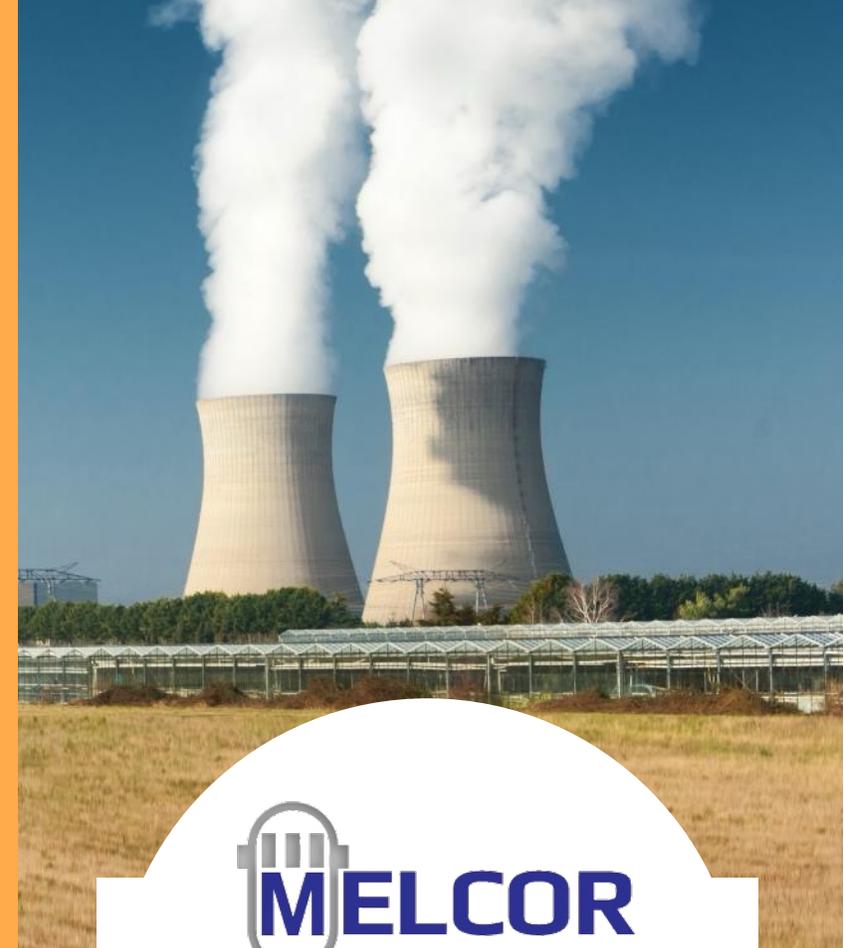




Sandia
National
Laboratories

Securing the future of Nuclear Energy



MELCOR MSR Modeling Capabilities for Licensing and Severe Accident Analysis

Matthew S. Christian, Lucas I. Albright, Bradley A. Beeny, Noel Belcourt, Brandon A. de Luna, Edward M. Duchnowski, Troy C. Haskin, Dallin J. Keesling, Casey Wagner, David L. Luxat



Sandia National Laboratories is a multimission laboratory managed and operated by National Technology and Engineering Solutions of Sandia, LLC., a wholly owned subsidiary of Honeywell International, Inc., for the U.S. Department of Energy's National Nuclear Security Administration under contract DE-NA0003525.

MELCOR for Advanced Nuclear Energy Technologies



Fully integrated, engineering-level code

- Thermal-hydraulic response of reactor coolant system, reactor cavity, reactor enclosures, and auxiliary buildings
- Core heat-up, degradation and relocation
- Core-concrete interaction
- Flammable gas production, transport and combustion
- Fission product release and transport behavior

Level of physics modeling consistent with

- State-of-knowledge
- Necessity to capture global plant response
- Reduced-order and correlation-based modeling

Traditional application

- Models constructed by user from basic components (control volumes, flow paths and heat structures)
- Demonstrated adaptability to range of reactor designs – LWR, LWR-SMR, FHR, HPR, HTGR, MSR, SFR, ATR, VVER, SFP...

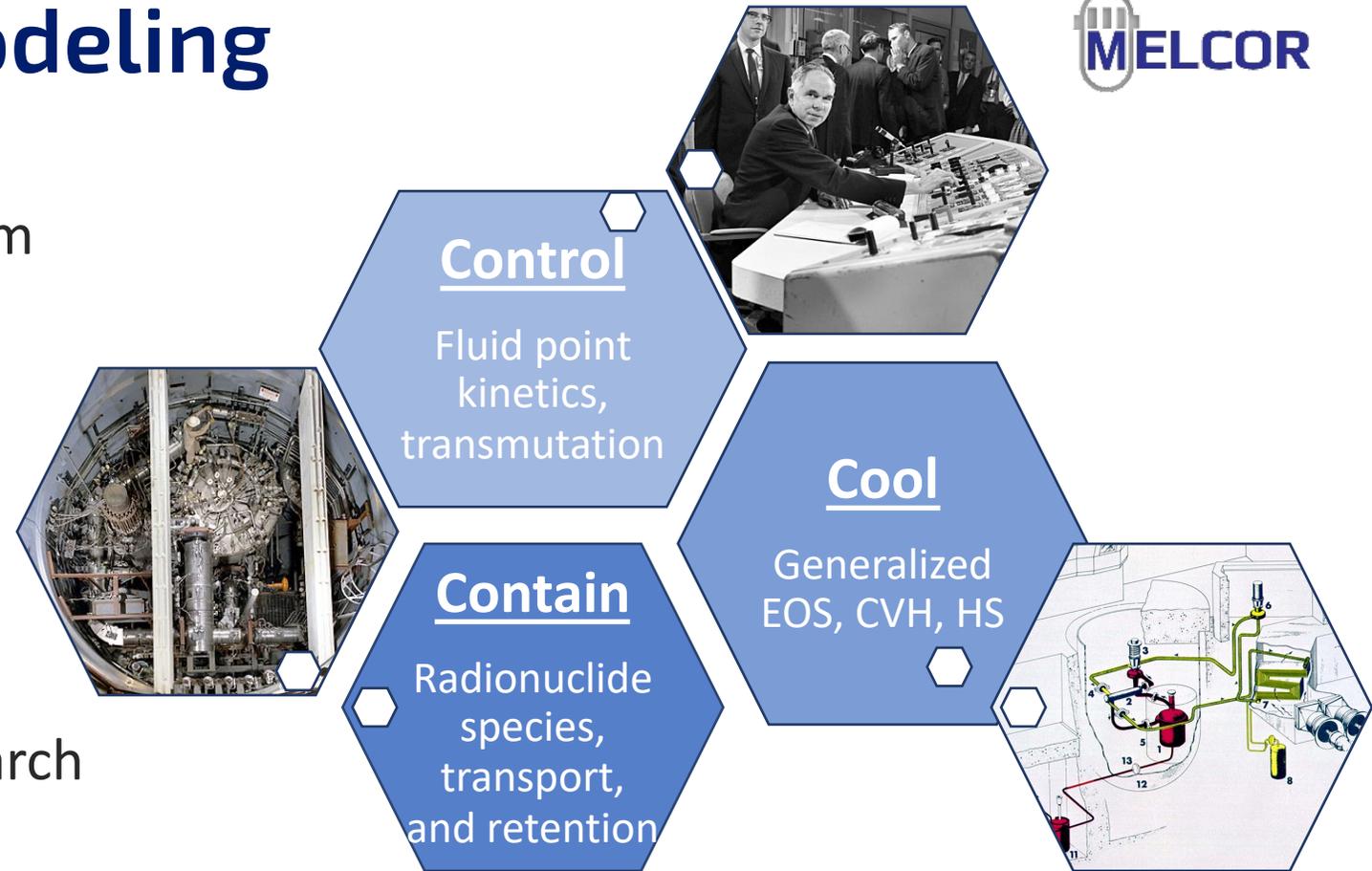
Molten Salt Reactor Modeling



- MELCOR Molten Salt Reactor source term modeling strategies revolve around 3 primary areas:

1. Control
2. Cool
3. Contain

- MELCOR capabilities have been tested using models for the Molten Salt Research Experiment (MSRE) and Molten Salt Breeder Reactor



Control Neutrons



- Fuel point kinetics – Derived from standard PRKEs and solved similarly
 - Feedback models
 - User-specified external input
 - Doppler
 - Fuel and moderator density
 - Flow reactivity feedback effects integrated into the equation set

- Transmutation – Depletion Module (under development)
 - Treats transmutation of the initial radionuclide inventory (radioactive decay and neutron interactions) during severe accidents including mass transfer between radionuclide classes

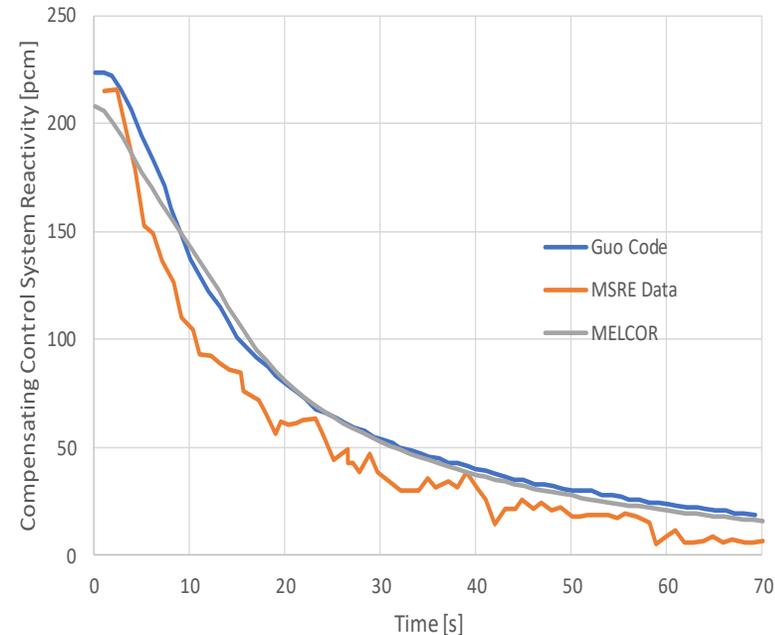
$$\frac{dP(t)}{dt} = \left(\frac{\rho(t) - \beta_{eff}}{\Lambda} \right) P(t) + \sum_{i=1}^6 \lambda_i C_i^c(t) + S_0$$

$$\frac{dC_i^c(t)}{dt} = \left(\frac{\beta_i}{\Lambda} \right) P(t) - (\lambda_i + 2/\tau_c) C_i^c(t) + \left(\frac{V_L}{V_C} \right) (\lambda_i + 2/\tau_L) C_i^l(t),$$

$i = 1 \dots 6$

$$\frac{dC_i^l(t)}{dt} = \left(\frac{V_C}{\tau_c V_L} \right) C_i^c(t) - (\lambda_i + 1/\tau_L) C_i^l(t),$$

$i = 1 \dots 6$



$$\frac{dN}{dt} = (A_\lambda + A_\phi \Phi) N(t) + S(t)$$

$N(t)$: vector of isotope number densities

A_λ : radioactive decay transition matrix

A_ϕ : neutron interaction transition matrix

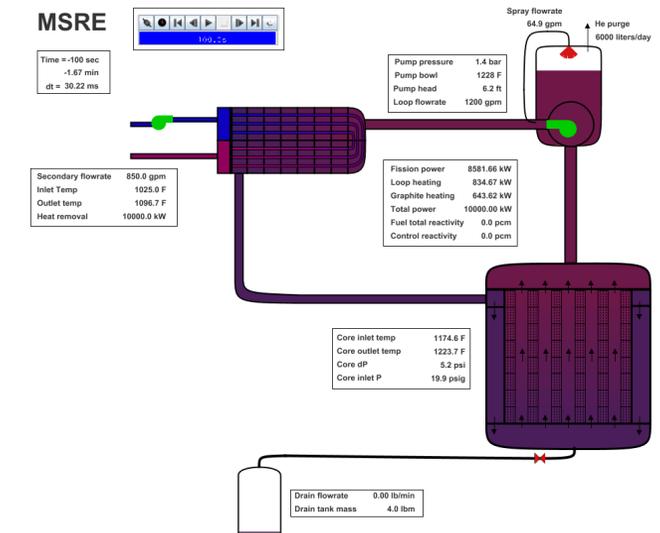
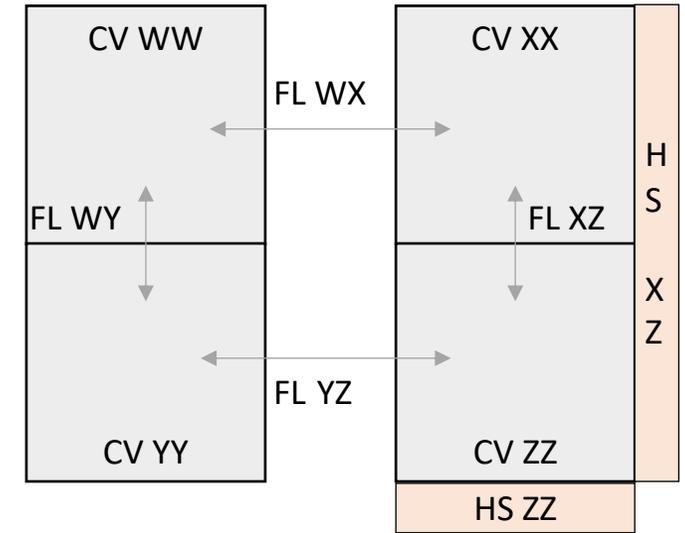
Φ : scalar neutron flux

$S(t)$: eternal isotope source

Keep the Reactor Cool

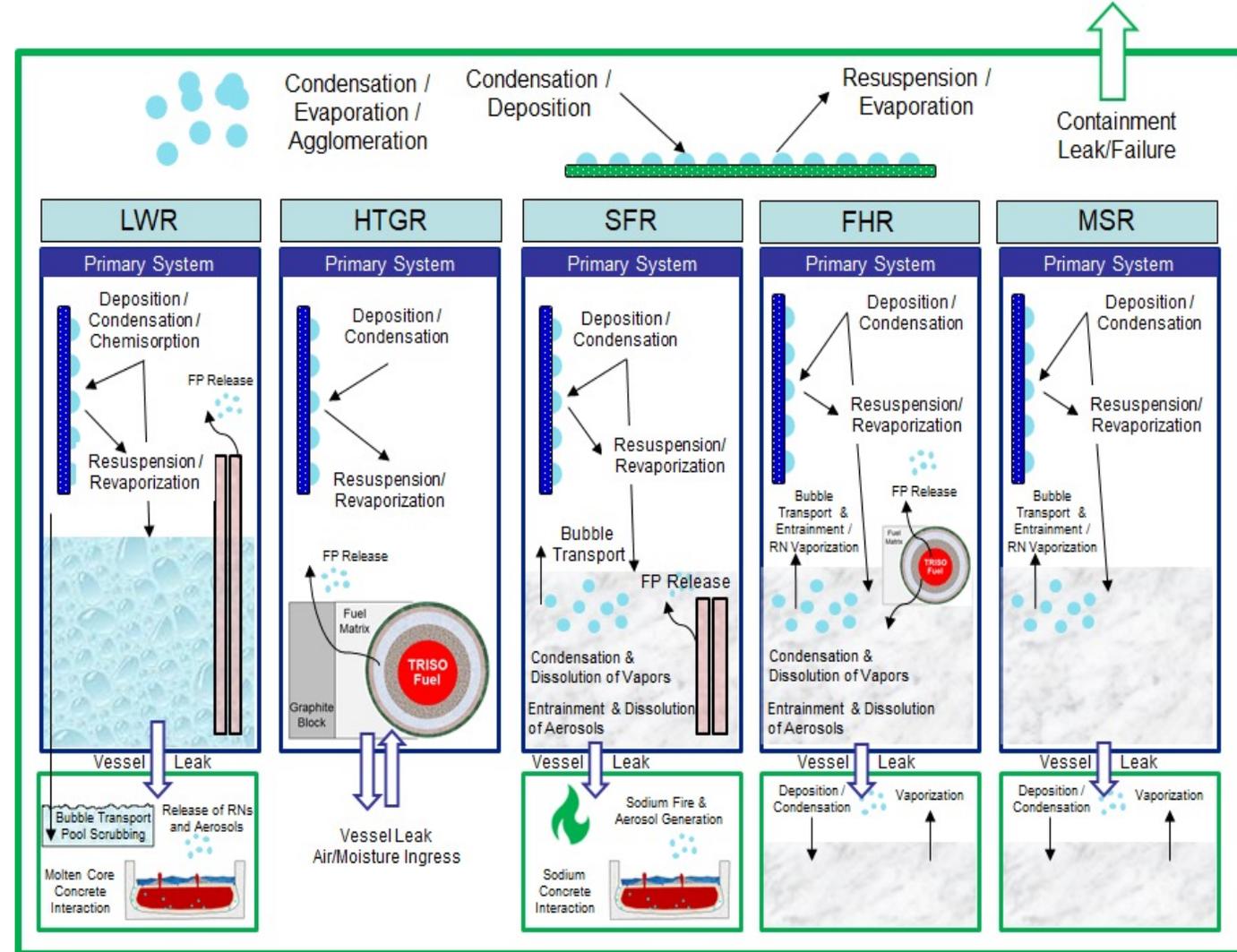


- Generalized EOS – Equations of state for multiple working fluids are presently available in MELCOR including water, sodium, and FLiBe.
 - MELEOS can generate equations of state for any fluid with the necessary data
- Thermal hydraulics – CVH/FL Packages
 - The CVH package defines control volumes (CV)
 - The FL package defines flow paths (FL)
- Heat Transfer – HS/CVH/COR Packages
 - The HS package defines heat structures (HS) that model radiative and conductive heat losses
 - The CVH package manages convective heat losses
 - The COR package controls heat losses of heat bearing and other core structures



Contain Radionuclide Release

- Radionuclide Species – DCH/RN1
 - Default 17 representative radionuclide species/elements (RN classes)
 - Species categorized by similar transport and retention mechanisms
 - Each species represents a set of elements and the corresponding collapsed isotopic masses
 - Does not currently model RN class masses at the isotopic level or mass transfer between RN classes
 - Users have the ability to redefine/add RN classes
- Radionuclide Transport and Retention
 - Transport and retention of RN classes modeled by various physiochemical processes
 - New MELCOR structure allows rapid implementation and benchmarking for models



RN Package Generalized Representation Degrees of Freedom

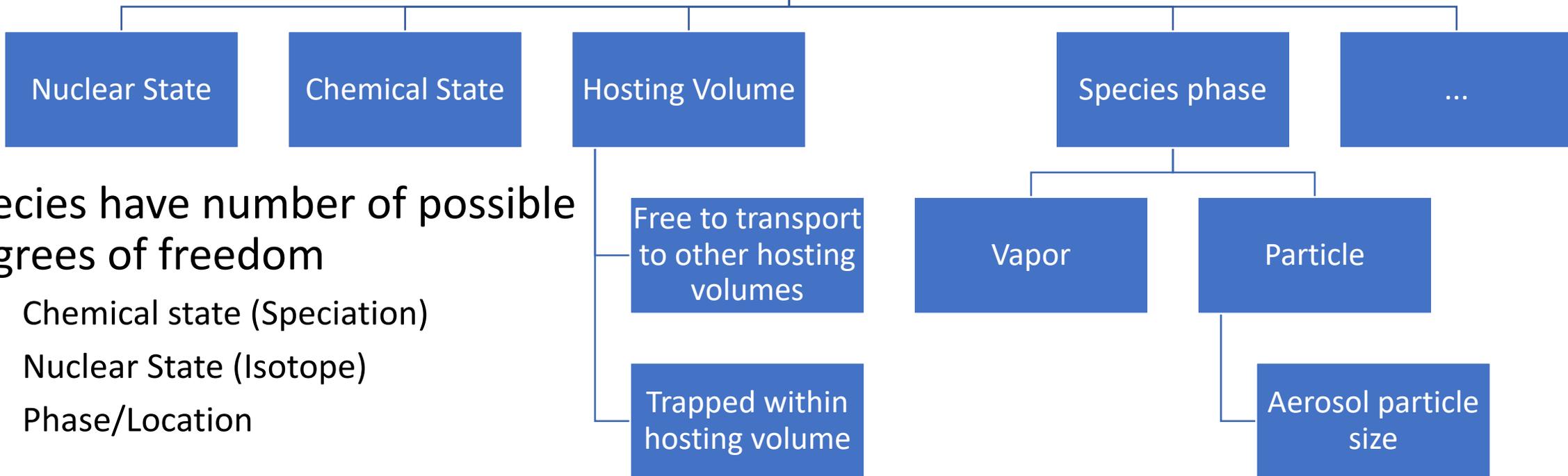


- Generalize RN package for species transport modeling
 - RN package initially intended to model radionuclide transport
 - Has evolved with MELCOR to generally characterize transport of any trace species

Generalized mathematical representation in software

- *Degrees of freedom designed to be expanded*

Hazardous Trace Species "State"

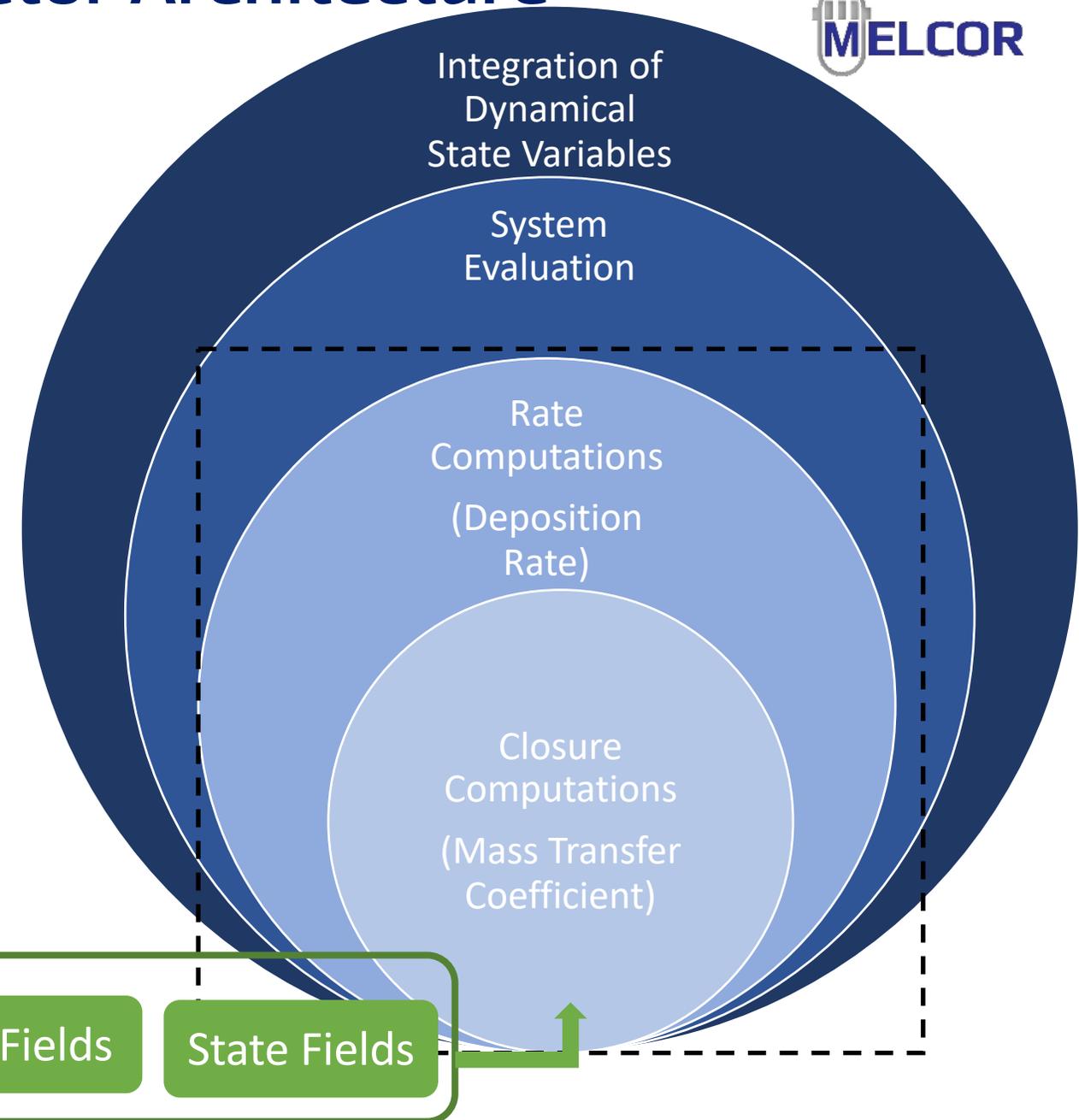


- Species have number of possible degrees of freedom
 - Chemical state (Speciation)
 - Nuclear State (Isotope)
 - Phase/Location

MELCOR's New Generalized Reactor Architecture



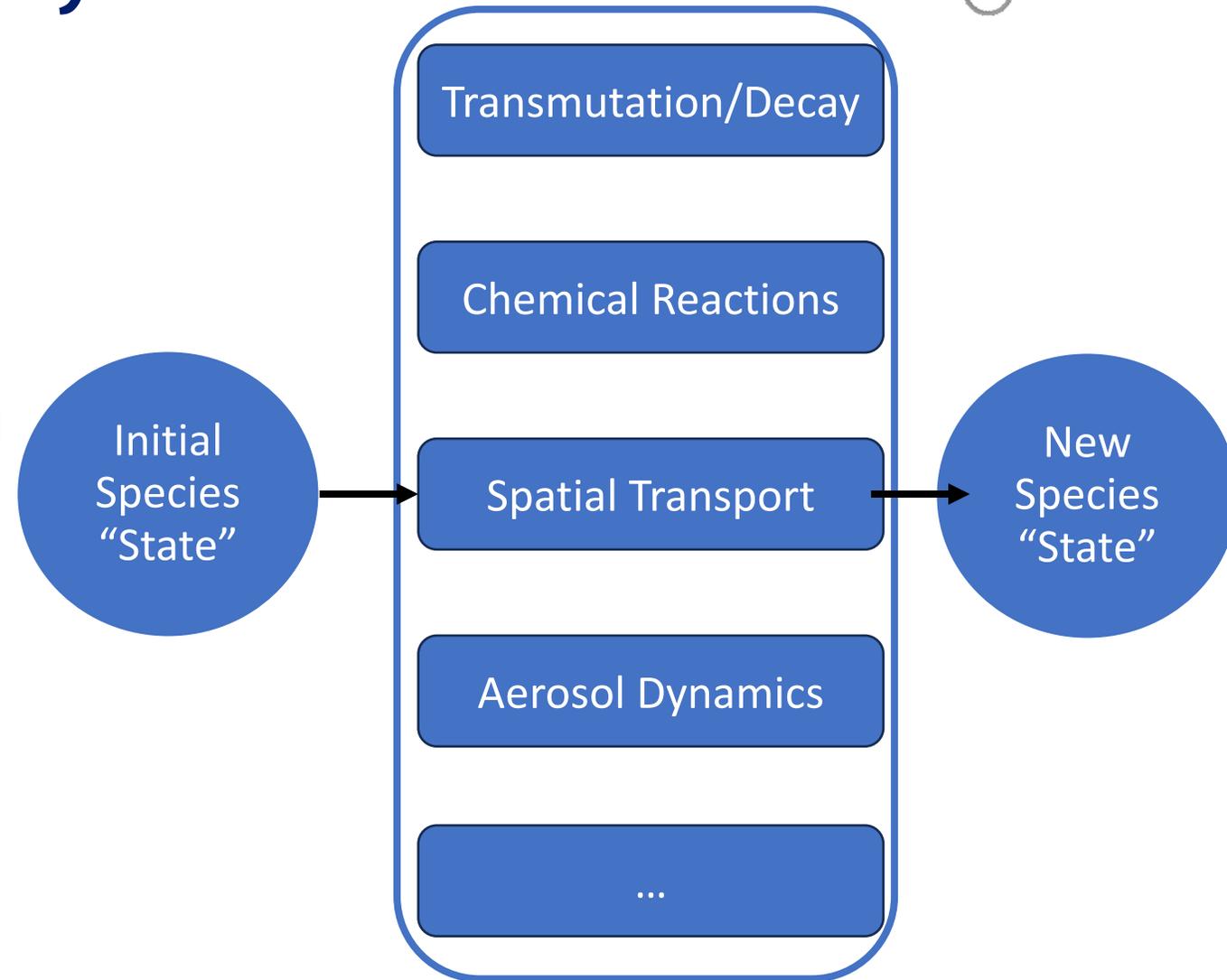
- MELCOR modernization established a new software platform
- Rapid capability implementation through physics/chemistry generalizations
- Generalized means re-using physics equations but changing system state variables
- Numerics **separated** from physics



RN Package Generalized Dynamics



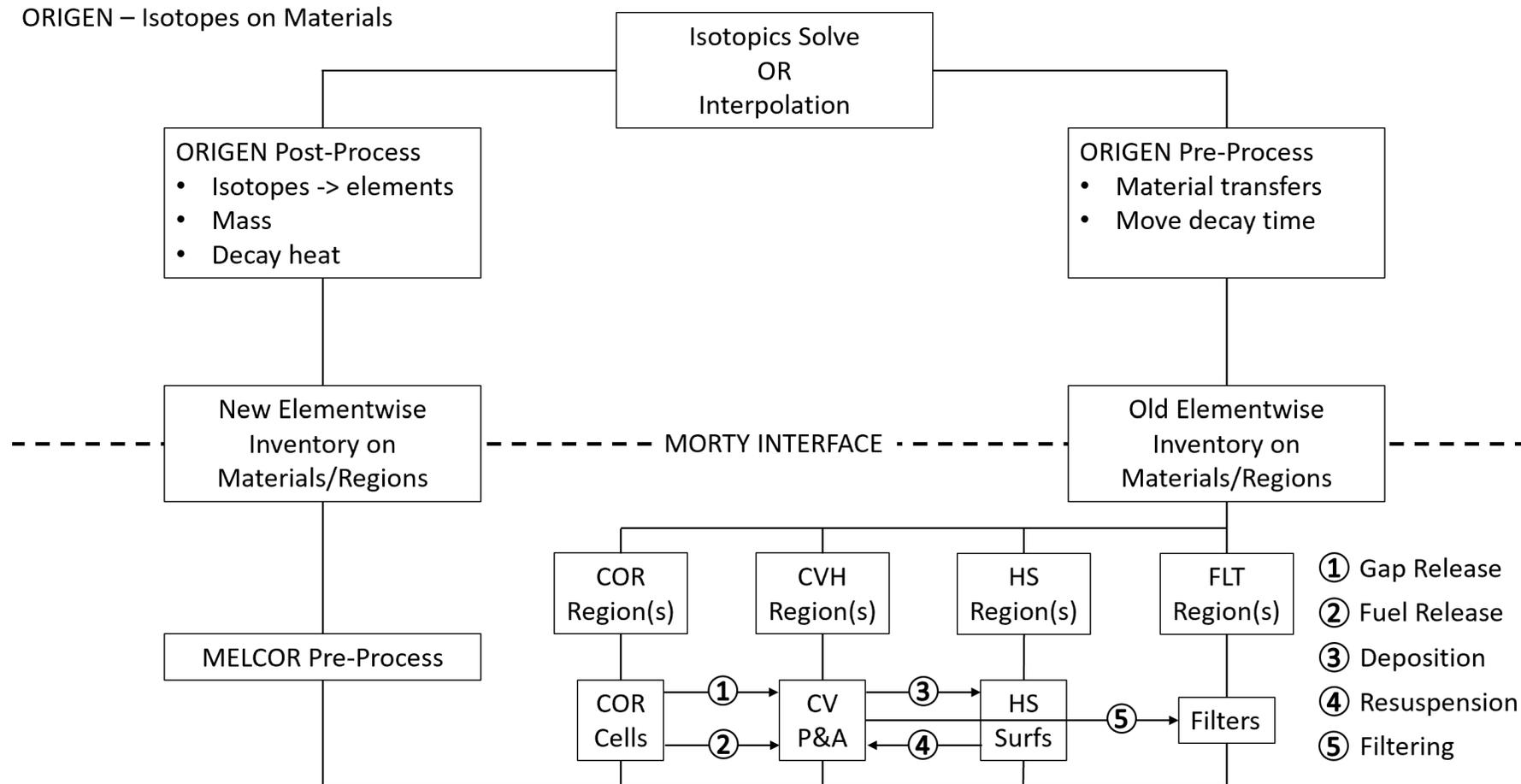
- Species transport is formulated as mass fluxes
- **Interfacial Transport:** Transport of species between “volumes”
- **Aerosol Dynamics:** Transformation of species within volumes associated with vapor and aerosol dynamics
- **Nuclear Processing:** Transformation of species within hosting volumes due to nuclear transmutation and decay
- **Chemical Processes:** Transformation of species within hosting volumes due to chemical reactions



ORIGEN-MELCOR Integration for Radionuclide Decay



ORIGEN provides isotopic analysis and computation of radionuclide transmutation, fission, and decay



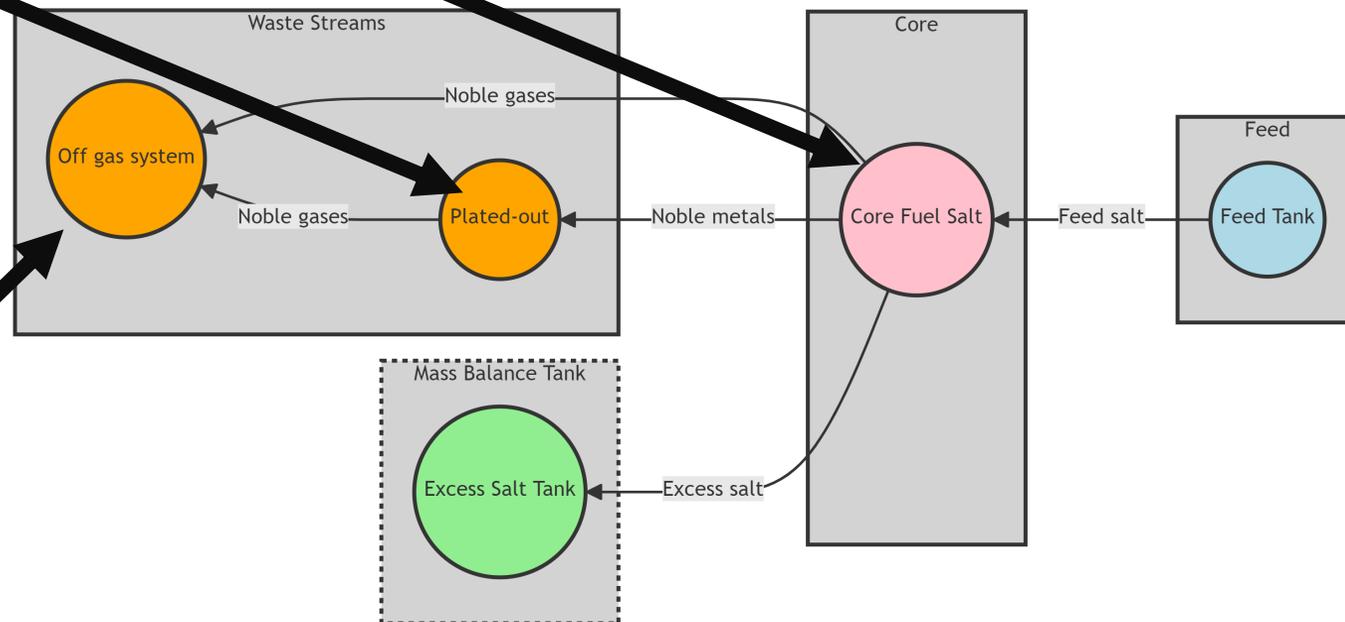
MELCOR – Elements/Classes on Regions

ORIGEN-MELCOR Integration Updates Species Concentrations



- Primary system radionuclides circulate through the core and exposed to neutron fission environment until shutdown
- Corrosion products and noble metals deposit and decay with transmutation to other elements and possibly change in solubility or phase
- Off-gas hold-up systems designed to decay Xe & Kr but also include transmutation to other elements and possibly change in phase (i.e., gas → aerosol)

ORIGEN/MELCOR integration allows independent tracking, decay, and transmutation of nuclides in multiple locations and their impact on radionuclide transport and the source term



Radionuclide form (aerosol or gas) and nuclide decay impact the source term and health consequences

[ORNL/SCALE MSBR drawing]

MELCOR's MSR Chemistry SCOPE

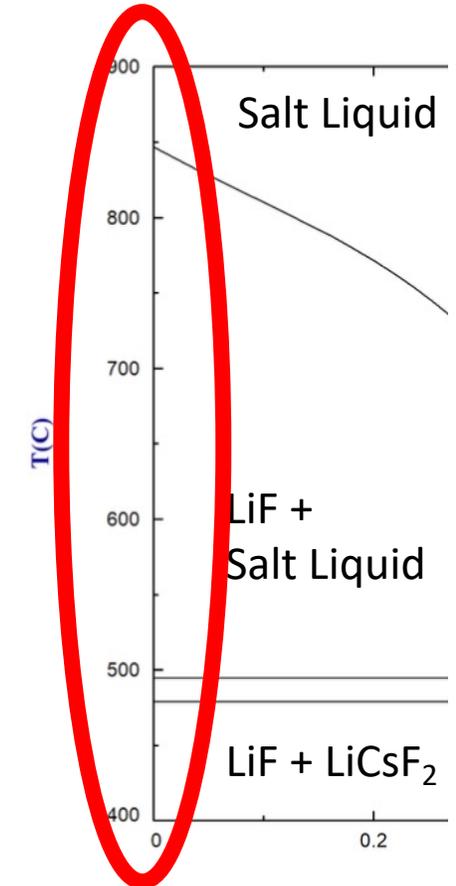


- MELCOR models chemistry in COR, EOS (fluid) and in RN packages
 - “Like” species are grouped to reduce computational time (e.g. Rb and Cs)
 - Chemical reactions considered if equilibrium constant is $O(10^0-10^5s)$
 - Molten Salt Thermodynamic Database helps to model reactor's chemical speciation and create fluid files
 - Can define a chemical reaction model in RN if not in MSTDB
 - Models events:
 - Halogen potential control
 - Environmental contamination
 - Reactor Refueling
 - Salt Spill
-

Assume Radionuclide Masses Minimally Affect Bulk Salt Thermochemical Properties



- Carrier and fuel salt mass is thousands of kgs while radionuclides (RNs) will be a few kgs
- RNs will quickly form products related to phase speciation, having little affect on bulk properties
- RN species will be driven largely by temperature, not mass changes
- RNs in salt can be treated in system with classical mass-transport equations (frozen chemistry)

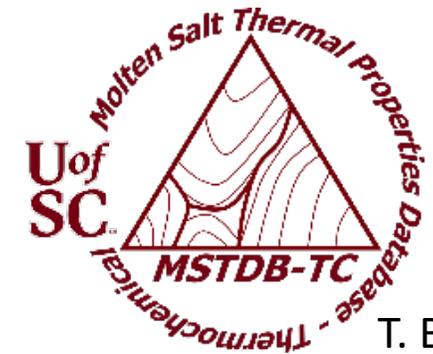


1% CsF mol frac with LiF

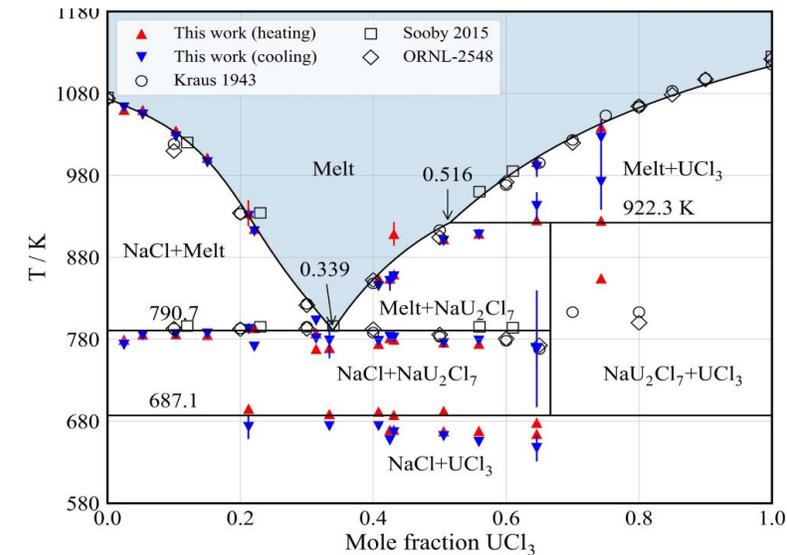
Chemical Databases and Gibbs Energy Solvers can model Molten Salt Properties



- The Molten Salt Thermochemical Database (MSTDB-TC) contains fitted Gibbs energy functions for salt systems
- Thermochemica uses databases, like MSTDB-TC, to model thermochemical properties (equilibrium speciation, melting point, speciation...)
- Thermochemical properties of multiple molten salt compositions can be modeled (limited by database systems)
- Common fluoride and chloride carrier, fuel and fission product systems available; iodine increasing



T. Besmann (USC)



Yingling et al., J Chem. Thermo 2023, 179, 105974



Thermochemica

M. Piro (McMaster)

MSTDB-TC and Thermochemica can be used to probe MSR chemistry

Thermochemica + MSTDB-TC Allow Investigation of how Reactor Chemistry affects Mass Transfer

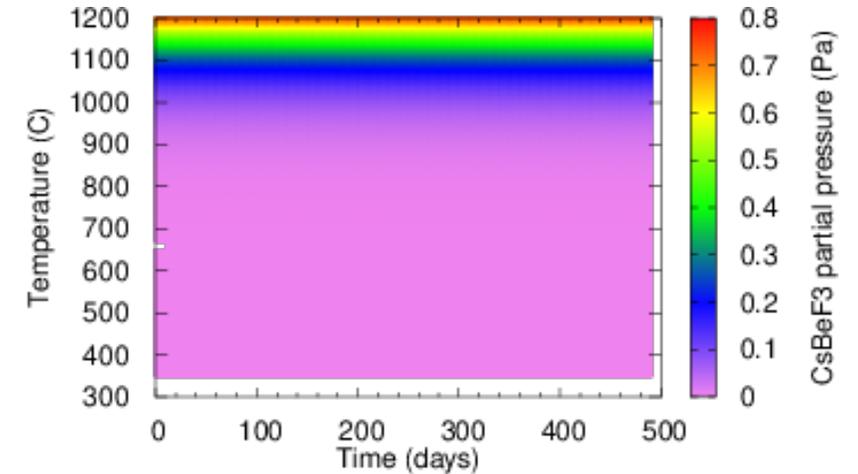


- MSR related topics that can be investigated are:
 - Fission product speciation for different reactor operations
 - Fuel cycle operation design
 - Corrosion control
 - **Parameterize mass transport coefficients in MELCOR**

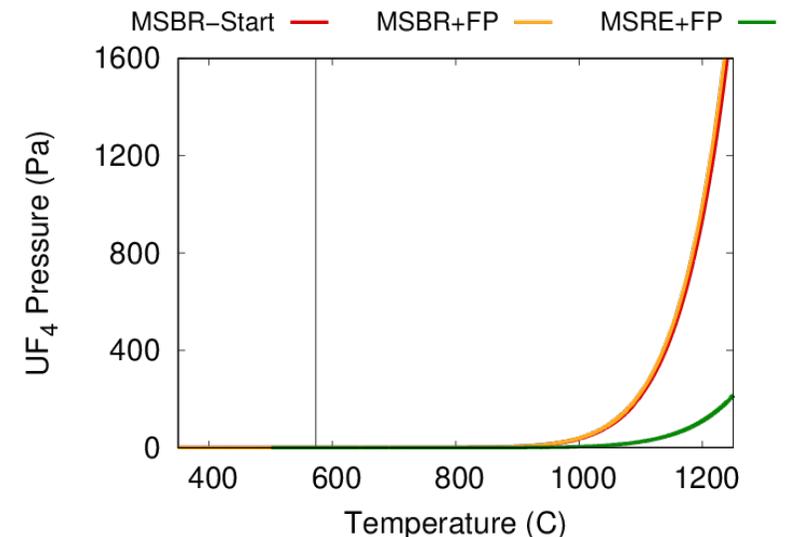
- Note: MSTDB-TC contains finite salt systems (e.g., Pb, Ti, S); use wisely

- New classes in RN can be created for off gas modeling from Thermochemica results

Use Thermochemica + MSTDB-TC to fit parameters, such as Antoine coefficients, RN transport

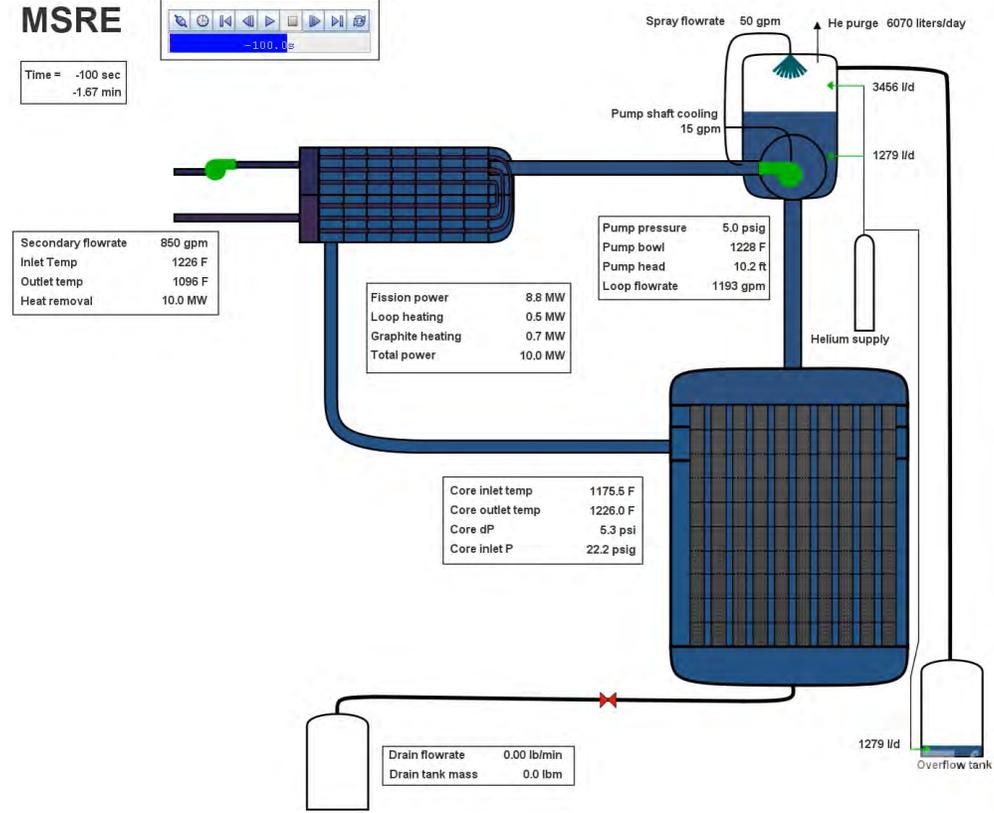


CsBeF₃ Pressure in MSRE 500 Days After Shutdown

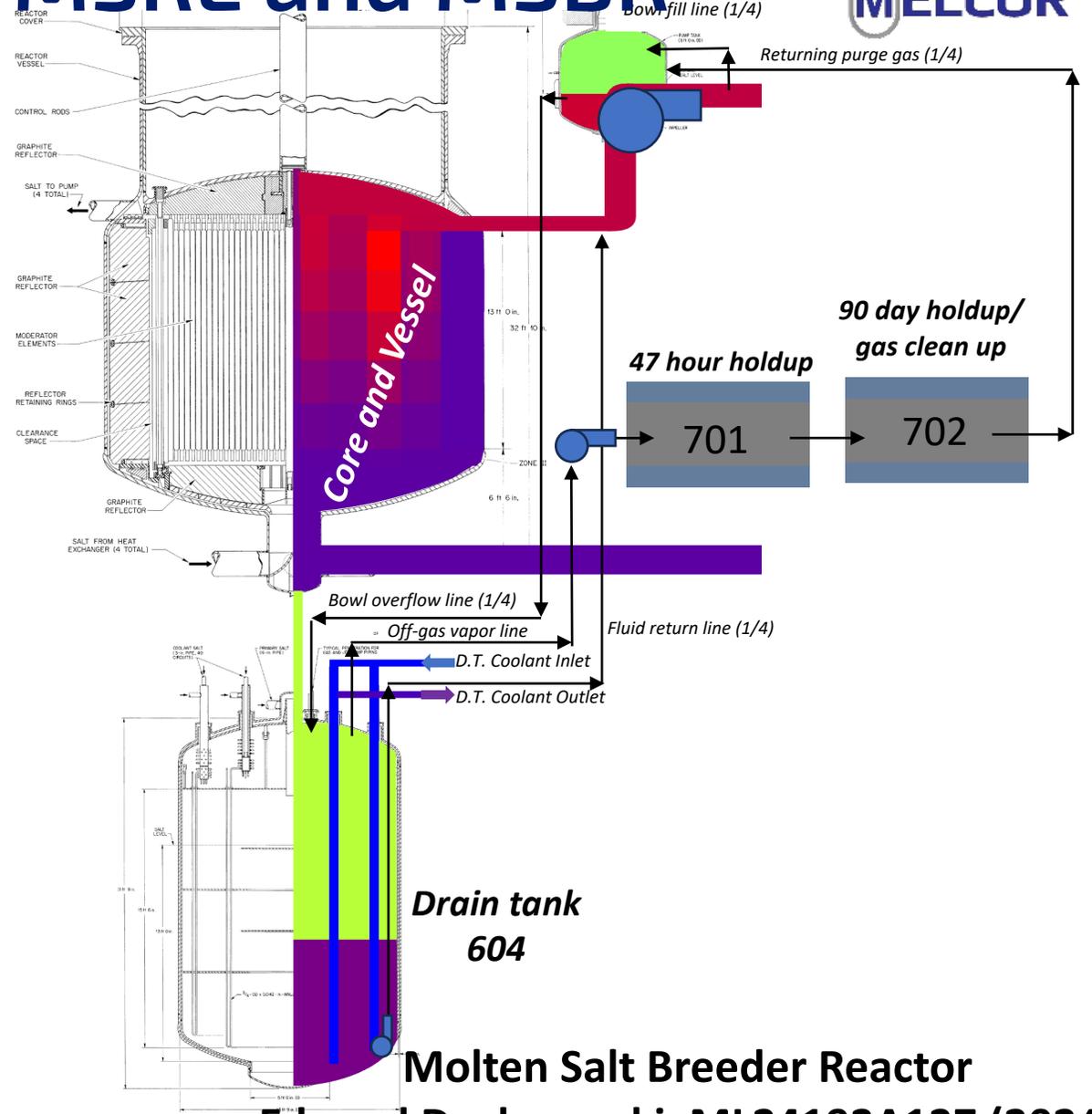


UF₄ Pressure in Different Systems

MELCOR Demonstrations of MSRE and MSBR



Molten Salt Reactor Experiment
Casey Wagner, SAND2022-12146PE (2022)

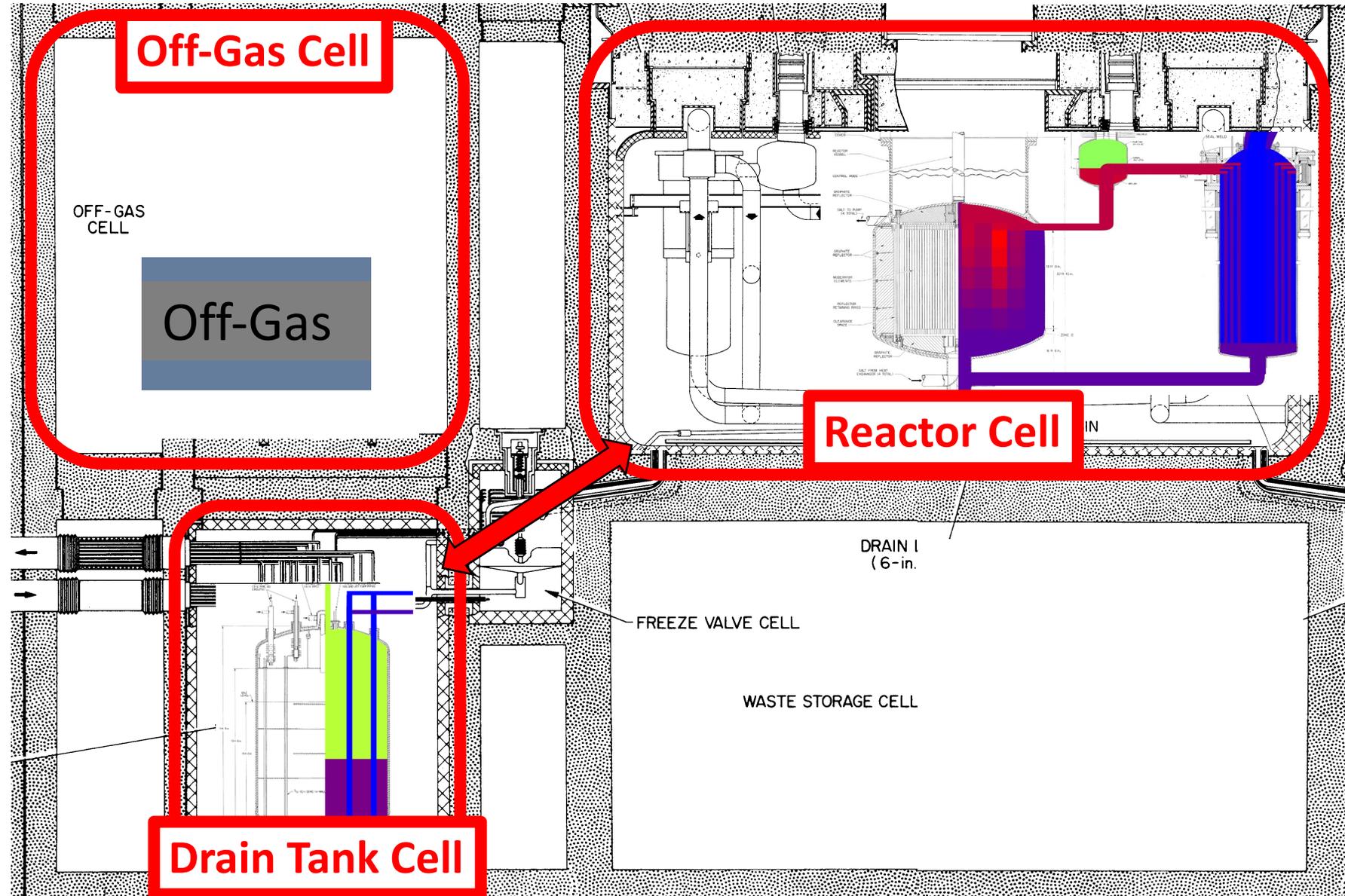


Molten Salt Breeder Reactor
Edward Duchnowski, ML24192A127 (2024)

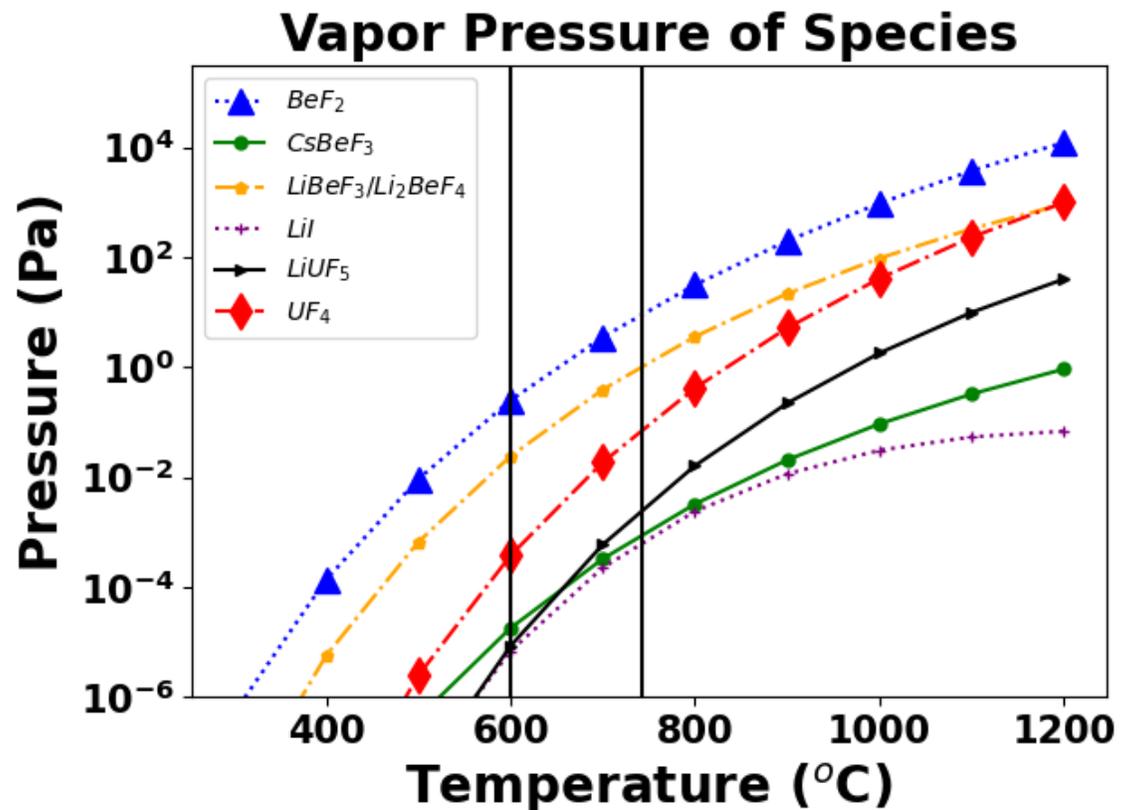
MELCOR MSBR Nodalization



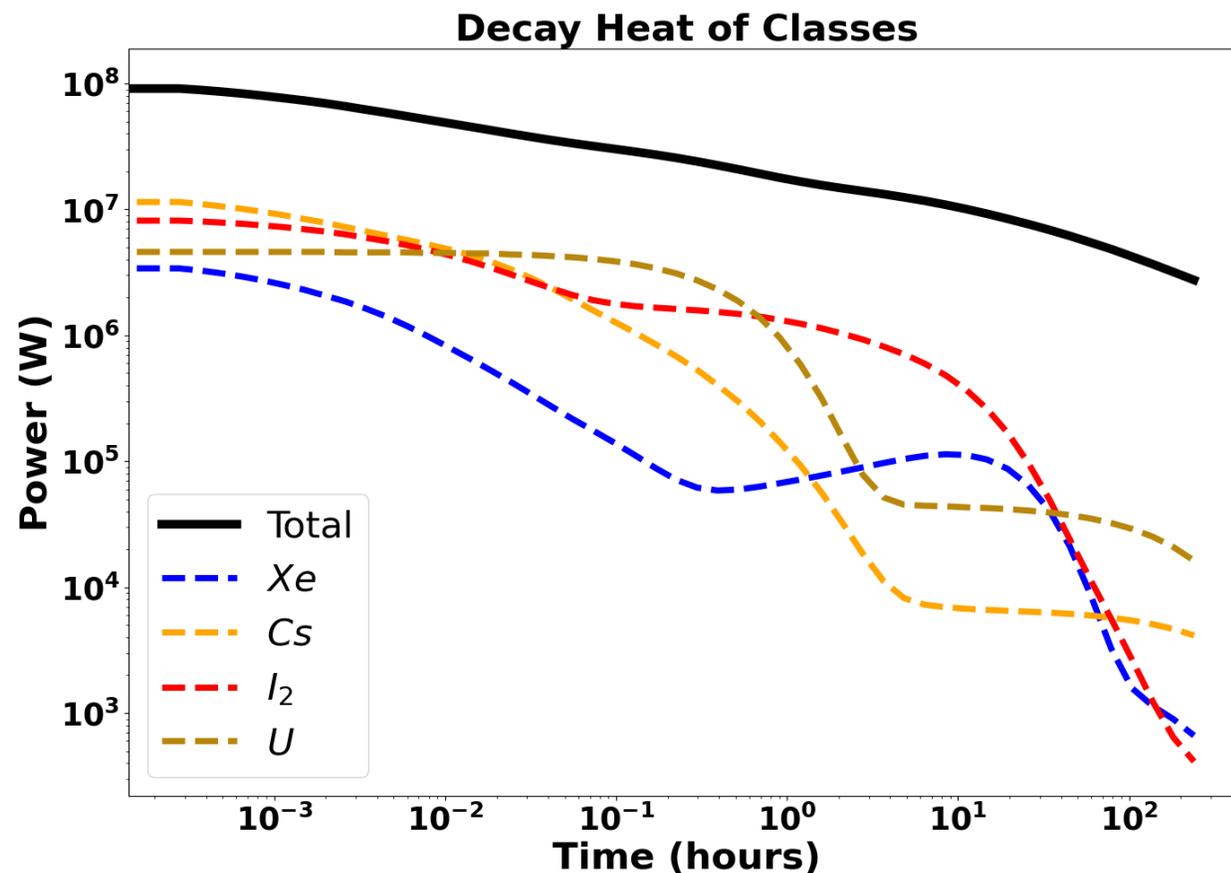
- Off-gas, drain tank, and primary loop in respective containment
- Drain tank cell and reactor cell connected through freeze valve cell
 - Essentially maintain equivalent environments during steady state operation



SCALE and Thermochemica Results Used for MELCOR



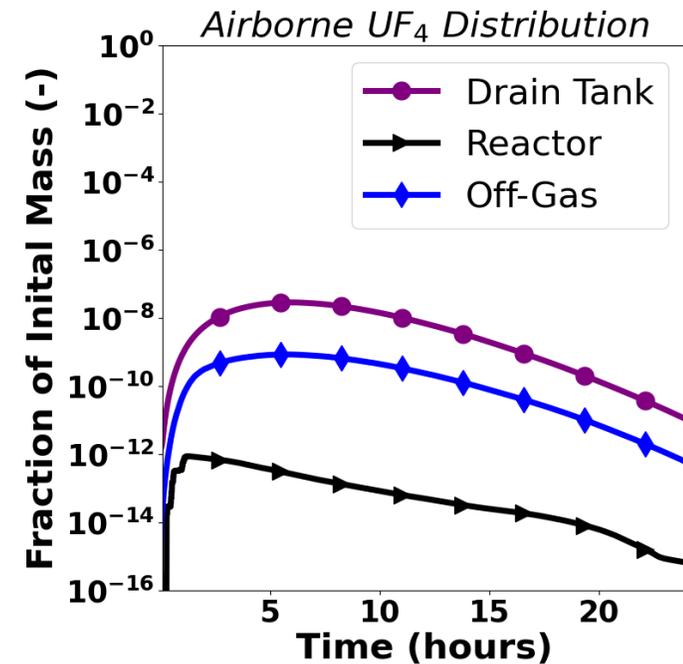
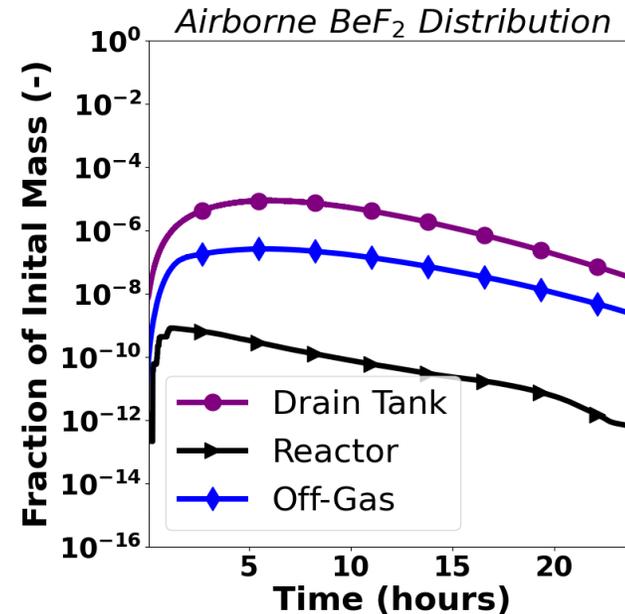
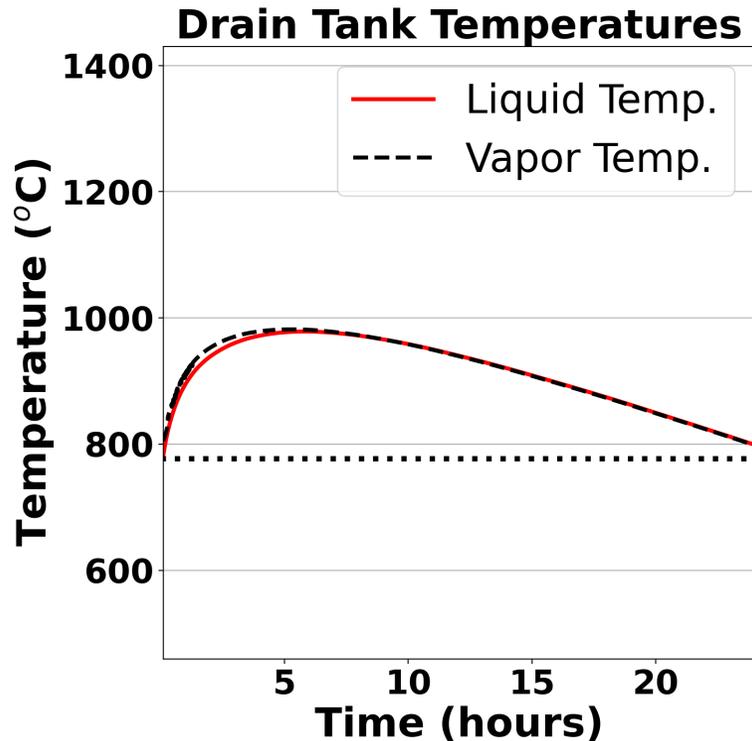
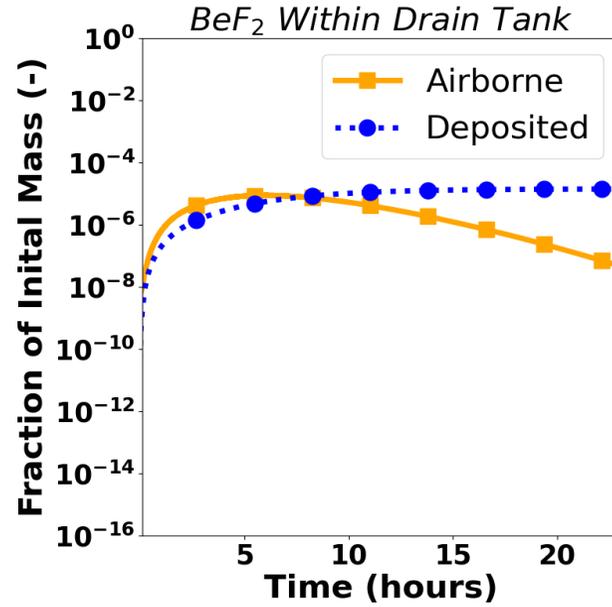
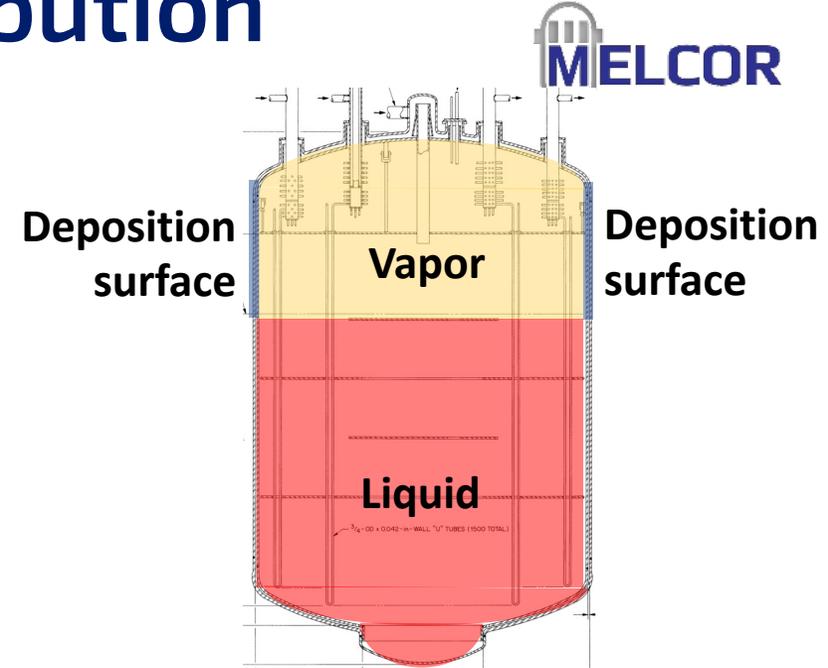
Thermochemica calculated vapor pressures using SCALE calculated RN inventories



Element decay heat from SCALE calculations

Emergency Drain: BeF₂ and UF₄ Distribution

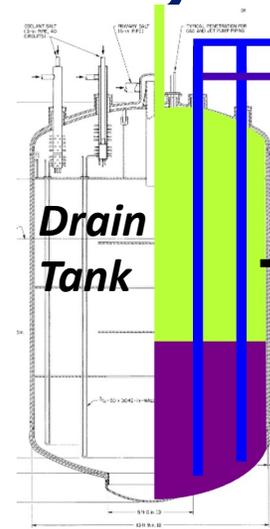
- Small amounts of BeF₂ and UF₄ continuously released
- Release rates vary as function of fuel salt temperature
- Airborne species condense back into pool and deposit on cool drain tank wall



Emergency Drain: Radioactive Decay In Off-Gas

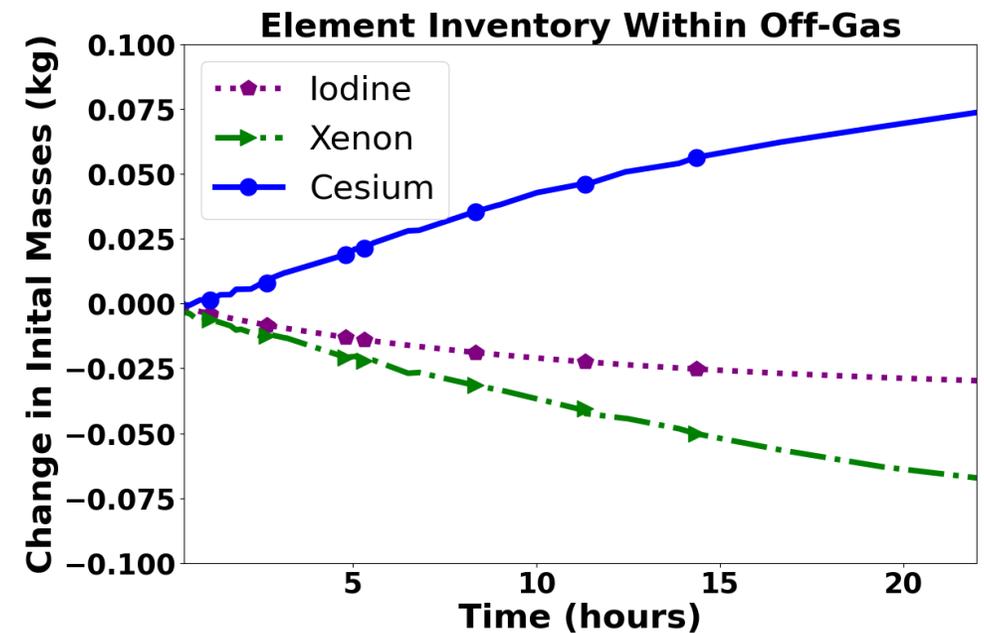
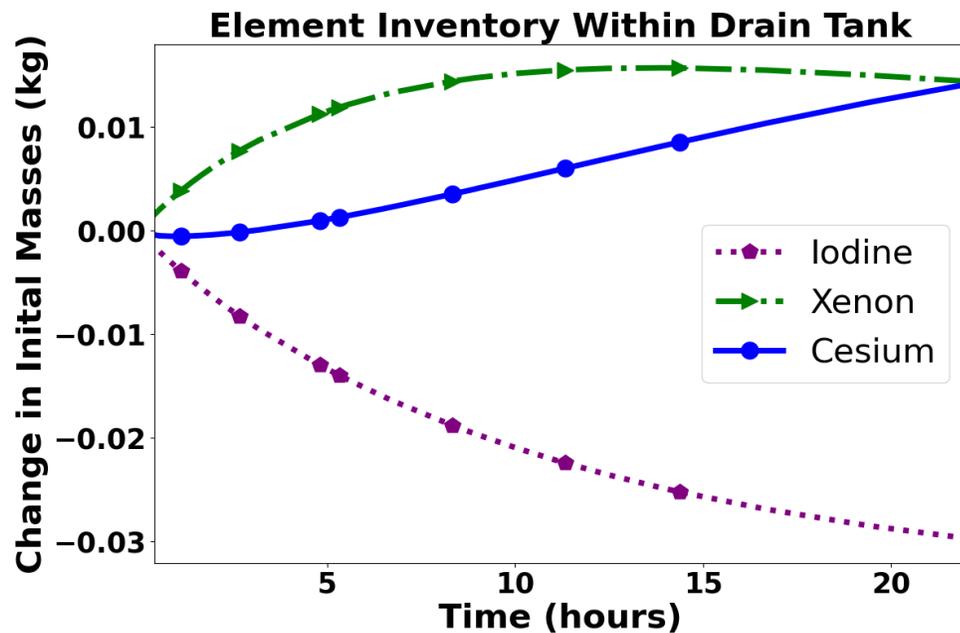


- Noble gases lumped into 'Xe' class
 - Common methodology to describe gaseous fission products
- Coupling of SCALE/ORIGEN with MELCOR allows for investigation of specific elements
 - Can **track the transport, production, and decay** of isotopes such as xenon-135 within the 'Xe' class



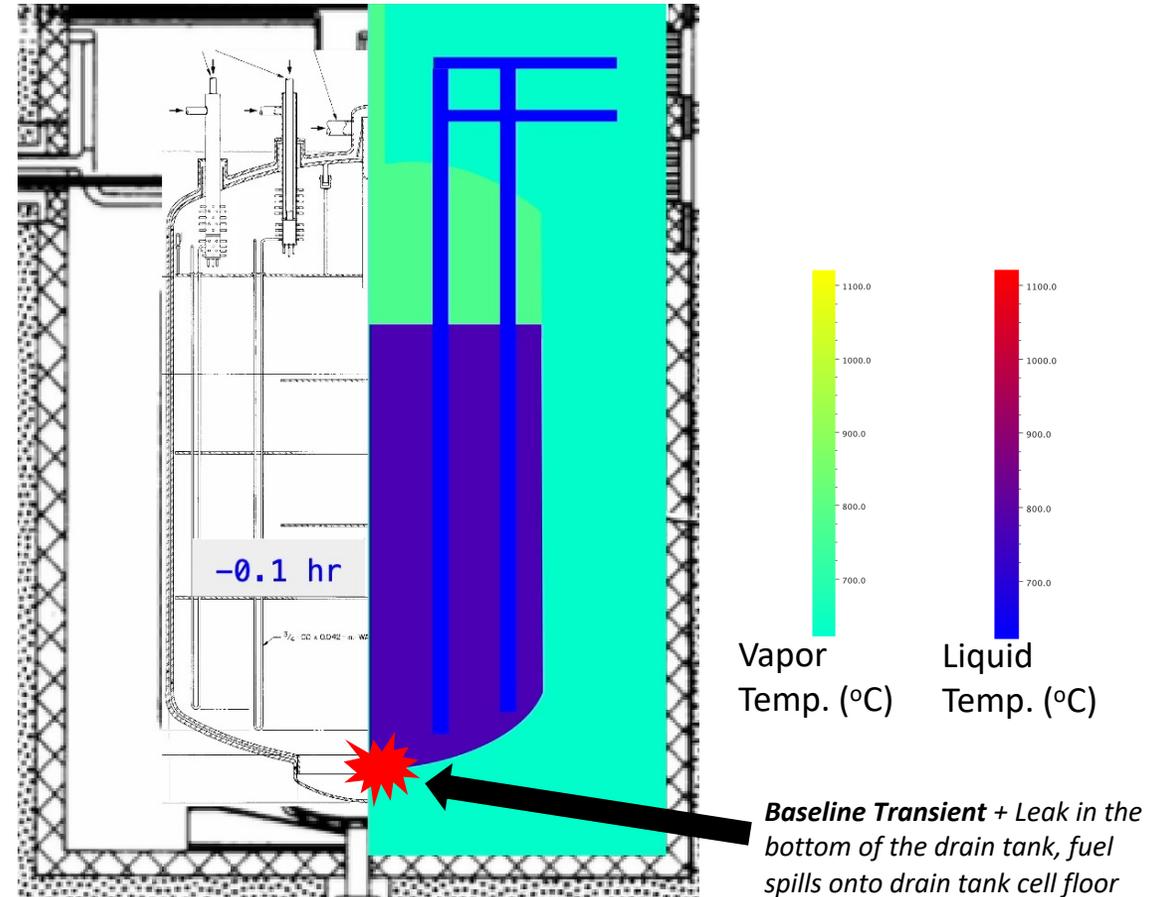
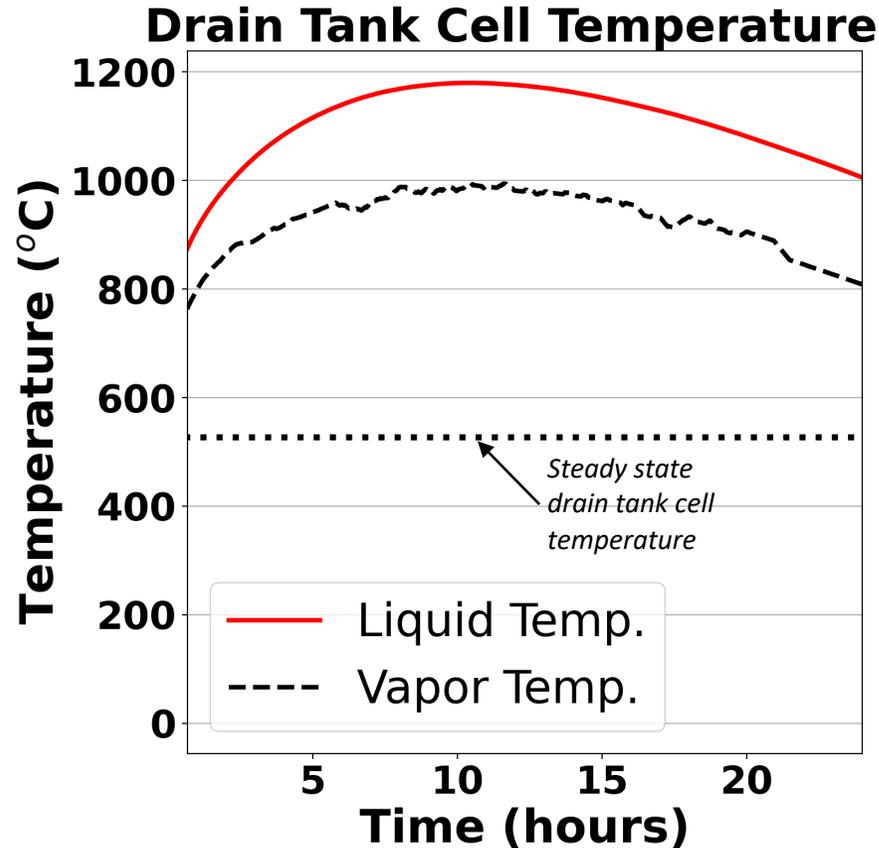
SCALE inputs of primary loop isotopic inventory

SCALE inputs of off-gas isotopic inventory



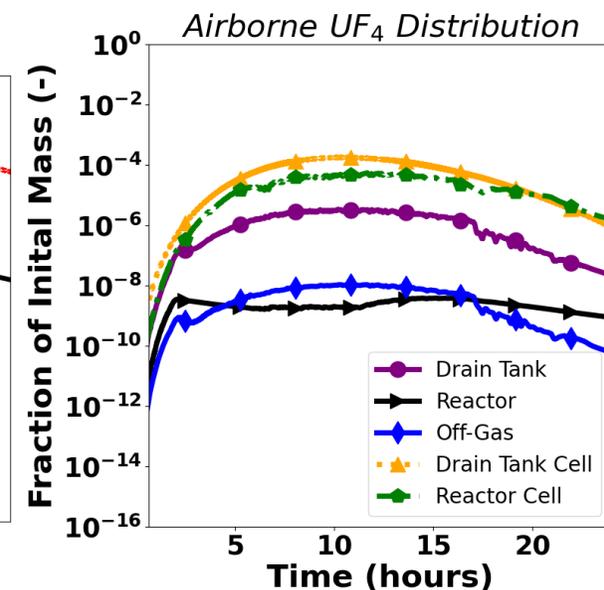
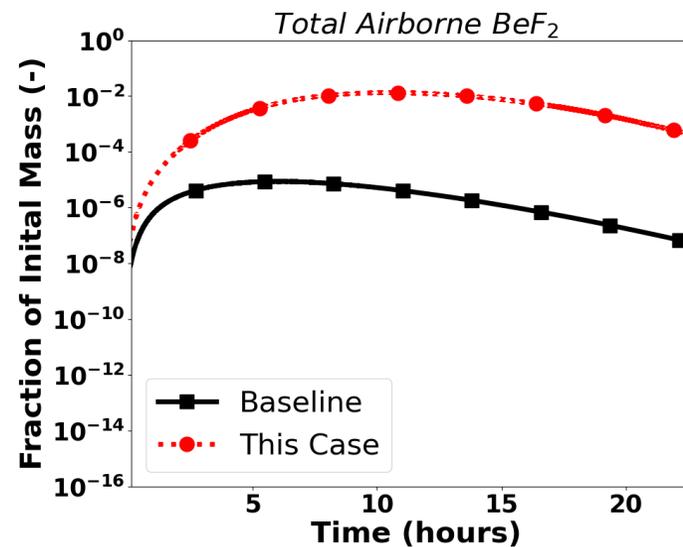
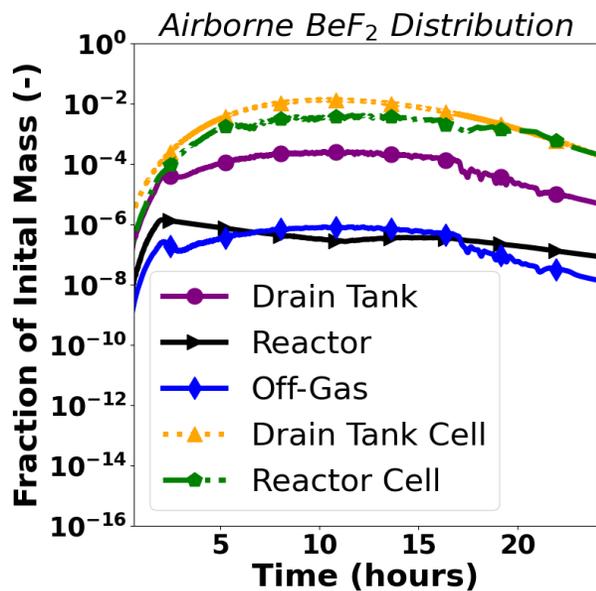
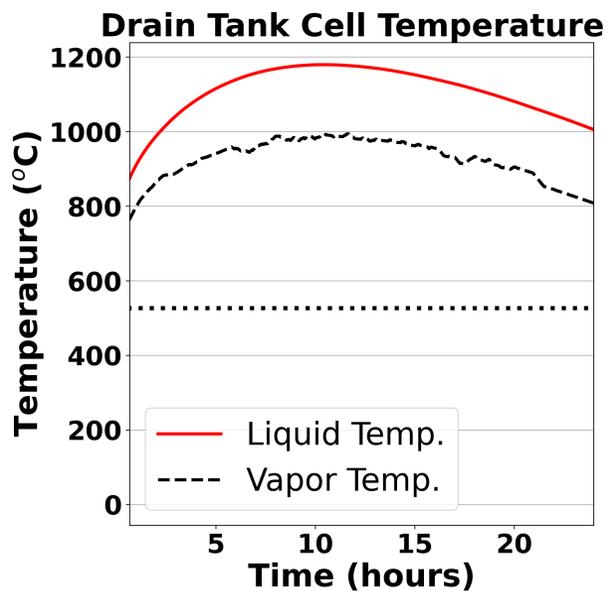
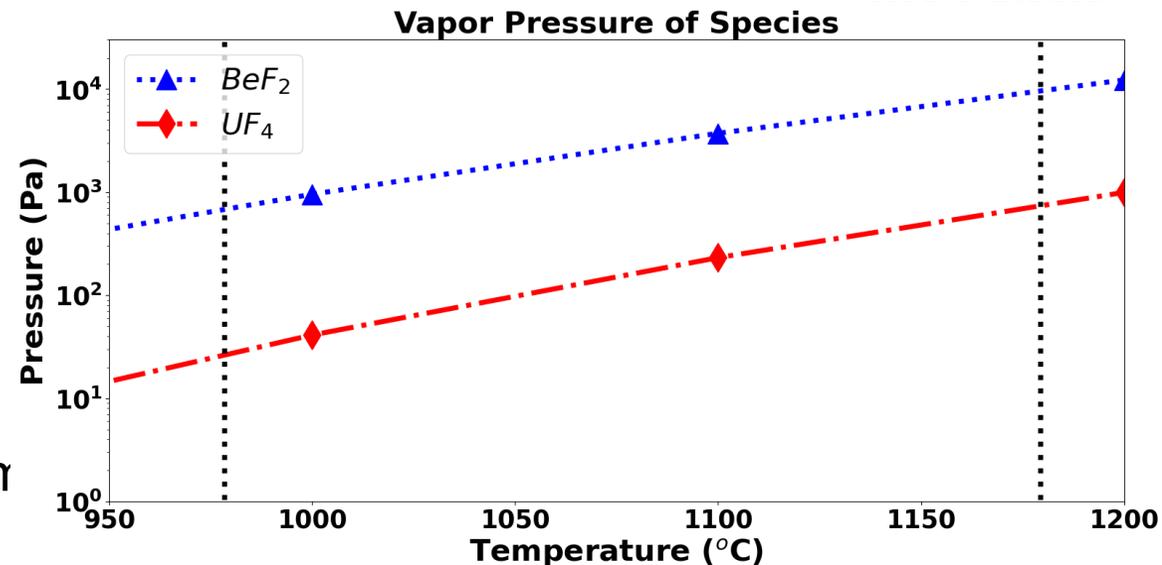
Drain Tank Leak Accident

- Leak in bottom of drain tank
 - Allows for all liquid to drain into drain tank cell
 - Liquid no longer cooled by drain tank heat removal system
 - Passive heat removal from drain tank cell floor and walls



Drain Tank Leak Accident

- Higher liquid temperatures due to insufficient cooling, around an order of magnitude higher vapor pressure
- Initially lower atmosphere pressure in drain tank cell than drain tank
- Much larger fraction of airborne BeF_2 and UF_4 than in baseline case**
 - Significant fraction of airborne species escape from drain tank cell to reactor cell



MELCOR Chemistry Road Map



- **Particle Deposition**
 - Solids (Noble metals, corrosion products, oxides)
 - Aerosols
 - EOS thermophysical property sensitivity studies
 - Viscosity
 - Density
 - Heat Capacity
 - Bubble formation and dynamics (Daniel Orea, Oak Ridge National Lab)
 - Bubble size/shape
 - Diffusion through liquids and solids, such as Xe into graphite
 - Bursting transfer from liquid to vapor phases
 - MSR/Super-Critical CO₂ Heat exchanger failure
-

Summary



- MELCOR MSR modeling centers around “Control, Cool and Contain” ethos
 - New Generalized MELCOR architecture allows for rapid implementation and evaluation for physical models
 - SCALE integration into MELCOR allows for evaluation of radionuclide decay during system analysis
 - Chemistry is informed through SCALE radionuclide calculations and MSTDB-TC calculations
 - Demonstrations have been carried out for MSRE and MSBR designs
 - MELCOR is being continuously developed to add important chemistry/physics models and to reduce uncertainties in MSR modeling.
-

Assume Radionuclide Masses Minimally Affect Bulk Salt Thermochemical Properties



- Sandia MELCOR Team



- Ted Besmann/Juliano Schrone-Pinto and tea



- Joanna McFarlane, Daniel Orea, Kevin Robb... (ORNL)



- Patricia Paviet (PNNL)



- YOUR NAME HERE

mschris@sandia.gov

Sandia National Laboratories is a multimission laboratory managed and operated by National Technology & Engineering Solutions of Sandia, LLC, a wholly owned subsidiary of Honeywell International Inc., for the U.S. Department of Energy's National Nuclear Security Administration under contract DE-NA0003525. Any subjective views or opinions that might be expressed in the paper do not necessarily represent the views of the U.S. Department of Energy or the United States Government.