

Cluster Dynamics Modeling of Defect Evolution in Neutron-irradiated Tungsten for Fusion Applications

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ABSTRACT

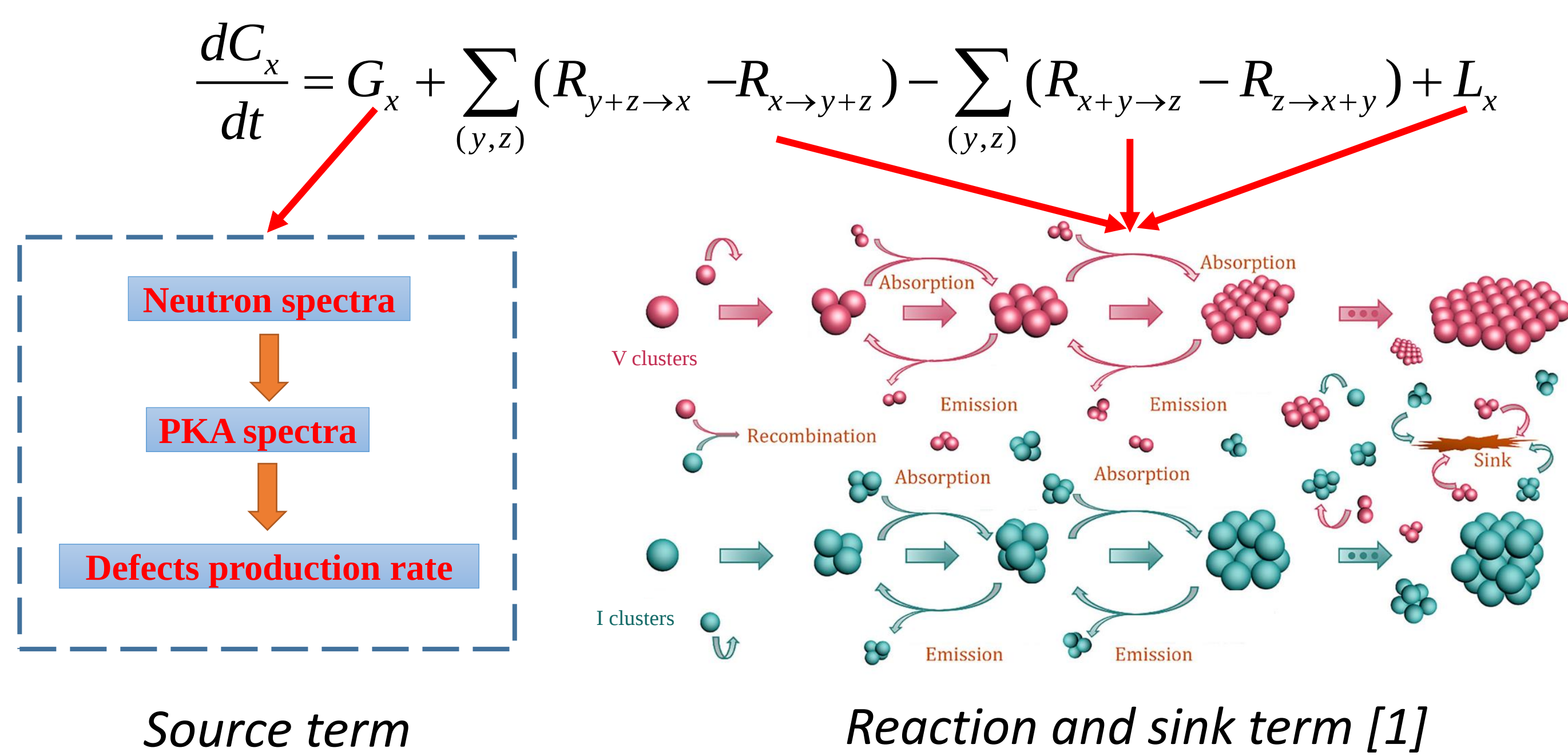
Tungsten is considered the most promising plasma-facing material, which can be exposed to high neutron loads at high temperature. However, a significant number of defects, such as dislocation loops, voids, and precipitates, are generated in W under neutron irradiation, leading to the degradation of its material properties. This study employed a cluster dynamics model that incorporates the one-dimensional migration of dislocation loops to simulate defect evolution in single-crystal and polycrystalline W under neutron irradiation at 600 – 1200 °C, and compared the results with experimental data obtained from the BR2 reactor. A general agreement with experimental results was achieved, and the underlying causes of the observed discrepancies were analyzed.

BACKGROUND

- Tungsten (W) is the primary plasma-facing material in fusion reactors, which suffers from microstructural defects such as dislocation loops, voids and transmutation precipitates, leading to severe property degradation.
- Experimental studies are costly and limited. While cluster dynamics (CD) models are key for simulating long-term defect evolution.
- There is a lack of simulations capable of directly and accurately comparing complex defect evolution processes in W. (rapid migration of dislocation loops, effects of transmutation-induced precipitates).
- This study uses a CD model that accounts for the rapid one-dimensional migration of dislocation loops and clusters larger than 10 nm. The simulation results are compared with experiments conducted on both single crystal and polycrystalline W in the BR2 reactor.

METHODS

- CD, based on the rate theory, is a mesoscale approach used to model the evolution of defects in materials.



- The reaction rates employed in this work are cited from Reference [2]

Reaction	Reaction rate coefficient k^+/k^-
Clustering reactions	
$V_x + V_y \rightarrow V_{x+y}; V_x + I_y \rightarrow V_{x-y} \text{ or } I_{y-x} \text{ (3D SIA)}$	$4\pi(r_x + r_y)(D_x + D_y)$
$I_x + I_y \rightarrow I_{x+y} \text{ (3D + 3D SIA)}$	$4\pi Z_{\text{int}}(r_x + r_y)(D_x + D_y)$
$V_x + I_y \rightarrow V_{x-y} \text{ or } I_{y-x} \text{ (1D SIA)}$	$(4\pi r_x + 2\pi r_y)D_x + \pi^2(r_x + r_y)^4 D_y C_x$
$I_x + I_y \rightarrow I_{x+y} \text{ (3D + 1D SIA)}$	$Z_{\text{int}}(4\pi r_x + 2\pi r_y)D_x + Z_{\text{int}}^4 \pi^2(r_x + r_y)^4 D_y C_x$
$I_x + I_y \rightarrow I_{x+y} \text{ (1D + 1D SIA)}$	$Z_{\text{int}}^4 \pi^2(r_x + r_y)^4 (D_x C_y + D_y C_x)$
Dissociation reactions	
$I_{x+1} \rightarrow I_x + I; V_{x+1} \rightarrow V_x + V; I_{x+1} \rightarrow I_{x+2} + V$	$\frac{k^+}{\Omega} \exp\left(\frac{-E_b^+}{k_B T}\right)$

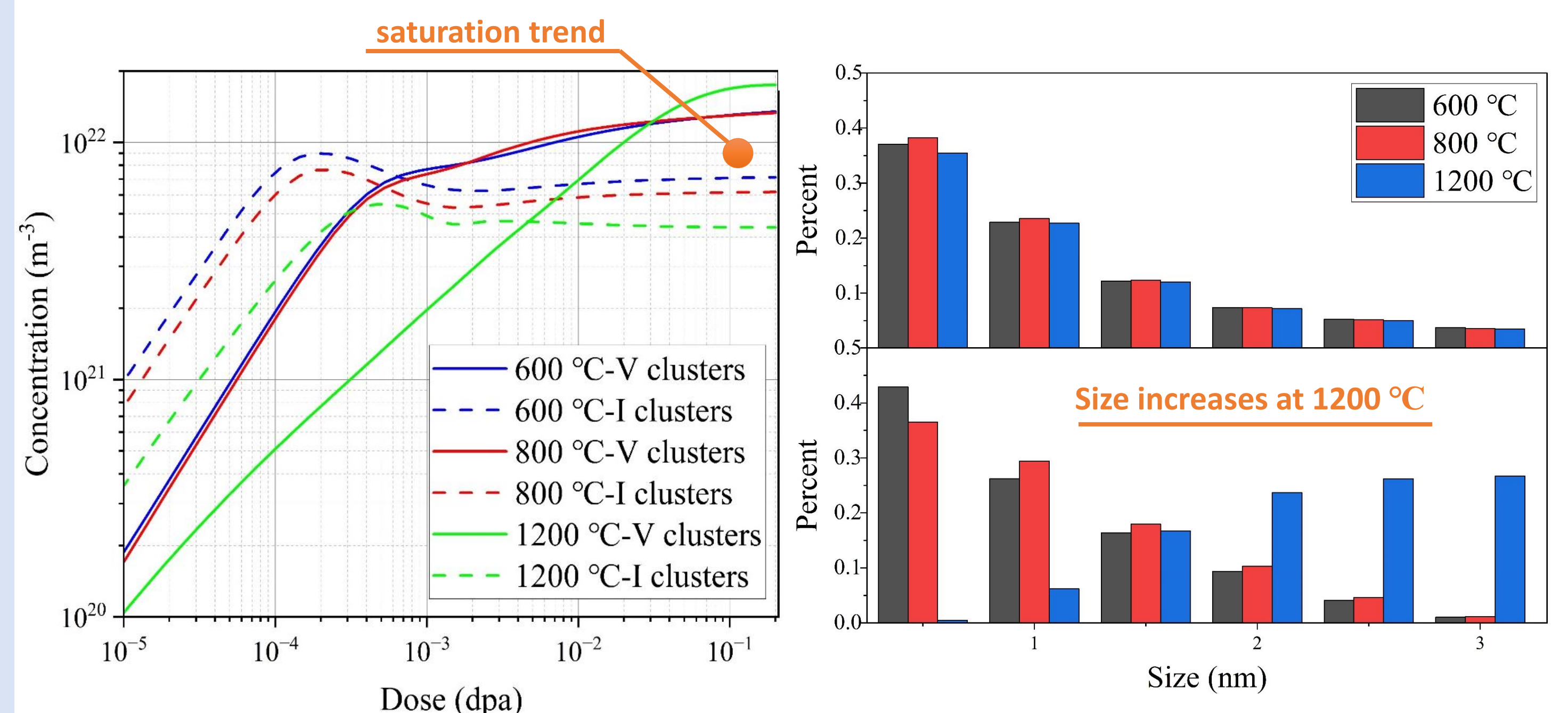
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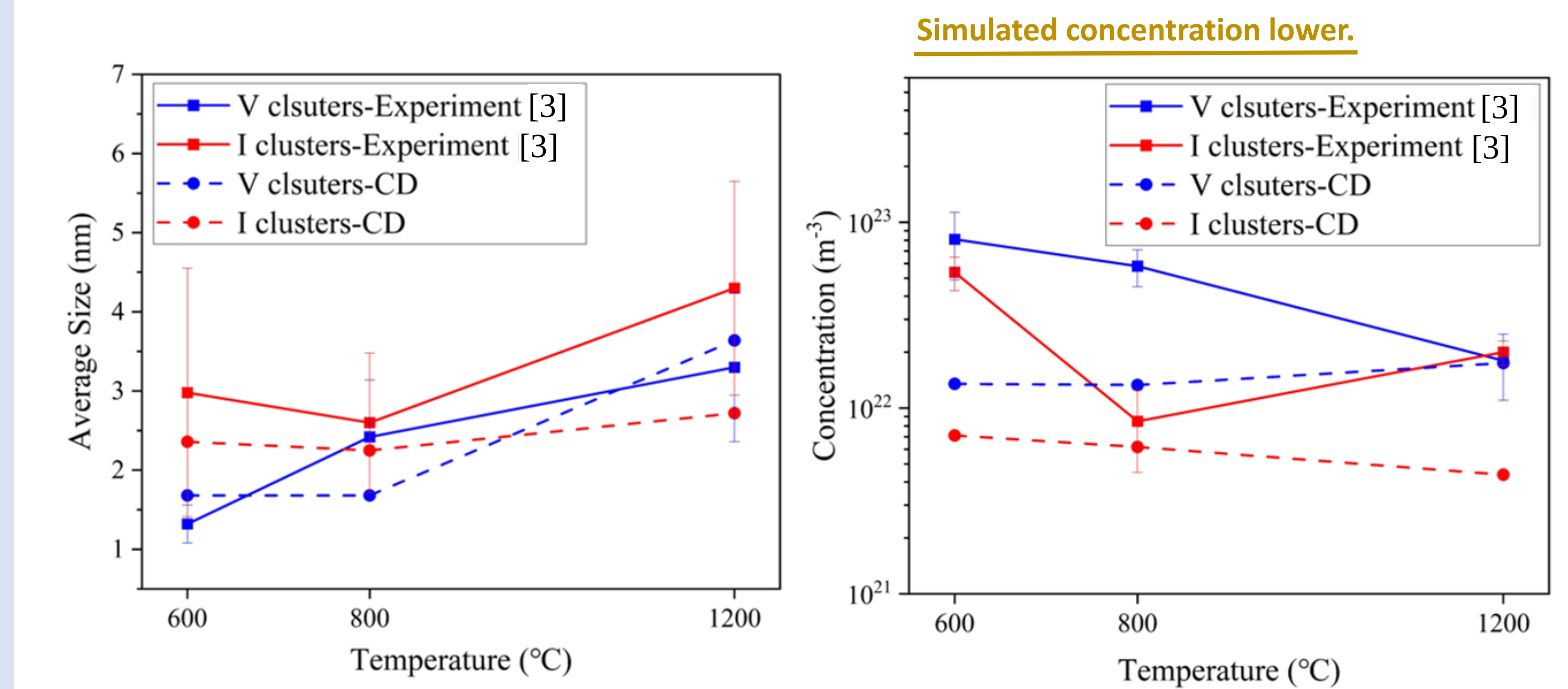
OUTCOME

Single crystal W simulation

- 600, 800, and 1200 °C up to 0.2 dpa in the BR2 reactor



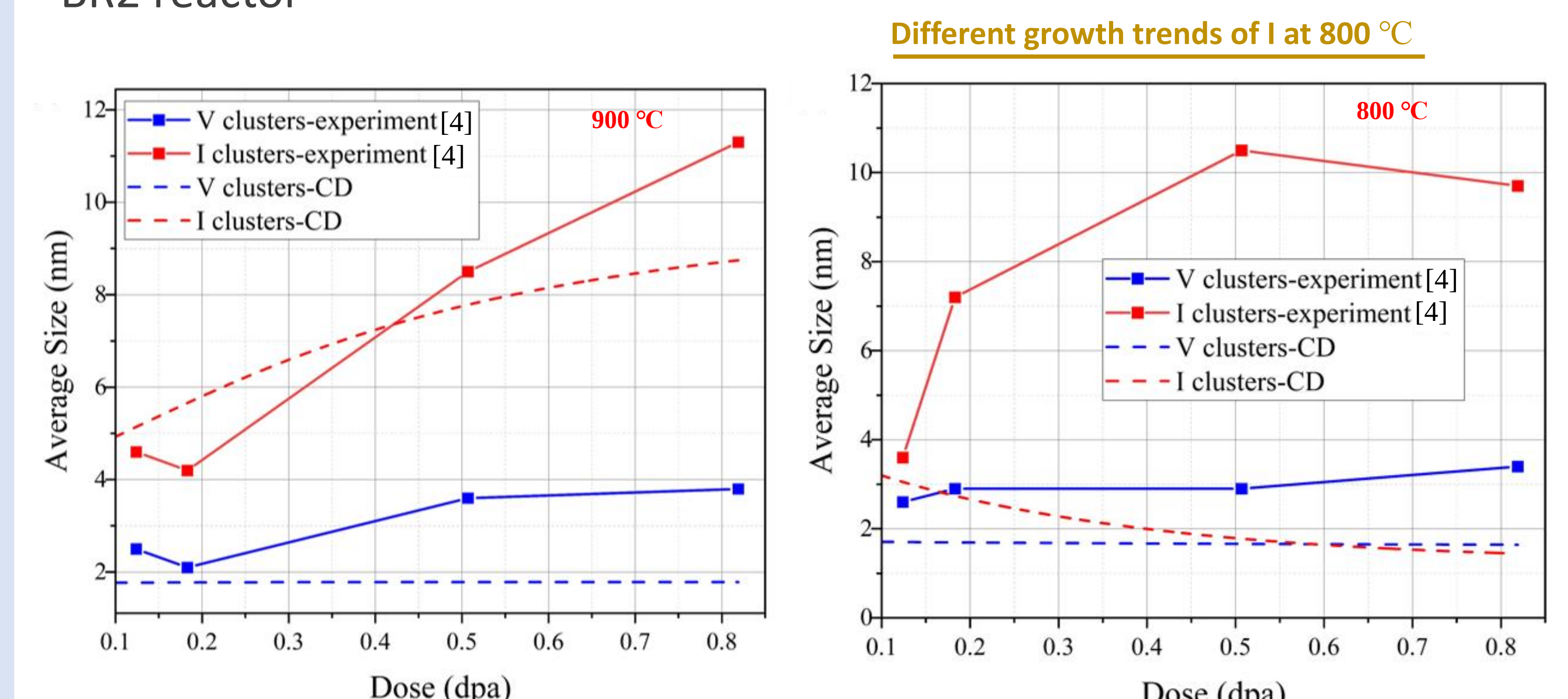
Total defect concentration and Size distribution (Size > 0.5 nm)



Comparison of average size and concentration with experiment

Polycrystalline W simulation

- 800 and 900 °C with doses of 0.124, 0.183, 0.507, and 0.819 dpa for the BR2 reactor



Relationship between average size and irradiation dose

CONCLUSION

- The simulation results demonstrate general agreement with the experimental data.
- Discrepancies are observed between the simulation and experimental results, specifically in the overall magnitude of defect concentration and the size of clusters in polycrystalline tungsten at lower temperatures.
- Further model improvements are necessary and will likely focus on cluster migration rates and the influence of transmutation-induced precipitates.