

Overview of Japanese Water Cooled Ceramic Breeder Test Blanket Module Manufacturing and Assembly Technologies

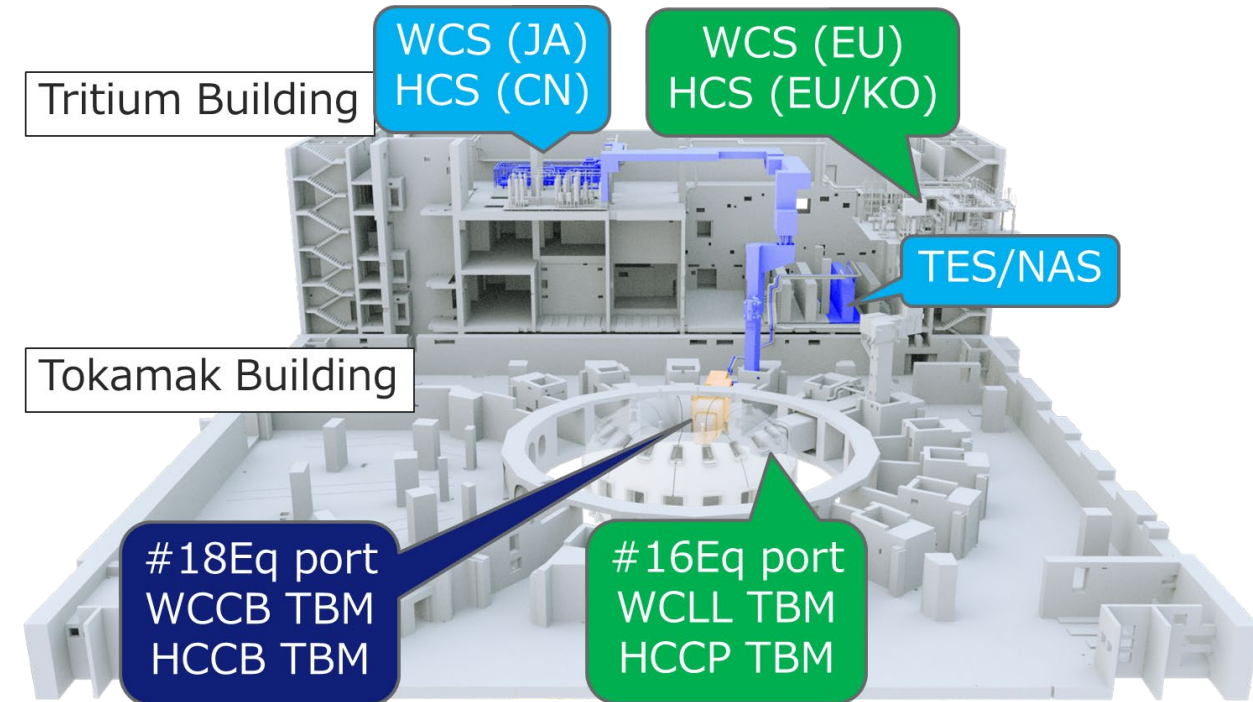
Takanori HIROSE

Wenhai GUAN, Jaehwan KIM, Motoki NAKAJIMA, Takashi
NOZAWA, Yoshinori KAWAMURA, Hiroyasu TANIGAWA

*Rokkasho Institute for Fusion Energy, National Institutes for Quantum Science and
Technology*

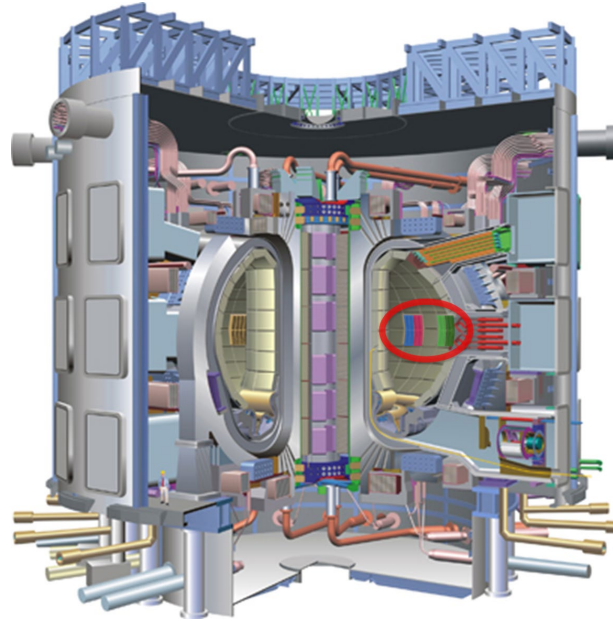
- 1. Background**
- 2. Overview of Japanese WCCB-TBM Design**
- 3. Structural Design of Container**
- 4. Key Manufacturing Processes**
- 5. Welding and Heat Treatment**
- 6. Assembly Sequence**

- ❑ Breeding blankets are essential for tritium production and heat extraction in fusion reactors.
- ❑ The Water-Cooled Ceramic Breeder (WCCB) concept leverages PWR experience and known material behavior under neutron irradiation.
- ❑ Japan is developing a WCCB Test Blanket Module (TBM) for ITER to validate this concept experimentally.



Water cooled ceramic breeder blanket

Structural material	Reduced activation ferritic martensitic steel, F82H
Tritium breeder	Φ1 mm Li_2TiO_3 pebble
Neutron multiplier	Φ1 mm Beryllium pebble
Coolant	15.5 MPa / 280 °C water
Sweep gas	0.14MPa helium gas with 1% H_2

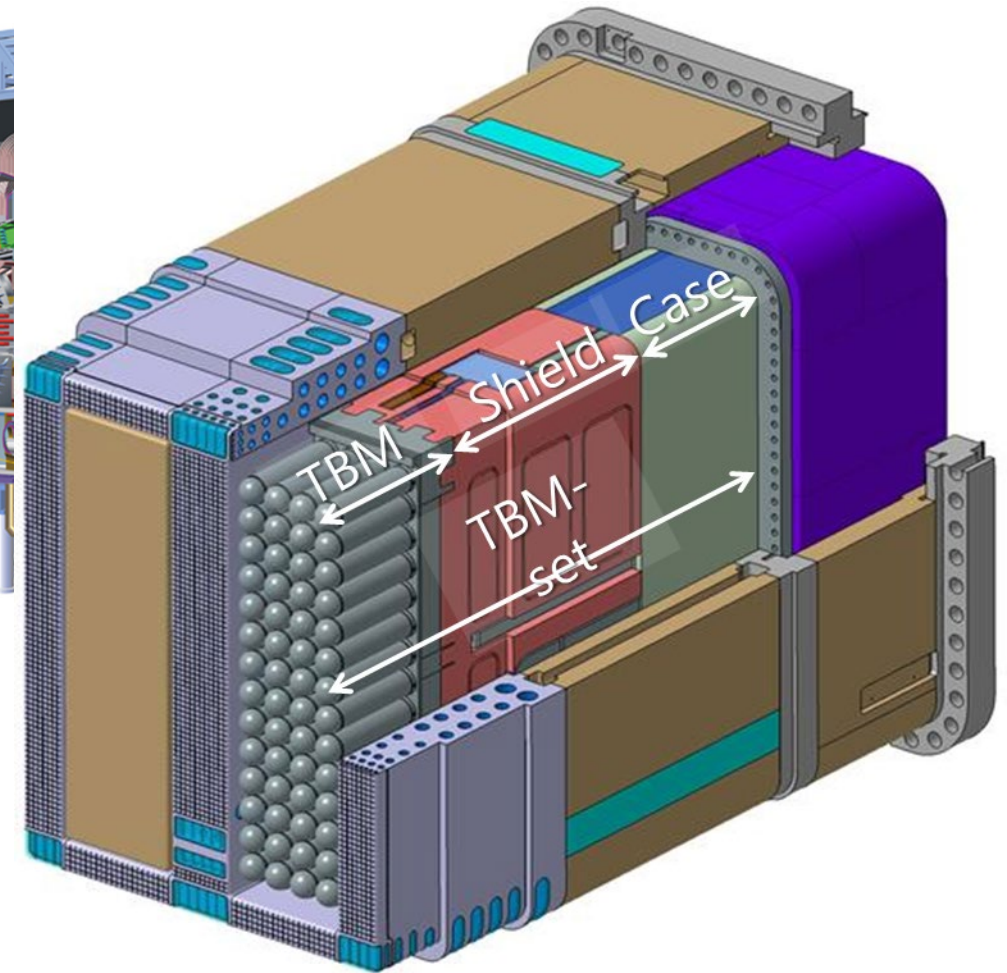
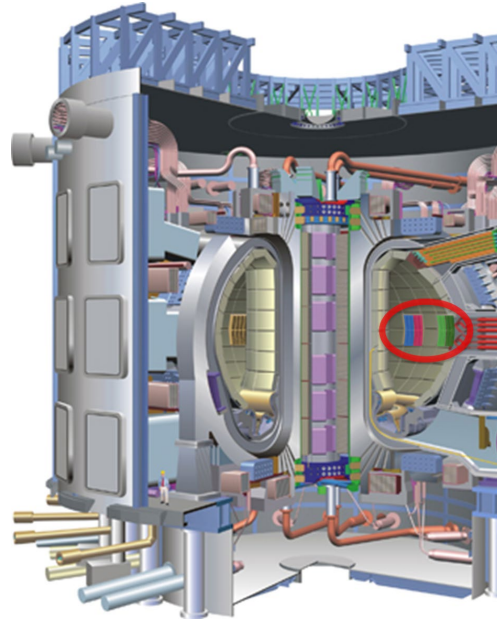


Cross-sectional schematic illustration of the TBM showing with material allocations and cooling channels.

<https://doi.org/10.1016/j.fusengdes.2020.112050>

Water cooled ceramic breeder blanket

Structural material	Reduced activation ferritic martensitic steel, F82H
Tritium breeder	Φ1 mm Li_2TiO_3 pebble
Neutron multiplier	Φ1 mm Beryllium pebble
Coolant	15.5 MPa / 280 °C water
Sweep gas	0.14MPa helium gas with 1% H_2

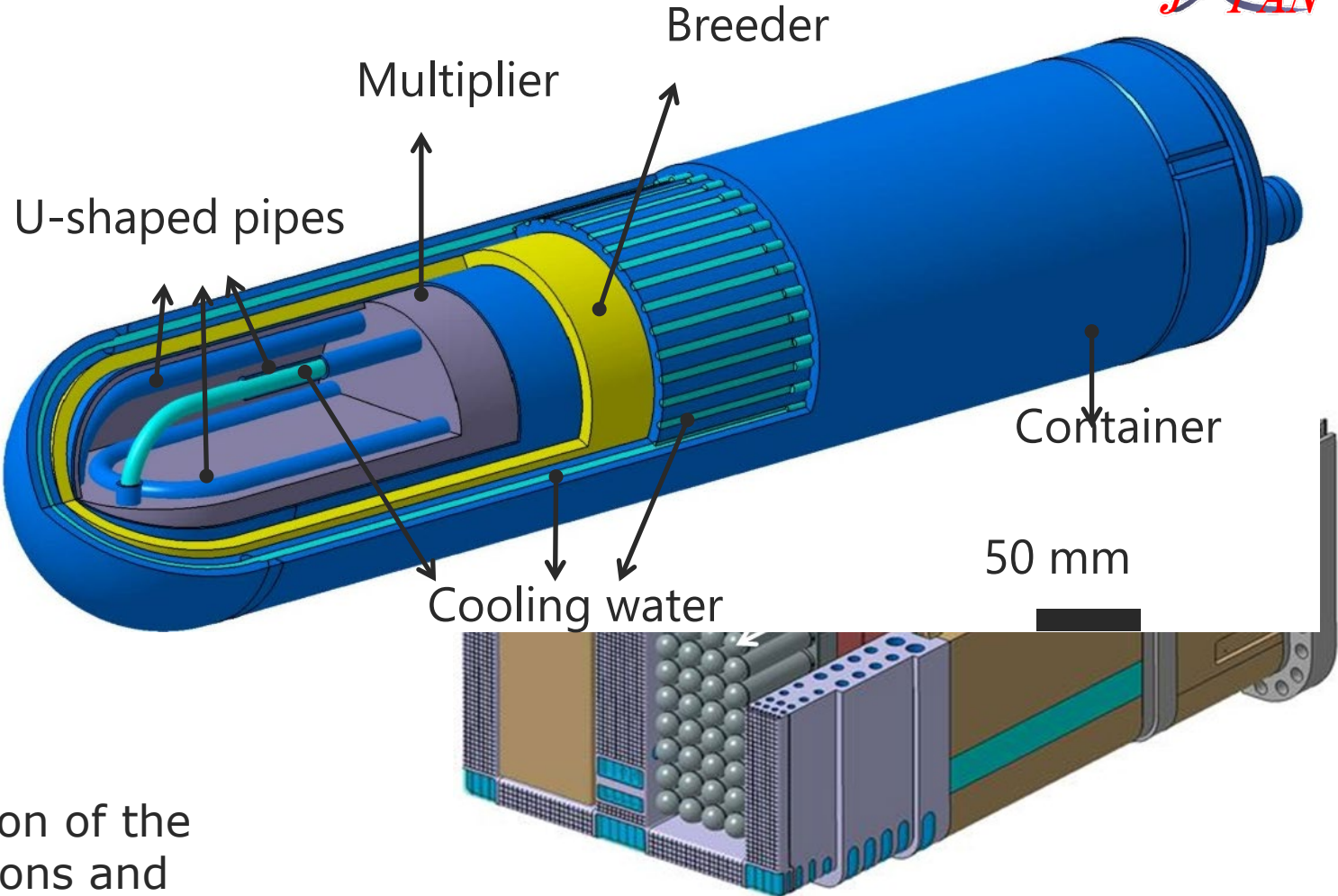


Cross-sectional schematic illustration of the TBM showing with material allocations and cooling channels.

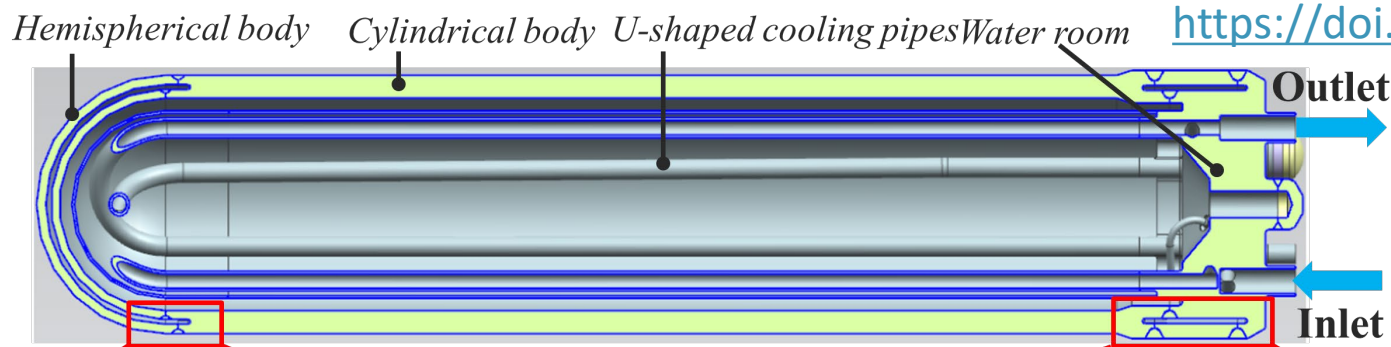
<https://doi.org/10.1016/j.fusengdes.2020.112050>

Water cooled ceramic breeder blanket	
Structural material	Reduced activation ferritic martensitic steel, F82H
Tritium breeder	Φ1 mm Li ₂ TiO ₃ pebble
Neutron multiplier	Φ1 mm Beryllium pebble
Coolant	15.5 MPa / 280 °C water
Sweep gas	0.14MPa helium gas with 1% H ₂

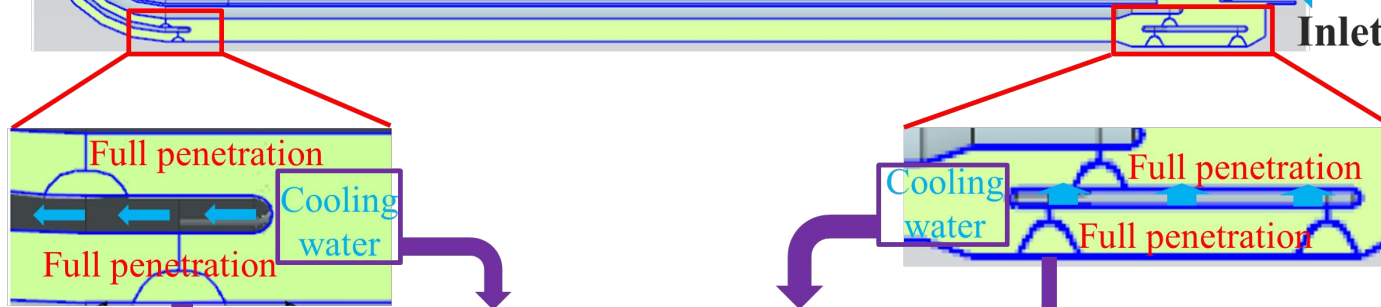
Cross-sectional schematic illustration of the TBM showing with material allocations and cooling channels.



<https://doi.org/10.1016/j.fusengdes.2020.112050>



<https://doi.org/10.1016/j.fusengdes.2023.113637>



Design conditions	
Pressure [MPa]	Temperature [°C]
17.0	343

Pressure boundary

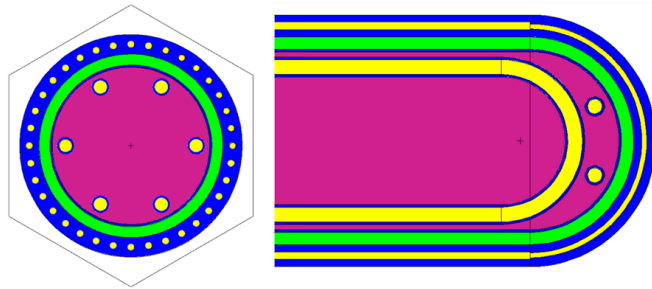
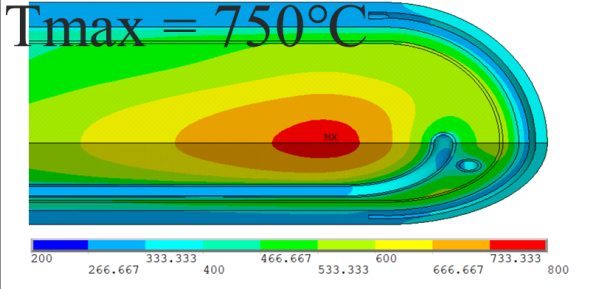
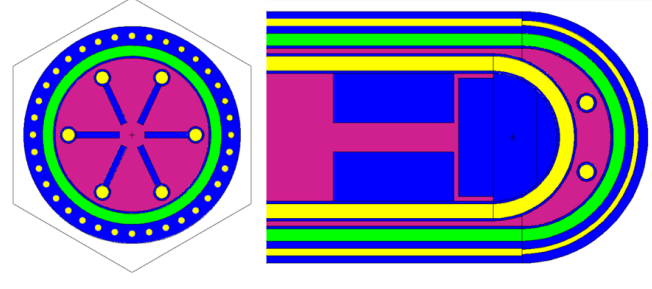
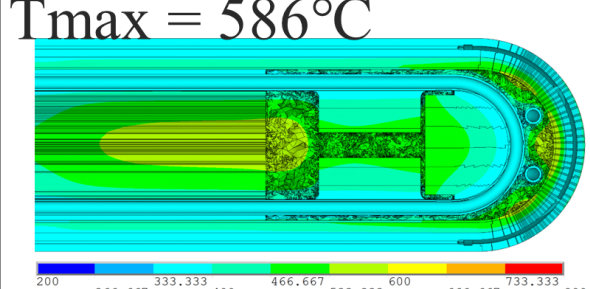
No harmonized standard defining material properties and weldability for F82H in RCC-MRx.

+

Third party validation (N2 ESPN CAT. IV is applied to WCCB TBM)

- ❑ Coolant manifold locates at the back, and distributed to container and internal cooling pipes.
- ❑ Helium gas flows into breeder layer to sweep out produced tritium.
- ❑ The cylindrical submodule remains within the elastic range under the expected loading conditions in ITER, and the cyclic loads are kept below the fatigue limit.

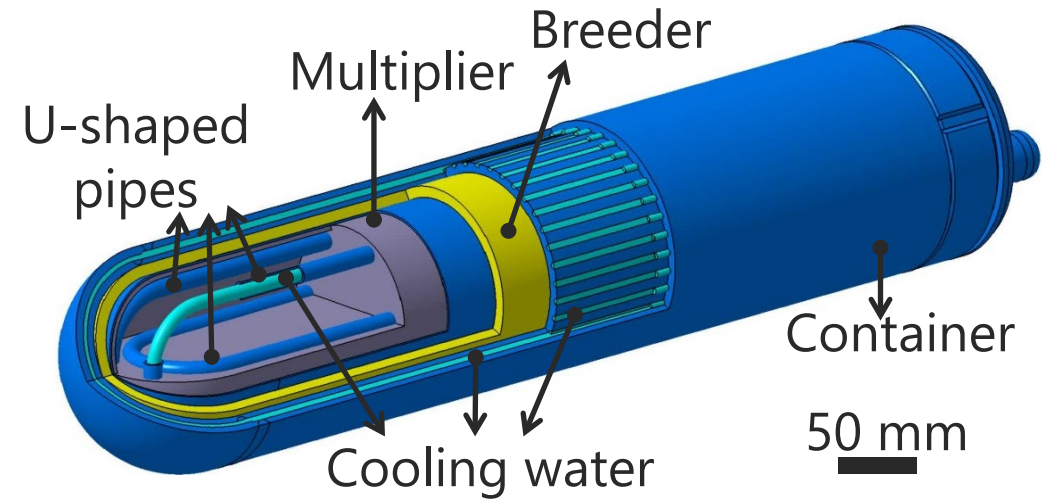
- ✓ The container's structural integrity including LOCA condition has been confirmed according to RCC-MRx Edition 2018 criteria.

	Submodule structure	Temperature distribution	TBR
U-shaped cooling pipes			1.12
Cooling pipes w/ fins			1.09

- Cooling fin was applied to decrease multiplier temperature to avoid temperature excursion due to Beryllium – water reaction in case of LOCA.

<https://doi.org/10.1016/j.fusengdes.2023.113637>

- ❑ Radial-axis cylindrical structure ensures:
 - Stable containment of high-pressure coolant
 - Efficient heat dissipation to vacuum vessel in case of failure
 - Even when cooling water leaks into its interior, the cylindrical container remains within the elastic range of F82H.



<https://doi.org/10.1016/j.fusengdes.2018.04.105>

MANUFACTURING AND ASSEMBLY



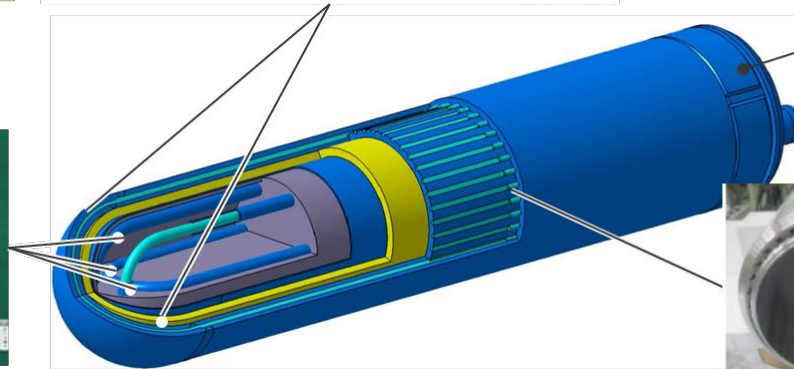
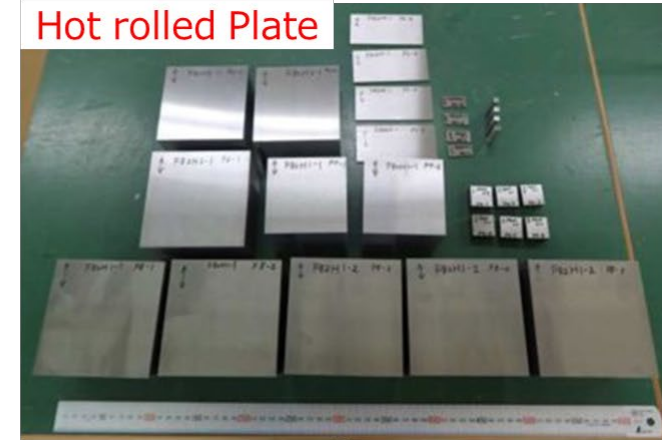
Slab

Billet

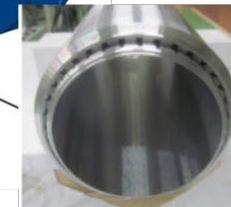
Forged Hemispherical body



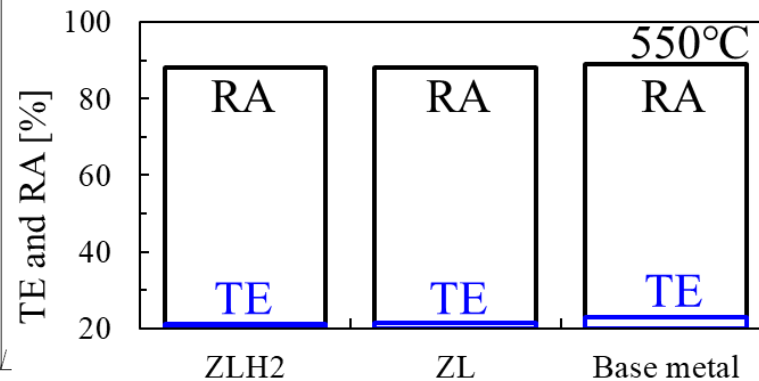
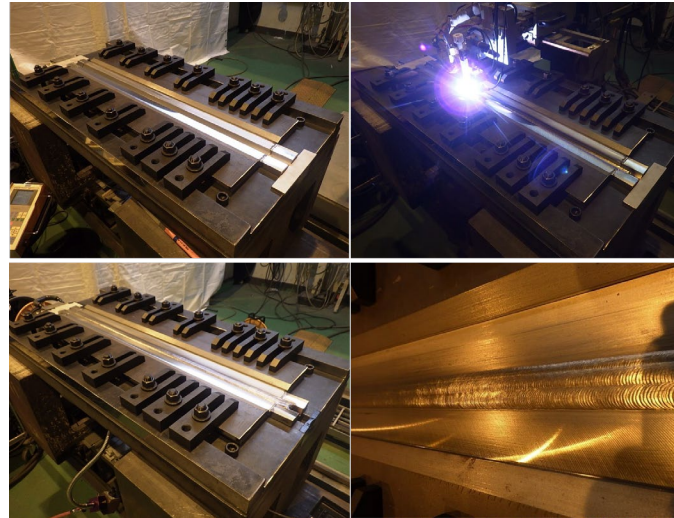
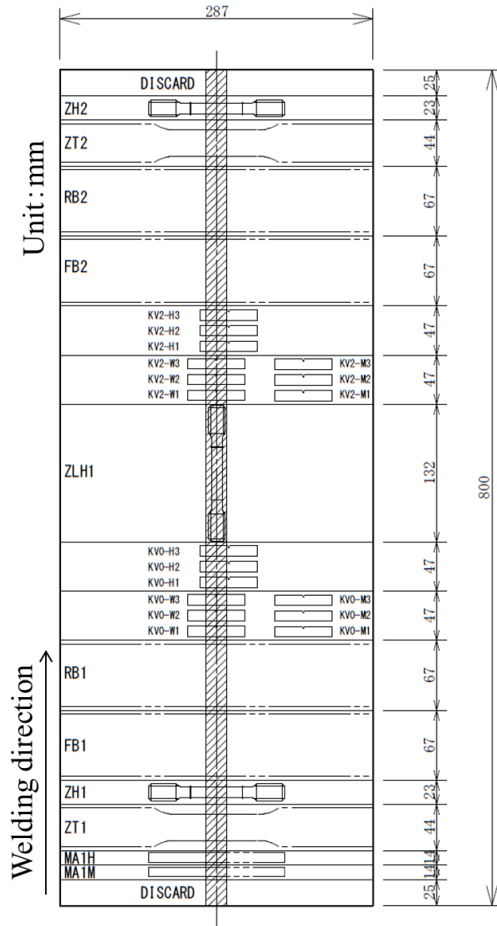
Hot rolled Plate



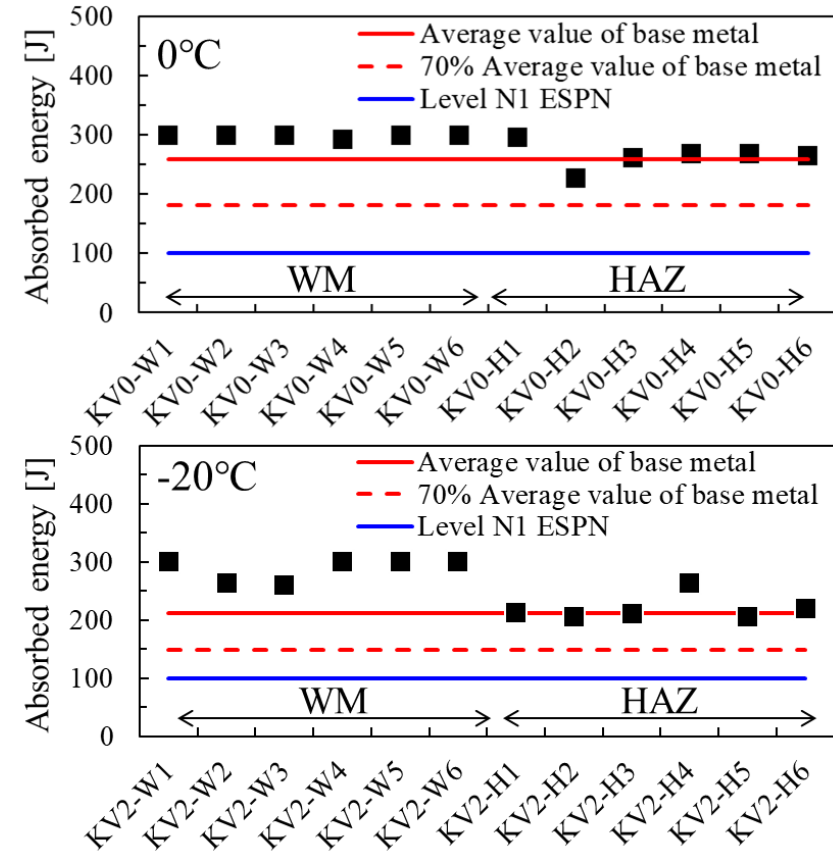
Forged Cylindrical body



- 5 tons heat of F82H has been produced taking account for European harmonized standard.
- F82H is to be used as TBM using NPMA without registering RCC-MRx.
- Material properties including long term properties were summarized as material file.



<https://doi.org/10.1016/j.fusengdes.2024.114202>



➤ General weldability of F82H was assessed with a third party inspector according to European harmonized standard. **No show stopper was found.**



WPS-R1006S

TIG welding between U-shaped pipe
and manifold

OD: 8 mm

Thick: 1 mm, I-butt weld w/o filler

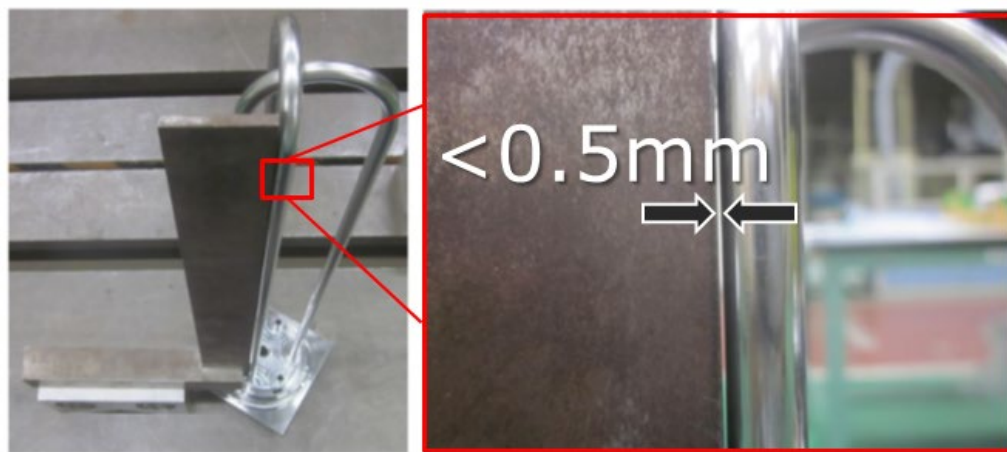
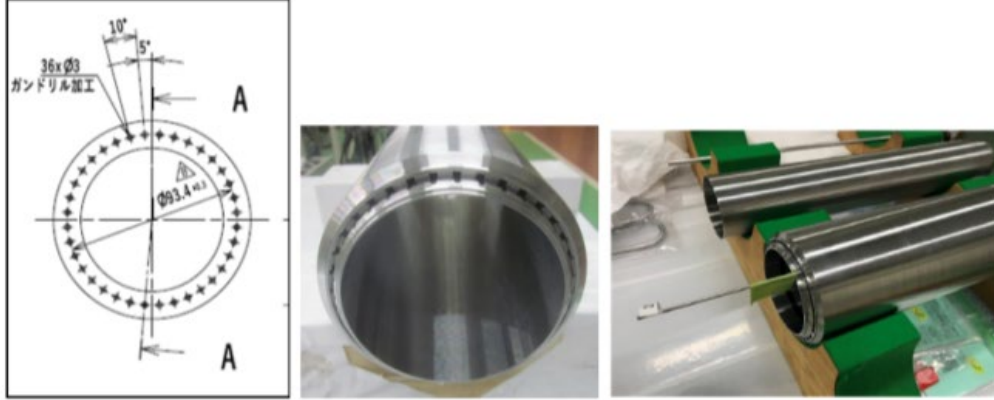
Single path w/ pure argon gas for
backing and shield

<https://doi.org/10.1016/j.fusengdes.2024.114227>

Material	Thick (mm)	Filler	Note
F82H	3.3	Φ1.2	Cylindrical body / Inner hemisphere
F82H	1	n/a	Cylindrical body / Separator
F82H	1.9	Φ1.0, 1.2, 1.6	Cylindrical body / Inner hemisphere, Backing
F82H	5.6	Φ1.2	Cylindrical body / Outer hemisphere
F82H	1	n/a	U-shaped pipe / manifold
F82H	0.35	n/a	U-shaped pipe / manifold, fillet
F82H	0.7	n/a	Sweep gas line
F82H	0.2	Φ1.0, 1.2, 1.6	Plug, fillet
F82H	0.35	Φ1.0, 1.2, 1.6	Plug & manifold, fillet
F82H	1	n/a	U-shaped pipe / manifold
F82H	5	Φ1.2	Cylindrical body / manifold
F82H	5.6	Φ1.0, 1.6, 2.0	Cylindrical body / manifold
F82H	5.6	Φ1.0, 1.6, 2.0	Cylindrical body / manifold
F82H/Inconel	1	n/a	Pipes
Inconel/SS316L	1	n/a	Pipes

TIG welds were applied. Full-penetration was achieved at the pressure boundaries.

<https://doi.org/10.1016/j.fusengdes.2024.114227>



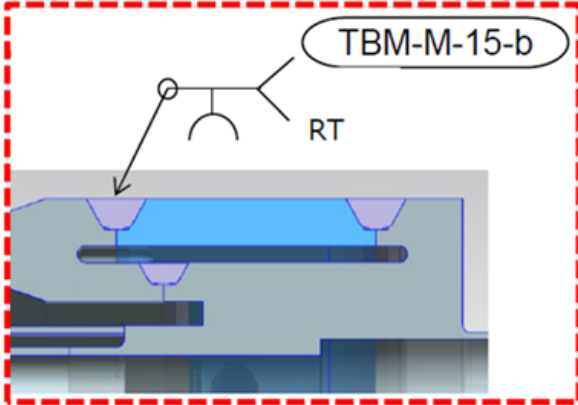
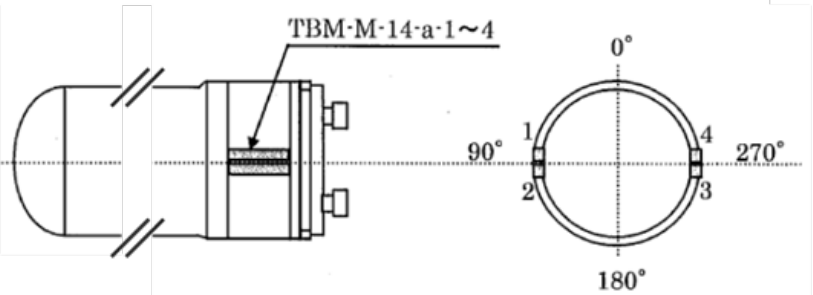
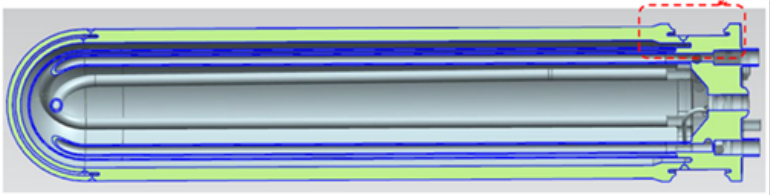
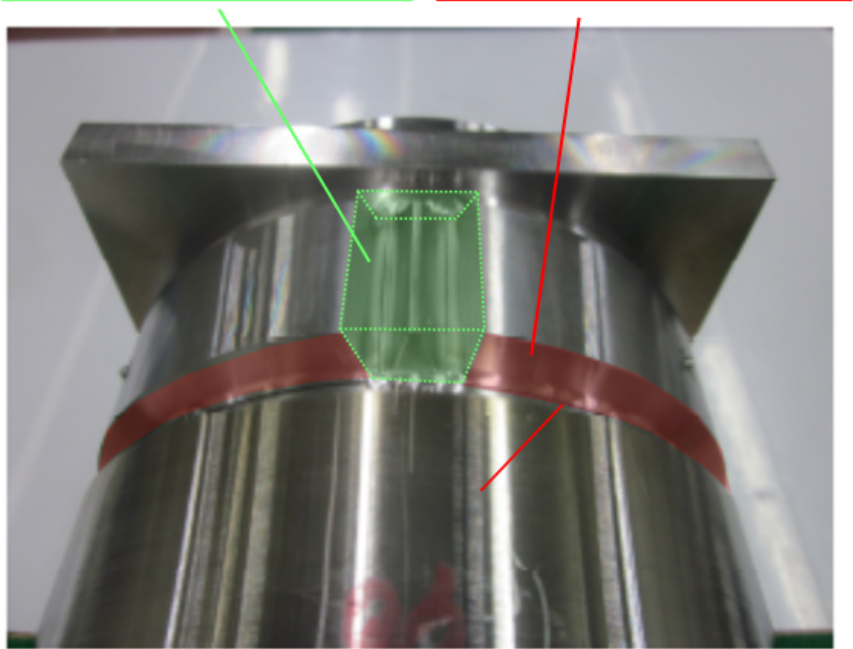
Inspection	Interest	Results
Penetrant tests	Elbow in the U-shaped tubes	No indication
	Welds	No indication
Dimensions	Drilled cooling channels	Insert angle < 5/100°
		Position in cylinder thickness < ±0.2 mm
Square	U-shaped tube	< 0.5 mm
Ovalization	U-shaped bend	$(d_{\max} - d_{\min})d_e < 7\%$ where d_e : outside diameter before bending
Distortion	Welds	Max. 2.2 mm in radial direction. Needs to be compensated by parts dimensions

Through the prototyping process, we obtained data on the dimensional tolerances and welding deformations of components that should be taken into account in the design and fabrication of the actual equipment.

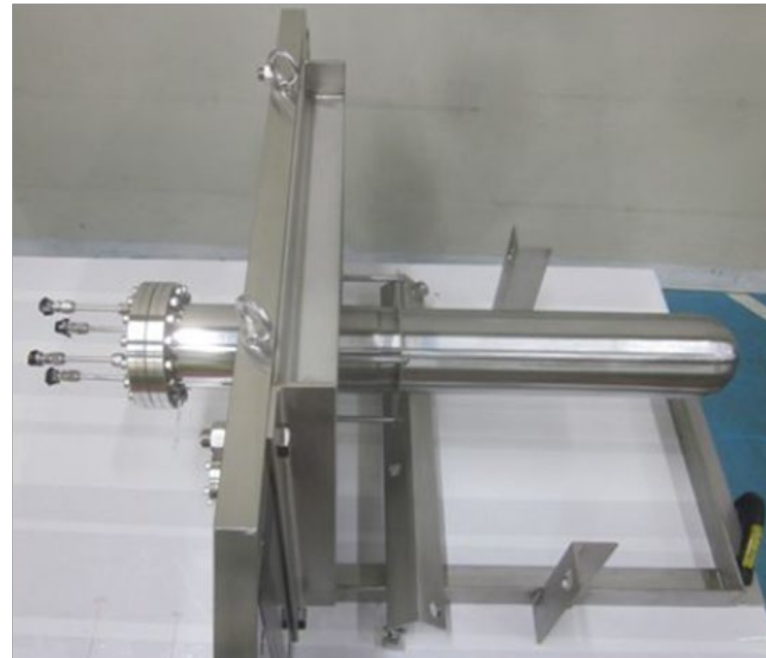
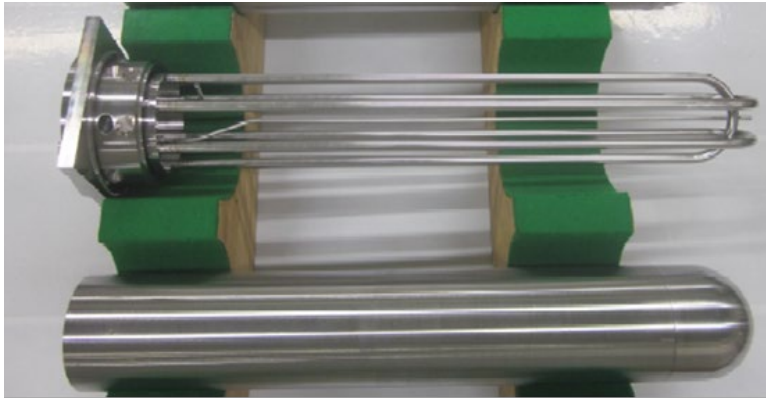
ID	Part	Thick. (mm)	Note
TBM-M-14-a	Coolant separator / Manifold	5.6	Hardly distinguish the weld from contents inside. According to RCC-MRx, alternative inspection is to be investigated.
TBM-M-15-b	Cylindrical body / Manifold	5.6	

TBM-M-14-a

TBM-M-15-b

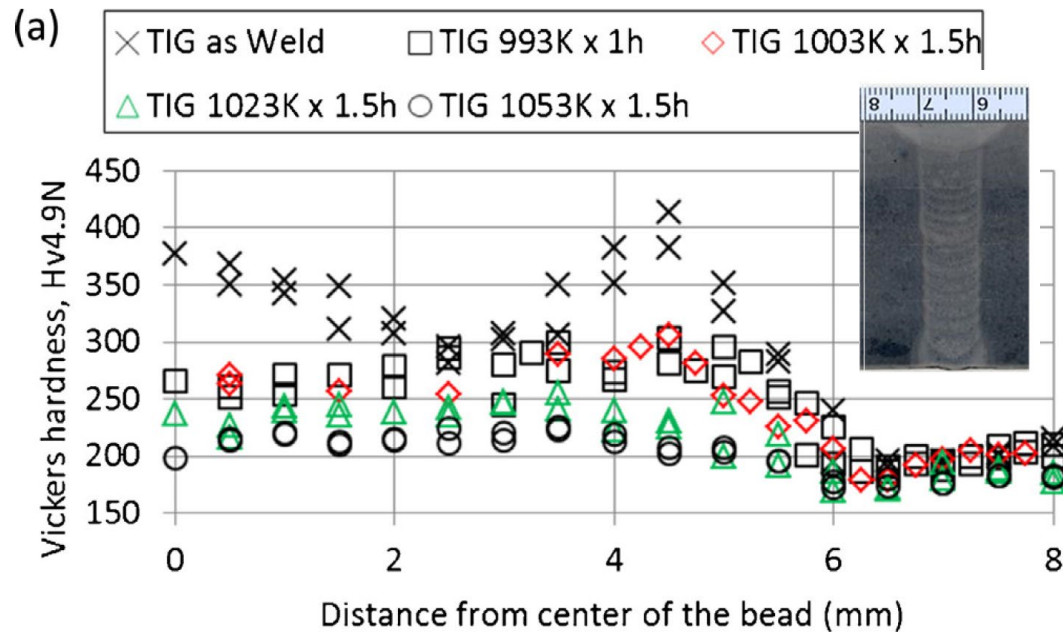


➤ The pressure resistance of the cooling water channel was confirmed by a 25.8 MPa hydraulic test with no leakage observed.



- ✓ Welding Procedure Specification have been developed for the manufacturing
 - ▣ It was identified the weld joints that are hardly radiographically inspected due to internal components, alternative inspection is to be investigated: ex) penetrant tests per layer, pressure test with bigger margin
 - ▣ Tolerance due to assembly and welding distortion have been assessed and it can be controlled by the dimensions of parts
- This first mock-up is being tested at EB high flux testing machine for its cooling capability.

- ❑ F82H should be PWHTed at 720°C+15/-0 for 60 min.
- ❑ Higher temperature or longer duration cause softening.



<https://doi.org/10.1016/j.fusengdes.2015.06.133>

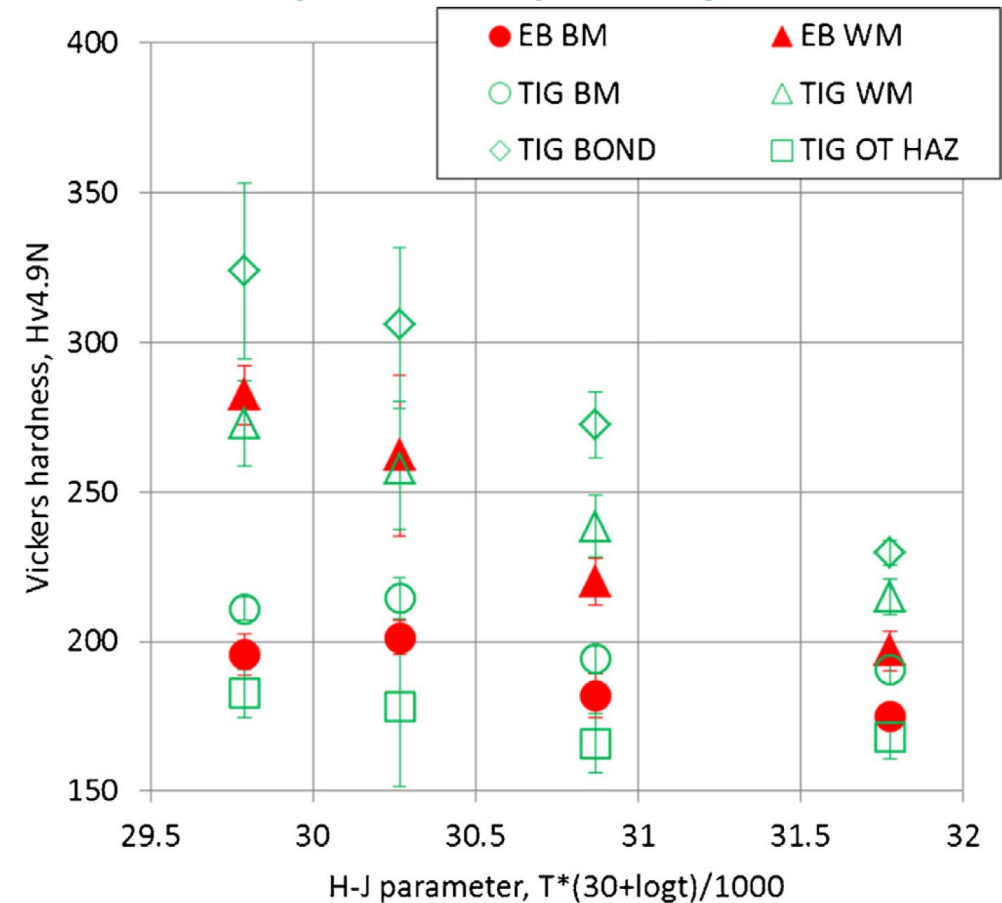


Fig. 5. PWHT effects on the hardness of F82H welds.

<https://doi.org/10.1016/j.jnucmat.2008.12.273>

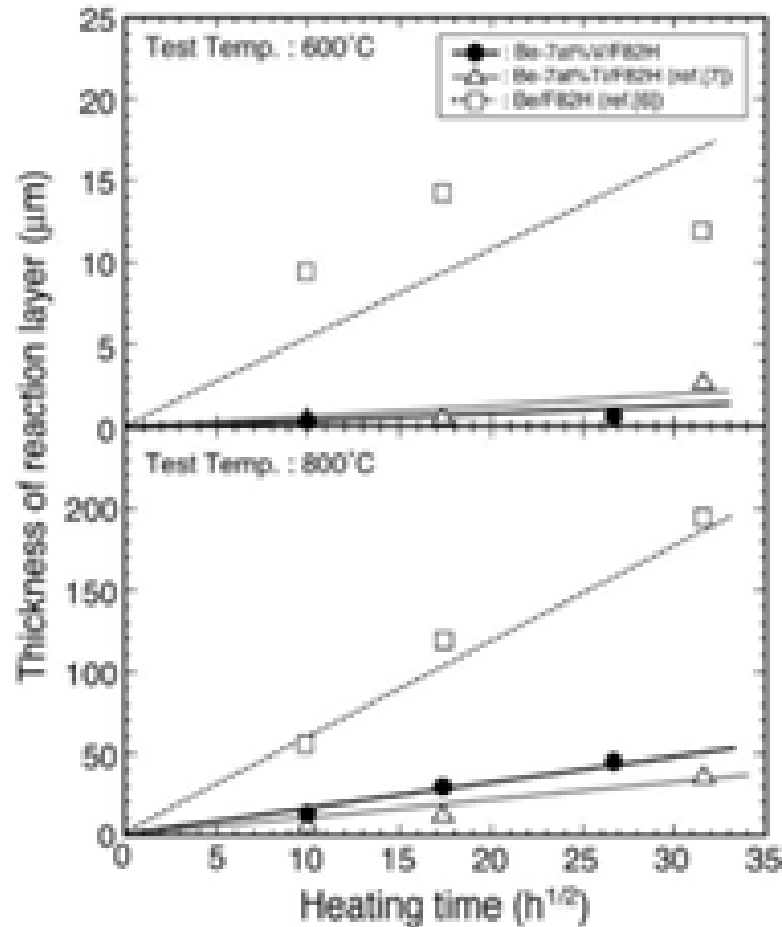


Fig. 5. Change in thickness of the reaction layer at the surface of F82H with annealing time in compatibility tests between Be-7 at 3V and F82H.

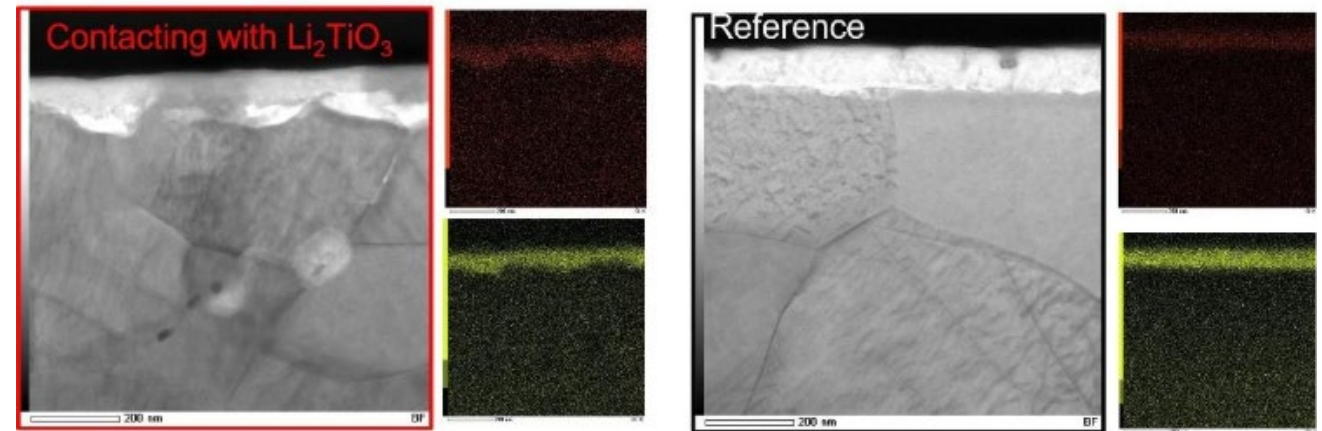


Figure. 9 TEM images of F82H contacting Li₂TiO₃ under PWHT conditions (left) and reference (right)

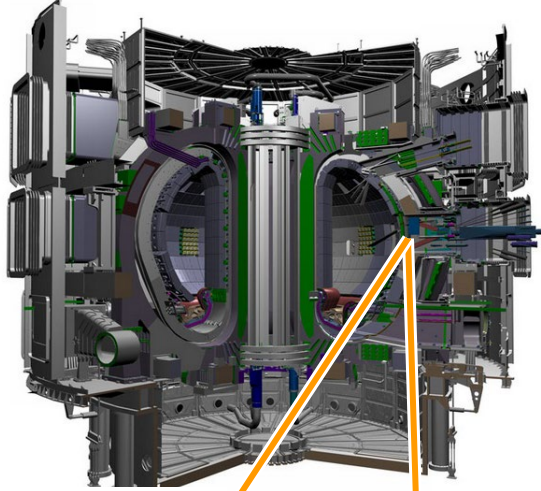
- ❑ Thickness of reaction layers of Beryllium and Li₂TiO₃ are 3.6μm and <0.2 μm, respectively.
- ✓ Post-weld heat treatment (PWHT) is not required for sealing the pebble filling ports by welding, as the material used is either Inconel or 316L stainless steel.

<https://doi.org/10.1016/j.fusengdes.2024.114227>

- ❑ The Japanese WCCB-TBM has demonstrated robust structural integrity and manufacturability under ITER-relevant conditions.
- ❑ Key technologies such as welding, heat treatment, and precision assembly have been studied through mock-up fabrication and testing.

- While nuclear performance remains a central focus in the development of breeding blanket technologies, it is equally vital to ensure manufacturability, inspectability, and compliance with regulations and corresponding standards.
- These aspects are not merely supporting factors—they are essential for realizing reliable and deployable fusion components. Our work demonstrates that a balanced approach integrating design, fabrication, and qualification is key to advancing toward practical fusion energy systems.

Breeding Blanket Module with box shape



ITER
<http://www.iter.org>

ITER Test Blanket Module (TBM) is tested to validate functions of Blanket in the fusion environment

Production of Fuel Tritium

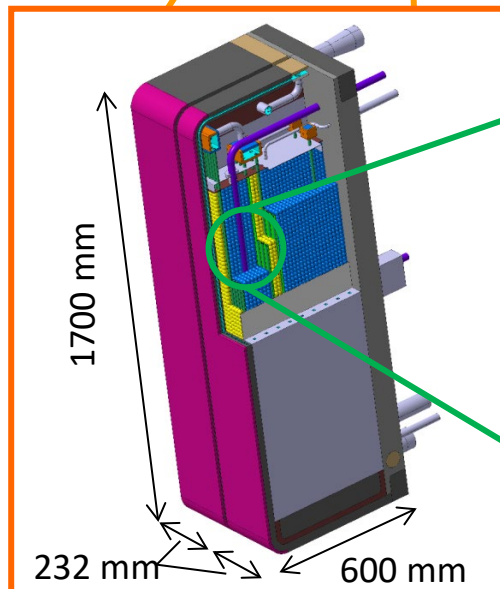
Energy Conversion

Structural Integrity

Armor tile = Beryllium for high heat flux.

First Wall = Ferritic/Martensitic Steel, F82H

(Fe-8Cr-2W) for irradiation damage.



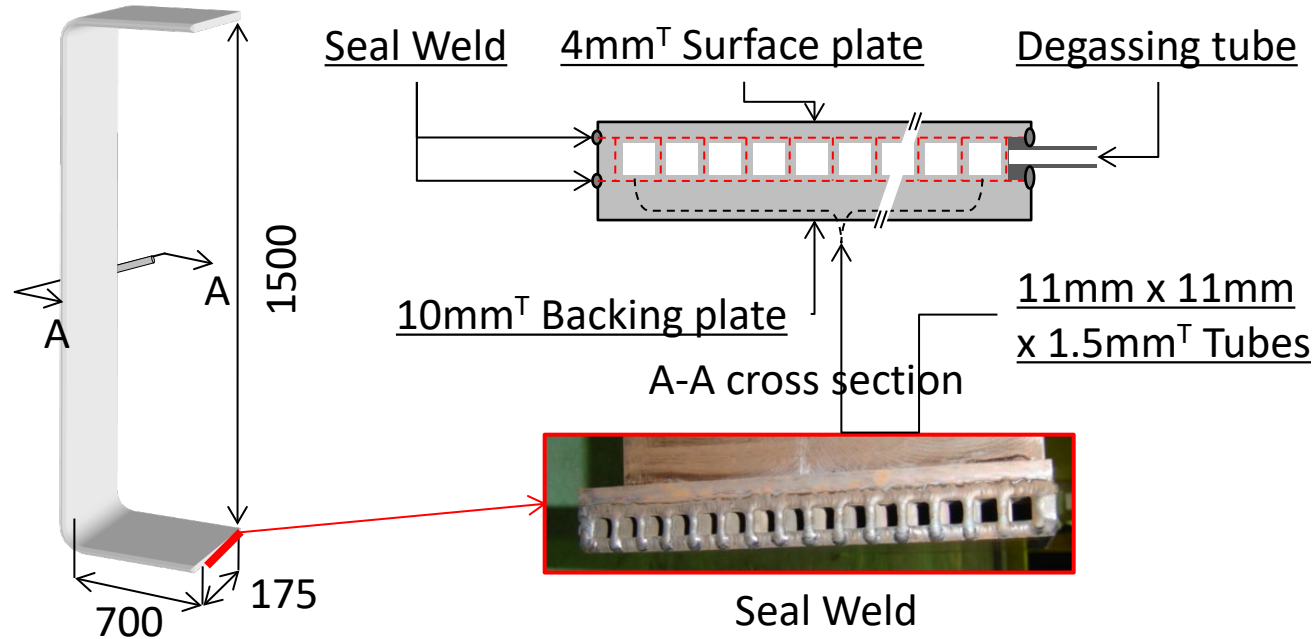
HIP has been developed as the most promising process for FW fabrication.

-U-Shaped First Wall with built-in cooling channels
at $1100^\circ\text{C}/150\text{MPa}$

-Dissimilar metal joint between Be and RAFM
at $750^\circ\text{C}/160\text{MPa}$

Water Cooled Solid Breeder (WCSB) Blanket

Fabrication Procedure of the FW



Bracing with grid

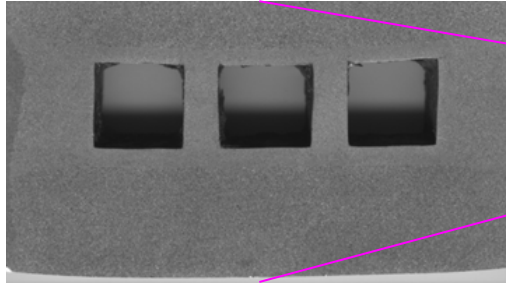
Fabrication Procedure of FW

1. F82H, tubes and plates
2. Acid washing to remove oxide
3. Bending to FW shape
4. Assembly with welding
5. Degassing at 600°C to reduce oxygen
6. HIP at **1100 °C x 150 MPa**
7. Heat treatment at 960°C and 750°C to obtain fine-grained microstructure

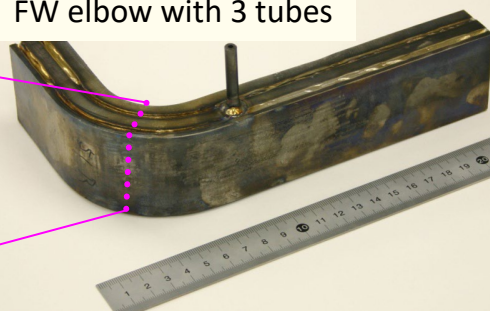
- ◆ HIP conditions are within limit of the available HIP units.
- ◆ Assembly gap is controlled below 0.5mm due to machining accuracy.
- ◆ Bracing successfully suppressed deformation during HIP process.

Fabrication of TBM-FW Mock-Up

Small scale partial mock-up

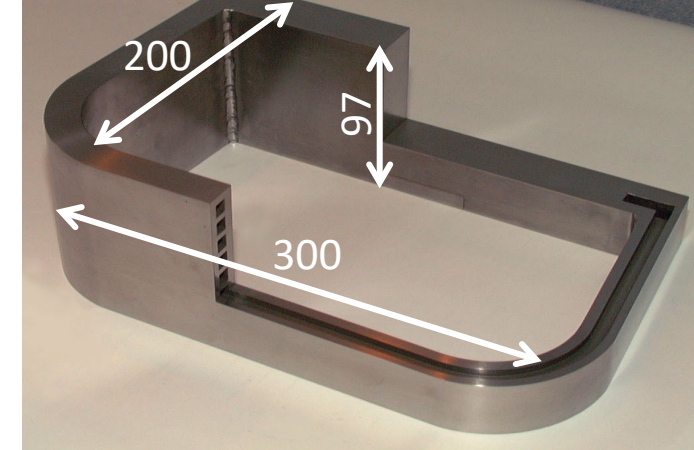


FW elbow with 3 tubes

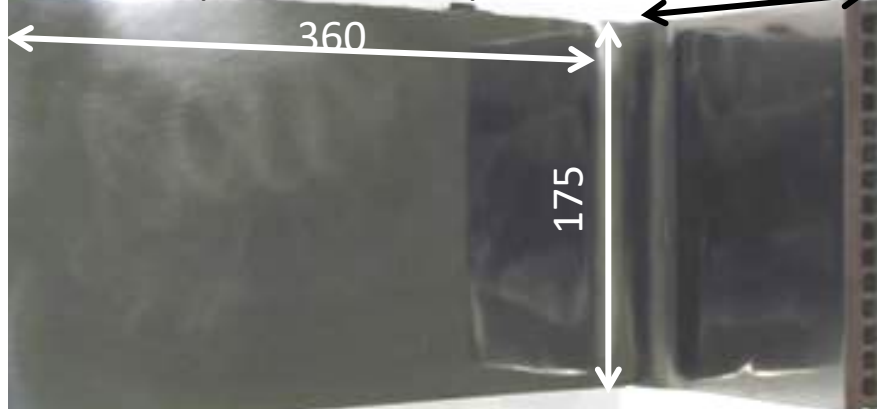


Small scale partial mock-up

Small scale TBM-FW

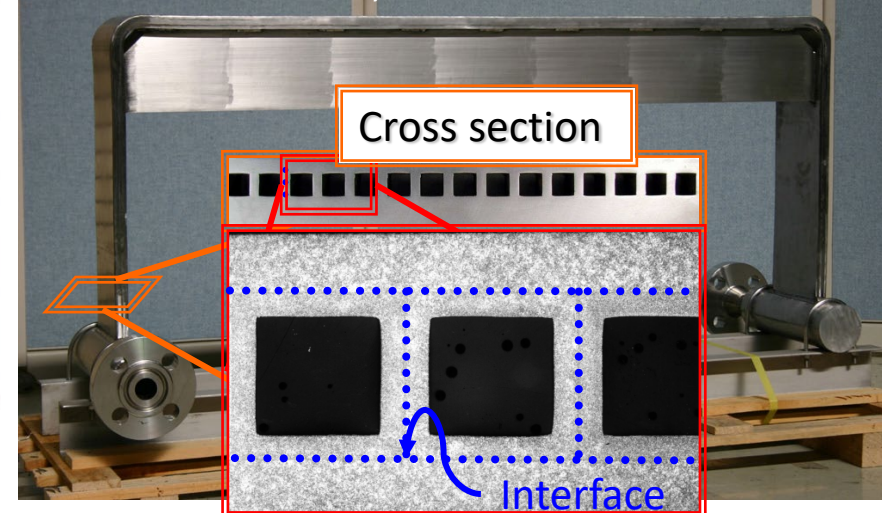


Full-scale partial mock-up



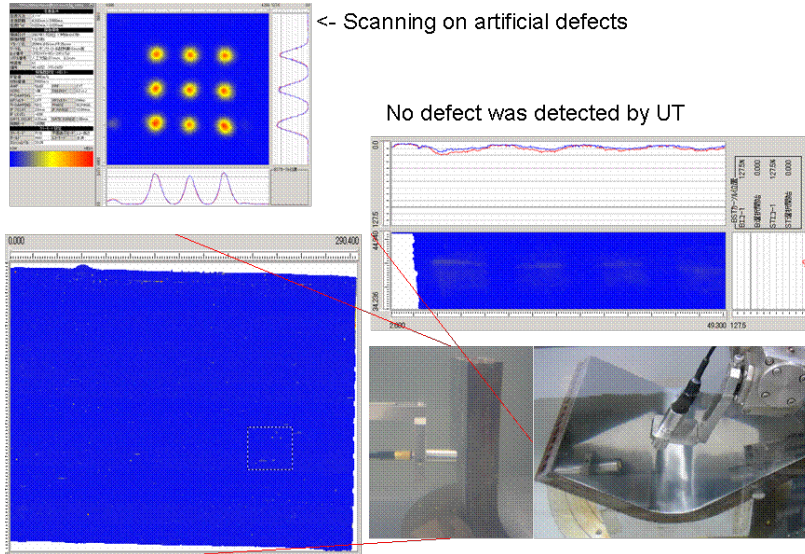
Destructive / non-destructive inspection

Full scale TBM-FW, 1500 x 176 x 600

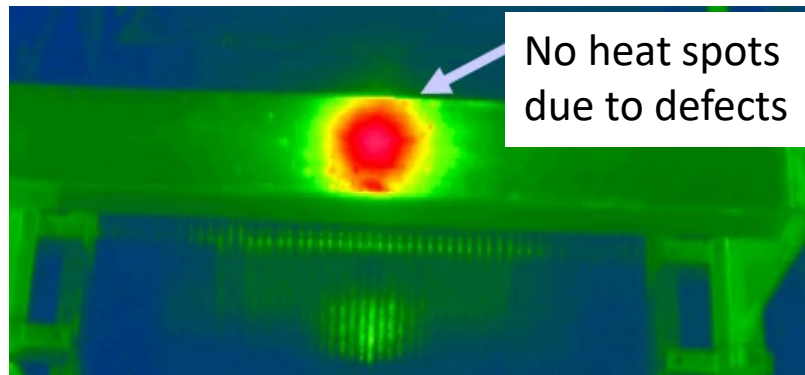


Fabrication of the TBM-FW was successfully validated in an industrial scale.

Inspection of the HIP Joint for TBM-FW



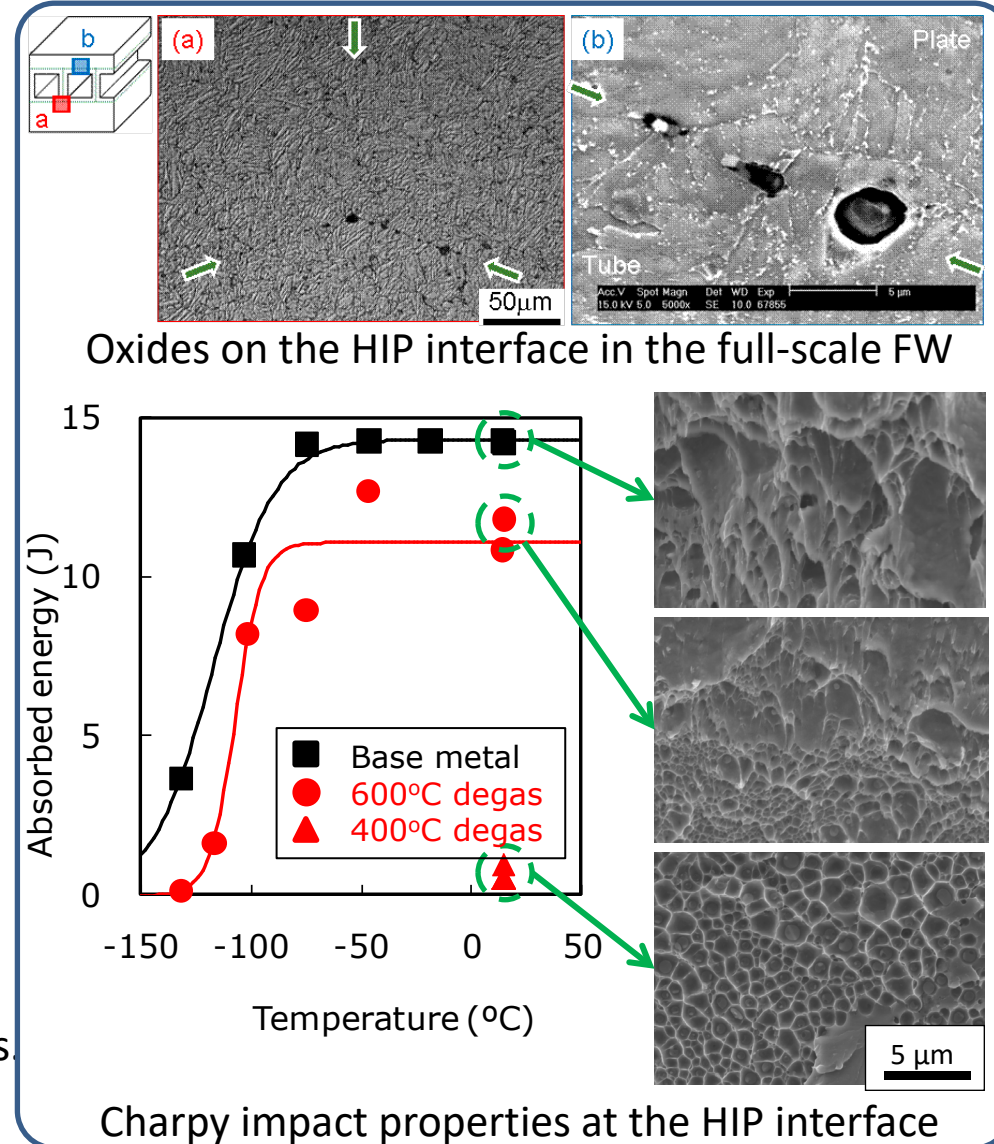
Ultrasonic inspection did not detect defects on the HIP interface.



High Heat Flux testing did not indicate defects.

IR image of high heat flux test using H⁺ ion.

Heat flux: 0.5MW/m², coolant: 15MPa/30°C water



Non detectable defects degrade the toughness at HIP interface

Optimization of HIP Conditions for TBM-FW

- Upper-Shelf-Energy (USE) in Charpy impact test is very sensitive to the quality of joint.
- Surface finishing is to remove surface layer including oxides and to improve contact condition.
- Degassing suppressed oxidation at the HIP interface.
- The USE after electrolytic polishing are very similar to that of base metal (BM).
- On the other hand, mirror like finishing does not improve the toughness without electrolytic polishing.

