Overview of IAEA Activities on Nuclear Fuel Cycle Options

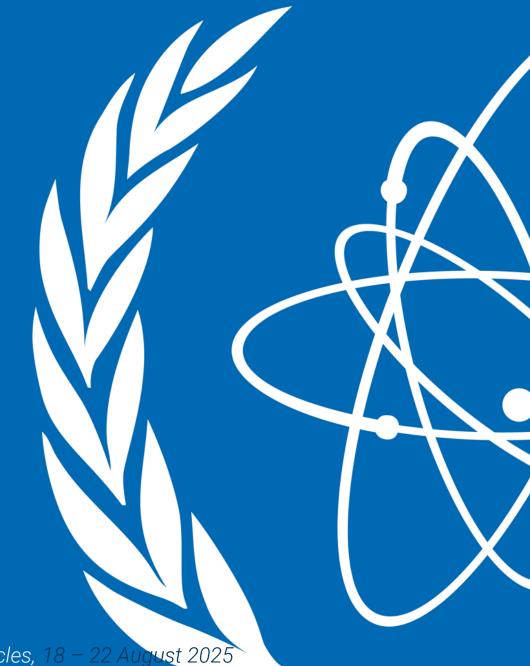
Ms Amparo González-Espartero, PhD

Technical Lead - Spent Fuel Management

Nuclear Fuel Cycle and Materials Section

Department of Nuclear Energy, IAEA

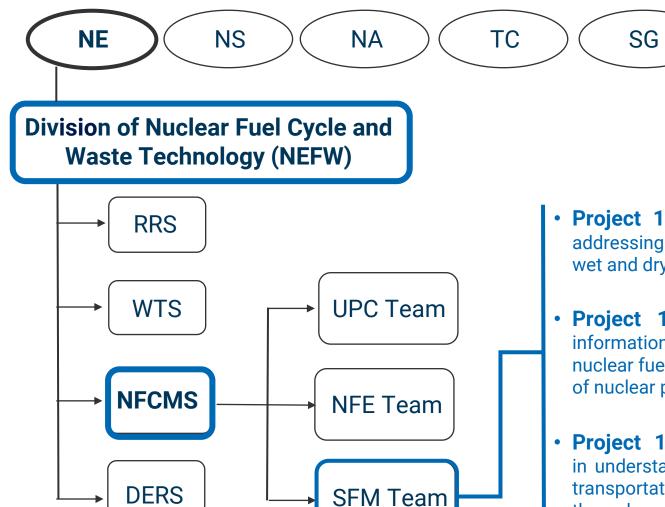
a.g.espartero@iaea.org



TM on Proliferation Resistant Features of Fast Reactors and Advanced Fuel Cycles, 18 – 22 August 2025

IAEA's Organization

Department of Nuclear Energy



Nuclear Fuel Cycle and Materials Section (NFCMS)



• **Project 1.2.3.001 Spent fuel storage:** To support MSs in understanding and addressing the challenges of an effective and safe storage of their SNF (through wet and dry technologies), including anticipating those generated by SMRs

MT

- **Project 1.2.3.002 Spent fuel recycling:** To facilitate discussion and information sharing among interested MSs on recent and future developments in nuclear fuel recycling processes and technologies for current and next generations of nuclear power reactors (e.g., LWRs, FRs, SMRs)
- Project 1.2.3.003 Radioactive materials transportation: To support MSs
 in understanding and addressing the challenges and issues related to the safe
 transportation of all kinds of radioactive and nuclear materials used or generated
 through nuclear fuel cycle activities, including SNF from current fleet of LWRs,
 Advanced Reactors and all different SMRs technologies

NFCMS mandate

To support sustainable, safe, secure, reliable and economic Nuclear Fuel Cycles associated with current and future generations of nuclear power reactors, by providing Member States with relevant technical information (guidance) based on operational experience and best practices

• Through:

- Organization of International conferences and workshops
- Publication of technical documents and reports
- Coordination of international research activities through Coordinated Research Projects (CRPs)
- Management of specific databases
- Issuance of e-Learnings









NFCMS: who we are and what we do



Technical Working Group on Nuclear Fuel IAEA Cycle Options and Spent Fuel Management (TWG-NFCO)

20 Member States (Belgium, Canada, China, Finland, France, Hungary, India, Japan, RoK, Mexico, Netherlands, Romania, Russia, Spain, Sweden, Slovakia, UK, UAE, Ukraine, USA).

Three International Organizations (EC, OECD/NEA and WNA)

TWG-NFCO focuses on nuclear fuel cycle options with an emphasis on:

- Spent fuel management (storage, recycling and transportation)
- Innovative fuel cycles (multirecycling, minor actinides management and P&T of long-lived fission products)
- Nuclear materials management

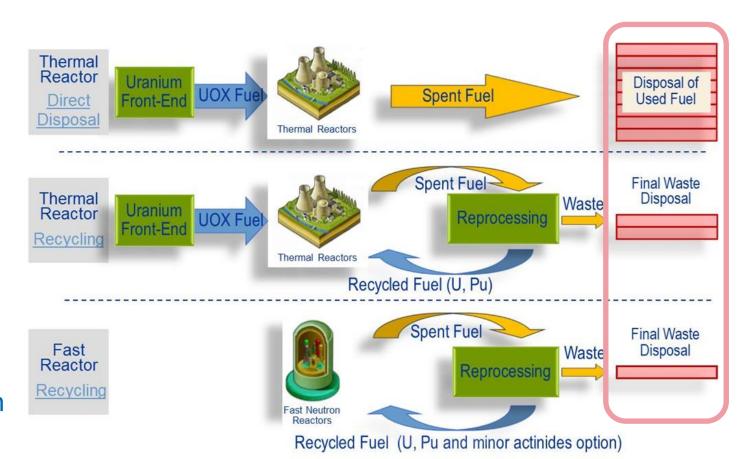
TWG-NFCO Meeting, 1-3 April 2025





Nuclear Fuel Cycle Options

- For Nuclear power to be sustainable, the nuclear fuel cycle must remain economically viable and competitive through the optimization of the use of fissile materials in reactor cores or the recycling of valuable materials
- This results in different fuel cycle
 options, some already implemented and
 others may be deployed in the future
- Potential future synergies between LWR-SMRs and AMRs will bring new spectrum of Nuclear Fuel Cycle Options

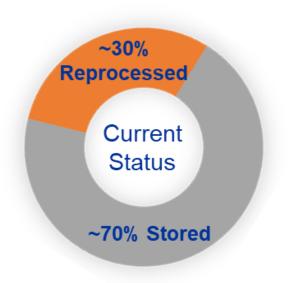


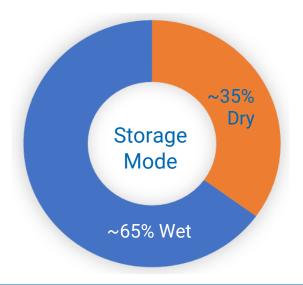
Each Type of Reactor has an Associated Nuclear Fuel Cycle

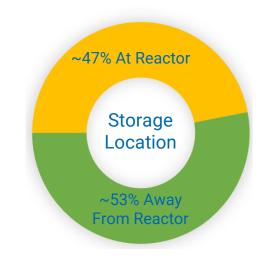
Spent Fuel Management: Status and Trends

~ 416 NPPs* in 31 countries discharging ~ 10 000 tHM per year

Global Inventory by the end 2024, ~ 430 000 tHM













(* August 2025, see PRIS database: https://pris.iaea.org/pris/)

- Global Inventory of spent fuel at the end 2023: ~ 430 ktHM, among which ~ 301 ktHM are in Storage
 - 47% at Reactor, 53% Away from Reactor (33% Wet Storage / 67% Dry Storage)
- Selection of storage technology depends on many factors: Fuel, Economics, Stakeholders' preference

Spent Fuel Management Strategies Worldwide

Today mainly countries with large nuclear power programmes recycle spent fuel: France, the Russian Federation, Japan, India and China.



Some countries have not yet made a final decision.

Most spent fuel is in interim storage.





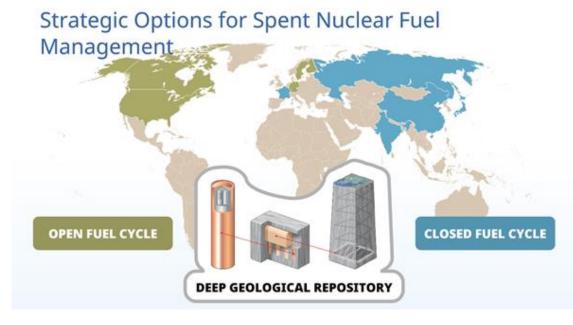
Several other countries have opted for direct disposal: Finland, Sweden, Canada, Germany



The Netherlands reprocess SNF from Borssele NPP abroad and stores High Level Waste at HABOG facility

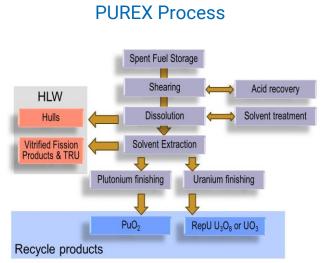






Spent Fuel Recycling through Reprocessing

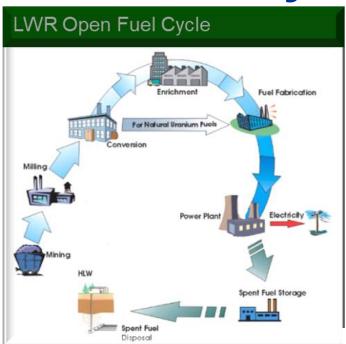
- Reuse of Reprocessed Pu as MOX in Light Water Reactors
 - More than 40 years of experience worldwide (44 LWRs have used MOX fuel at industrial scale since 1986)
 - Loading cores partially with MOX (25-50%) and the remainder with UOX fuel
 - Recent reactor designs can accommodate 100% MOX cores
 - Demonstrating Multiple Recycling in LWRs contributes to transitioning to Fast Reactors
- Reuse of **Rep-Pu as MOX in Fast Reactors**: implemented in Russia in BN-800
- Reuse of Rep-U as Enriched Reprocessed U (ERU) in Thermal Reactors (PWRs, VVERs, RBMKs, AGRs, PHWRs)
 - More than 30 years of experience worldwide (TRs can accommodate 100% Rep-U cores)



- Recycling Spent Fuel is a mature technology (Pu recycling in LWRs saves up to 25% of natural uranium resources)
- Reference options exist worldwide (PUREX)
- Reprocessing capacities exist in France, Russia, India, Japan and China

Towards Enhanced Nuclear Energy Sustainability through Fast Reactor Systems





Environment-friendly innovative fuel cycles:

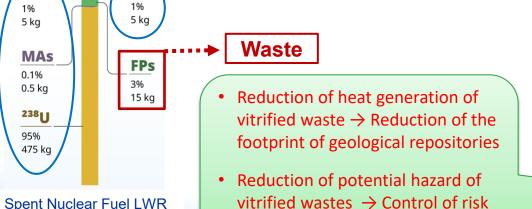
- Natural resources preservation
- Minimizing the burden of waste to be disposed of (reducing footprint of final repository)
- Economically viable fuel cycles through recycling valuable materials
- Enhance safety and security
- _

Fresh fuel 500 kg UO₂

Towards Fully Closed Cycle

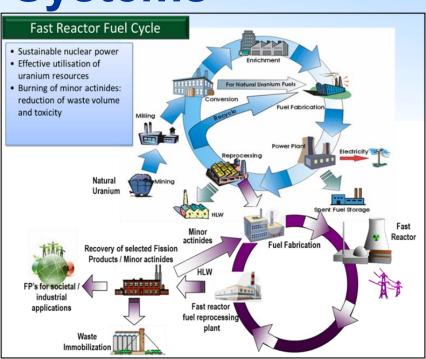
Multirecycling of valuable materials through innovative systems

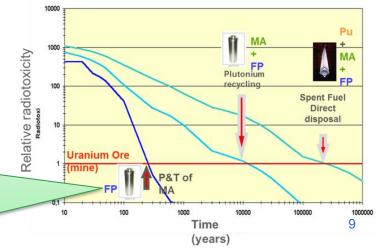
- Breeding Pu
- Burning Pu
- Burning/Transmuting MAs



for future generations

Asset





Advanced Partitioning Processes

Hydro-Metallurgical Processes

GANEX

Uranium extraction followed by group extraction of all actinides

UREX +

- Separation of U & Tc by UREX
 Recovery Cs & Sr by CCD-PEG
 Recovery of Pu & Np by NPEX
 Recovery of Am, Cm and Ln by TRUEX
 Separation of Am and Cm from Ln

DIAMEX, TODGA

Separation of Minor Actinides and Lanthanides from HLLW

Advanced Aqueous **Partitioning** Methods

TRUEX

Transuranic element extraction from HLLW

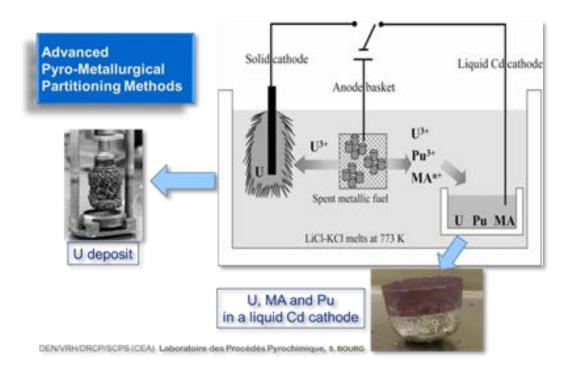
SANEX, ARTIST, **TALSPEAK**

Separation of Am, Cm from Lanthanides

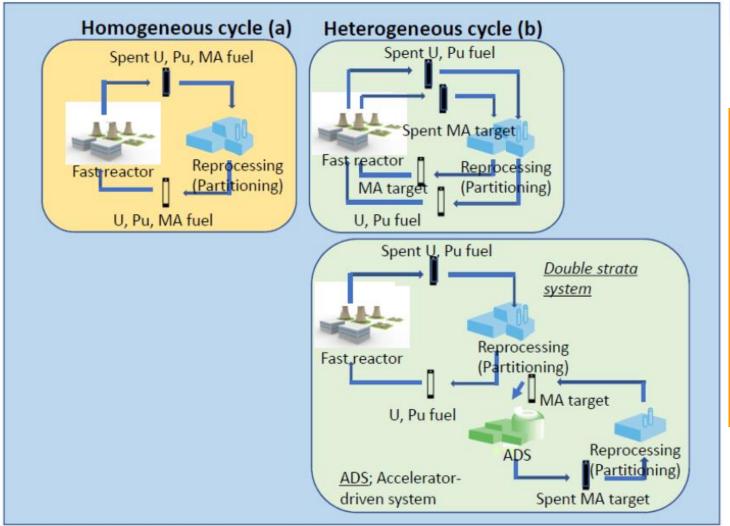
SESAME

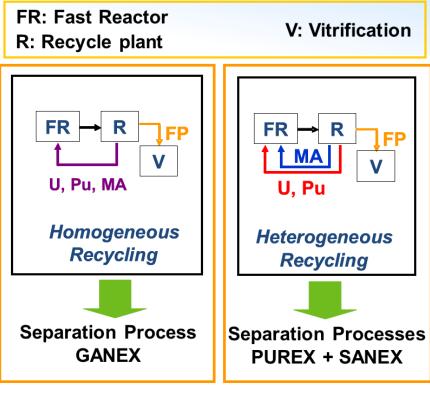
Separation of Am from Cm

Pyro-Metallurgical Processes



Fully Closed Fuel Cycle Options





Hydro-Metallurgical Processes

Demonstrating Multiple Recycling in LWRs Contribute to Transitioning to Fast Reactors

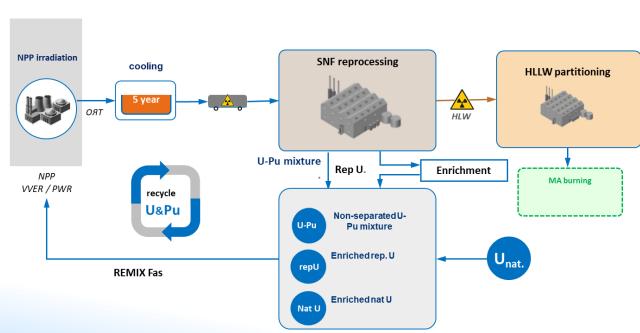


In France

- MIX scheme is composed of MOX fuel rods with enriched uranium matrix (instead of depleted uranium as for current MOX fuel)
- MOX-MR scheme mixes plutonium batches coming from reprocessed spent UOX, current spent MOX fuels, and a minor amount of multi-recycled plutonium. Plutonium is then blended with depleted uranium.

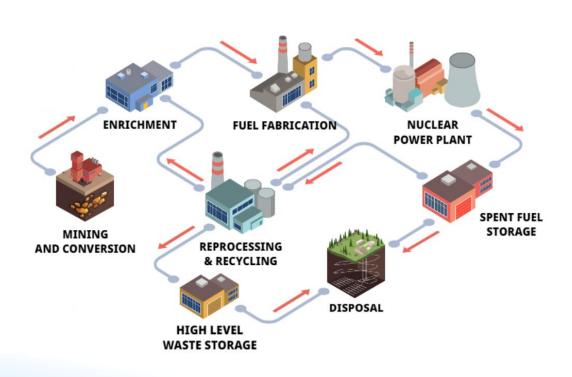
In Russia

- REMIX fuel is made up with:
 - Mixture of U+Pu, co-extracted from LWRs' SNF reprocessing
 - Enriched uranium (natural or RepU)





Main Challenges for SNF Reprocessing and Recycling



Reprocessing and Recycling

- Reprocessing is specialized on current fuels and capacities are limited
- Ageing of available Rep&Rec facilities, specially in Europe
- Accelerate implementation at industrial scale of Multi-recycling in LWRs as transition to AMRs
- Demonstrate and scale up multi-recycling through Advanced Fuel Cycles for innovative reactors

Integration of existing and new fuel cycles is key for sustainability

French decision to pursue the reprocessing/recycling strategy till 2100

- The 4th Nuclear Policy Council chaired by the French President of 17th March 2025, confirmed the programme to renew the back-end nuclear fuel cycle facilities at La Hague, as well as the policy guidelines for achieving closure of the nuclear fuel cycle in France in the second half of the century
- On March 7th, 2024, the Minister of Economy and Finance, publicly announced at Orano La Hague, the decision to pursue the reprocessing-recycling strategy beyond 2040 with:
 - La Hague and Melox plants operational life extended beyond 2040
 - New MOX fuel fabrication plant, new wet storage capacities at La Hague site by 2040 to meet the needs of the existing nuclear fleet and the EPR2s
 - New spent fuel reprocessing plant at La Hague site, by 2045/2050

MOX









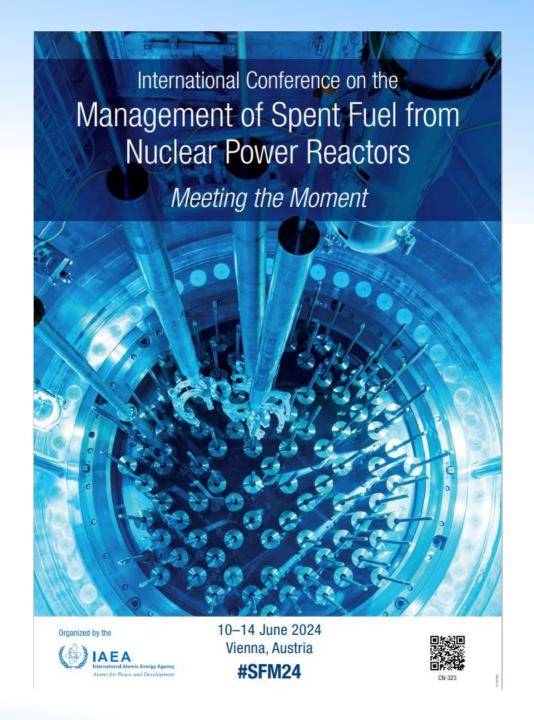
IAEA's Activities in Support of Spent Fuel Management

- Publications
- Scientific/Technical Events
- Coordinated Research Projects
- e-Tools

Scientific/Technical Events

- Conference SFM'24, 10-14 June 2024
 - In cooperation with NEA-OECD, WNA, EC, WNTI
 - 300 in person participants from 58 countries and 6 Int Org
 - 86 women and 214 men present
 - About 220 participants online
 - Proceedings in progress





#SFM24 Panel Discussions





Panel on National Programmes (II) **Hans Wanner** Hans Wanner Consulting **Umesh Dani** Bartosz Sosnik Moderator Bhabha Atomic Research Centre Polskie Elektrownie Jądrowe (BARC) India **Nos Balint Muhammad Shoaib Public Limited Company for Pakistan Atomic Energy** Radioactive Waste Management Commission (PAEC) (PURAM) Pakistan



Scientific Conference Programme from 8:30h to 18:30h

	Monday, 10 June		Tuesday, 11 June		Wednesday, 12 June		Thursday, 13 June		Friday, 14 June
	Opening Session R. M. Gross(Director General International Atomic Energy Agency (M.M. Gross) Director General International Atomic Energy Agency (M.M. D. Magawood, Director General World Nuclear Association (WMA) W. D. Magawood, Director General World Nuclear Association (WMA) W. Engelmann, Director for Nuclear Safety and Security Joint Resociation Desire - European Commission (MC-Cit) W. P. Bouhan, Managing Director of Springen Nuclear Transport Solutions (MS), Board Member World Nuclear Transport Institute (WMT) J. M. Gago, Conference Chairman, Spain Coffee/Tea Break Coffee/Tea Break Panel on National Strategies for Sport Fuel Management (I) T. Manneville, French General Directorate for Energy and Climate, FRA T. Künderg, Ministry of Infrastructure and Water Management, NET & Scinick, Publish Elektrowne Japtowe, POL H. J. Park, Institute for Korea Sport Nuclear Fuel, ROK T. Justons, Nuclear Decommissioning Authority, UK (IDF81)	\$6.2 ADV	ID#64 Challenges and Opportunities of Advanced Nuclear Reactors for the Bask End of the Fuel Cycle, C. Evans, FRA ID#64 The WRADD Project - Starting with the End in Mind: Explaring Have Innovative Nuclear Systems Require Innovative Waste Management Solutions, I. McKamarina, U.S. ID#65 Printerial Impacts of SAR Deployment on Multinational Cooperation at the Bask End of the Fuel Cycle, J. Kinghom-Aliks, Ed. ID#65 Printerial Impacts of SAR Deployment on Multinational Cooperation at the Bask End of the Fuel Cycle, J. Kinghom-Aliks, Ed. ID#65 Highest Startingles, J. J. Minitock, RAP. ID#65 Highest Startingles, J. J. Minitock, RAP. ID#65 High Space Fuel Management Strategy in the U.K. LE, Kasks, U.K. ID#65 Assessment of Performance and Spent Fuel Characteristics of a Beneric 100-MWe-Class SFR-SMR, D.T. Wayasserk, C.W. Coffee/Toa Break Coffee/Toa Break Option Committee	\$2.2 \$T0	Did 5 Progression and Challenges in Technically Undersinning the Unided Kingdom's Saralety for the Management of Speet AGR Oxide Fired, R. Heacey, K.M. 194156 Assessment of the Effects of Long-Term Wet Strungs on Canadian Speet Nacions Fired, G. Cotts-Sarocke, CAV 194176 Descretion on Sieve Cooling Rate Effect of Unimadelet Zinckly-4-Horide Reprintation United Try Storage Condition, D.H. Kook, ROK 19457 Writedspring Drying and Dry Storage Oxideos for Stanless—Sheet Clad Advanced Cas Reactor Speet Fired, C. Johngher, UK 19450 Writedspring Drying and Dry Storage Oxideos for Stanless—Sheet Clad Advanced Cas Reactor Speet Fired, C. Johngher, UK 19450 Writedspring Drying and Dry Storage Oxideos, C. Scheder, CFT 194131 Re-Examining Speet Fired Cladding Integrity Limits, J. Fattenski, USA 1951 New Doal Purpose Casks for Speet Fired of Foreign WWER MPPs and High Level Waste form Speet Fired Processing, M.E. Bush, H.S. 19450 Sheet Sheet Sheet Fired Cladding Steps Facility for the Storage of Fired Fired, A Smartys, LIT 194109 Performance Enhancement of the National Cask for Oxenge Sheet Sheet Fired Cladding Cask for Speet Fired Cladding Cask for Cask for Speet Fired Cladding Cask for Oxenge Speed Cask for Speet Fired Cladding Cask for Oxenge Speed Cask for Speet Fired Cladding Cask for Oxenge Speed Cask for Speed Cask for Speet Fired Cladding Cask for Oxenge Speed Cask for Speed Cask for Oxenge Speed Cask for Speed Cask for Oxenge Speed Cask for Oxenge Cas	\$5.2 DIS	DNSS An Overview of the United Kingdom Strategy for Management of Sport Nuclear Past, Focusing on Metatic United Fasts and Challenges Presented for Decological Disposal, MrJ. Austro, CV. DN15 Design and Evaluation of a Korean-Style High-Efficiency Disposal System for Demestic Water, Kr. Zue. ROX DN16 Esignaring Encapsulation Finelegaes for Disposal of Sport Nuclear High-Efficiency Disposal System for Demestic Water, Kr. Zue. ROX DN18 Predicting Decay Heat by Combining Fuel Parameters with Grown Shrindown Office United Machine Learning, Kr. Sollins, SWED DN17B Developing Wastle Acceptance Criteria for Advanced Reactor Wastle Forms in the Universal Camilier System. J. Sollins, USA DN11ERC of the Host Rock Thermal Properties on the Disposal Area Requirements for Sport Fuel and Whitfield High Level Wastle in a Geological Repository, B. B. Acar, Tut? Ceffice/Tea Break Paster Session 5 DIS Paned on Innovation and Integration C. Evens, World Nuclear Association (WMN) (DISS3) L. Igistics Prior, Joint Research Center – European Commission (JRD-C) E-Gartrid, EKANA, FRA E-Petit de Mange, OLLO, USA (DN91) B. Mort, University of Euerpool, UK (DN103) A. Sturn University of Euerpool, UK (DN103)	S3.1 TRA	ID#114 Preparing Safe and Efficient Loading and Transport Operations of Spent Nuclear Faul J. Sulphac, FM4
13:15-14:45	J. Lubinski, Nuclear Regulatory Commission, USA Lunch Break	12:40-14:00	Heat from Spent Fuel Assemblies: A Geogen NPP Project, S. Caruso, SW Lunch Break	12:40-14:00	D#S2 Spert Nuclear Fust Management for Hollec International's SMR-300, R. Marin, SPA Lunch Break	13:00-14:20	Lunch Bresk	12:00-13:00	J. A. Gago, Conference Chairman, Spain
		12:50-13:50	Side Event 1: "The French Fuel Cycle: An Integrated System for a Sustainable Low-Carbon Energy", sponsored by France			13:10-14:10	Side Event 2: "Meeting the Moment: Paving the Way for Tomorrow's SFM Workforce" (Co-organized by NEI, NYIC and MEA-WINI, sponsored by NEI (USA)		L. Exrard. Deputy Director General and Head of the Department of Nuclear Safety and Security, IAEA. M. Chudskoy, Deputy Director General and Head of the Department of Nuclear Energy, IAEA.
14:45-16:00 P2	Panel on National Strategies for Spent Fuel Management (II) 8. Mds, Public Limited Company for Radioactive Waste Management (FIRANA), HITN L. Dank Brashe Atomic Research Centre (BARC), RVD M. Shoato, Palvistan Atomic Energy Commission (PAEC), PAK (IDW19) A.J. Chichevin, ROSATOM, RUS	14:00-15:40 \$2.3 \$TO	DPT2 Preparing for Estended Starage of SNF in Germany from the Point of New of the Licensing Authority, J. Palmes, GPT DPST Experience of Regulstory Oversight for the Controllion, Commissioning and Operation of Sport Fuel Starage Facilities in Unions, A Shapathak, UKP DPST Regulatory Framework for Speed Fuel Management for Shares Neiger Power Programme, M. Asserbade, GPM 10465 Environmental Audice and Public Engagement in the US Consent-Bissed Shing Process, M.Z. Bell, USA (DPSS Sport Pure Management in the Slovak Republic, J. Vactor, SLFI	14:00-16:00 P3	Panel on Navigading Stakeholders Engagement L Prizzert Nuclear Waste Management Organization (NAMO), CAN M Aloa, Lappeersanta-Labit University of Technology (LUT), FIN (DIXT ²) R. Zinswin, Association Nationale des Comités et Commissions. Locales d'information (ANCCI, IFA J. étoelen, Centrale Organisatie Voor Radioactief Ahral (COVRA), NET M. Miller Fernández, Empresa Nacional de Residuse Radiactivos, S.A. (PANESA), FIN T. Wester, Nationale Gesellschaft für die Lagenung radioaktiver Actitie (NACCI, IFA)	14:20-15:40 \$7.1 INT	DV34 Addressing Nucleur Spent Fuel Management Challerges: A Key to Adeviating Rigorians Skeptidism Towards Nucleur Power Generatins, J. Simon, IAB 1 DV72 An RISM Augmented Separtise Tool Helping for Continuous Montholog		Track titles 1. NAT - National Strategies for Sperit Fuel Management 2. STD - Strategies for Sperit Fuel Management Transportability 3. TRA - Transportation in the Back End of the Fuel Cycle
16:00-16:30	Break	15:40-16:00	Break	16:00-16:30	Break	15:40-16:10	Break		4. REC - Recycling of Spent Fuel
	Poster Session 1 NAT		Poster Session 2.1 STO		Poster Session 2.2 STO		Poster Session 7 INT		5. DIS - Disposal of SNF, HLW and Other Waste Forms in Deep Geological Repositories (DGR)
16:30-18:30 \$6.1 ADV	DUES Enteractivization of Spent Field for Selected Small Modular Reactions and Implications for the Back End Full Dycle, X. Wang, CAV. 1988 A Tool to Estimate Isotopic Exclusion for Actinides Transmutation Decidated for Fast Moline Staff Reactions: A Composative Study of Exchinic Incore Kery Parameters for the Preliminary Design of a Field Moliton Satt Reaction (F-MSR), E-E. Dukus, FRM. 10451 A Cassessing the Puterhal for Moliton-Saft Reaction (MSR) Technology in Bill's Substantiable Nuclear Energy Enteres, L. Van den Durgot, BEL.	S4.1 REC	DB 105 Solvent Extraction Experiments for Uranium, Putronium, and Replantum On-Recovery with Dissaker Solutions Derived from Invadiated Nuclear Fuels I, M. Nakahasa, JW. DB 413 Recycling of PMR Spent Fuel in a Fast Reactor, CB. Lee, ROK DB 112 MOX and UDX Fuel Recycling in Pressurized Water Reactors, G. Nace; FAPA DB 17 Fundamental Approaches to HTGR SNF Reprocessing Technology Development, E.D. Famonousa, RUS DB 12 Nationalized Recompliant is Compared Resident Tolland Nuclear DB 12 Nationalized Recompliant is Compared Resident Tolland Nuclear DB 12 Nationalized Recompliant is Compared Resident Tolland Nuclear	16:30-18:30 S5.1 DIS	ID4120 CIBEO, Readeness of the French CIGE Project at the Contribution Licence Examination Stage, 4th Allertimes, FRA DIB60 Recent Programs within the United Kingdom RBD Programme for the Geological Disposal of High Head Generaling Wastes, J.J. Dansford, 4K. Diberson, S. Debeston, S. Debest	16:10-17:50 \$7.2 INT	10445 About Speet Nuclear Fuel in Argentina: Should Wellforry L. Grogogy. Aris 1095 The Value of an integrated View on Speet Nuclear Fuel and Rediscative Waster Management Strategies Reducing Flancial Risks to Stakeholders. L. Van der Durget, BCE. 10474 ROWO.A Horistic Approach to the Optimization of Deep Sectogical Depend of High Level Waste, E. Viscospoulos, SWI 104115 Advances in Developing a LWR Multirecycling System. C. Foras, State Viscospoulos, Control of Control o		6. ADV - Impacts of Advanced Nuclear Energy Systems on the Back-End of the Fael Optie 7. INT - Achieving Integrated Spent Fuel Management Panel titles P1 - Fanel on National Strategies for Spent Fuel Management (i)
	ID437 The French R&D Collaborative Project ISAC on Fast MSR Dedicated to Actinides Transmication, MS. Chevalud, FRA DW142 NRC Research Activities in Spent Fuel Storage and Management of Advanced Fuels for Advanced Reactors,	\$4.2 REC	ID99 Technological Approaches to Spent Accident Tolerant Nuclear Fuel Processing, J.A. Podmover, RUS ID951 Outlines for Building the Future of French Recycling Facilities, H. Vincenth, FMA		IDW71 Adaptation of Current Final Disposal Strategy and Methods in Finland for Spent Fuel from SMRs, P. Keto, FW IDW63 Spent Nuclear Fuel and the ConfinAR Geo Project.		one confutty men		P2 - Panel on National Strategies for Spent Fuel Management (II) P3 - Panel on Navigating Stakeholders Engagement: Sharing Insights
	T. Bovce. USA		IDW131 The ASOF Project - Advanced Separation for the Optimal		L. Kniznik, ARG	1			and Lessons Learned in Spent Fuel Management Strategy

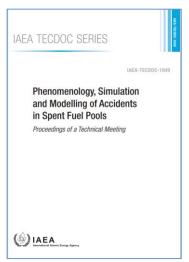
Track titles						
1. NAT - National Strategies for Spent Fuel Management						
2. STO - Storage of SNF and Vitrified HLW and Subsequent Transportability						
3. TRA - Transportation in the Back End of the Fuel Cycle						
4. REC - Recycling of Spent Fuel						
5. DIS - Disposal of SNE, HLW and Other Waste Forms in Deep Geological Repositories (DGR)						
6. ADV - Impacts of Advanced Nuclear Energy Systems on the Back End of the Fuel Cycle						
7. INT - Achieving Integrated Spent Fuel Management						

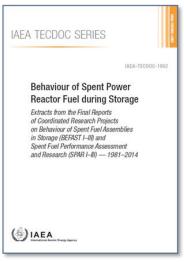
Panel titles
P1 - Panel on National Strategies for Spent Fuel Management (I)
P2 - Panel on National Strategies for Spent Fuel Management (II)
P3 - Panel on Navigating Stakeholders Engagement: Sharing Insights and Lessons Learned in Spent Fuel Management Strategy Implementation in Member States
P4 - Panel on Innovation and Integration: Approaches for Managing Spent Fuel from Advanced Reactors (e.g. SMRs,)

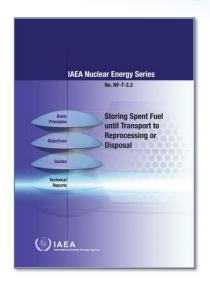
• 14 Sessions in 7 Tracks

Spent Fuel from Advanced Reactors (e.g. SMRs, ...)

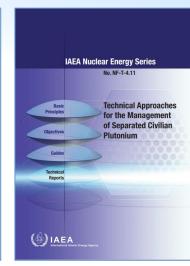
 77 contributed oral presentations 13 E-posters, 21 posters from 29 Member States and 2 international organizations **IAEA Publications on Spent Fuel Management**

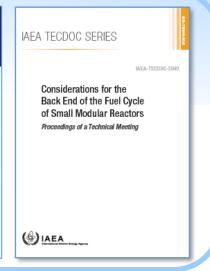






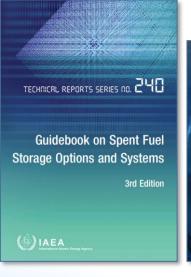






https://www.iaea.org/publications

- TECDOCs
- NE Series publications
- Interactive Books with pictograms, animations and downloadable pictures and charts



AVailable ON-LINE AFR Wet Storage (~ 52,800 tHM) Western Europe 45%

Fuel Storage Options

and Systems

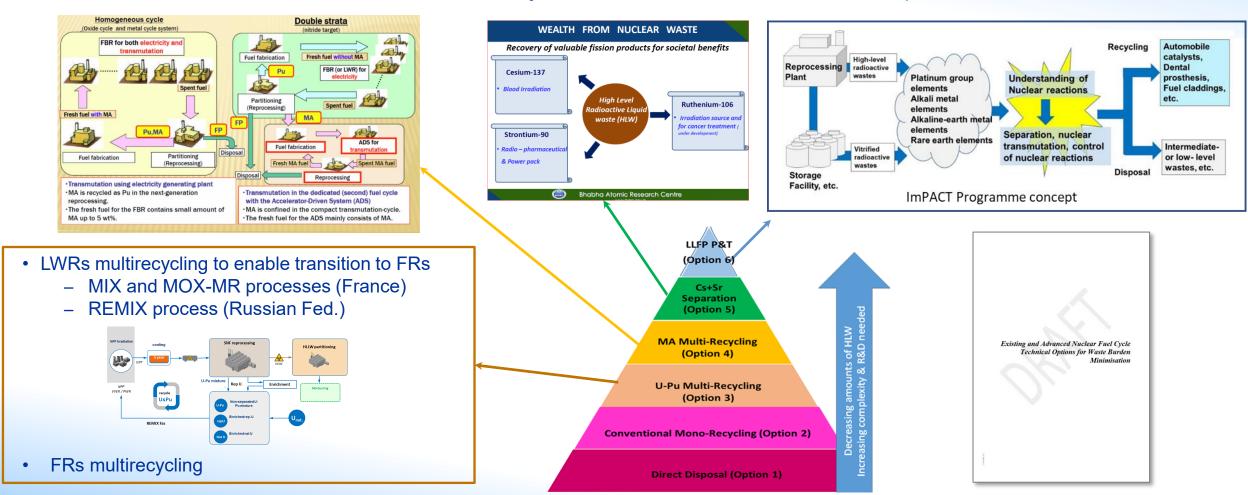
Third Edition



IAEA activities on Advanced Fuel Cycles



Nuclear Energy Series on "Existing and Advanced Nuclear Fuel Cycle Technical Options for Waste Burden Minimization": To describe relevant information on Nuclear Fuel Cycle Options in terms of nuclear materials and wastes involved and nuclear fuel cycle facilities and infrastructures required



IAEA Activities on Recycling

IAEA Technical Meeting (EVT2105850) on Backend of the Fuel Cycle Considerations for Small Modular Reactors, 20-23 September 2022

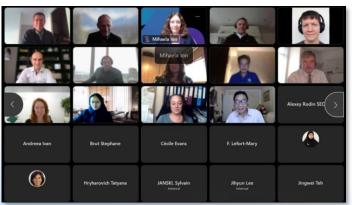


107 Participating Experts from32 Member States

3 International Organizations

40 people in person and 67 on-line





~ 40 Presentations and Extended Abstracts

IAEA Technical Meeting on Backend of the Fuel Cycle Considerations for SMRs

IAEA TECDOC SERIES

IAEA-TECDOC-2040

Considerations for the Back End of the Fuel Cycle of Small Modular Reactors

Proceedings of a Technical Meeting

IAEA-TECDOC-2040 Dec 2023

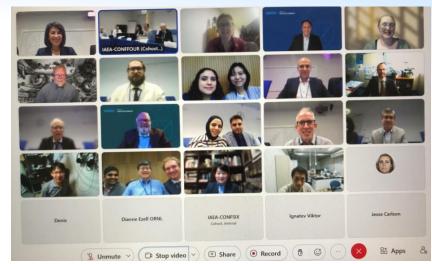


Summary of Presentations and Discussions during the Technical Sessions on

- IAEA Activities
 - SMR Developments and Associated Nuclear Fuel Cycle Options, Fuel Designs, Safety,
 Security, Safeguards, Economics, Transportation
- International Organization's Activities and Perspectives
 - o EC/JRC, OECD/NEA and ERDO
- Member States' Activities and Perspectives
- Three Breakout Sessions (Storage, Reprocessing&Recycling, Transportation, Disposal)
 - LWR type
 - HTGR type
 - AMRs (LMFRs and MSRs) type
- General Discussion
- Conclusions and Future Areas of Work

International Workshop on the Chemistry of Fuel Cycles for Molten Salt Reactor Technologies, 2-6 Oct. 2023, in cooperation with the OECD/NEA

- Held in Vienna on 2-6 October 2023, co-organised by the NEA and the IAEA
- 44 participants (28 onsite + 16 online) including 4 developers of MSRs (Copenhagen Atomics, Seaborg Technologies, Naarea and Terrapower), R&D organisations, regulators and industry experts from 13 countries and the EU
- Several technical sessions including technical presentations, "thoughts from the floor" sessions and panel discussions with all the presenters
- Breakout sessions for the participants to engage in further discussion and work together to identify gaps and needs: experimental underpinning; irradiations; monitoring and operation; irradiated fuel treatment (material and waste streams)





Publication gathering Main Findings, Topical Discussions, Break Out Sessions, Conclusions and Recommendations is under preparation



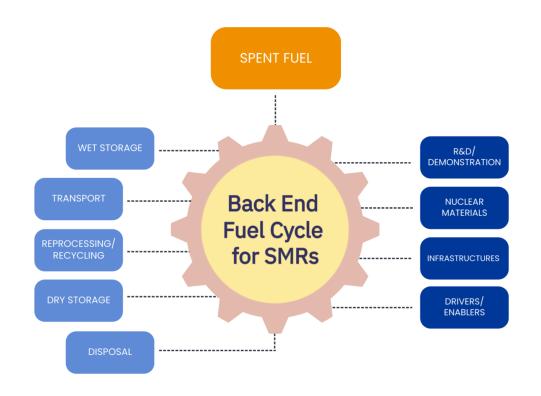
Coordinated Research Project on <u>Challenges</u>, <u>Gaps and Opportunities for Managing Spent Fuel</u> from <u>SMRs</u>

Understanding the implications of the management of new spent fuels is paramount to make informed decisions

MAIN OBJECTIVES:

- To identify viable nuclear fuel cycle options for the different SMR technologies
- To identify common technologies/similarities for various reactor types and/or significant differences
- To prepare a list of generic key parameters for countries to perform their analysis incorporating their specific context
- Development of <u>specific roadmaps</u> for managing spent fuel from the different SMR technologies, identifying what can be derived, optimized or adapted from existing practices, or what needs to be fully developed

SMR-COGS, CRP T13021





First Research Coordination Meeting of SMR-COGS CRP held on 11 to 15 November 2024 in Vienna

STATUS of the Coordinated Research Project SMR-COGS

- 14 Research Contracts from ARG, ARM, CPR, CZR, EGY, INS, LIT, MEX, POL, ROM(2), UKR(3)
- o 18 Research Agreements from CAN(2), CPR, DEN, EGY, JOR, NOR, SIN, SPA, SWE, TUR, UK(2), USA(5)



Industry, Operators, Researchers, Regulators, etc.

Nuclear Energy Programmes: Embarking (Phase 1, 2 and 3), Expanding, Mature and Not Nuclear (DEN and NOR)

Observers: OECD/NEA, FIN, FRA, NET, RUS

45+ participants from 25 countries

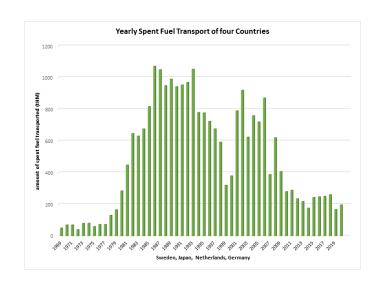
Transportation of Spent Fuel

Technical Meeting on Operational Experiences on Spent Fuel and High Level Waste Transportation, 17-21 October 2022

- Spent fuel has been regularly transported for decades
- TM will review draft TECDOC on Operational Experience containing case studies from seven countries
- Will be opportunity to gather and discuss further information
- Tecdoc is under Drafting Process, gathering country cases



Detailed regulations require long preparation times







Upcoming and Recent IAEA Events on Spent Fuel Management (1/2)

- 1. Technical Meeting on Operating Experience and Lessons-learned on Managing Non-standard/Exotic Legacy Power and Research Reactor Spent Fuels, **18 21 February 2025**, **Vienna** (Material accessible at https://conferences.iaea.org/event/399)
- 2. Technical Meeting on Spent Fuel Behaviour and Performance of Storage Systems, 23-27 June 2025, Seoul, RoK
- 3. Technical Meeting on the Management of Spent Fuel (Pebbles and Compacts) from High Temperature Reactors, **7 11 July 2025, Vienna** (Material accessible at https://conferences.iaea.org/event/414)
- 7. Technical Meeting on Proliferation Resistant Features of Fast Reactors and Advanced Fuel Cycles, **18 22 August 2025, Vienna**
- 8. Workshop on Molten Salt Taxonomy (<u>In collaboration with the OECD/NEA and EC-JRC</u>), **November 2025, Vienna**

Joint IAEA-NEA-EC/JRC Workshop on the Taxonomy and Related Terminology of Fuel Cycles for Molten Salt Reactors

3 - 7 November 2025







Objectives

- To identify and establish a taxonomy for the fuel cycle options related to molten salt reactor technologies.
- To draft a suitable terminology for the different stages of the fuel cycles to facilitate communication on this complex topic.

Topics

- MSR Fuel Cycle Scenarios
 - MSR fuel cycle scenario modelling: covering once-through to full recycling options
 - Flowsheet studies
 - Safeguards and MC&A considerations
- MSR Fuel Cycle Technologies
 - Clean-up and chemical control of fuel salt (with focus on fission products and contaminants removal)
 - Recycling of actinides
 - Influence of waste streams on fuel cycle options
 - Off-gassing techniques
 - Online, inline, offline, at-line, onsite, and offsite treatment options
 - How fuel cycle technologies affect the coupling of chemistry and physics
- Impacts of the input streams on the fuel cycle
 - Fuel source (e.g.: fresh fuel, recycled from spent fuel from other reactors)
 - Fuel and blanket salts
 - Fissile/fertile material (e.g.: U/Pu, Th/U, fuels with minor actinides)

Spent Fuel Management Meetings in 2025 (2/2)

- Technical Meeting on Status and Trends for SNF and RW Management, 28 31 October 2025,
 Vienna
- 10. Technical Meeting on Policies and Strategies for Managing Spent Fuel and Radioactive Waste,
 1 5 December 2025, Vienna

International Conferences in 2026

 International Conference on Fast Reactors and Related Fuel Cycles (FR26) in Beijing (China) from 18 to 21 May 2026 (In collaboration with IAEA/NPTDS)

- International Conference on Fuel Supply Chain for Sustainable Nuclear Power Development in Vienna from 13 to 15 October 2026 (IAEA/NFCMS)
 - Topic 1. Industry Prospects and Challenges Facing Raising Fuel Supply Demand
 - Topic 2. Supply and demand of raw materials for nuclear fuel supply
 - Topic 3. Advanced nuclear fuels for innovative reactor technologies
 - Topic 4. Industrial and Innovative technologies for recycling nuclear materials

IAEA e-Learnings on Spent Fuel Management

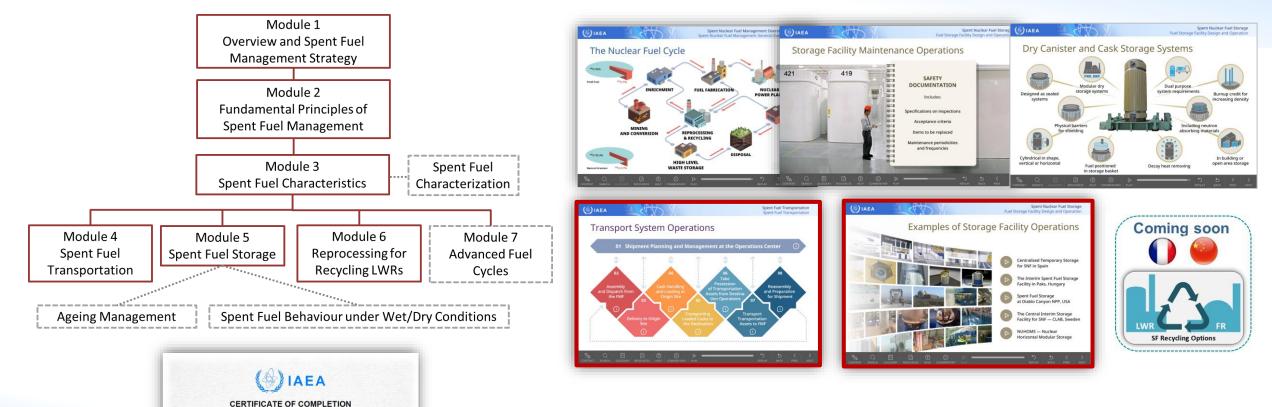
6 Modules (11 Lectures) already available in 🏰 🔵 🔵 🥌





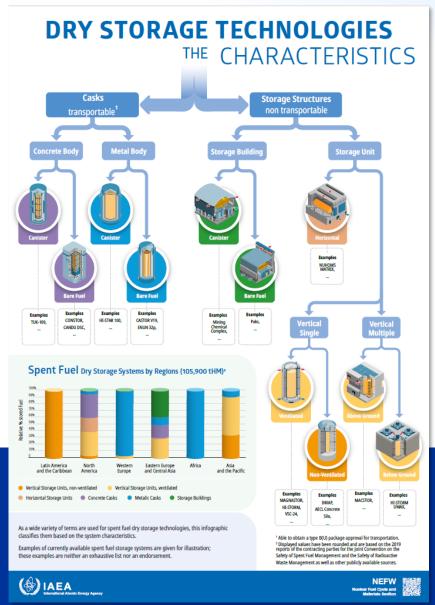


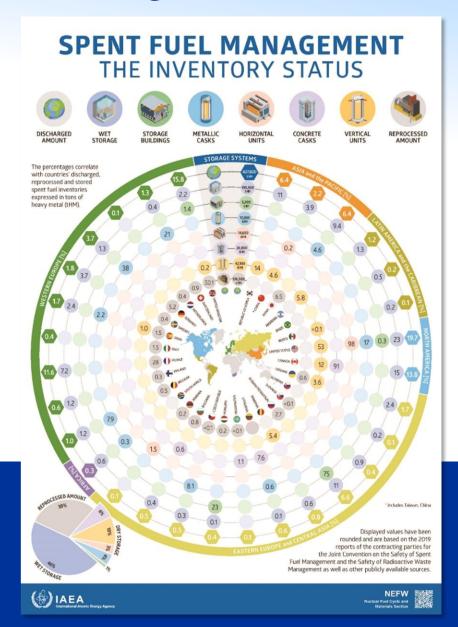




https://www.iaea.org/services/education-and-training/online-learning

IAEA Infographics on Spent Fuel Management





Nuclear Fuel Cycle Simulation System





INFCIS is a resource for technical and statistical information about nuclear fuel cycle activities worldwide, as reported to the IAEA.

The system includes four databases and one computer simulation system:

Nuclear Fuel Cycle Facilities Database (NFCFDB)

NFCFDB covers civilian nuclear fuel cycle facilities around the world. It contains information on operational and non-operational, planned, and cancelled facilities. All stages of nuclear fuel cycle activities are covered, starting from uranium ore production to spent fuel storage facilities. Nuclear Fuel Cycle can be defined as the set of processes to make use of nuclear materials and to return it to normal state. It starts with the mining of unused nuclear materials from the nature and ends with the safe disposal of used nuclear material in the nature.

World Distribution of Uranium Deposits Database (UDEPO)

UDEPO is a database of uranium deposits in the world. It contains information on the classification, geological characteristics, geographical distribution and technical characteristics of the uranium deposits worldwide. The website provides filtering and search capabilities that include the geological classification of the deposits, current status and country. UDEPO also provides tools to create dynamic summary reports.

Post Irradiation Examination Facilities Database (PIEDB)

PIEDB is a catalogue of post irradiation facilities (hot cells) worldwide. It includes a complete survey of the main a complete survey of the main and the complete survey of the complete survey of the main and the complete survey of th

Nuclear Fuel Cycle Simulation System (NFCSS)

NFCSS, formerly known as VISTA, is a scenario-based computer model for the estimation of fuel cycle material, service requirements and actinide arisings. The NFCSS is a computer simulation system that uses simplified approaches to calculate fuel cycle requirements. The site provides a detailed description of the simulation system, an example scenario with results, and a simple calculation tool that can be used to calculate annual material flow in a selected nuclear fuel cycle option.

World Thorium Deposits and Resources (ThDEPO)

Access NFCFDB

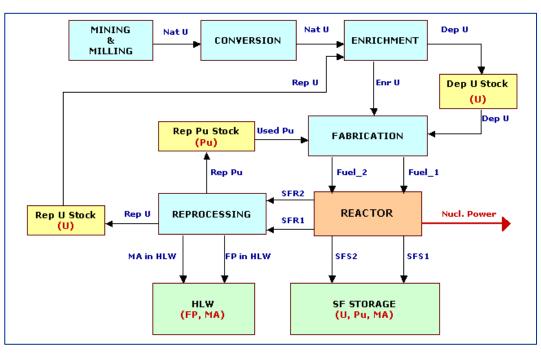
Access UDEPO

Access PIEDB

Access NFCSS

Access ThDEPO

Overall Fuel Cycle Material Flow Diagram





Spent Fuel Management Network

SFM.Contact-Point@iaea.org



Welcome to the IAEA International Network on Spent Fuel Management - SFM Net

The spent fuel management (SFM) network is a forum for the sharing of practical experience and international developments on spent fuel management.

Its main objectives are to facilitate the efficient exchange of information, communication and cooperation amongst professionals working in the back end of the fuel cycle – from its removal from a reactor core to its final disposition (i.e. SNF wet and dry storage, transportation, handling and retrieval, reprocessing and recycling, economics of the back-end of nuclear fuel cycle, damaged SNF management, stakeholder involvement, communication issues, etc.)

The establishment of the SFM Net is aimed at fostering safe, sustainable and efficient spent nuclear fuel management practices across all IAEA Member States.

For further information or questions please contact SFM.Contact-Point@iaea.org.



Events 2025

- Technical Meeting on Operating Experience and Lessons Learned on Managing Non-Standard Lagacy Spent Fuels from Power and Research Reactor (18 – 21 February 2025) evi23046/8
- Third Coordination Research Meeting on Spent Fuel Research and Assessment (24 28 March 2025) EVT2404557
- 23rd Meeting of the Technical Working Group on Nuclear Fuel Cycle Options and Spent Fuel Management (01 04 April 2025) EVI 24034034
- Technical Meeting on the Behaviour of Spent Fuel and Cladding During Storage and the Performance of Spent Fuel Storage Systems (23 27 June 2025) EVID-408673
- Technical Meeting on the Management of Spent Fuel (Pebble Beds and Prismatic) from High Temperature Gas Cooled Reactors (7 11 July 2025) Application
- Joint (AEA-NEA Workshop on the Taxonomy and Related Terminology of Fuel Cycles for Molten Salt Reactors (29 September 03 October 2025) EVIZE04560
- Workshop on the Challenges in Managing Spent Evolutionary Advanced Technology Fuels (10 14 November 2025) #V12205116
- Technical Meeting on the Operation and Maintenance of Storage and Transportation Casks (09 12 December 2025) EVID-404726

Latest News



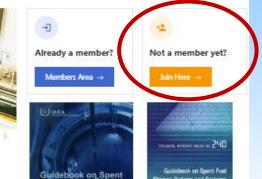
NEW CRP: Challenges, Gaps and Opportunities for Managing Spent Fuel from Small Modular Reactors (T13021)



IAEA's Learning Management System!

With the addition of 3 new modules, the course on

Spent Fald Storage is available on IAEA's Learning



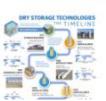
Fuel Storage Options

and Systems



New infographics now available!









Thank You

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