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ABSTRACT

CAD geometry supports engineering-level fidelity in NBI-induced ion-loss analysis

Using a KSTAR PFC CAD geometry and an efficient 3D collision-detection routine, we found (Fig. 1):

- Diagonal, band-like, and non-axisymmetric ion loss distribution by the combined poloidal turn and toroidal drift of fast ions.
- Localized heat flux peaks at protrusions and leading edges of the tungsten divertor.

Parameter scan over NB source (beam path), energy, and magnetic equilibrium

• Observed results can suggest ion loss mitigation guidance (limited to scanned range).

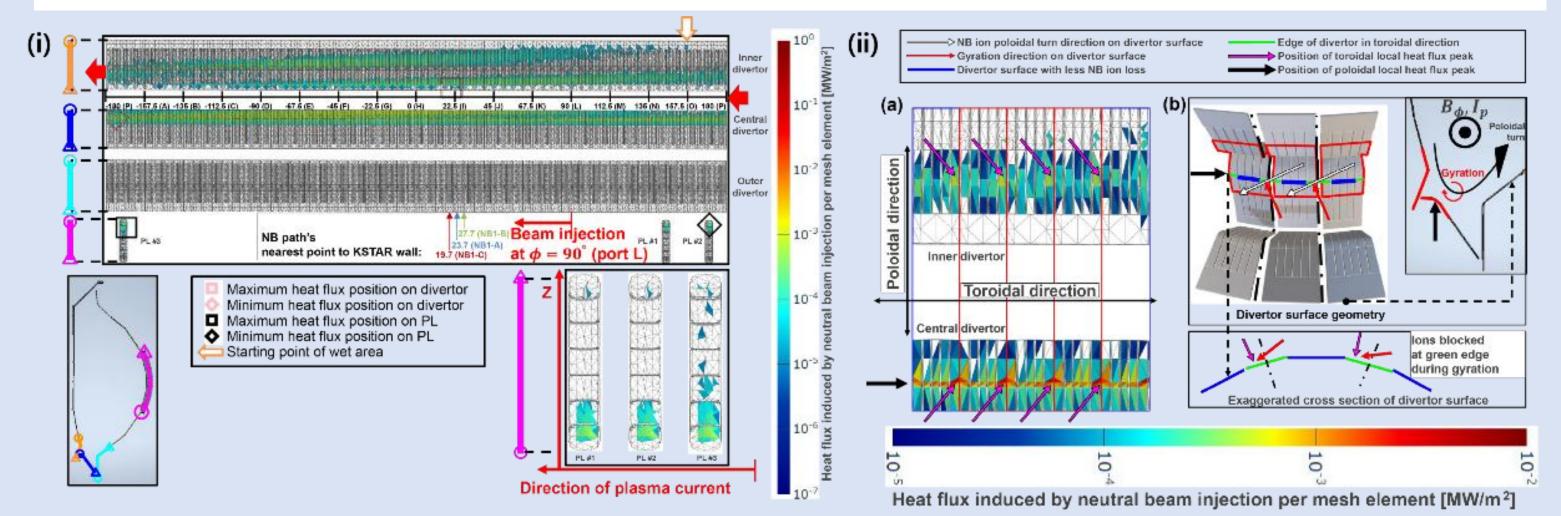


FIG. 1. (i) NuBDeC heat-flux map on unfolded divertor and PL surfaces for the reference case (100 keV NB1-C; equilibrium I in Fig. 2-i). (ii-a) Zoom of the inner/central divertor at 130–160° toroidal angle. (ii-b) 3D view of inner, central, and outer divertors; red outlines correspond to the three red rectangles in (ii-a).

BACKGROUND

- NBI heating generates fast ions; a fraction escapes confinement and deposits heat flux on PFCs, risking localized overheating [1-3].
- Accurate heat-load modeling requires geometry fidelity: early axisymmetric (2D poloidal) vessels were common [4, 5], whereas recent work adopts full 3D CAD models for higher-fidelity predictions [1, 6].
- CAD-to-simulation pipelines bring engineering-level geometry into synthetic diagnostics and integrated modeling frameworks for various fusion devices (W7-X, MAST-U, AUG, KSTAR, VNS and so on.) [7-11].
- High-fidelity geometry aids systematic detection of subtle but critical defects and ties model predictions to actionable engineering safeguards [12, 13].

METHODS

Monte Carlo orbit-following NuBDeC code for NBI simulation.

• Bi-Gaussian NB source is used. Ionization is sampled via the random-threshold method [14, 15].

Orbit integration: full Lorentz motion with gyration (RK4): $\dot{\mathbf{x}} = \mathbf{v}$, $\dot{\mathbf{v}} = (\mathbf{q}/\mathbf{m})\mathbf{v} \times \mathbf{B}$, $\mathbf{E} = 0$

• **B** is the magnetic equilibrium field (from equilibrium file); q/m is the charge-to-mass ratio.

CAD-to-simulation KSTAR PFC mesh + fast-ion path

• At each time step, the straight path segment between successive ion positions is tested against a CADderived PFC triangle mesh to record wall hits (Fig. 2-iii). Defeatured KSTAR PFC surfaces were tessellated into an unstructured mesh (376,747 triangles; 5-cm mesh size).

Two-phase 3D collision detection

• Space partitioning eliminates almost all non-colliding triangles with simple arithmetic (\approx 99.9% culled) \rightarrow An exact segment-triangle test confirms hits on the few candidates; full workflow and algorithms are summarized in [16, 17].

Heat flux due to NBI-induced fast ion loss (Γ_i)

- On intersection with triangle j, the i-th sample ion's kinetic energy E_i is deposited on that element and recorded to its PFC region (divertor, PL, other; shown in Fig. 2-iii).
- For a beam pulse of duration Δt , the NBI-induced ion-loss heat flux on element j is:

$$E_i = \frac{1}{2}\omega_i m_i \left(v_{i,\parallel}^2 + \frac{2\mu_i B}{m_i}\right)$$
, $\Gamma_j = \sum_i \frac{E_i}{A_i \Delta t}$, $(A_j: \text{triangle area, } \omega_i: \text{particle weight.})$

SIMULATION SETUPS & RESULTS

- Scan space: KSTAR NB1-A/B/C sources; beam energy = 60 or 100 keV; equilibrium: $\{I_p: 0.50/0.75/1.00 \text{ MA}; \beta_p: 1.0/2.0; \text{ two strike point configurations} \}$ (Fig. 3-i).
- For beam deposition calculations, electron density and temperature profiles are in Fig. 3-ii (typical KSTAR H-mode profiles).
- Magnetic ripple effects are neglected (approximately 0.04% amplitude).
- Each setup tracks 2.4×10^6 Monte Carlo sample ion particles.
- Summary of the results: Ion-loss heat flux and power fraction ranges (42 setups; 1 MW NBI; Fig. 3)
 - Divertor: heat flux max. $< \approx 30 \text{ kW m}^{-2}$; power fraction < 0.5%
 - PL: heat flux max. $< \approx 40 \text{ kW m}^{-2}$; power fraction < 0.3%
 - Other PFCs: heat flux max. $< \approx 15 \text{ kW m}^{-2}$; power fraction < 0.35%

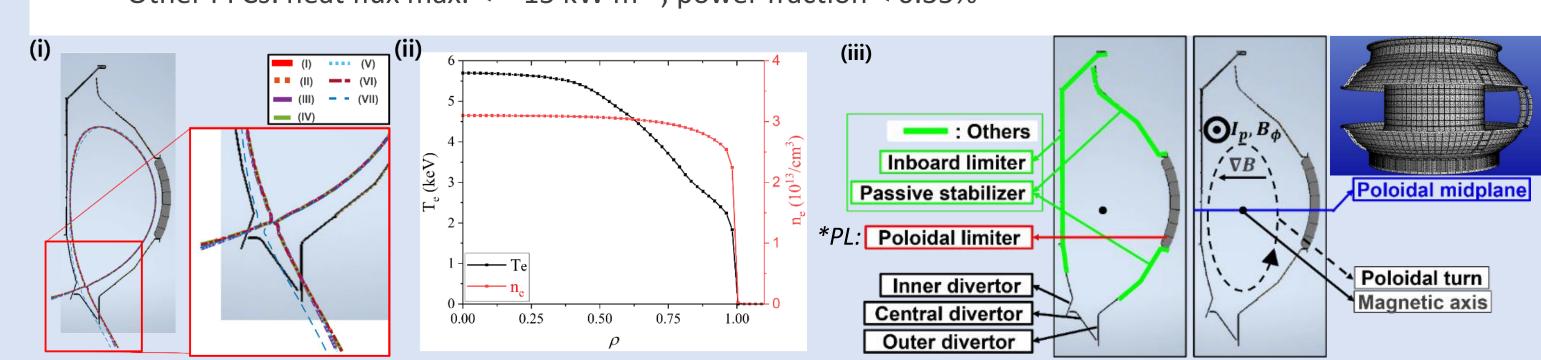


FIG. 2. (i) Toroidal cross section of magnetic separatrices for seven magnetic equilibria in the NuBDeC simulation parameter scan. Equilibrium case I is the reference setup ($I_p=1.00~MA$, $\beta_p=1.0$). (ii) One-dimensional profiles of electron temperature (T_e) and density (N_e) over $\rho = \sqrt{\psi_N} = \sqrt{(\psi - \psi_{axis})}/(\psi_{LCFS} - \psi_{axis})$. (iii) Toroidal cross section of KSTAR PFC surfaces illustrating the poloidal turn, poloidal midplane, and the following PFC regions. Categorization of the simulation results: divertor, *PL, and others.

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[2] KURKI-SUONIO, T., et al., J. Plasma Phys. **84** 6 (2018) 745840603. [3] AKERS, R., et al., 26th IAEA FEC (Proc. Int. Conf. Kyoto, Japan, 2016), IAEA, Vienna (2016). [4] KIM, J. Y., et al., AIP Adv. **6** 10 (2016) 105013. [5] BAK, J., et al., Nucl. Mater. Energy **12** (2017) 1270–1276. [6] ÄKÄSLOMPOLO, S., et al., Nucl. Fusion **58** 8 (2018) 082010. [7] LEVINESS, A., et al., Nucl. Fusion 64 9 (2024) 096034.

[1] WARD, S. H., et al., Nucl. Fusion 62 12 (2022) 126014.

[9] LUNT, T., et al., Phys. Rev. Lett. **130** 14 (2023) 145102.

[8] VELARDE, L., et al., Plasma Phys. Control. Fusion 67 1 (2024) 015024.

[11] FRANKE, T., et al., Fusion Eng. Des. **215** (2025) 115035. [12] JUAREZ, R., et al., Nat. Energy **6** 2 (2021) 150–157. [13] JUAREZ, R., et al., Nat. Commun. 15 1 (2024) 8563. [14] NA, B., et al., Fusion Eng. Des. **185** (2022) 113320. [15] RHEE, T., et al., Phys. Plasmas **26** 11 (2019) 112504. [16] MOON, T., et al., Comput. Phys. Commun. **309** (2025) 109490. [17] ERICSON, C., CRC Press, "Boca Raton" (2004).

[10] KWON, J.-M., et al., IEEE Trans. Plasma Sci. **52** 9 (2024) 3910–3916.

PARAMETER-DRIVEN CHANGES IN ION-LOSS DISTRIBUTION

Beam energy and source (Figs. 3-5)

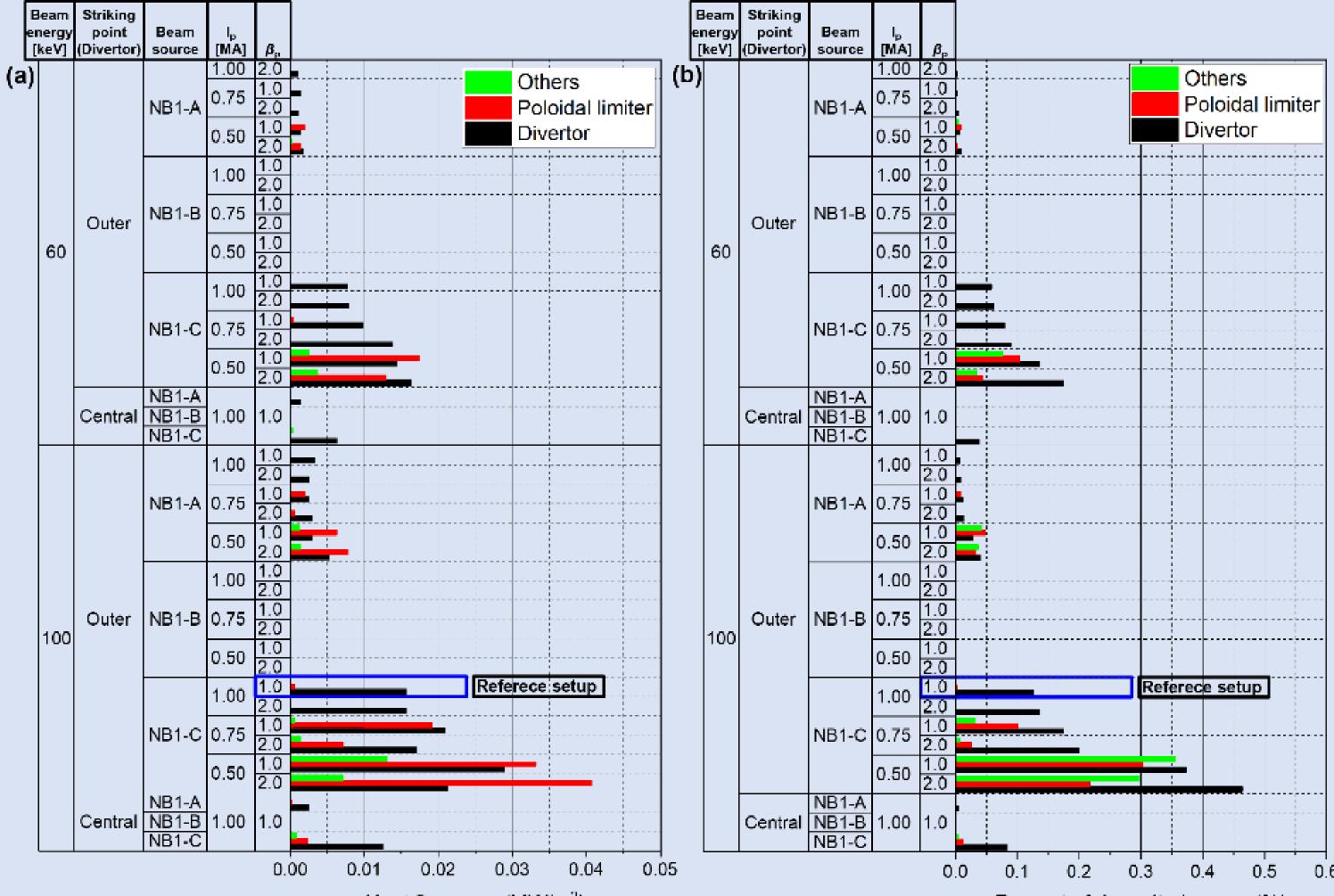
- •100 \rightarrow 60 keV: poloidal wetting narrows; peak heat flux $\downarrow \approx$ 75–80%;
- deposited power fraction $\downarrow \approx 90-95\%$.
- •NB1-C: highest losses; NB1-A: 3–4 times lower than NB1-C; NB1-B: no measurable losses.

Plasma current and poloidal beta

- • $I_p \downarrow \rightarrow$ higher safety factor (q) and stronger radial drift \rightarrow broader distribution along poloidal-turn (Fig. 6-ii-d).
- • $\beta_p \uparrow$ (Shafranov shift) \rightarrow increased divertor and PL deposition (Fig. 3).

Strike-point relocation

- •Moving the outer strike point to the central divertor \rightarrow divertor wetted area is expanded;
- Deposition expands to inboard limiter; PL wetting shifts to higher Z.
- Produces broader ion-loss distribution changes than β_p variations (Fig. 5-i-e, 5-ii).



Heat flux max. (MW/m²) Percent of deposited power (%) FIG. 3. Maximum heat flux and percentage of power deposition for varying beam energies, sources, I_p , β_p , and outer strike point position of the magnetic separatrix on the divertor. The three PFC regions in the legend are illustrated in Fig. 2-iii.

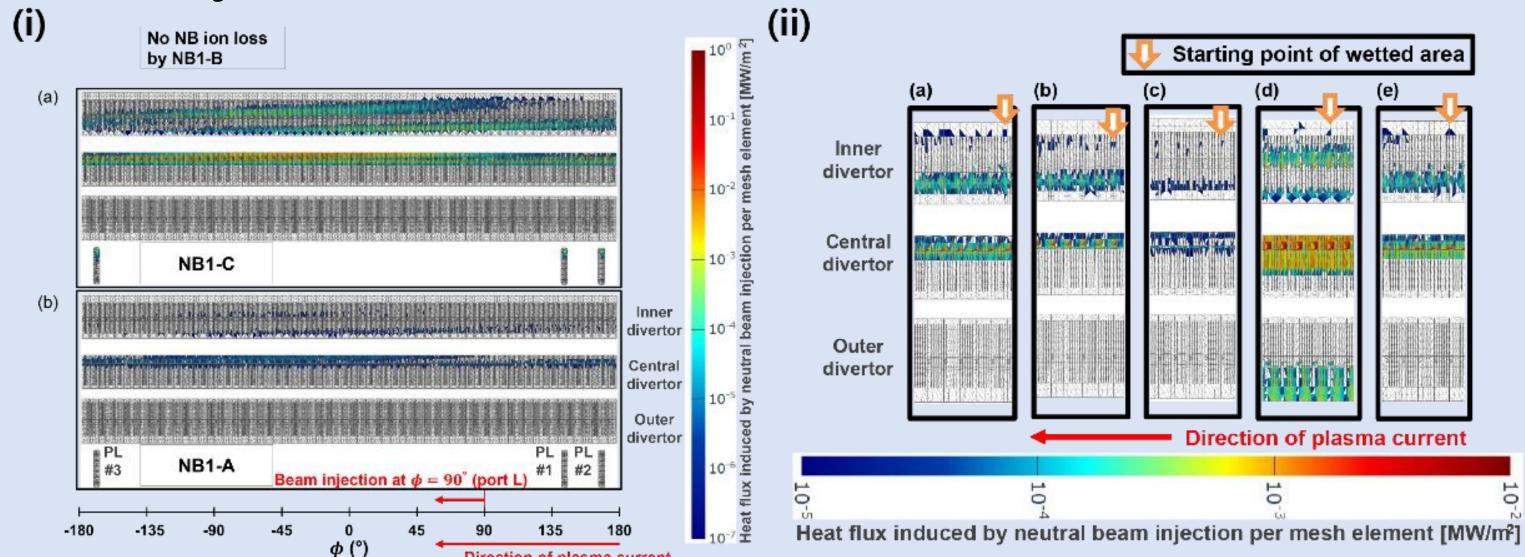
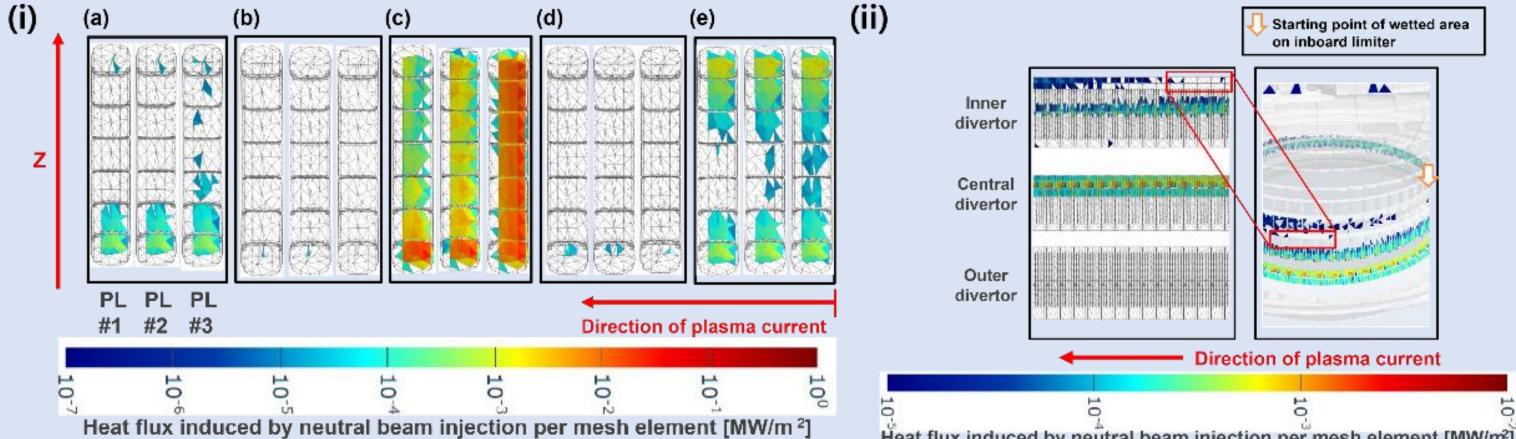


FIG. 4. (i) NuBDeC beam-source variants: (i-a) reference; (i-b) single-parameter change to NB1-A; NB1-B shows no ion loss. (ii) Heat-flux maps from ion loss on the unfolded divertor: (ii-a) reference; (ii-b) 60 keV; (ii-c) NB1-A; (ii-d) I_p = 0.50 MA; (ii-e) β_p = 2.0. The white arrow (orange outline) marks the start of the plasma-wetted area.



Heat flux induced by neutral beam injection per mesh element [MW/m²] FIG. 5. (i) Unfolded-PL heat-flux maps from NuBDeC NBI-induced ion loss: (i-a) reference; single-parameter variants—(i-b) NB1-A source, (i-c) I_p = 0.50 MA, (i-d) β_p = 2.0, (i-e) outer separatrix strike on the central divertor. PL heat flux vanishes for 60 keV NB1-C. (ii) Unfolded-divertor map for the outer strike on the central divertor (cf. Fig. 4-ii); the wetted area reaches the inner limiter. The white arrow (orange outline) marks the start of the plasma-wetted area.

DISCUSSION AND CONCLUSION

- CAD-based NBI simulation predicts non-axisymmetric, geometry-sensitive heat-flux bands with peaks at toroidal leading edges and upstream KSTAR PFC surfaces.
- Across 42 setups: divertor/PL ion-loss heat flux max. remain < 30–40 kW m⁻² with sub-percent power fractions.
- Three of the main factors that control the heat flux patterns are analyzed:
 - 1. (Beam deposition location) Deposition (ionization) sets the poloidal-turn and loss position. NB injection close to high-field side (HFS) \rightarrow deposition close to HFS; changes in I_p , β_p reshape flux surfaces \rightarrow deposition shifts.
 - 2. (Magnetic geometry shifts) Moving flux surfaces or the outer strike point repositions losses even for the same deposition, by changing ion loss distribution.
 - 3. (Radial drift scaling) Drift grows with higher beam energy $(v_{\parallel} \uparrow)$ and lower $I_p(q \uparrow)$, broadening the ion loss distribution and raising divertor loading.
- This workflow can provide a systematic basis to tune NBI and mitigate localized hot spots. The resulting heatflux datasets are visualization-ready in virtual environments such as the V-KSTAR platform and are intended for further use in synthetic diagnostics and integrated simulation, providing engineering-level fidelity and seamless geometrical context on which the data can be mapped.
- For the future work, non-axisymmetric fields (e.g., RMP), MHD instabilities, and atomic processes (thermalization, charge-exchange) will be considered.