



# HEXANA: design and development of a sodium-cooled fast modular reactor to decarbonize industry

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**PRT-HXN-25-0640 rev. A**

## Company overview

- Founded in 2023 – Spin-off from **CEA**
- Laureate of the Innovative Nuclear Reactors initiative under the **France 2030 program**
- **€27M secured** in funding (€17M in equity)
- 60 employees based in Aix-en-Provence, Paris & Brussels

## Current industrial challenge

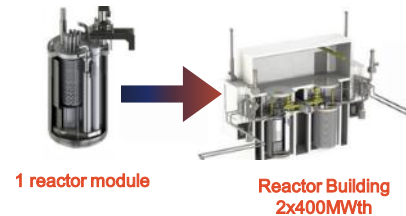
- Heat represents **75% of global industrial energy needs**, of which 75% is above 150°C (IEA)
- **90%** of this heat is **fossil fuel-based**

## HEXANA's energy platform

- **2 sodium-cooled fast neutron reactors (SFR):**
  - **Capacity:** 800 MWth output (excluding heat storage capacity), equivalent to 335 MWe
  - **Maturity:** **440 years of operation** worldwide, 120 years in Europe, 6 reactors operated
  - **Dual-reactor strategy:** no downtime during maintenance or refueling, continuous, uninterrupted power supply
  - **A high-temperature heat storage system**

## Products & services

- **High-temperature heat** (up to 465°C)
- **Ultra-low-carbon power** generation (2 gCO<sub>2eq</sub>/kWh)
- **Reliable & highly-flexible** energy system

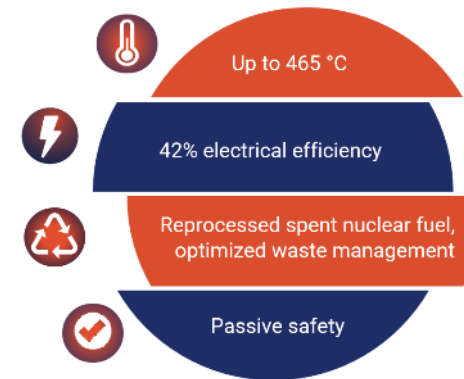


## End customers: energy intensive industries

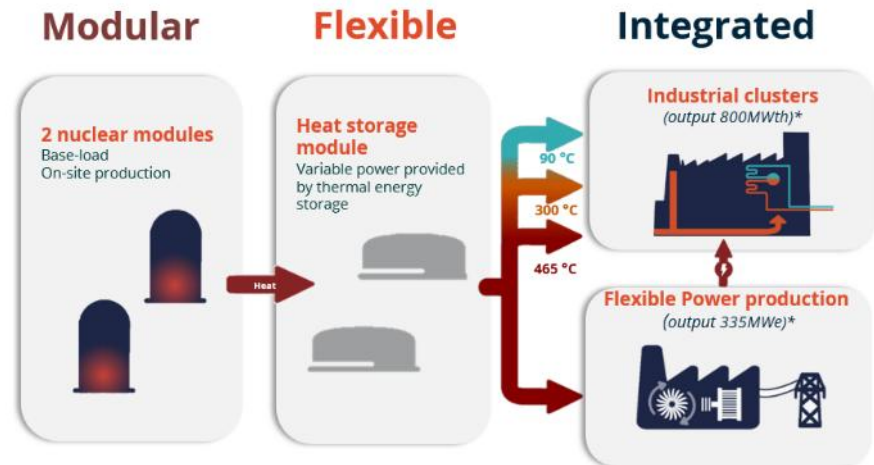
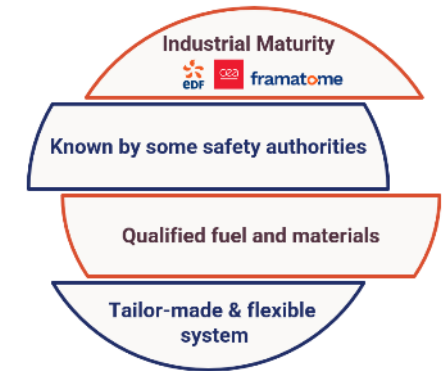


## Our added value

### Favorable technical specifications to match industry needs



### The most mature technology from GEN4 reactors



\*Capacity of 2 reactors only (excludes additional capacity from thermal storage)

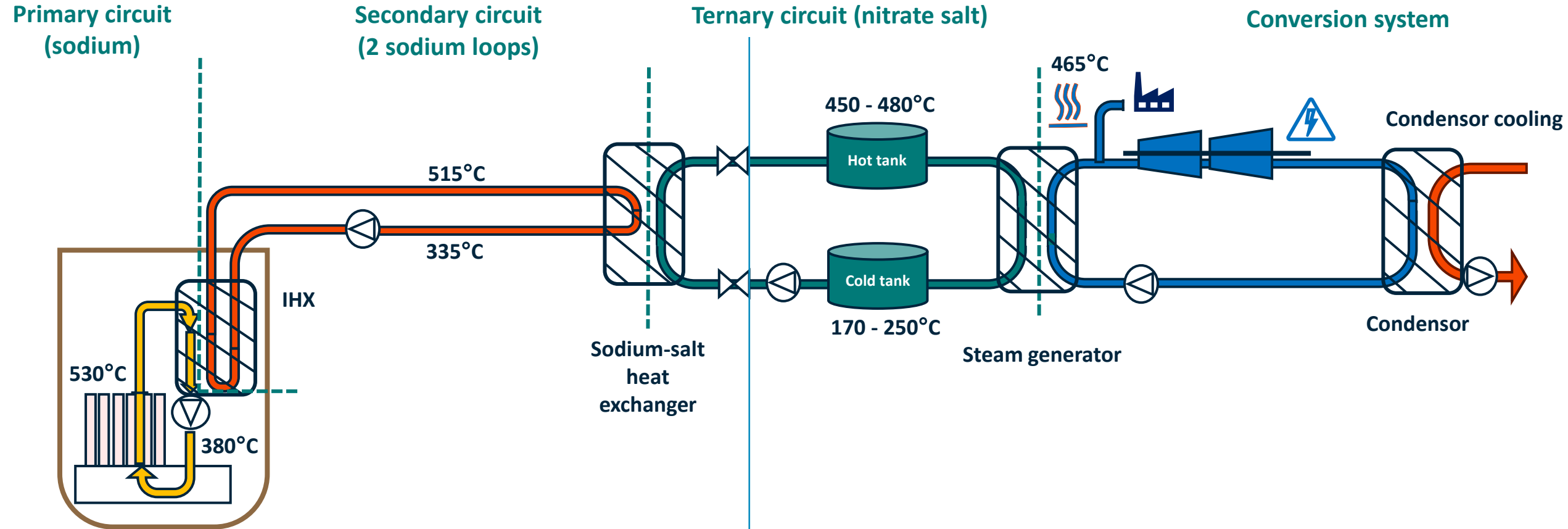
# Technology



# How it works: main fluid circuits

## Nuclear island

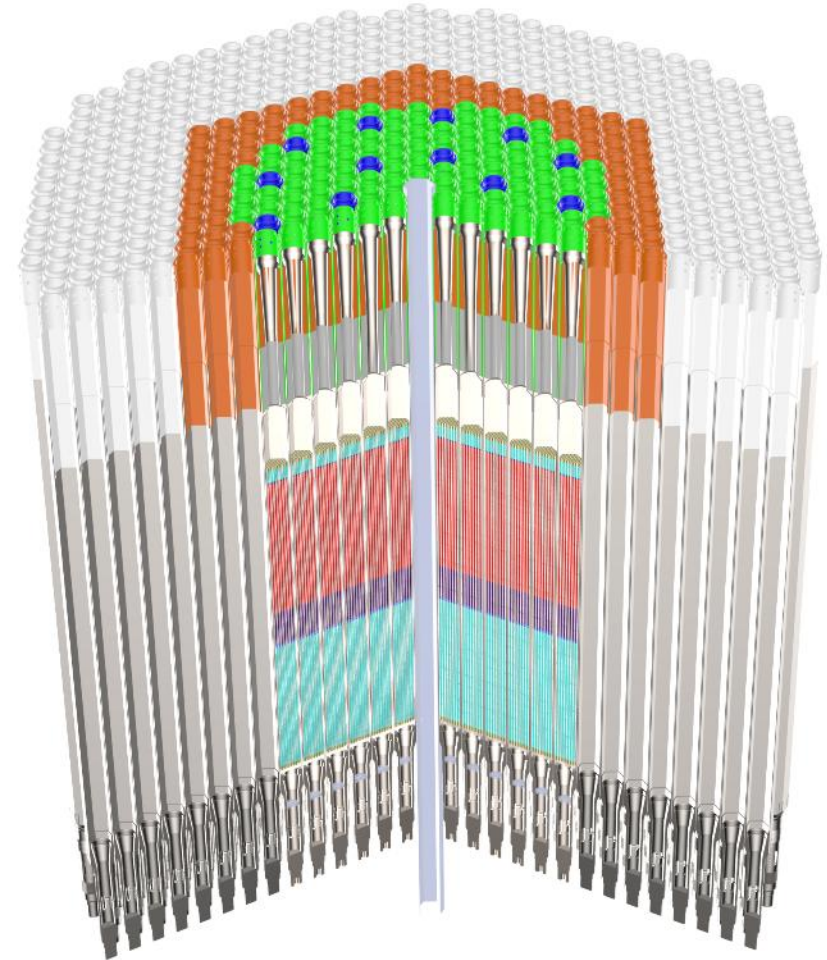
## Conventional island



# Core characteristics

## Core data (FOAK)

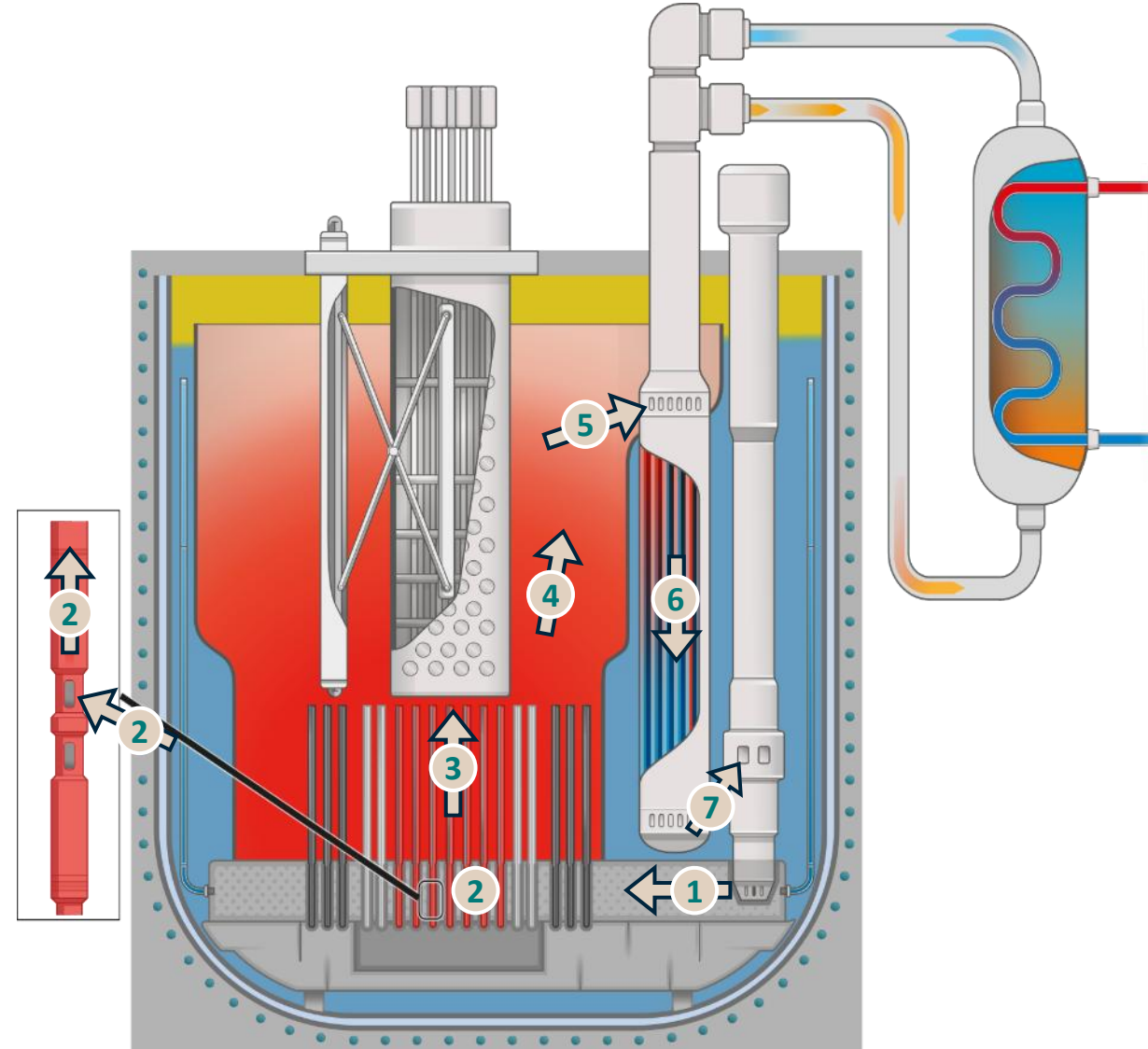
- **Fuel type:** MOX ( $\text{UO}_2$ ,  $\text{PuO}_2$ )
- **Fuel assembly geometry:** same as Superphenix
- **Core inlet/outlet temperature:** 380°C/530°C
- **Number of fuel assemblies:** 132
- **Fuel cycle:** 12-18 months
- **Reloading scheme:** 1/3 of core



# Primary circuit

## Sodium flow in the reactor vessel

1. **Forced circulation** of cold sodium: 2 primary pumps
2. **Injection of cold sodium (380°C)** at the bottom of the fuel assemblies (93% of primary flow rate, 5 bar relative pressure)
3. **Sodium heated** by convection along fuel pins, exiting core at 530°C
4. **Mixing and transfer** of sodium into a hot collector
5. **Hot sodium** enters 2 intermediate heat exchangers
6. **Primary sodium cooled** along intermediate heat exchanger tubes (heat transferred to secondary sodium)
7. **Cold primary sodium** exits at the bottom of the exchanger into a cold collector and is drawn by the primary pumps

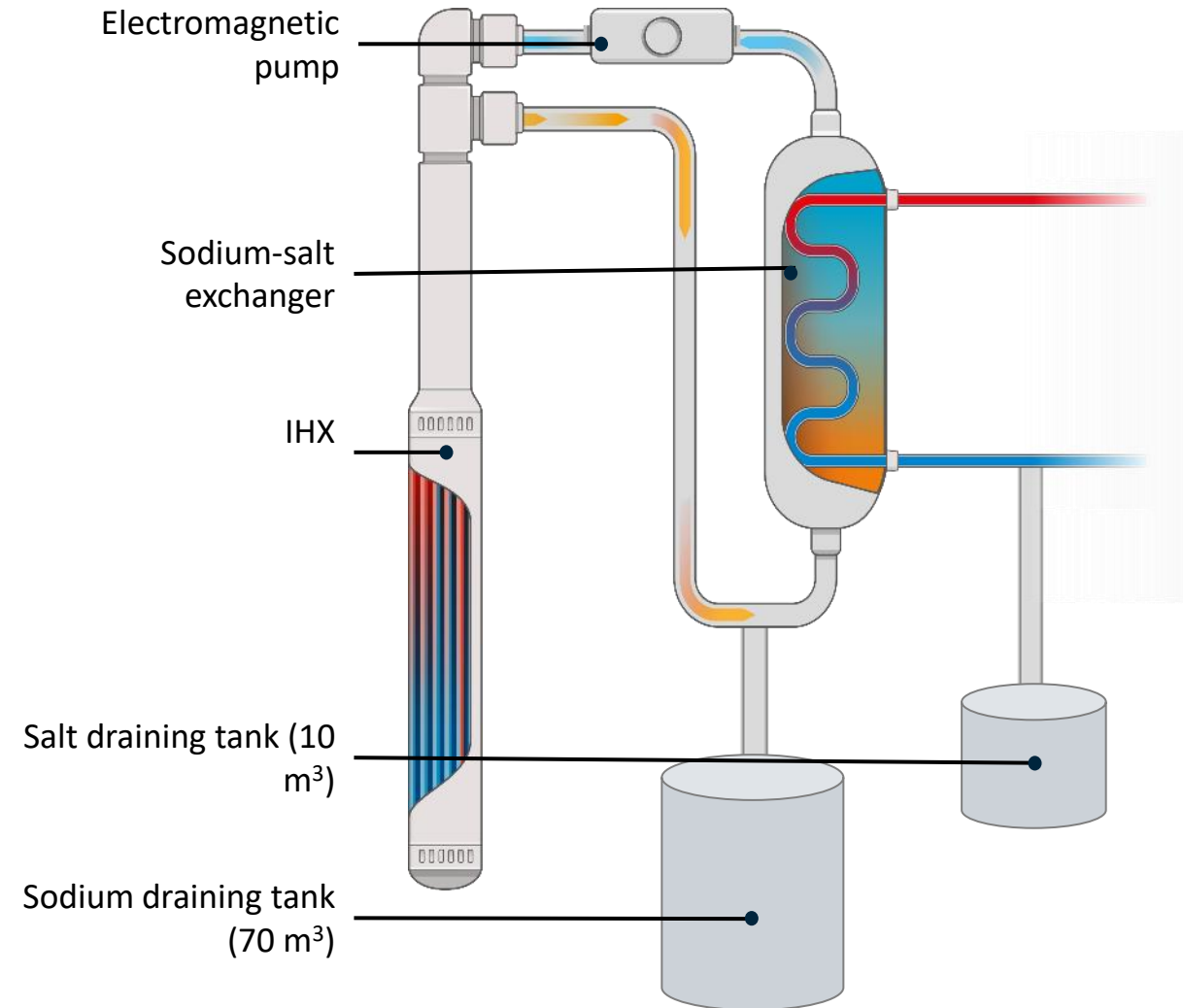




# Secondary circuit

## Description

- 2 independent secondary loops per reactor
- Non-radioactive sodium
- Direct heat exchange with thermal storage system through **sodium-salt heat exchangers**
- Connected to decay heat removal loops (with passive **sodium-air heat exchangers**)
  - Air as ultimate heat sink = infinite cold source in accident scenarios



# Conventional Island – Heat storage system

## Heat storage technology

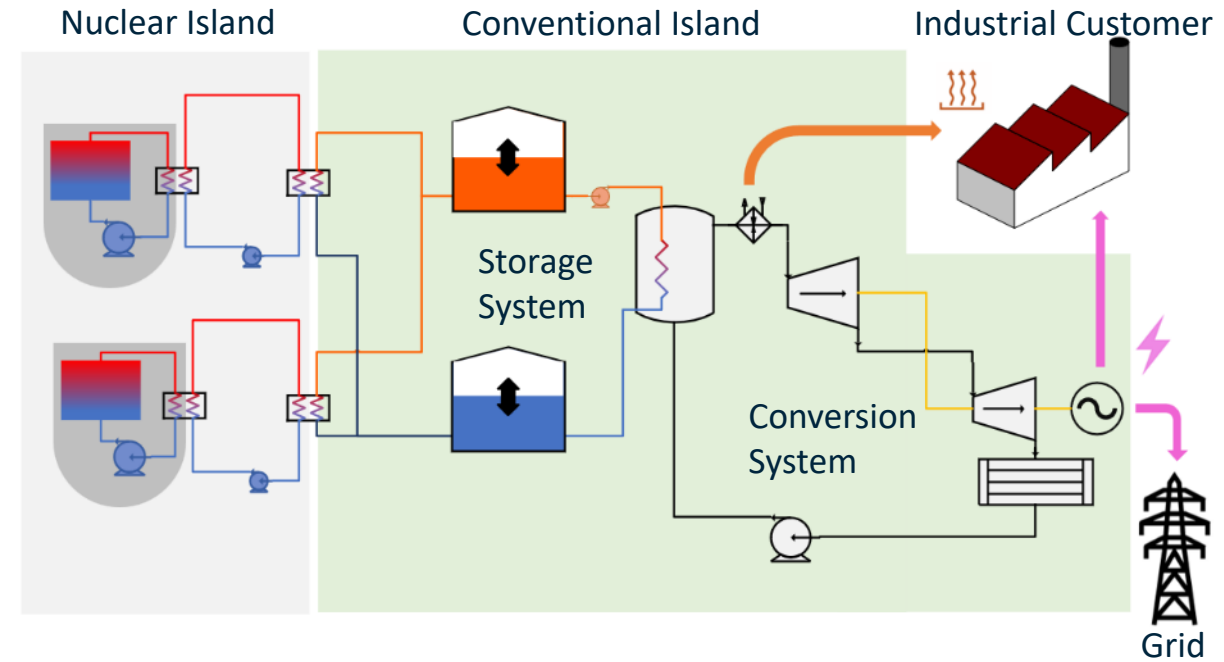
- **Storage medium:** molten nitrate salts
- **Storage architecture:** hot/cold tanks
- **Charging circuit:** sodium-salt heat exchangers
- **Power:** steam generator

## Load following strategy

- **Reactor:** 100% power at all times
- **Heat storage:** used for high flexibility

## High flexibility

- **Storage capacity:** 300 MWh up to 2.5 GWh





# Conventional Island – Energy conversion system

## Conversion system

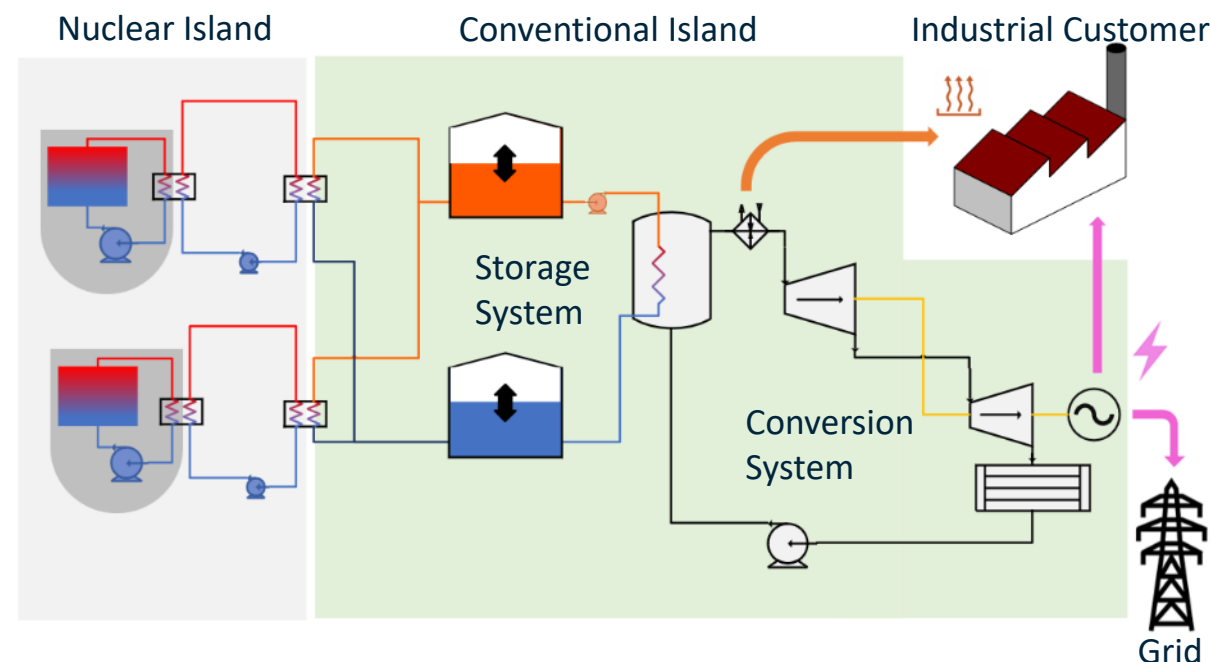
- **Conversion cycle:** Rankine (water/steam)
- **Power:** up to 2 x 400 MWth or 330 MWe
- **Cogeneration:** heat up to 465°C
- **Equipment grade:** non nuclear

## High flexibility

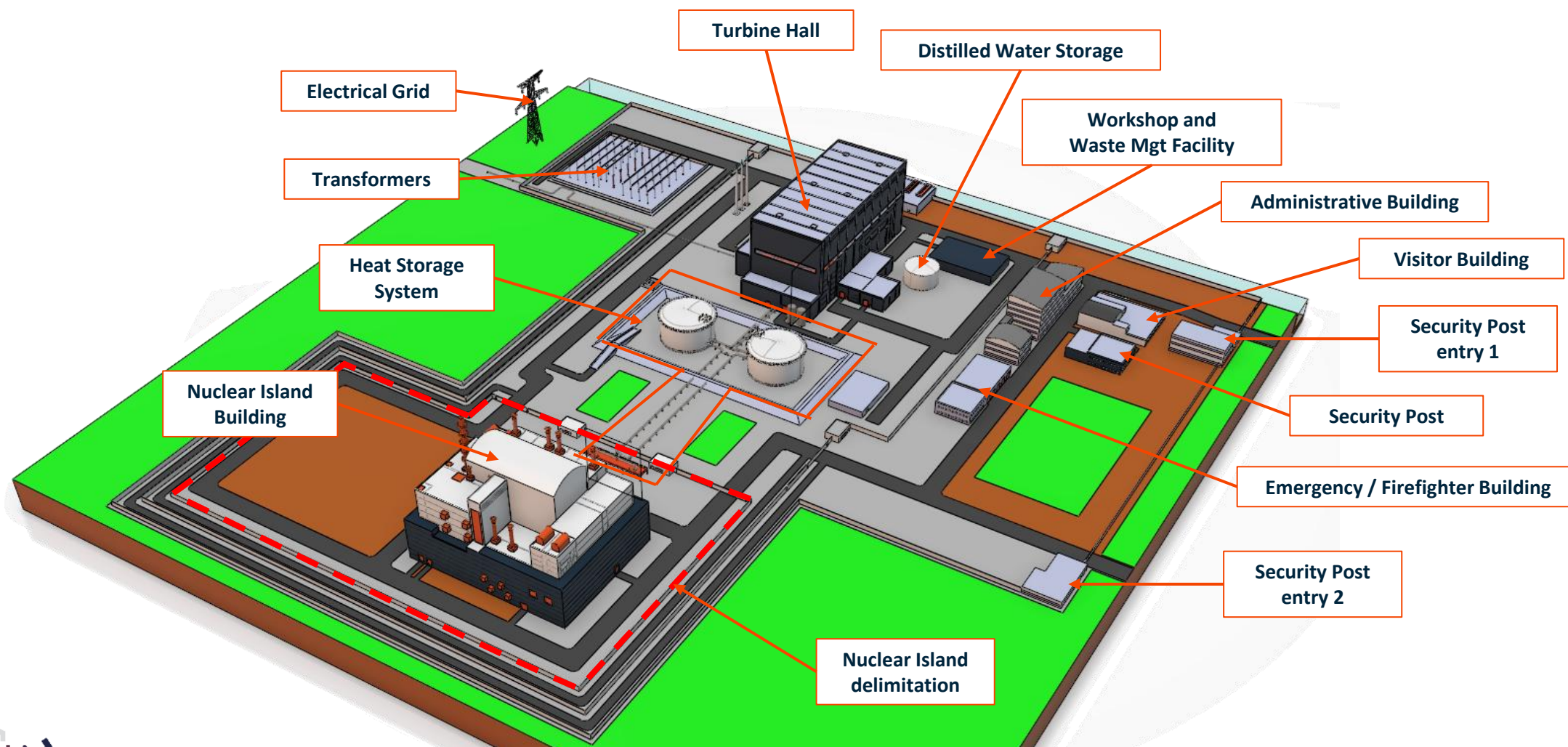
- **Power ramp:** 20% NP/min
- **Main limitations:** Thermal dilation issues

## Cold source and thermal pollution

- **Heat sink:** ocean/sea or river
- **Thermal pollution:** high efficiency (high temperature) → less thermal pollution than LWRs



# Typical site layout



# HEXANA: a highly flexible energy platform

## ■ Three levels of flexibility:

### • Design flexibility

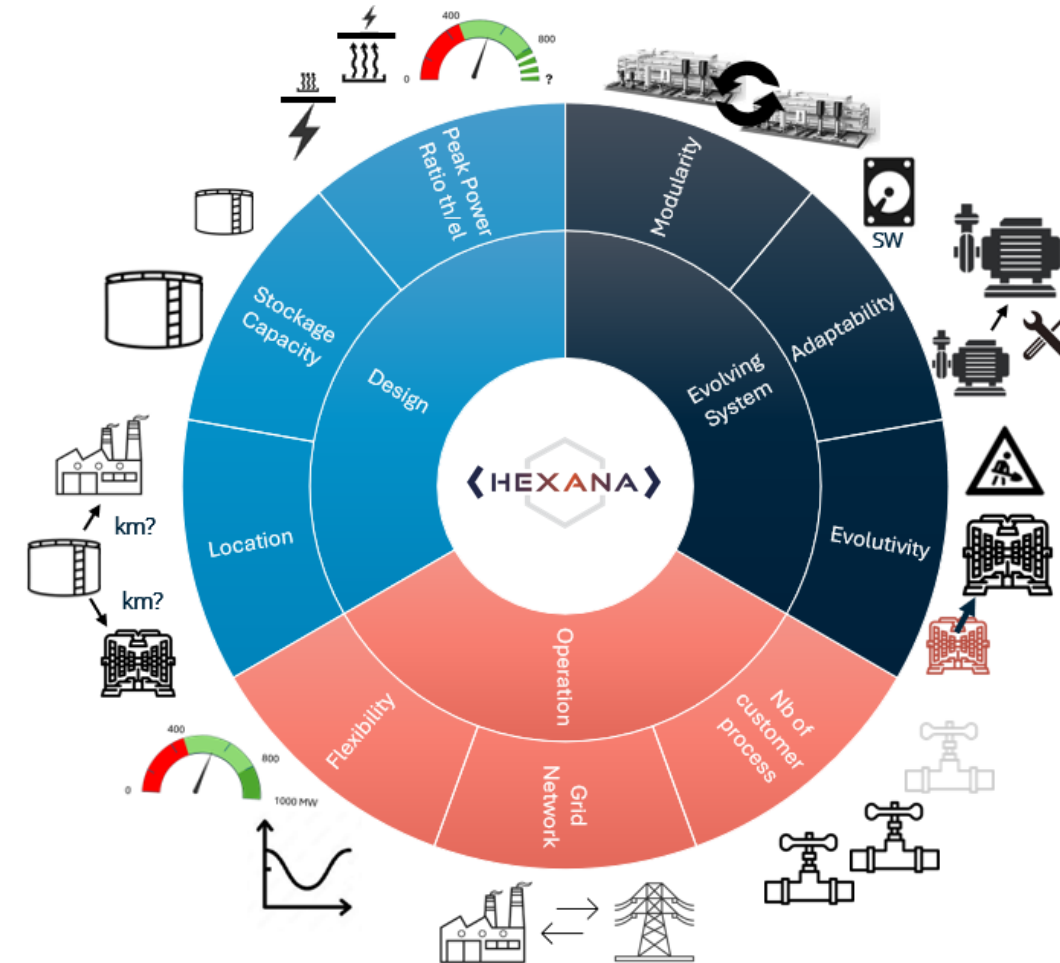
- Peak & power range limits
- CHP ratio (thermal/electric)
- Storage capacity
- Distance between Nuclear Island, storage and conversion

### • Operational flexibility

- Power variation (baseline: 20%/min)
- Power dispatch (heat vs. electric, industrial customer vs. grid)
- Number and temperatures of steam supplies

### • Evolutive flexibility

- Modularity: swap or add modules
- Adaptability: software, minor hardware changes
- Evolutivity: major change (turbine change, storage upgrade)



# R&D needs



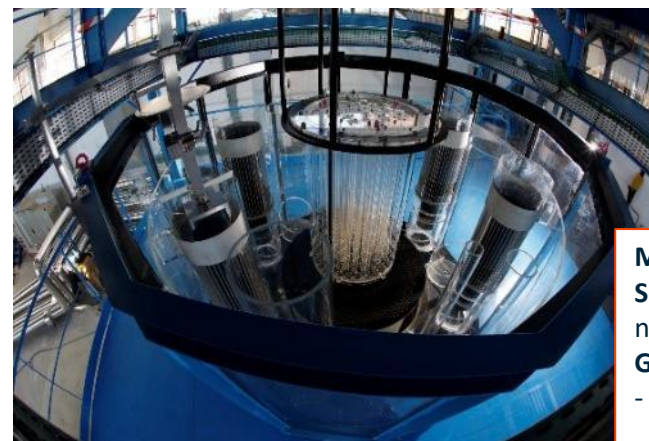
# Technical innovation: challenges and opportunities

Challenges		Opportunities
Modular layout	Rethinking of system interfaces	Optimized assembly and commissioning sequence
Compact reactor vessel	Constraints around component "packaging" and in-vessel flow distribution	Transportable design
Fuel and reactor internals handling system	Mechanical qualification required	Reduced maintenance duration (hence increased system availability)
Salt-based thermal storage system	Development and qualification of new sodium-salt heat exchangers required	Better system flexibility + eliminates sodium-water interaction risk in historic SFRs

# Existing and planned collaborations with CEA (France)

## ■ Water-based thermal-hydraulic installations

- Validation of the **geometric configuration** of the HEXANA primary circuit (compact reactor, asymmetrical component positions) with respect to forced convection, natural convection, gas entrainment, vibration
- Validation of **thermal-hydraulic codes** (CATHARE 3, STAR-CCM+)



Source: CEA

**MICAS – CEA Cadarache (France)**

**Scale:**  $\sim 1/6$  (matching Froude and Richardson numbers)

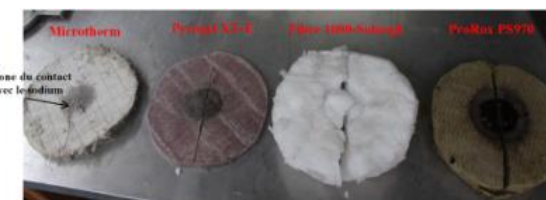
**Goals:**

- Simulate hydraulic flow in the upper plenum of an SFR
- Study gas entrainment
- Provide data to validate numerical models



# Existing and planned collaborations with CEA (France)

- Water-based thermal-hydraulic installations
- Sodium leak detection and location
  - CEA's work on multi-layer insulation + optical fibers
  - Interest in pursuing additional tests tailored to HEXANA's design needs (special components such as valves)

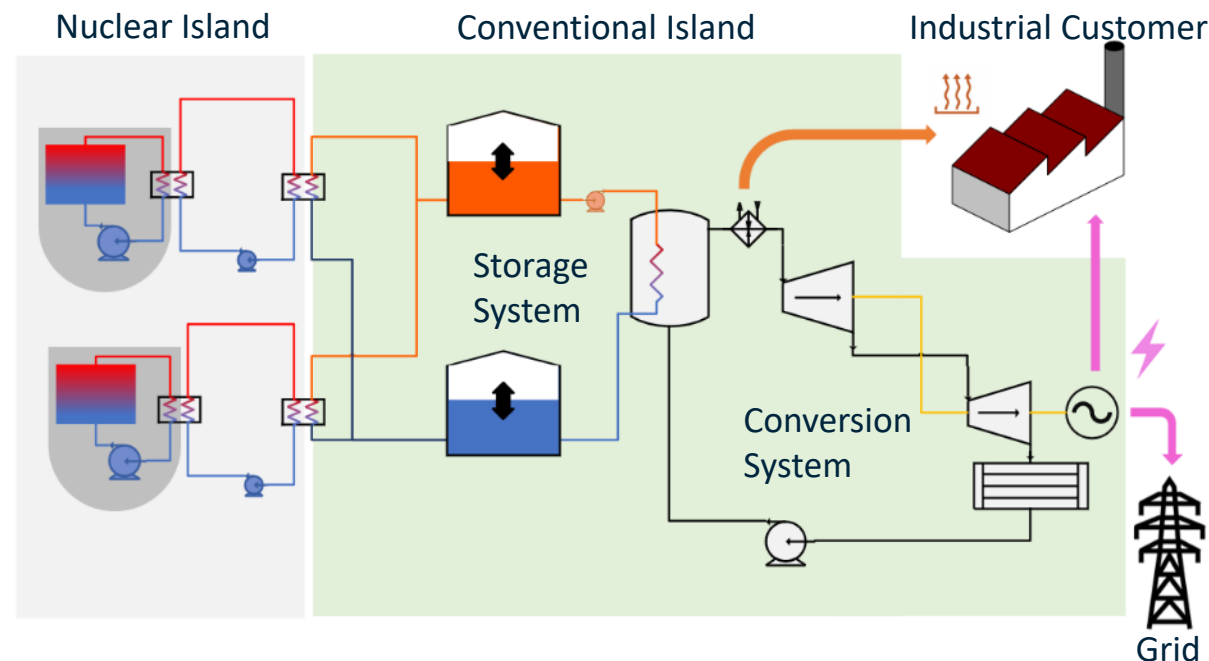


Source: CEA



# Existing and planned collaborations with CEA (France)

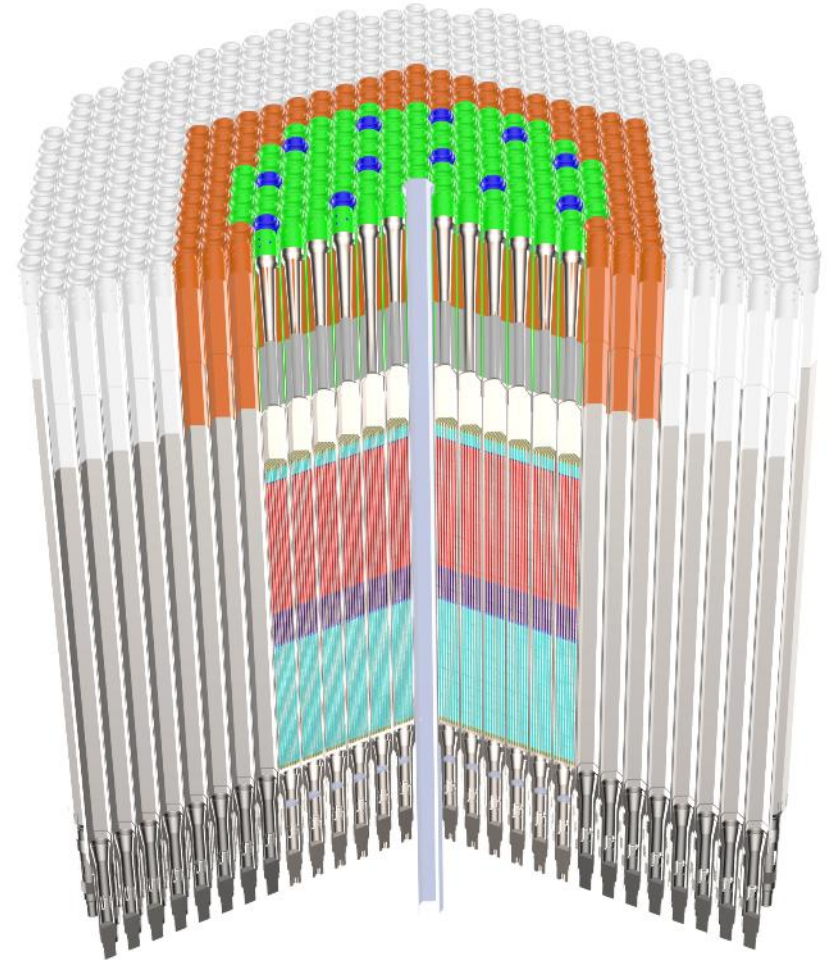
- Water-based thermal-hydraulic installations
- Sodium leak detection and location
- Salt-steam loop for thermal storage
  - Ongoing collaboration on the CEDALION salt-steam loop
  - Qualification of new components (steam generator), new fluid (YARA salt), operations (overall dynamics)



# Other R&D needs by subsystem

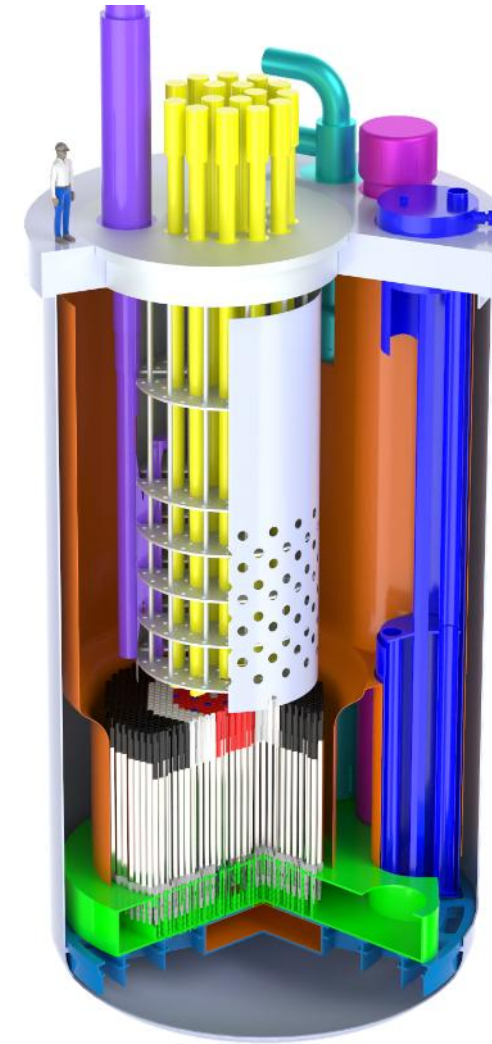
## ■ Fuel and other core components

- Supplement existing qualification data for the intended operating range for **core materials** ( $B_4C$ ,  $MgO$ , borated steel, hafnium for rods and neutron shields)
- Improve **irradiation resistance of fuel cladding** to extend cycle times
- **Mechanical modifications to fuel assemblies** to improve CAPEX (non-integrated fissile part, absence of hexagonal tube, etc.)
- Extension of MOX fuel qualification to **low linear power and high burn-up**
- Study of the **automatic drop of shutdown elements** into the core upon reaching the melting point of a thermal fuse to improve the diversity and passivity of reactivity control methods



# Other R&D needs by subsystem

- Fuel and other core components
- Reactor
  - Qualification of a fuel handling system using a **pantograph arm** in air, then in sodium
  - Qualification of the **core fusion mitigation system** (retention, cooling, choice of materials)
  - Development and qualification of **multifunctional casks** for washing, cooling, and transporting core components



# Other R&D needs by subsystem

- Fuel and other core components
- Reactor
- Reactor pit
  - Qualification of a new residual heat removal system
  - Qualification of in-service inspection and repair systems

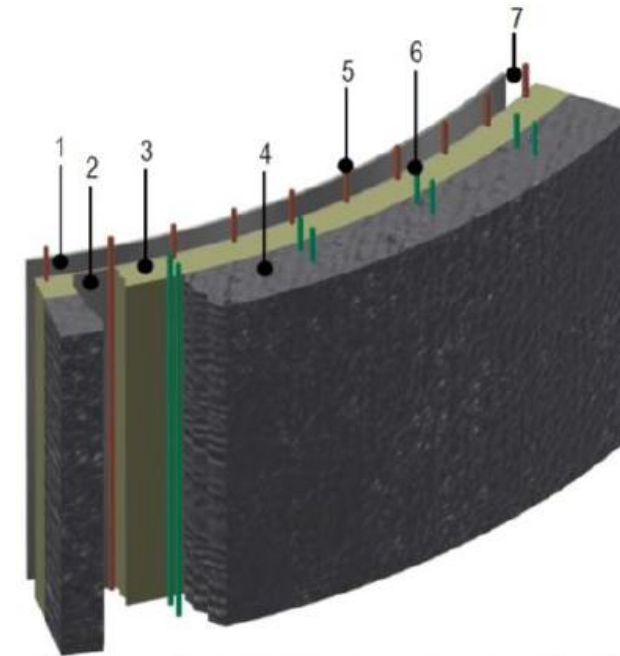


Figure 2: Detail of the reactor pit design

1 – Reactor vessel  
2 – Liner  
3 – Insulation

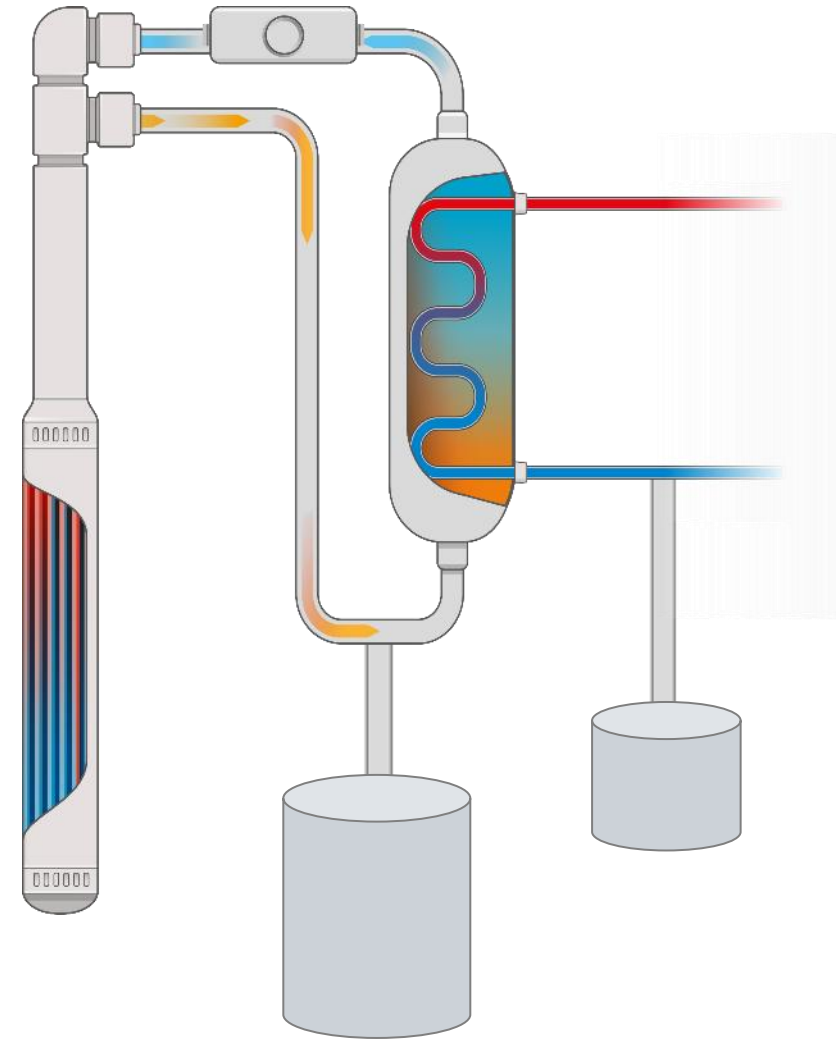
4 – Concrete/metal structure  
5 – Oil cooling system (DHRS-3.1)  
6 – Water cooling system (DHRS-3.2)

7 – Gap

Source: ESFR-Smart

# Other R&D needs by subsystem

- Fuel and other core components
- Reactor
- Reactor pit
- Secondary sodium loop and interfaces
  - Qualification of **new components** (salt/sodium exchanger, expansion bellows, electromagnetic pump, flanges), instrumentation
  - Qualification of **new processes** (unloading, draining, sodium purification, cold traps, blockage indicators)



# Specific needs for qualification of instrumentation

## ■ Opportunities to continue the work initiated by CEA

- **ASTRID project innovations**
  - **Ultrasonic non-destructive testing:** development of piezoelectric and/or electromagnetic multi-element sensors
  - Leak detection: **multi-layer insulation** and/or use of optical fibers
- **Optical fibers**
  - Adaptation of **coating materials** (or protective sheaths) for use in high-temperature sodium

## ■ New needs

- **Chemistry control**
  - Measurement of cover gas quality by **laser-induced breakdown spectroscopy** for detection of leakage and/or fuel cladding breaks
- **Detection of fuel cladding breaks**
  - **High count rate gamma spectrometry:** use of the ADONIS gamma spectrometry system, developed by CEA LIST
  - **Detection of delayed neutrons**
- **Qualification of in-core neutron detectors in a sodium environment**





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