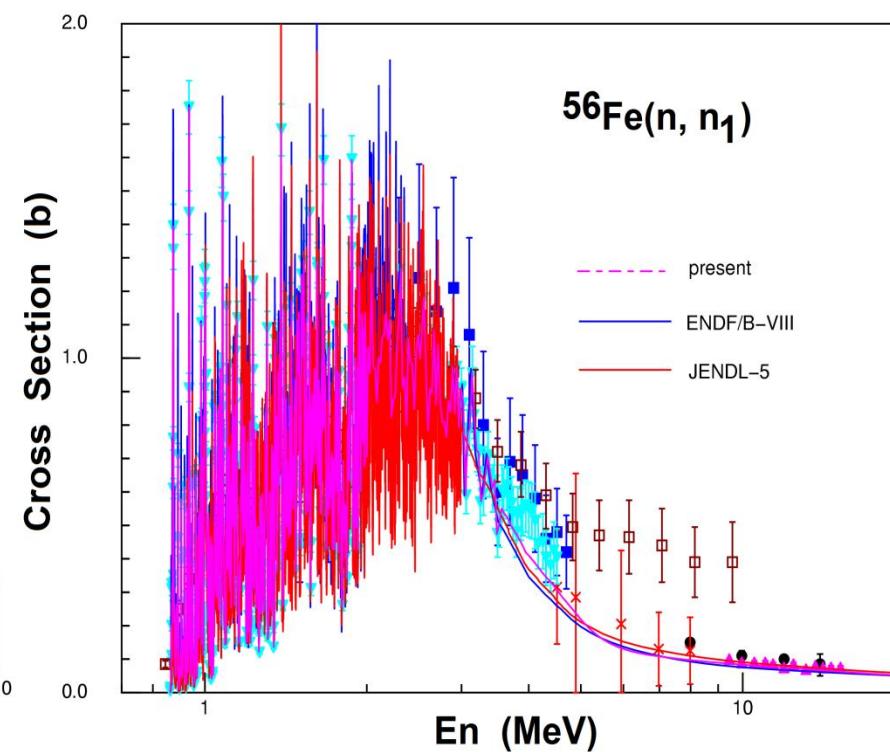
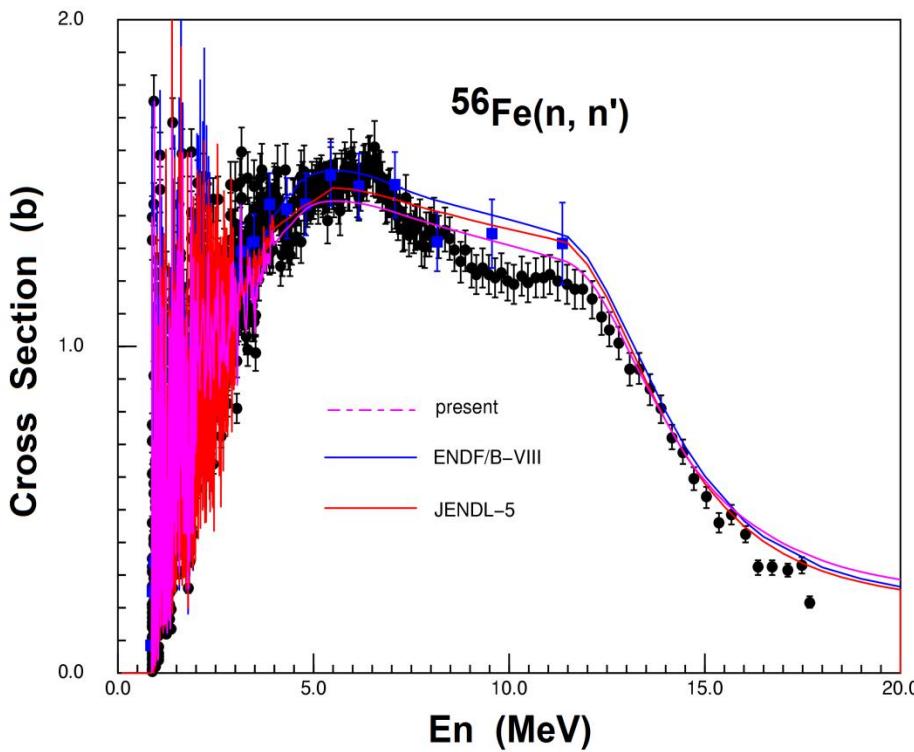


5. n+ Fe nuclear reaction data

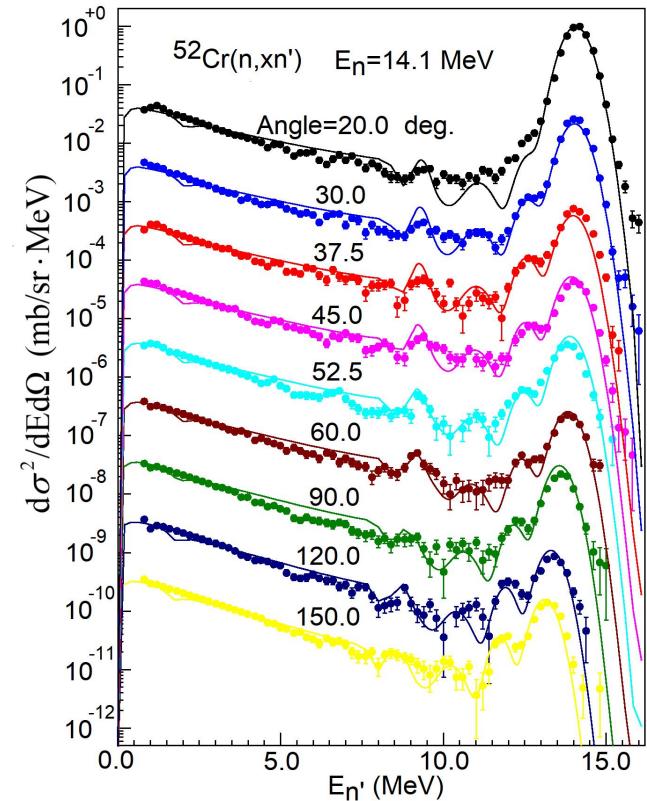
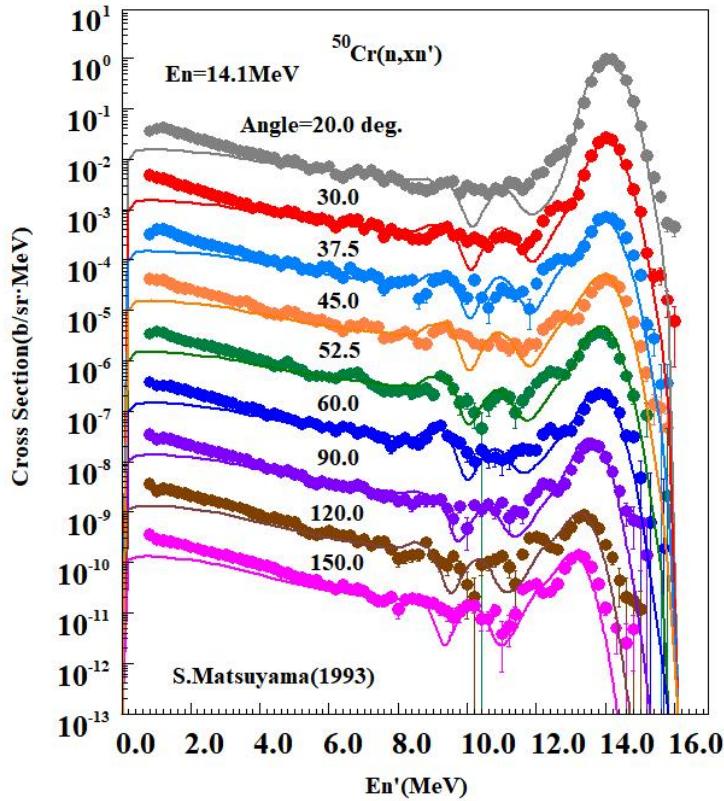
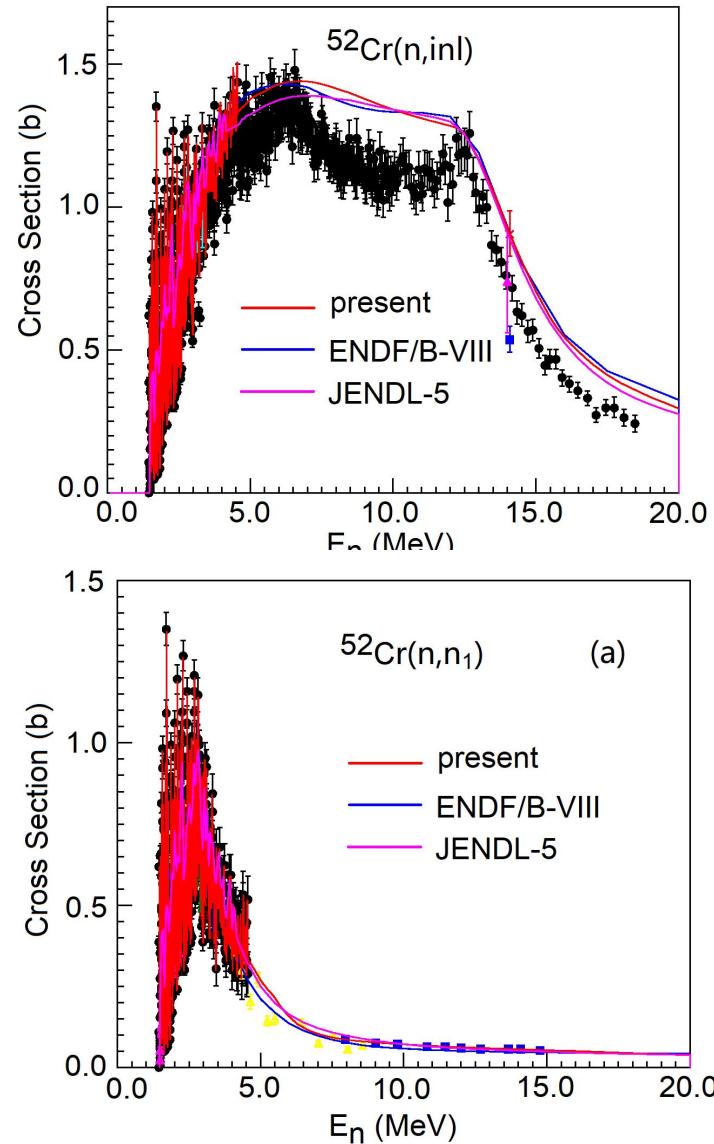
$^{54, 56-58}\text{Fe}$ have been re-calculated and evaluated in CNDC.

The evaluation is updated via the prompt gamma measurements.

Other improvements by Haicheng are also performed via considering the Benchmark testing experiments(not included here)

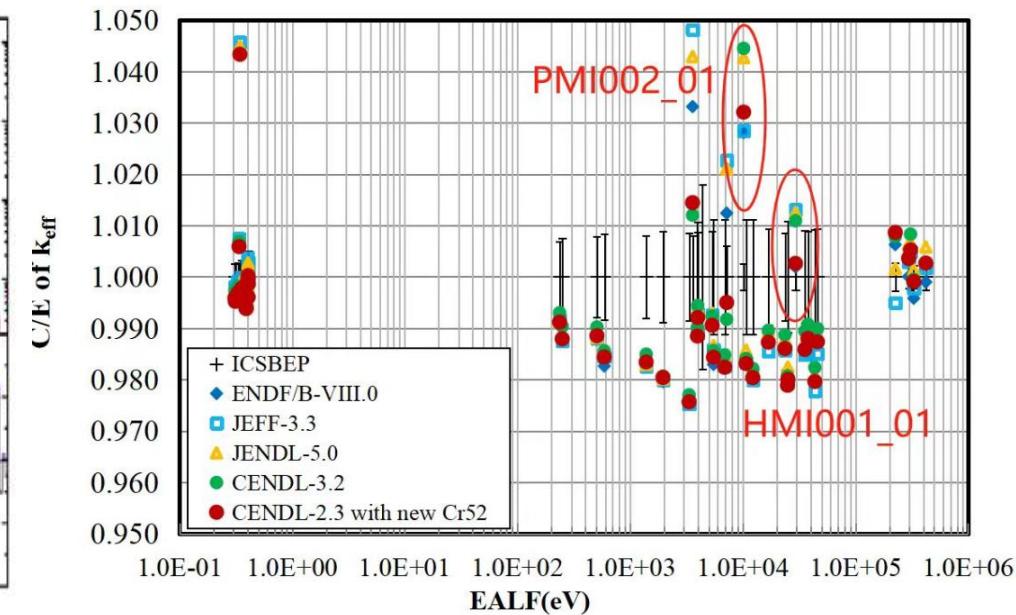
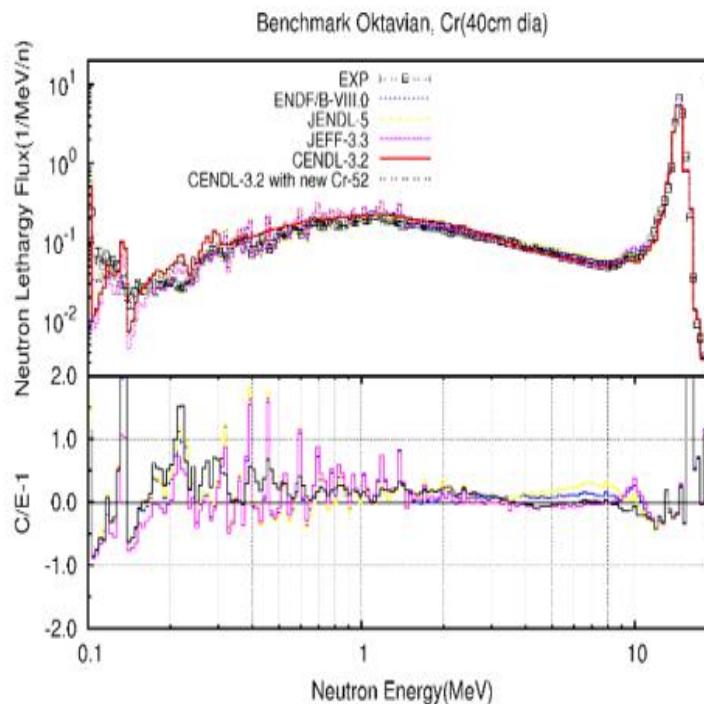
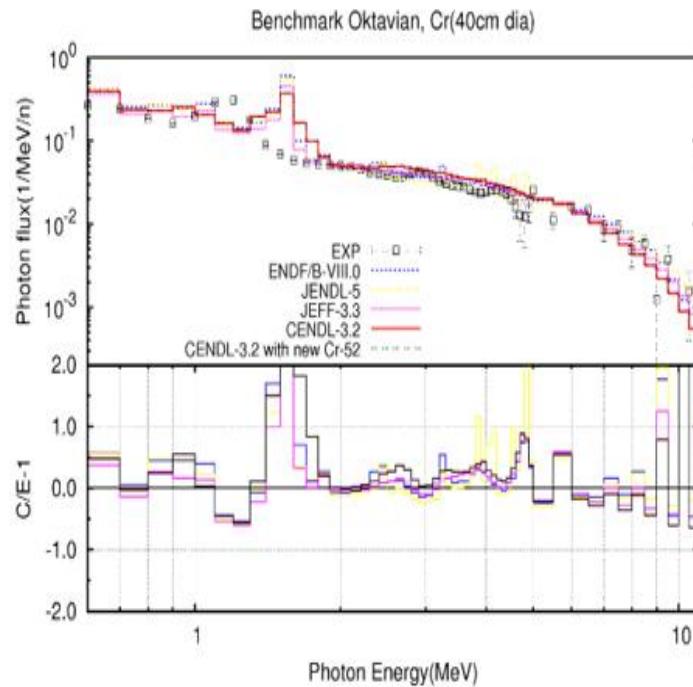


6. n+ Cr nuclear reaction data



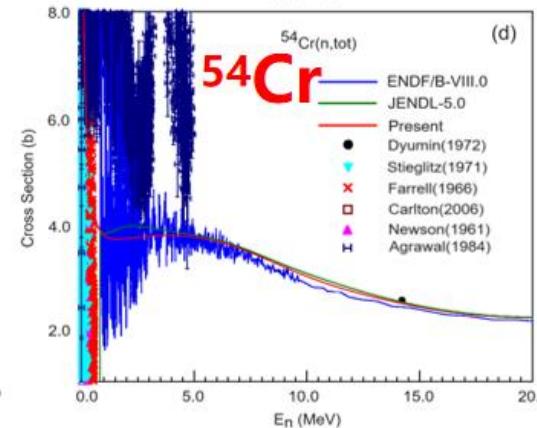
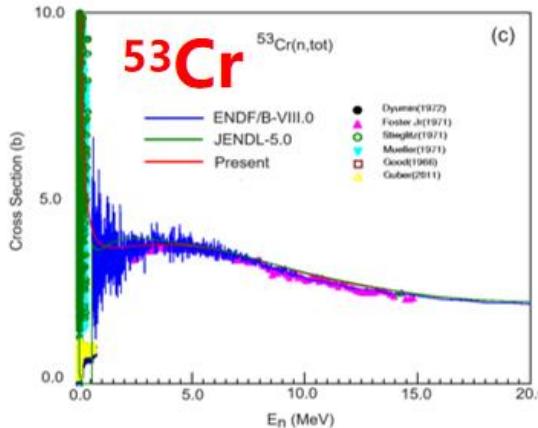
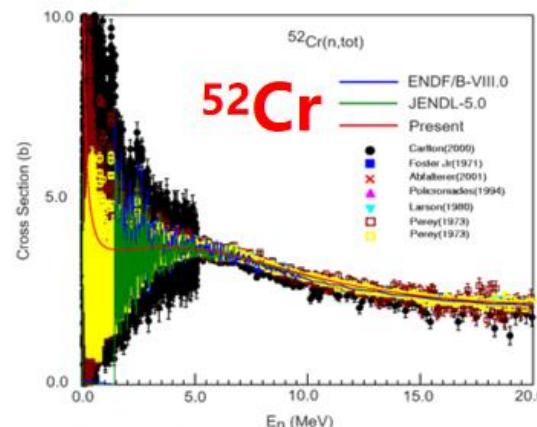
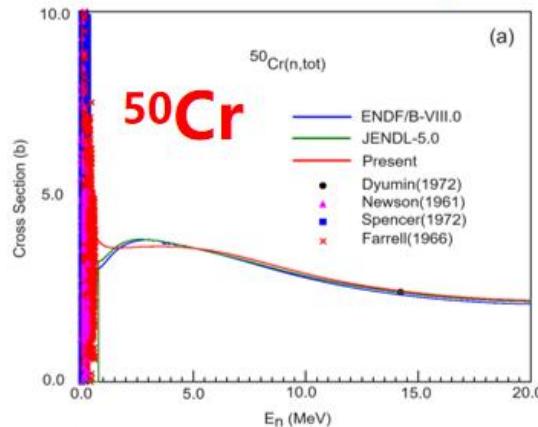
6. n+ Cr nuclear reaction data

Benchmark testing results for Cr-52

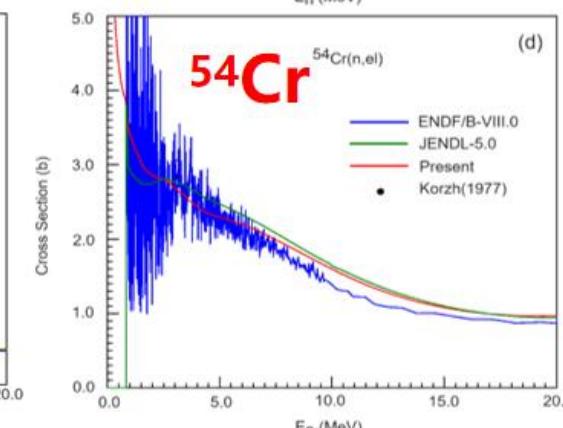
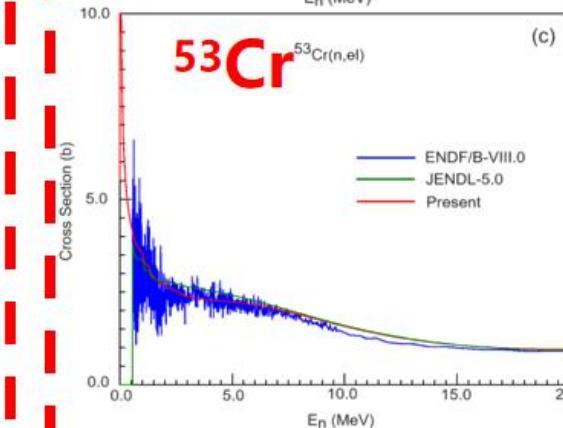
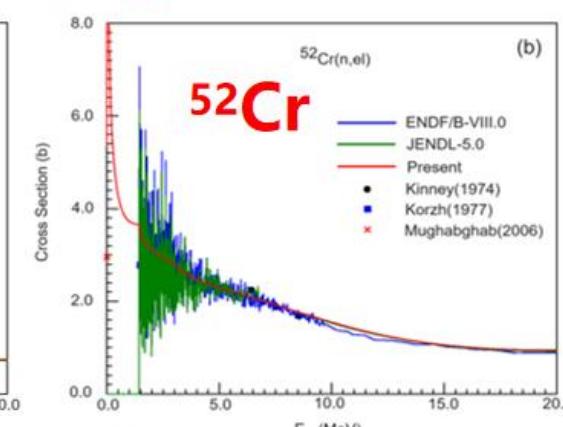
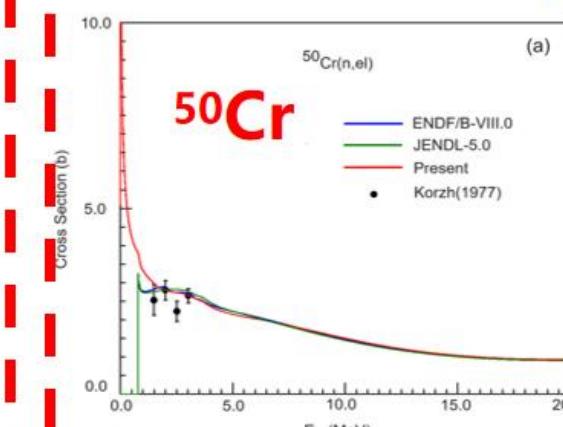


6. n+ Cr nuclear reaction data

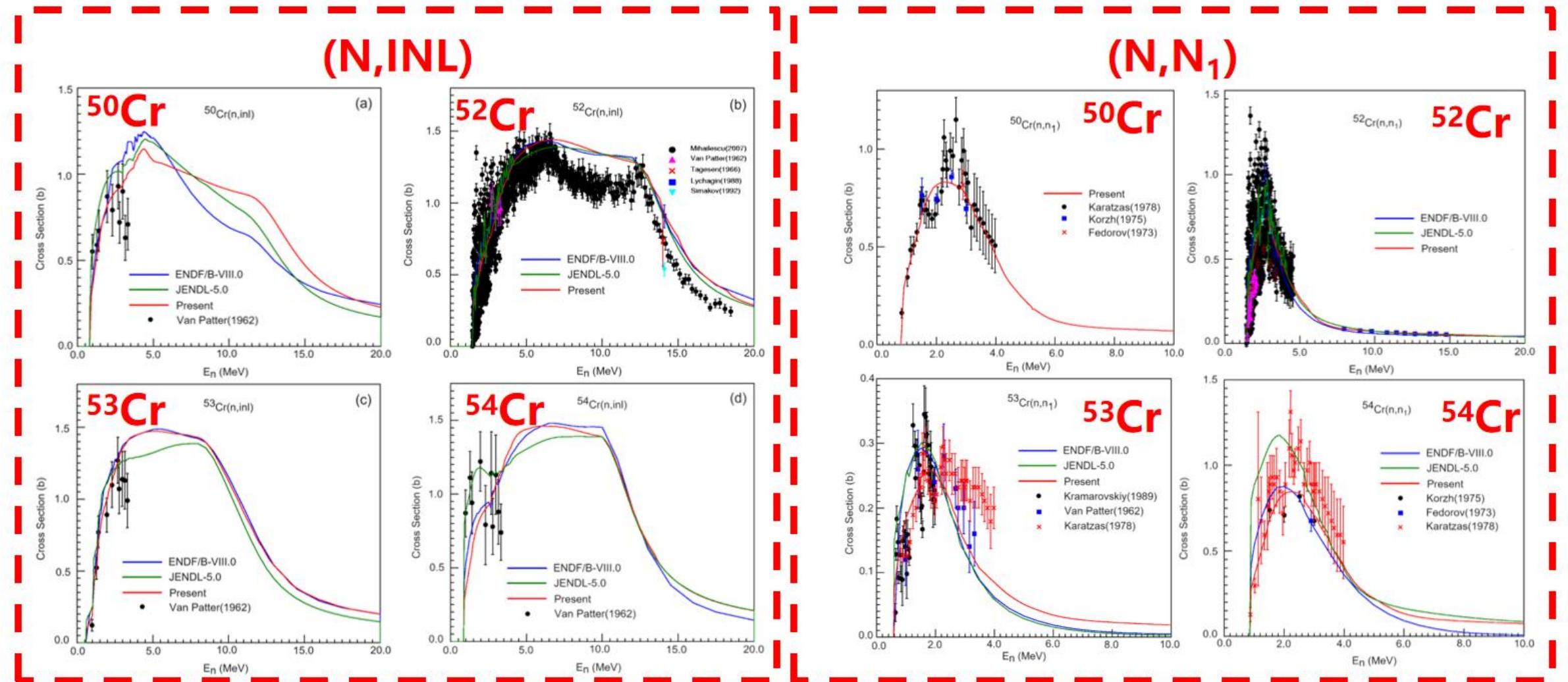
(N,TOT)



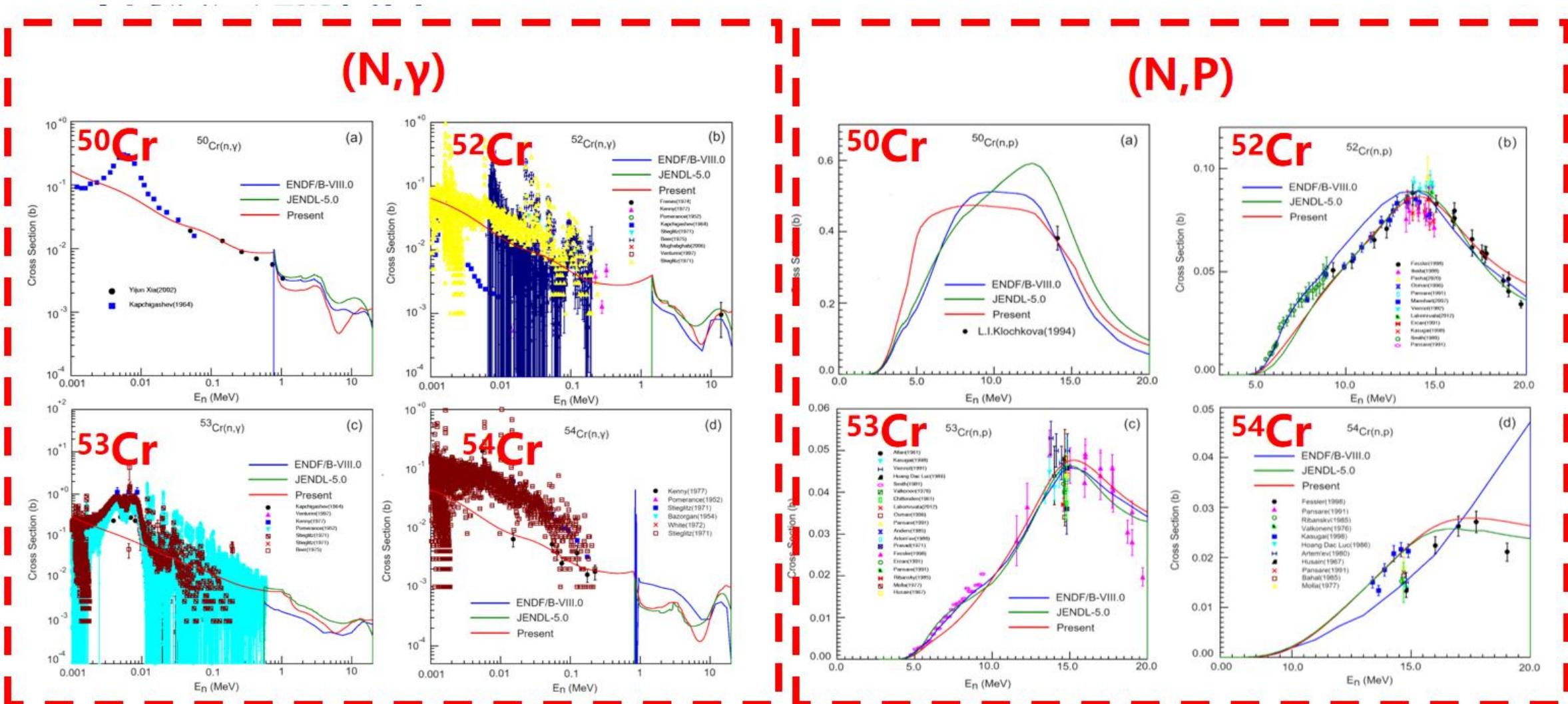
(N,EL)



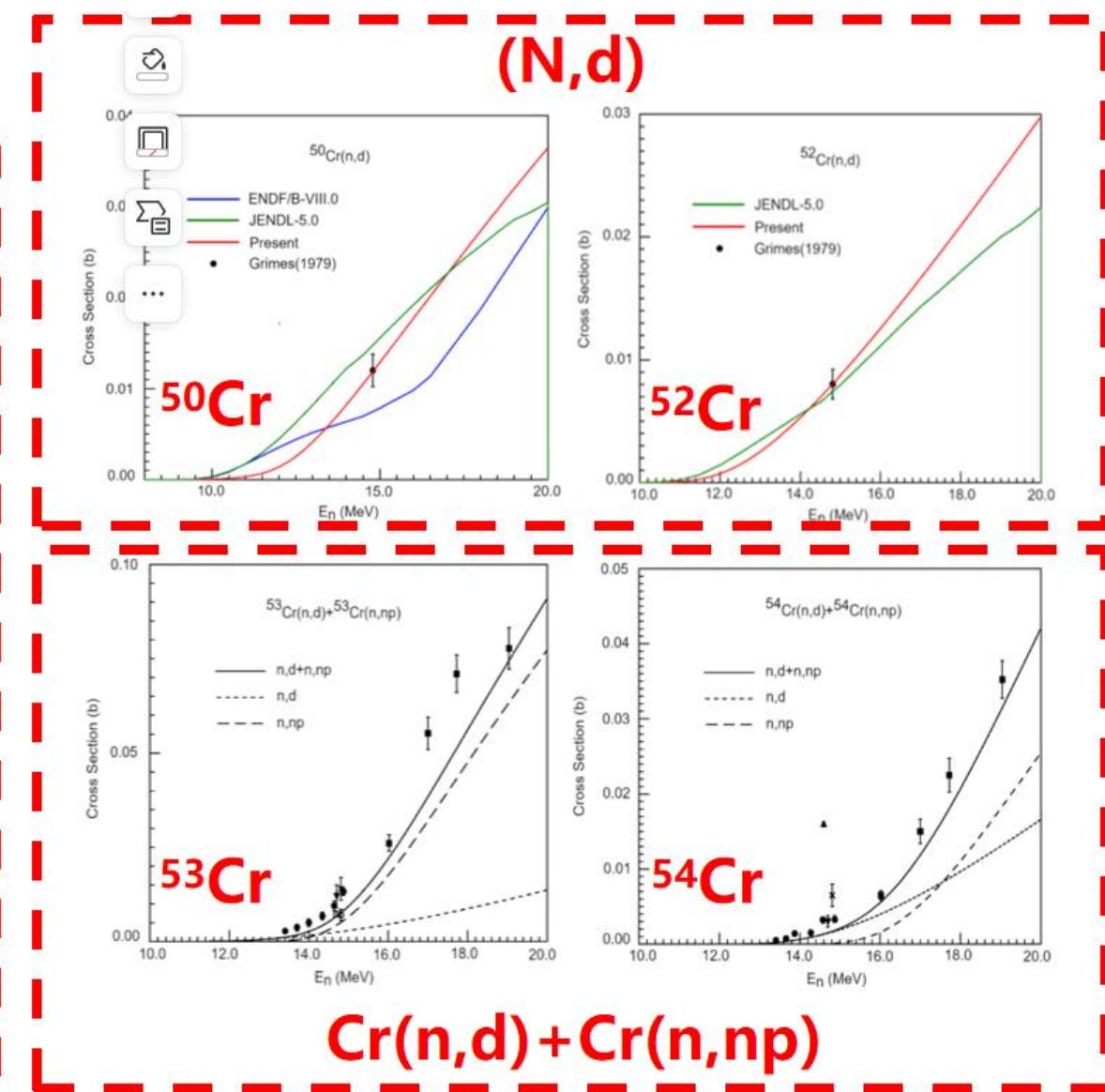
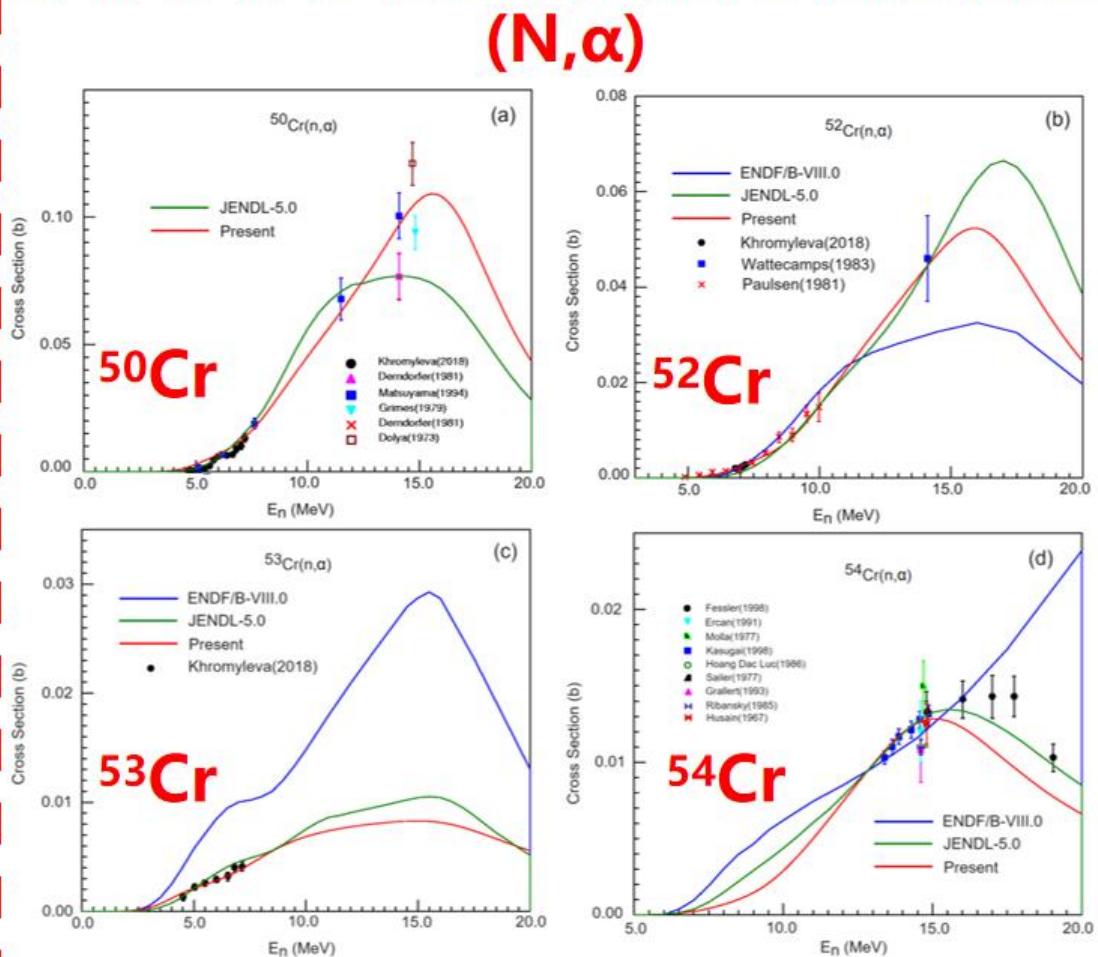
6. n+ Cr nuclear reaction data



6. n+ Cr nuclear reaction data

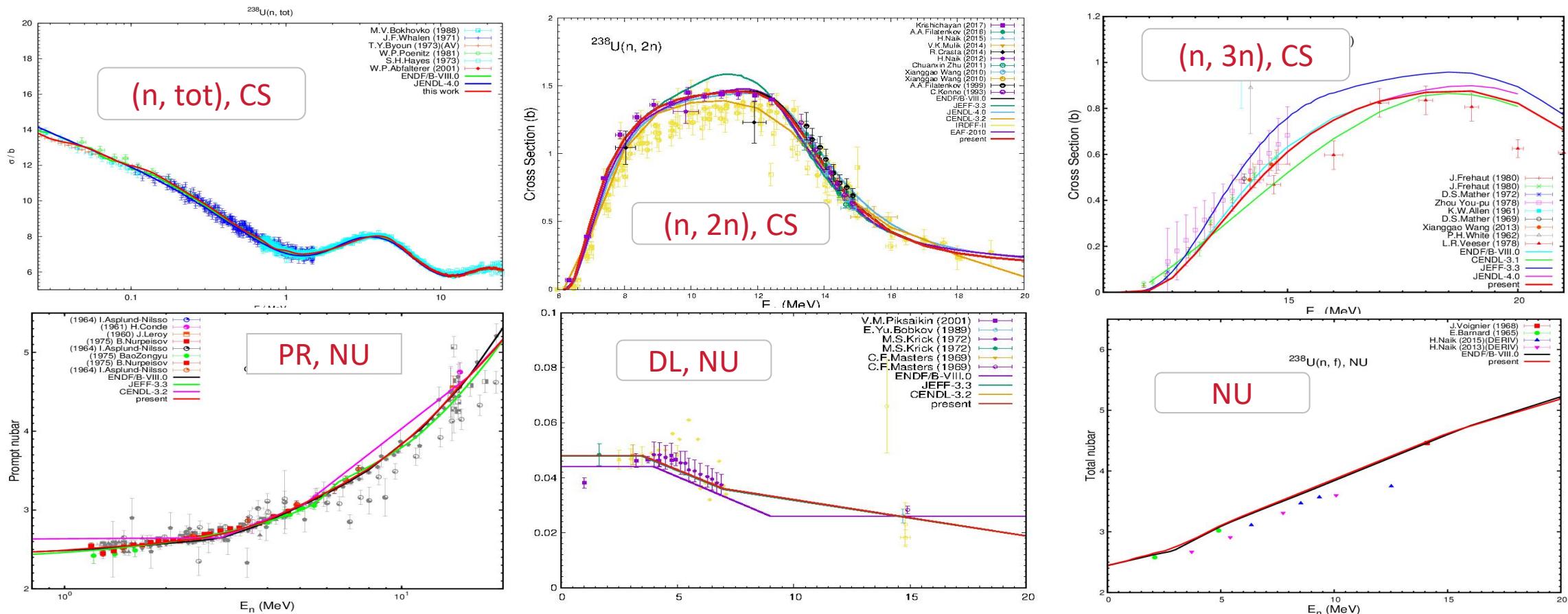


6. n^+ Cr nuclear reaction data



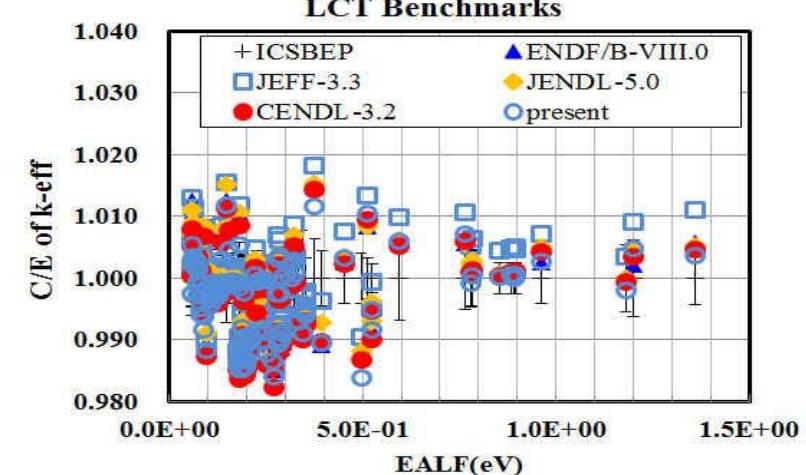
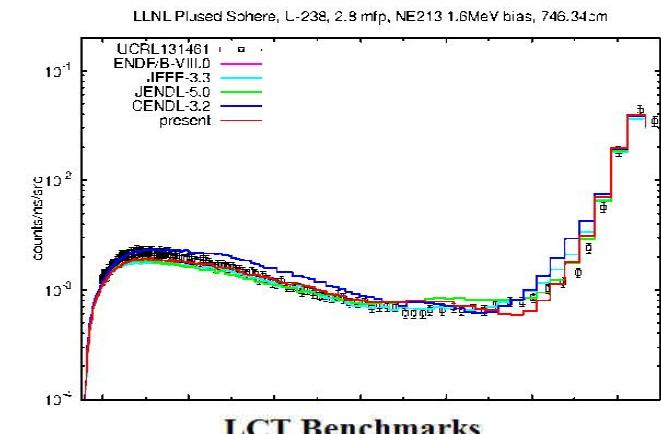
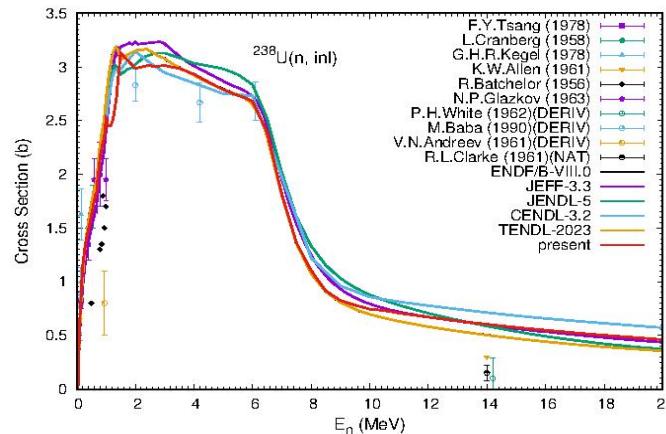
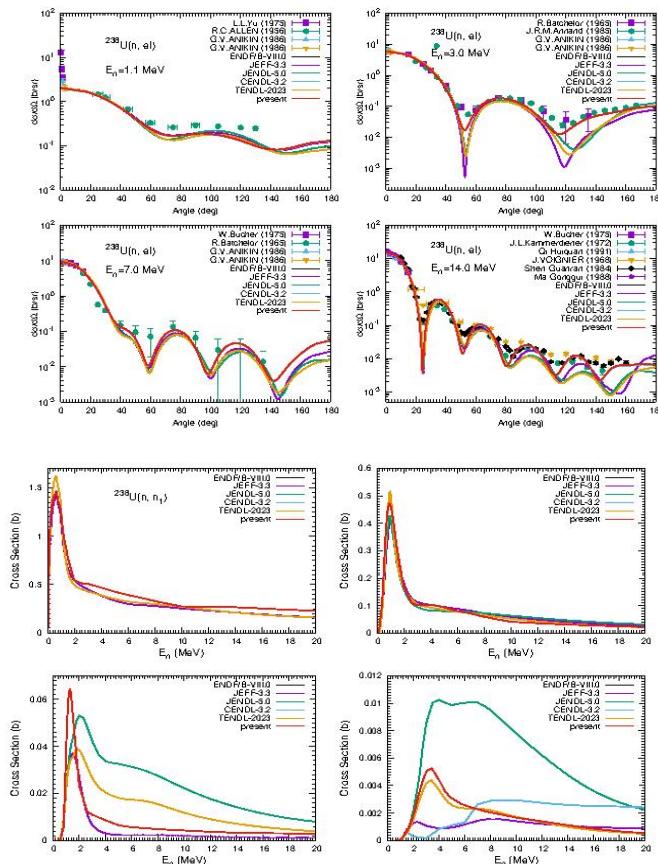
7. $n + {}^{238}\text{U}$

- Based on **experimental data analysis**, Nu-bar, (n,tot), (n, γ), (n,f), (n,2n) and (n,3n) cross sections evaluated.
- Experimental data evaluation based on factors such as experimental methods, experimental objectives, neutron monochromaticity, detector resolution capability, sample quantification, monitor cross-section selection, data correction, and uncertainty analysis.



7. $n + {}^{238}\text{U}$

- New **theoretical calculations** based on Hauser-Feshbach and pre-equilibrium carried out.
- **ENDF-6 file** obtained with resonance parameters and prompt fission neutron spectrum taken from ENDF/B-VIII.0.
- Guided by **integral benchmark**, (n, inl) , (n, γ) and (n, f) cross sections underwent multiple adjustments.
- The final benchmark results indicates a **significant improvement** in the final data quality.



Outlines

Introduction

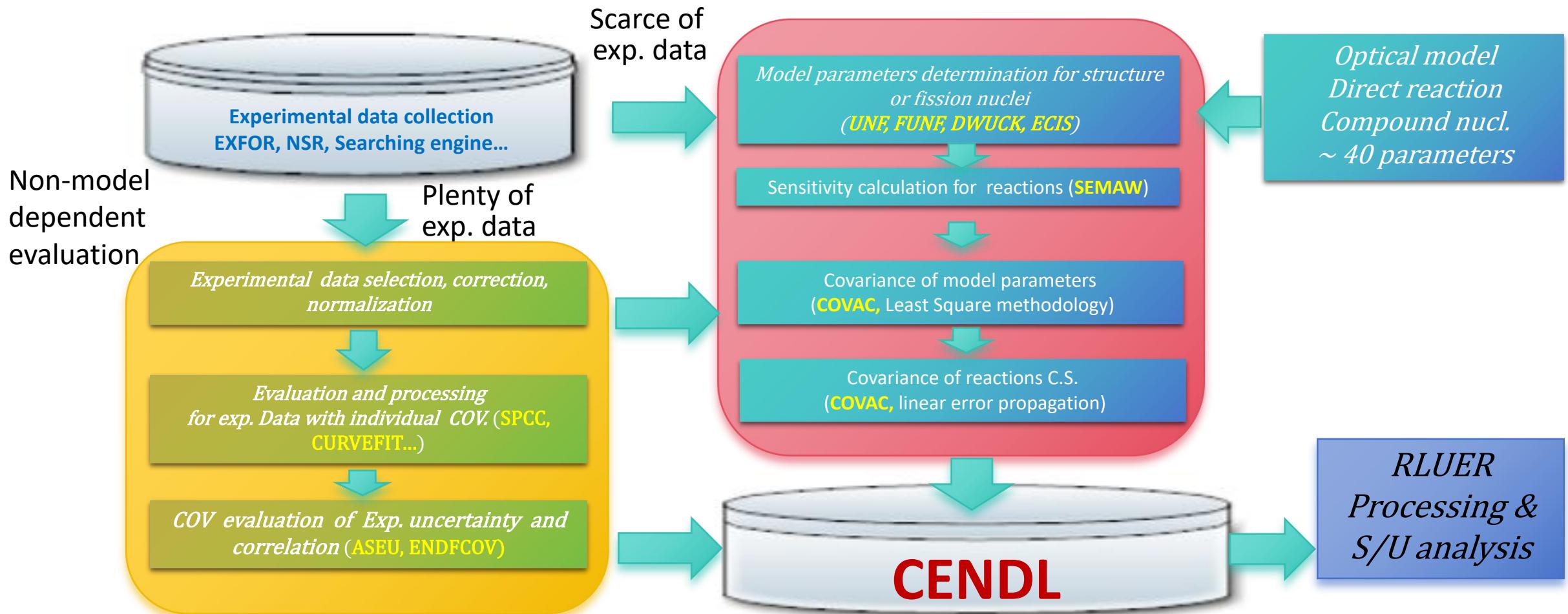
Nuclear Reaction Study

Perspective

Nuclear Reaction Study

- Platform for Theoretical Models
- Nuclear Data Improvement
- Covariance Evaluations
- AI and Nuclear Data Evaluation

1. Application of Bayes deterministic to broad mass range



Correlations among single (or multiple) set(s) of experimental data are vital elements to get an 'honest' covariance.

1. Application of LS to broad mass range

Descriptions to the current COV scheme at CNDC:

- ① Tech. for non-model & model dependent
- ② Energies for structure & smooth regions
- ③ COV data types for NI & NC
- ④ Tech. deal with single & multiple measurements
- ⑤ Tech. for parameter sensitivity selection
- ⑥ COV matrix positive definition treatment

| (n,tot) (n,tot) | (n,tot) (n,inl) | (n,tot) (n, γ) | (n,tot) (n,p) | (n,tot) (n,d) | (n,tot) (n,t) | (n,tot) (n,2n) | (n,tot) (n,np) | (n,tot) (n,n α) |
|--------------------|--------------------|----------------------------------|-------------------------|-------------------------|-------------------------|--------------------------|--------------------------|------------------------------------|
| NM | (n,inl) (n,inl) | (n,inl) (n, γ) | (n,inl) (n,p) | (n,inl) (n,d) | (n,inl) (n,t) | (n,inl) (n,2n) | (n,inl) (n,np) | (n,inl) (n,n α) |
| | | (n, γ) (n, γ) | (n, γ) (n,p) | (n, γ) (n,d) | (n, γ) (n,t) | (n, γ) (n,2n) | (n, γ) (n,np) | (n, γ) (n,n α) |
| | | | (n,p) (n,p) | (n,p) (n,d) | (n,p) (n,t) | (n,p) (n,2n) | (n,p) (n,np) | (n,p) (n,n α) |
| | | | | NM | (n,d) (n,d) | (n,d) (n,2n) | (n,p) (n,np) | (n,p) (n,n α) |
| | | | | | (n,t) (n,t) | (n,t) (n,2n) | (n,t) (n,np) | (n,t) (n,n α) |
| | | | | | | (n,2n) (n,2n) | (n,2n) (n,np) | (n,2n) (n,n α) |
| | | | | | | | (n,np) (n,np) | (n,np) (n,n α) |
| | | | | | | | | (n,n α) (n,n α) |

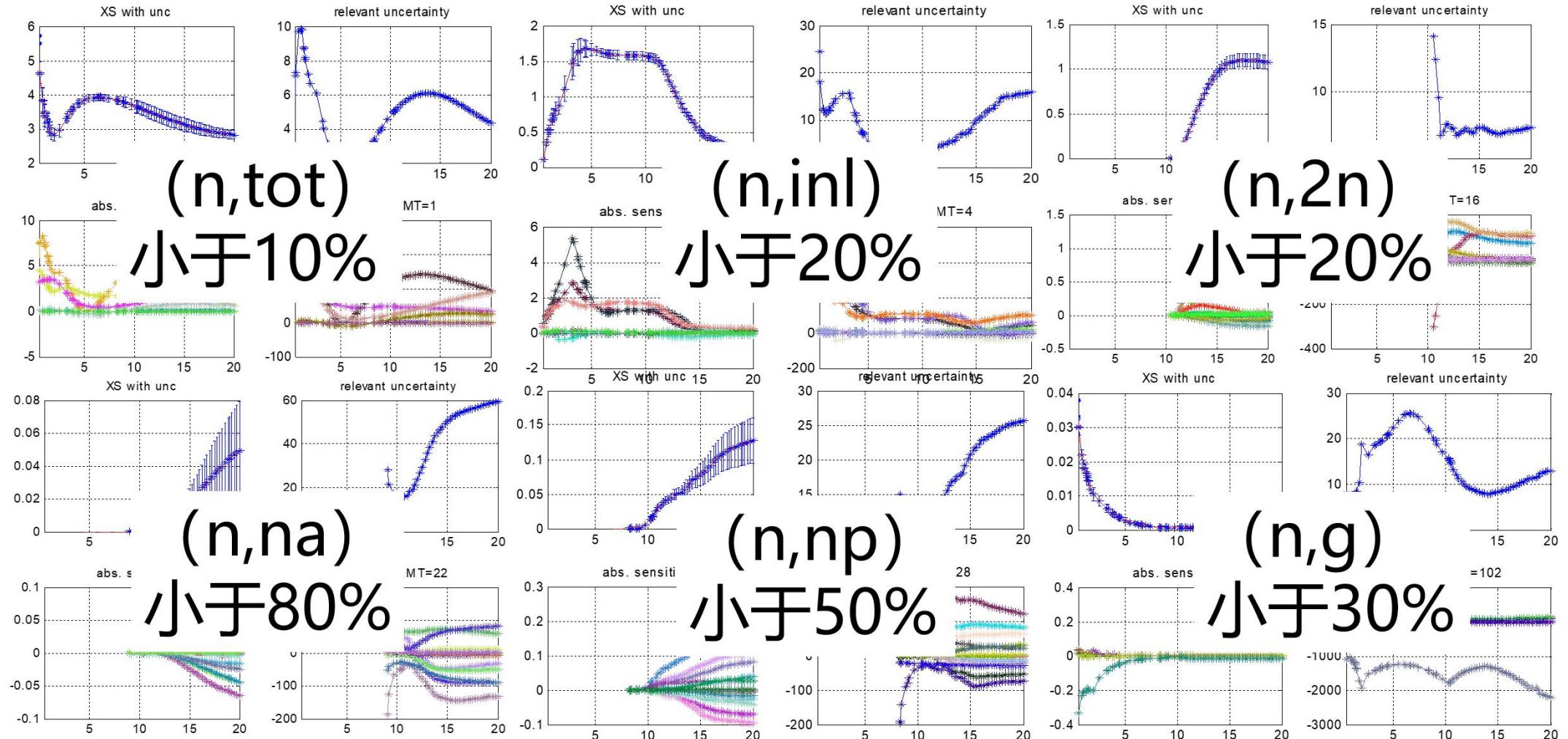
9个反应道以及彼此之间的关联

| (n,tot) (n,tot) | (n,tot) (n,inl) | (n,tot) (n, γ) | (n,tot) (n,p) | (n,tot) (n,d) | (n,tot) (n,t) | (n,tot) (n,2n) | (n,tot) (n,np) | (n,tot) (n,n α) |
|--------------------|--------------------|---------------------------|----------------------------------|-------------------------|-------------------------|-------------------------|--------------------------|----------------------------|
| | (n,inl) (n,inl) | (n,inl) (n, γ) | (n,inl) (n,p) | (n,inl) (n,d) | (n,inl) (n,t) | (n,inl) (n,2n) | (n,inl) (n,np) | (n,inl) (n,n α) |
| | | MODEL | (n, γ) (n, γ) | (n, γ) (n,p) | (n, γ) (n,d) | (n, γ) (n,t) | (n, γ) (n,2n) | (n, γ) (n,np) |
| | | | MODEL | (n,p) (n,p) | (n,p) (n,d) | (n,p) (n,t) | (n,p) (n,2n) | (n,p) (n,np) |
| | | | | MODEL | (n,d) (n,d) | (n,d) (n,t) | (n,p) (n,np) | (n,p) (n,n α) |
| | | | | | MODEL | (n,t) (n,t) | (n,t) (n,np) | (n,t) (n,n α) |
| | | | | | | MODEL | (n,2n) (n,2n) | (n,2n) (n,np) |
| | | | | | | | (n,np) (n,np) | (n,np) (n,n α) |
| | | | | | | | | MODEL |

9个反应道以及彼此之间的关联
ENDF文件中，MF=32, 33包含19111行，占到Zr90全套数据文件的
2019/1/12

1. Application of LS to broad mass range

55Mn ~ 160Gd



2. Application of Unified Monte Carlo (UMC-B) for n+48Ti

$$p(\sigma) = C L(y_E, V_E | \sigma) p_0(\sigma | \sigma_C, V_C)$$

$$\langle \sigma_i \rangle = \int_S \sigma_i p(\sigma) d\sigma$$

$$(V)_{ij} = \langle \sigma_i \sigma_j \rangle - \langle \sigma_i \rangle \langle \sigma_j \rangle$$

$$L(y_E, V_E | \sigma) \sim \exp \left\{ -\frac{1}{2} [(y - y_E)^T \bullet V_E^{-1} \bullet (y - y_E)] \right\}$$

$$p_0(\sigma | \sigma_C, V_C) \sim \exp \left\{ -\frac{1}{2} [(\sigma - \sigma_C)^T \bullet V_C^{-1} \bullet (\sigma - \sigma_C)] \right\}$$

y_e, V_e : measured quantities with "n" elements

σ_C, V_C : calculated using nuclear models with "m" elements

UMC-B:

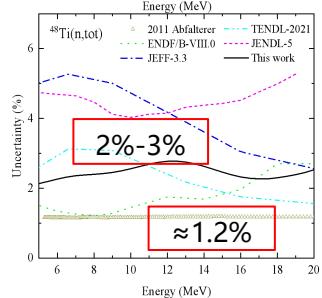
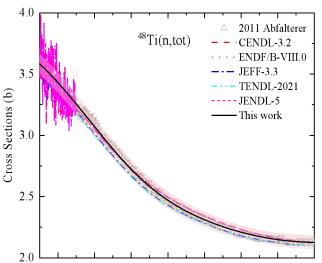
For each random set $\{\sigma_i\}$ we calculate

$$\omega_k = \exp \left\{ -\frac{1}{2} [(y_k - y_E)^T \bullet V_E^{-1} \bullet (y_k - y_E)] \right\}$$

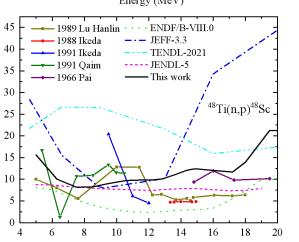
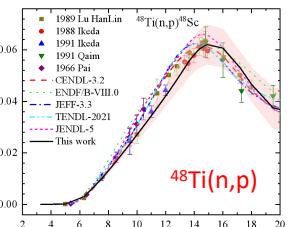
$$\sigma_i = \left[\sum_{k=1,K} \omega_k \sigma_{Cik} \right] / \left[\sum_{k=1,K} \omega_k \right]$$

$$(V_\sigma)_{ij} = \left[\sum_{k=1,K} \omega_k \sigma_{Cik} \sigma_{Cjk} \right] / \left[\sum_{k=1,K} \omega_k \right] - \sigma_i \sigma_j$$

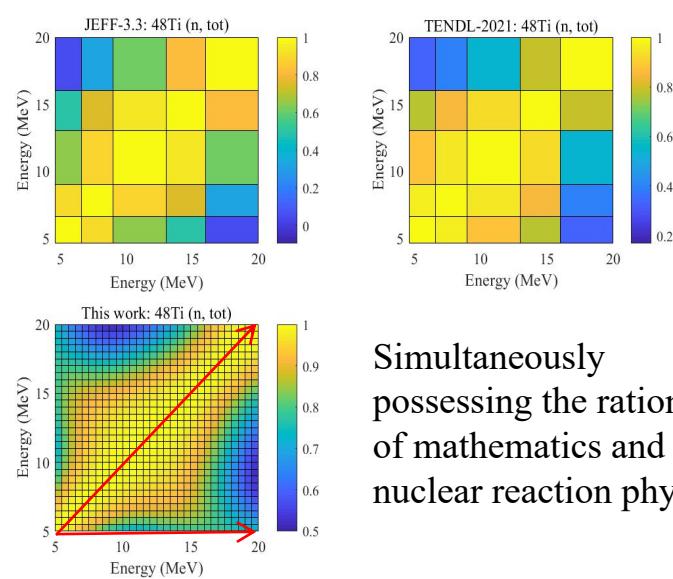
total cross section:



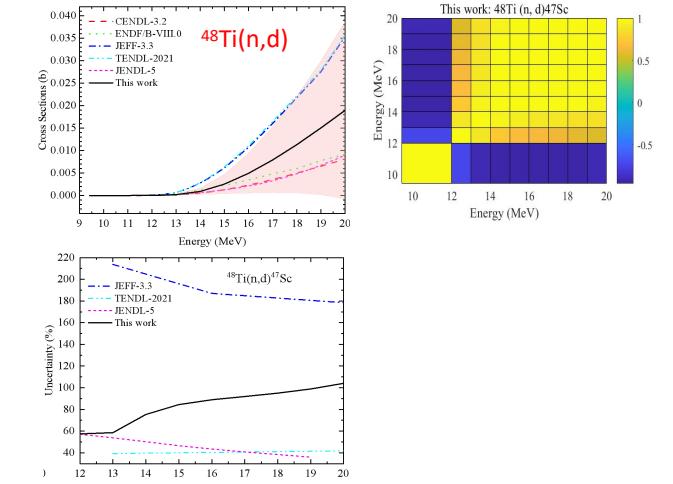
rich experimental data:



This work was in collaboration with R. Capote(IAEA),
Chinese Physics C Vol. 48, No. 7 (2024) 074101; doi: 10.1088/1674-1137/ad432c



lack of experimental data:



Outlines

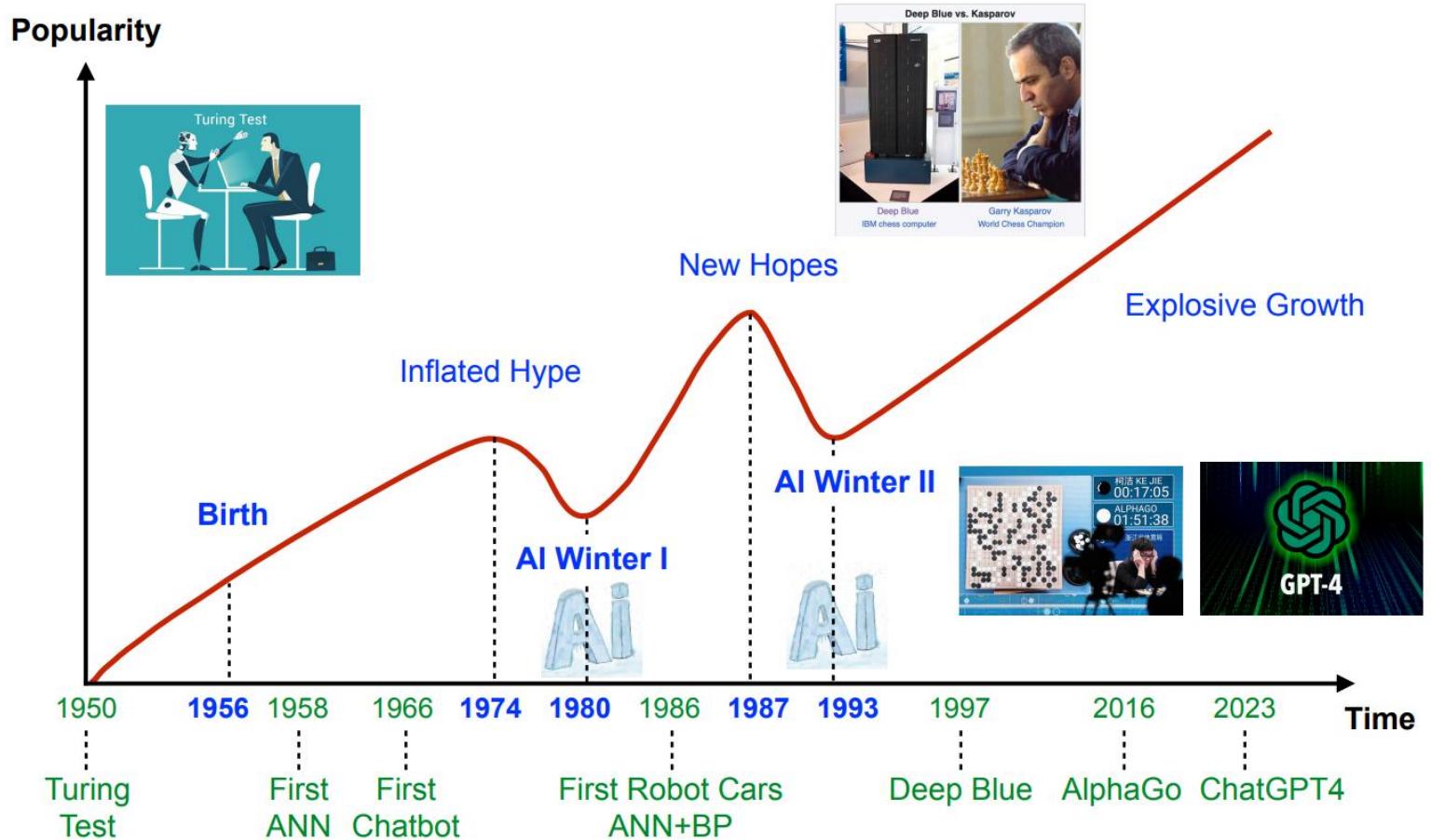
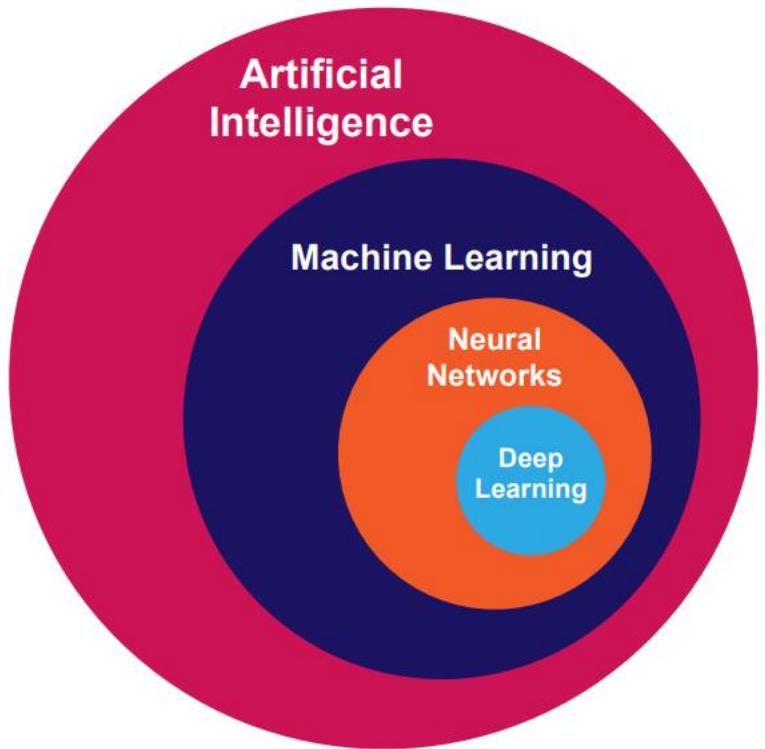
Introduction

Nuclear Reaction Study

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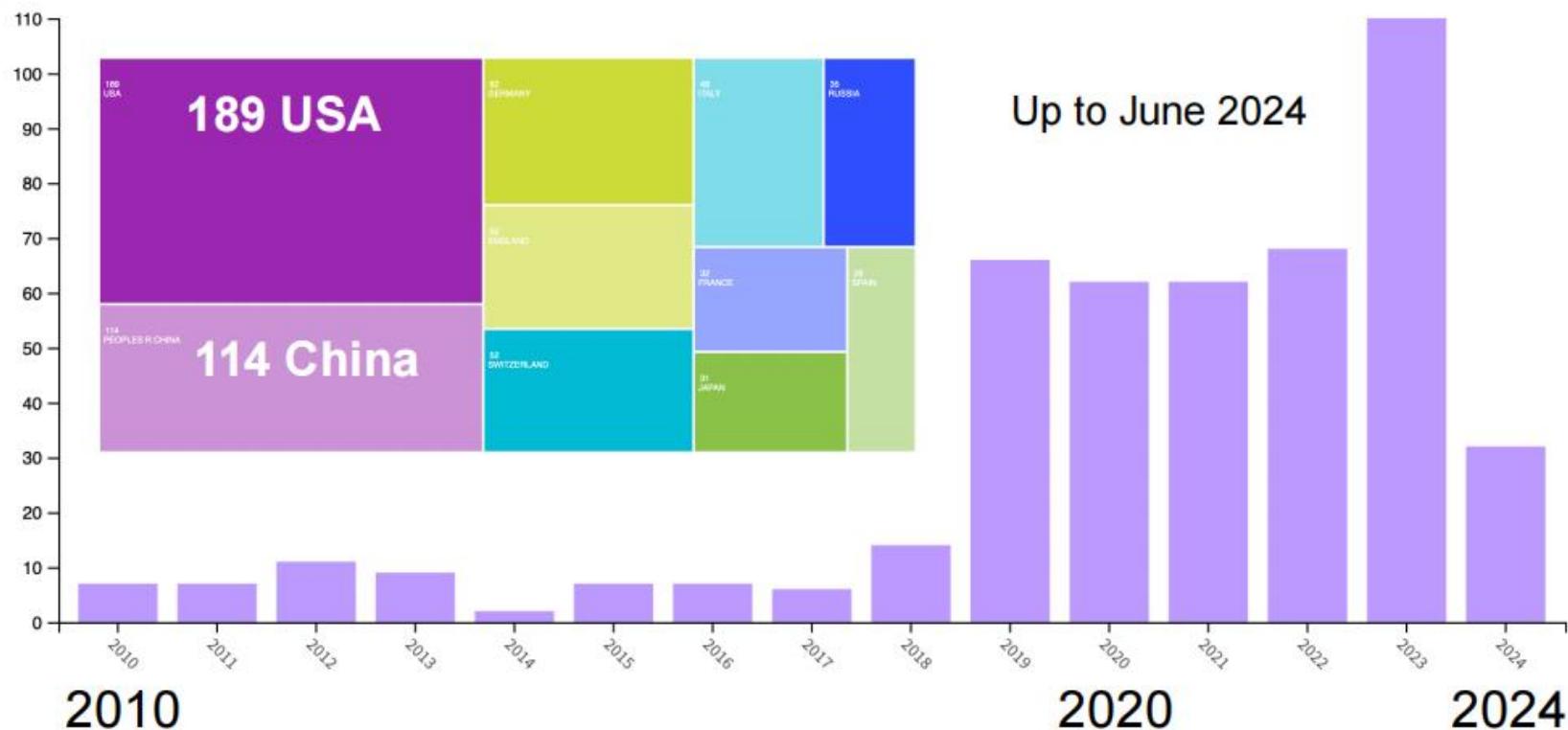


Machine learning in nuclear physics

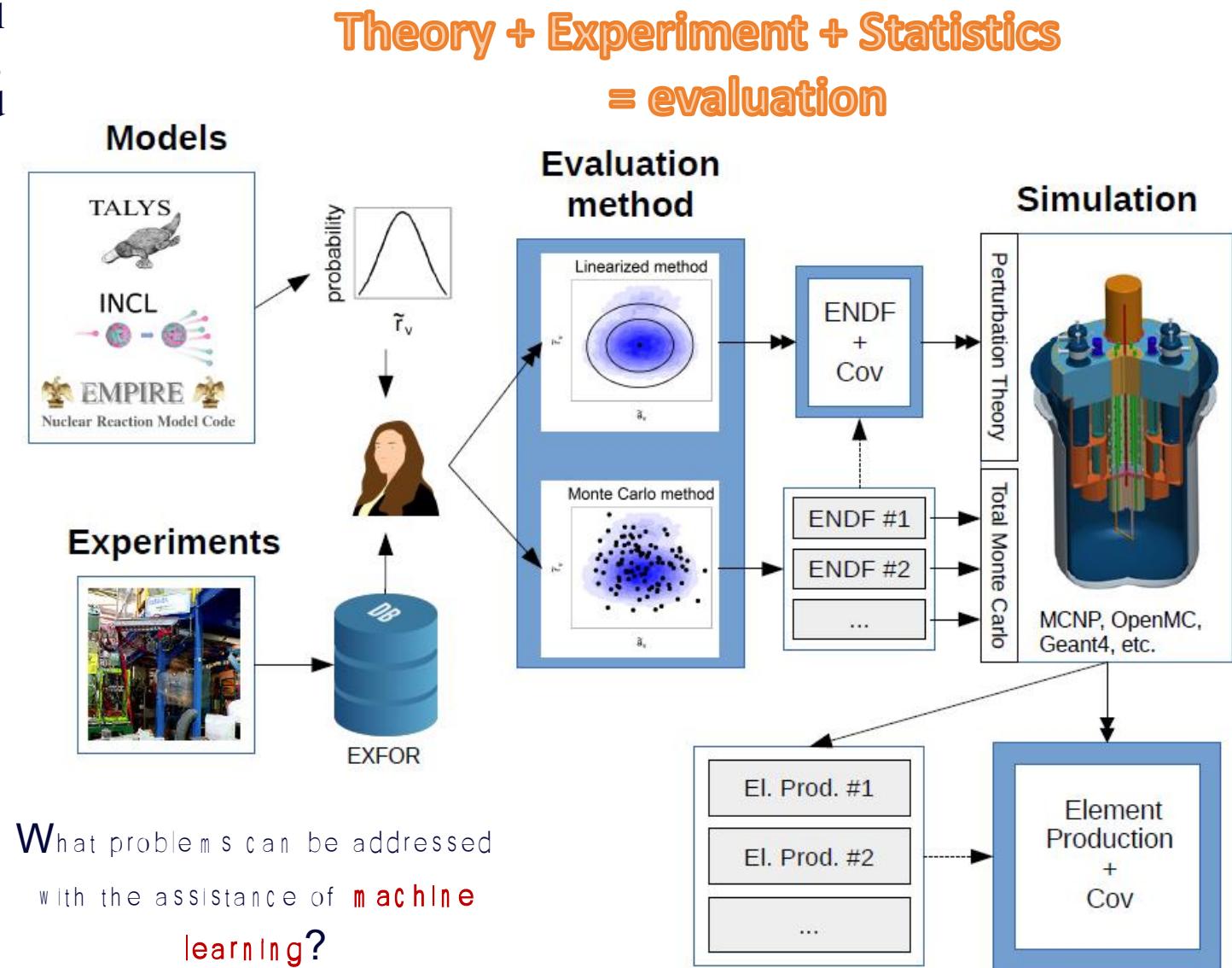
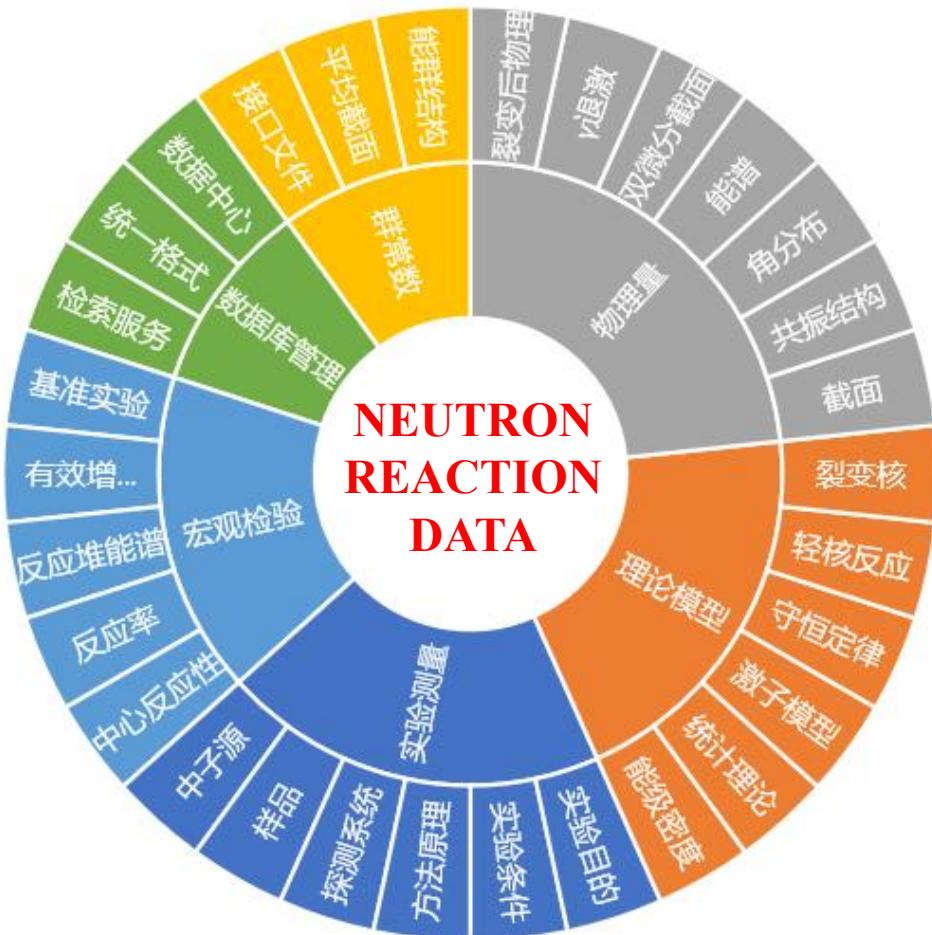
483 publications selected from Web of Science Core Collection

114/483 ~ 23.6%

(TI=(machine learning) OR AB=(machine learning)) AND WC=(Physics, Nuclear)

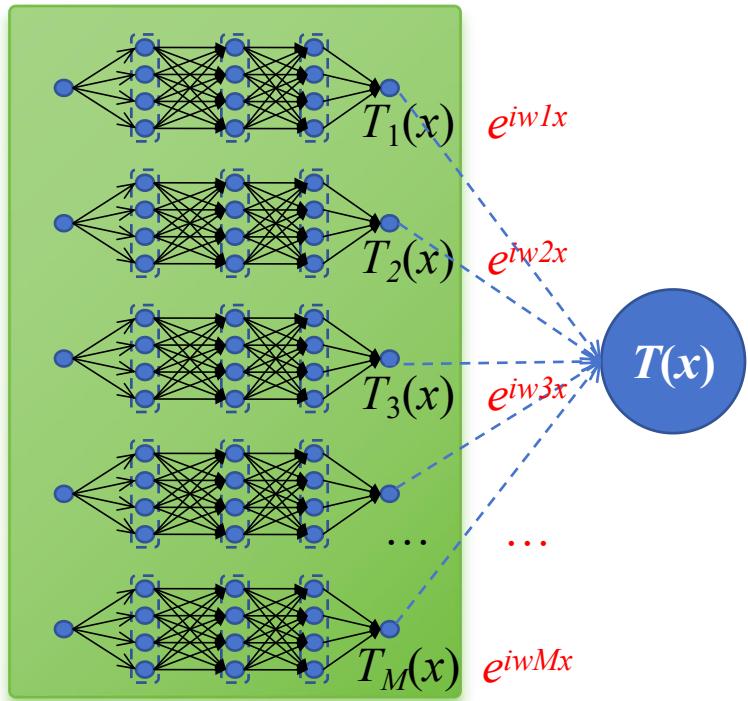


Nuclear data, which encompasses experimental measurements, theoretical models, evaluation, processing, validation, as well as database management and dissemination, necessitates highly specialized expertise.

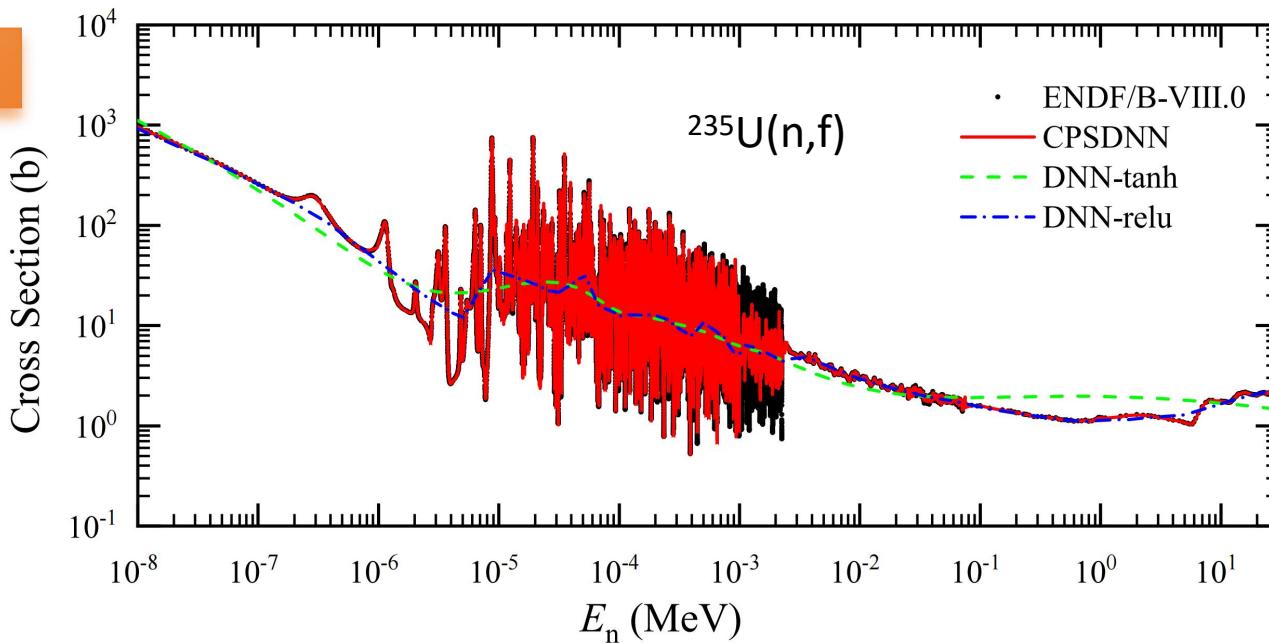


- Coupled Phase Shift Deep Neural Network(CPSDNN) Approach for Studying Resonance Cross Sections of $^{235}\text{U}(\text{n},\text{f})$ Reaction

CPSDNN to deal with high-frequency oscillating data



The neural network structure diagram of CPSDNN. Each $F_m(x)$ ($m = 0, 1, \dots, M$) represents a neural network. The $T(x)$ represents the training process of neural network.



The comparison of the raw cross sections, the predicted results of DNNs and CPSDNN for $^{235}\text{U}(\text{n},\text{f})$ reaction.

The black points are the evaluated data derived from ENDF/B-VIII.0. The green and the blue lines are the predicted results of DNNs with the tanh and relu activation functions, respectively.

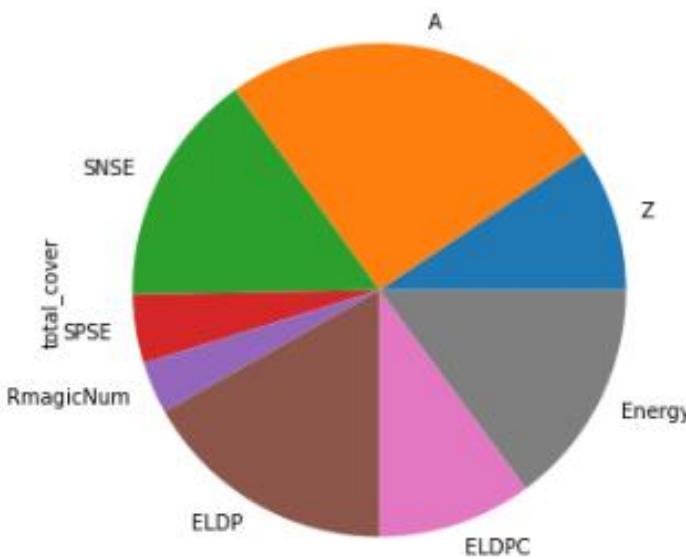
The red line is the predicted results of CPSDNN.

Systematic study of (n,2n) cross section adopting ANN and DT

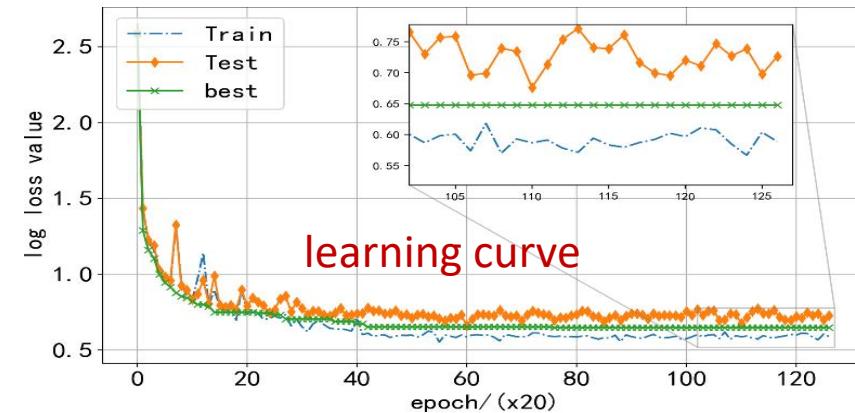


China nuclear industry college

The proportion of data exhibiting a prediction deviation of less than 10% surpasses 85%, enabling the successful calculation of covariance for cross-sectional data predictions across various energy levels.

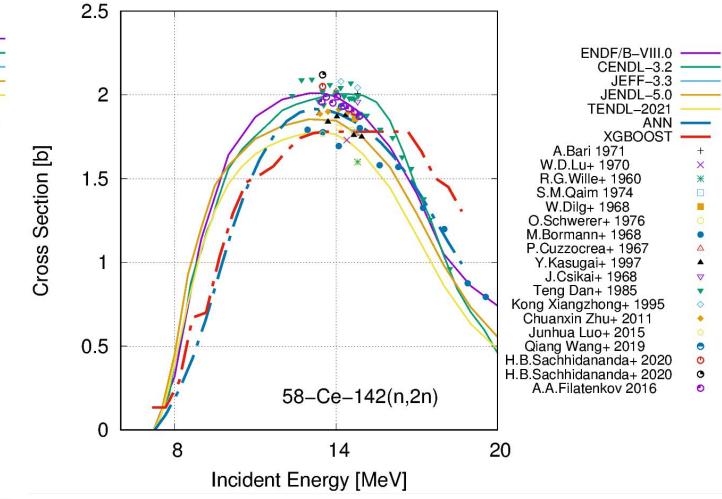
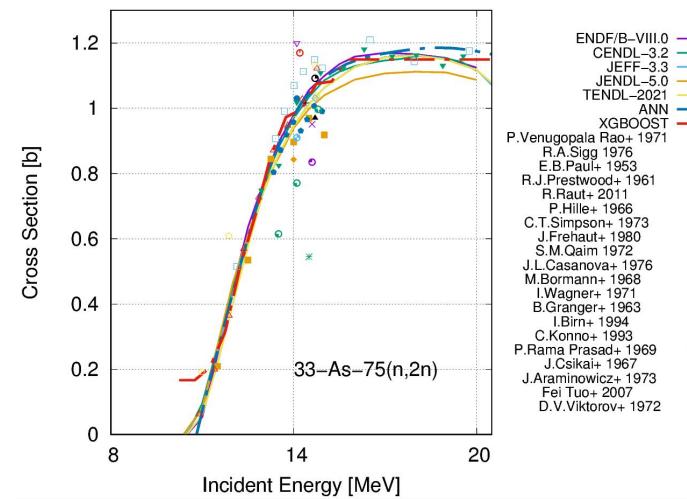


EPJ Conferences, 294, 04008 (2024)



learning curve

Comparison of ML results, evaluation and experimental data



Machine learning techniques uncover systematic patterns within nuclear reaction cross sections.

Outlines

Introduction

Nuclear Reaction Study

Perspective

Nuclear Reaction Study

- Platform for Theoretical Models
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New Evaluation Project and Data in CENDL-4.0

Motivation: Precision, Total number, Types

the coming end of 2025

| ID | Reaction | JENDL-5 2021 | ENDF/B-VIII.0 2018 | JEFF-3.3 2017 | CENDL-3.2 2020 | CENDL-4.0 2025 | BROND-3.1 2016 | TENDL 2021 |
|----|--|----------------------|-----------------------|------------------|-------------------|--------------------|-------------------|---------------|
| 1 | Photo-Nuclear Data (G) | 2684 | 163 | 0 | 0 | 266 | 0 | 2804 |
| 2 | Photo-Atomic Interaction Data (PHOTO) | 100 | 100 | 0 | 0 | 0 | 0 | 0 |
| 3 | Radioactive Decay Data (DECAY) | 4071 | 3821 | 3852 | 0 | 2354 (A=66~172) | 0 | 0 |
| 4 | Spontaneous Fission Product Yields (S/FPY) | 10 | 9 | 3 | 0 | 9 | 0 | 0 |
| 5 | Atomic Relaxation Data (ARD) | 100 | 100 | 0 | 0 | 0 | 0 | 0 |
| 6 | Incident-Neutron Data (N) | 795 | 557 | 562 | 272 | 410 | 372 | 2813 |
| 7 | Neutron-Induced Fission Product Yields (N/FPY) | 31 | 31 | 19 | 0 | 31 | 0 | 0 |
| 8 | Thermal Neutron Scattering Data (TSL) | 62 | 34 | 20 | 0 | 0 | 0 | 0 |
| 9 | Electro-Atomic Interaction Data (E) | 100 | 100 | 0 | 0 | 0 | 0 | 0 |
| 10 | Incident-Proton Data (P) | 239 | 49 | 0 | 0 | 78 | 0 | 2812 |
| 11 | Incident-Deuteron Data (D) | 9 | 5 | 0 | 0 | 0 | 0 | 2903 |
| 12 | Incident-Triton Data (T) | 0 | 5 | 0 | 0 | 0 | 0 | 2810 |
| 13 | Incident-He3 Data (He3) | 0 | 3 | 0 | 0 | 0 | 0 | 2809 |
| 14 | Incident-Alpha Data (He4) | 18 | 1 | 0 | 0 | 0 | 0 | 2808 |
| 15 | Activation Data | Special purposed lib | 0 | 0 | 0 | 818 | 0 | 0 |

Many thanks to the collaborators and CENDL project:

All the great contributions from CNDC and CNDCN(China Nuclear Data Coordination Network)

IAEA: Roberto, Arjan, Vivian, Otsuka et al. from IAEA, and CRP 20466

Thank you for your attention!