

# Understanding radial density flattening in the far-SOL of H-mode plasmas on JET

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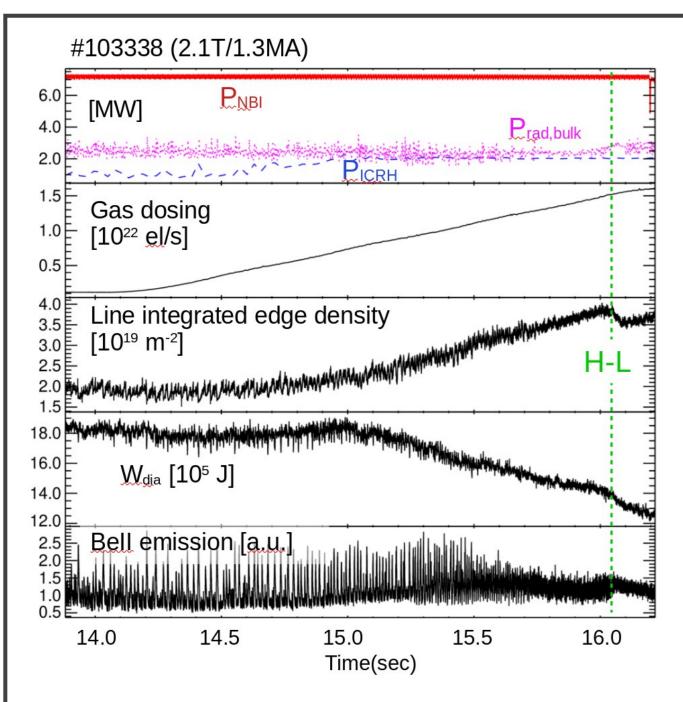
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#### 1. BACKGROUND

- Main Chamber (MC) W sputtering could far outweigh other W sources in new full-W ITER [Romazanov NF2022, Eksaeva PhysScripta 2022]
- Insufficient understanding of density flattening in far-SOL
- $\rightarrow$  at present, no reliable prediction for n<sub>e</sub> reaching MC tiles in ITER
- → at present, no reliable prediction for MC W source in ITER

#### 2. New H-mode density ramp experiments on JET

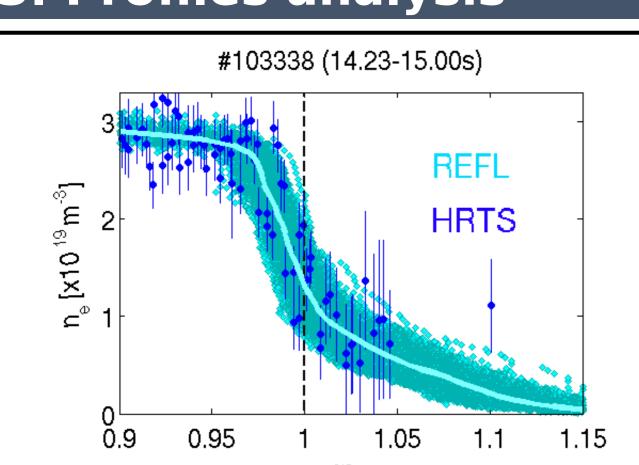


- Fuelling ramps up to H-L limit
- Gas or pellet fuelling ramps
- Discharge pairs with different triangularity, OSP geometry,  $Ip(L_{||})$
- Scenarios designed to take advantage of latest diagnostic improvements (reflectometry...)
- Different edge profile classifications explored ( $T_{e,OSP}$ ,  $v_{e,sep}$ \*,  $\alpha_t$ ,...)

 $\alpha_{\rm t} \sim q_{\rm cyl}^2 R \left( n_{\rm e,sep} / T_{\rm e,sep}^2 \right) Z_{\rm eff}$ 

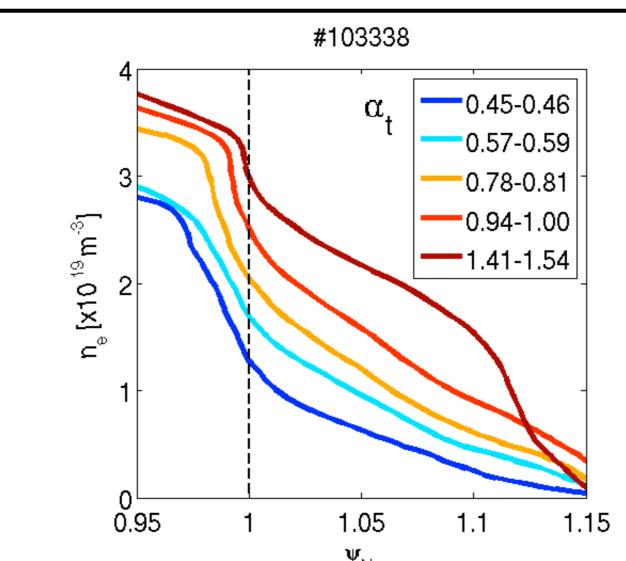
for exact formula see eqn. 10 in Eich NF 2020

#### 3. Profiles analysis



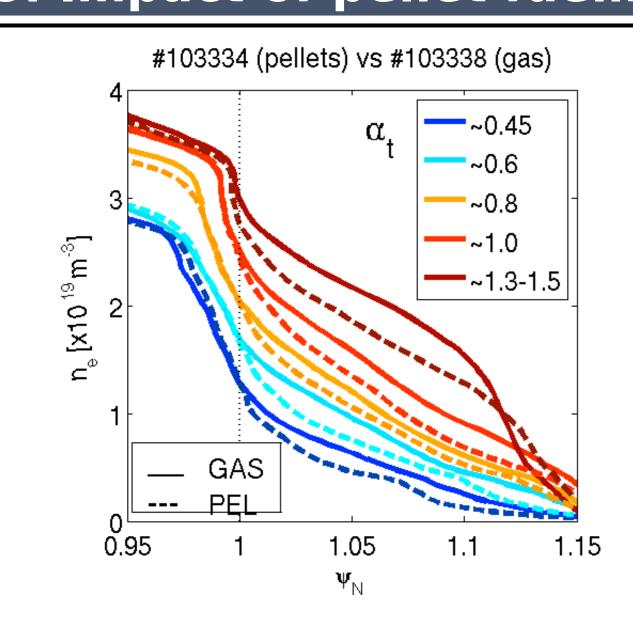
- Absolute positioning of reflectometry constrained with interferometry
- HRTS positioned iteratively through power balance [Eich NF 2020]
- Level of consistency yields error bars

### 4. Impact of gas puffing

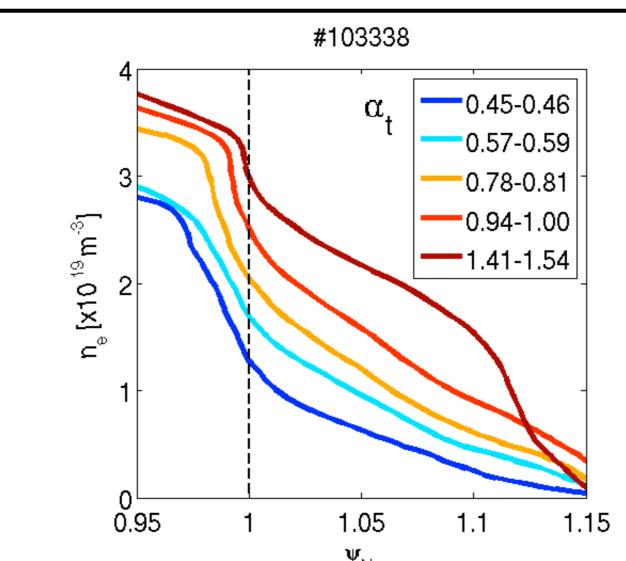


- with increasing  $\alpha_t$

### 5. Impact of pellet fuelling

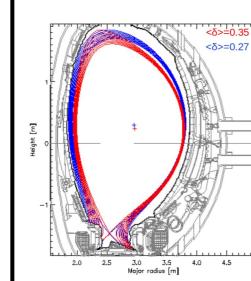


- At fixed  $\alpha_t$ , pellet fuelling yields lower absolute n<sub>e</sub> both in nearand far-SOL (but not n<sub>e,sep</sub> itself)
- However, difference originates mostly from near-SOL (where profiles diverge)
- Radial decay in far-SOL similar
- Density shoulder formation also with pellets

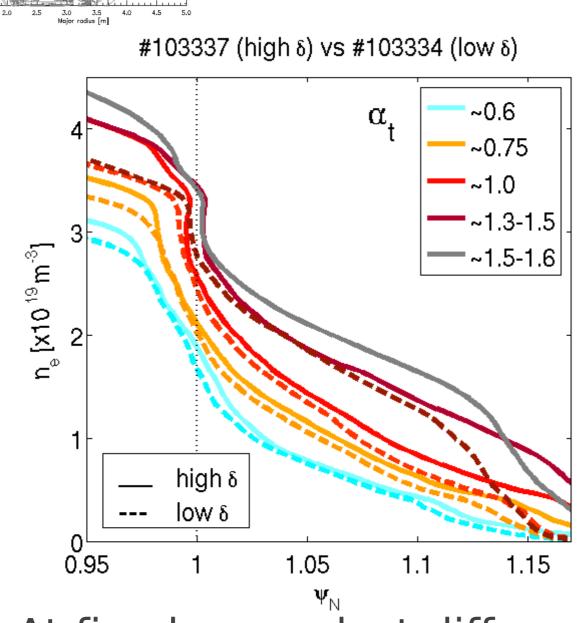


- Far-SOL density gradually flattens
- $\alpha_t$  well above 1: density shoulder

#### 6. Impact of triangularity

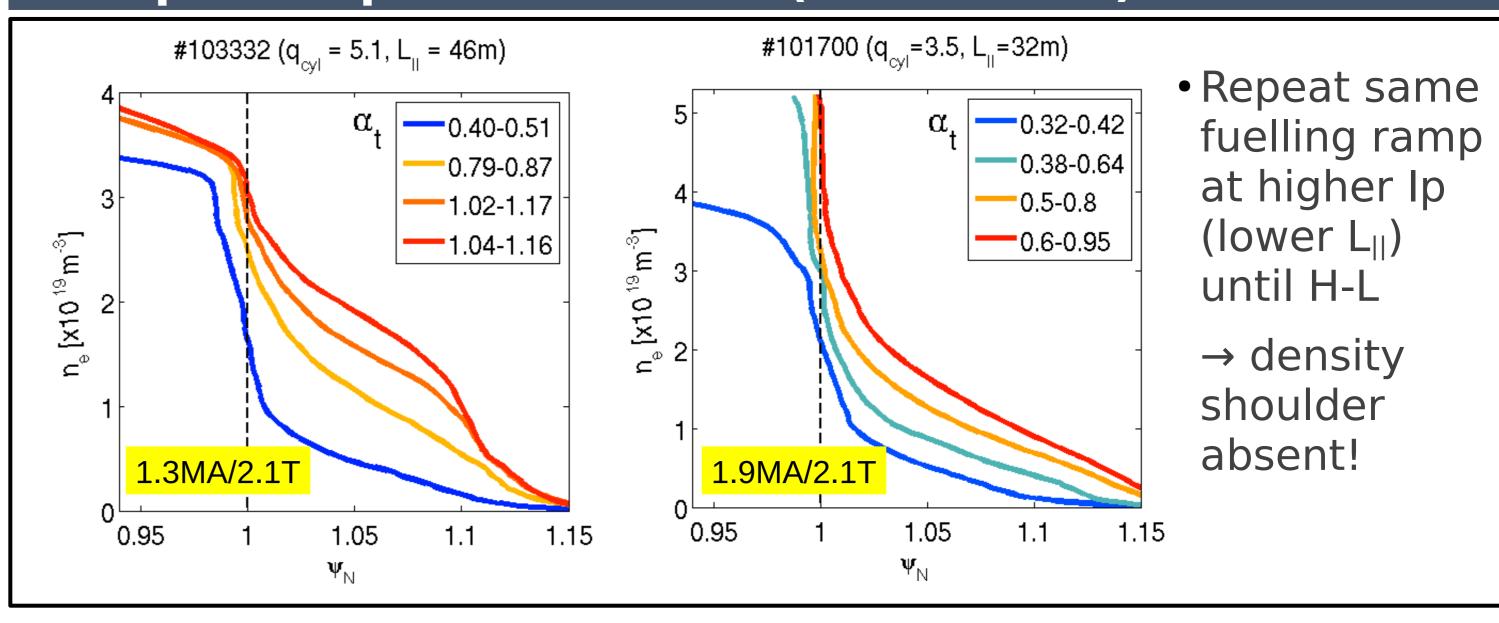


- Upper triangularity increased by 50%
- Div.geometry fixed

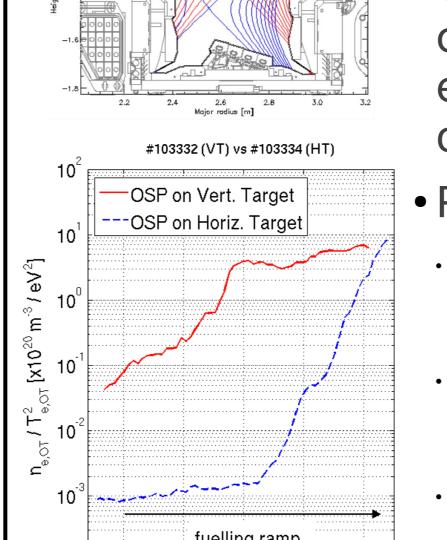


- At fixed  $\alpha_t$ : modest difference
- Increased  $\delta$  opens access to higher absolute ne (higher density shoulder) in SOL

#### 7. Impact of plasma current (at fixed Btor)

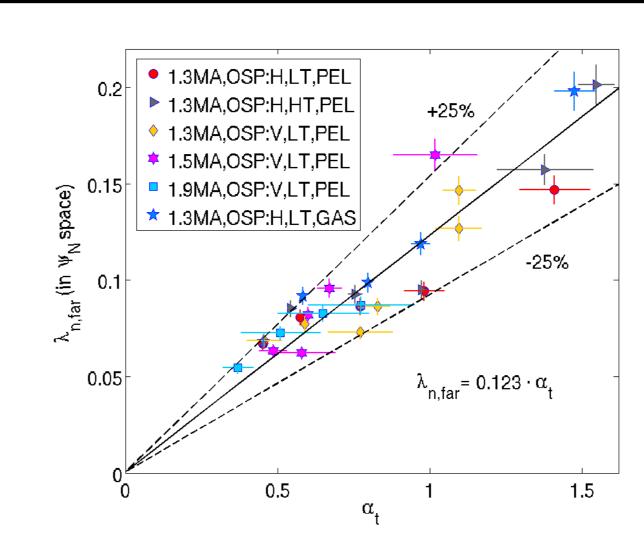


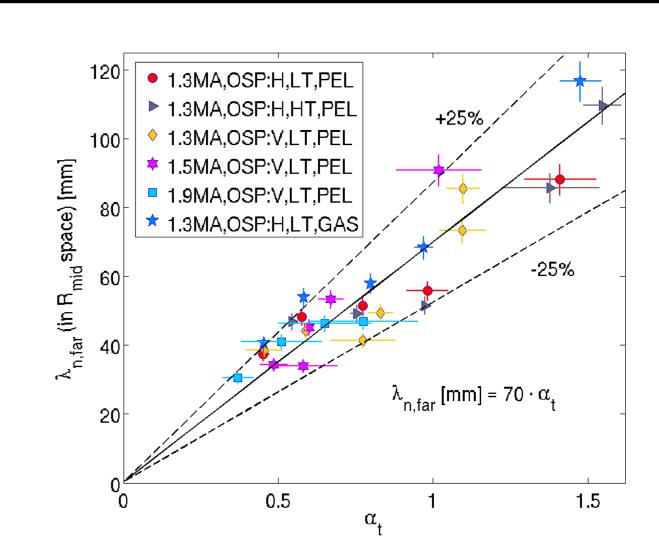
## 8. Role of divertor geometry



- Horiz. vs Vert. Target
- Very different OSP collisionalities (governs electrical disconnection .of filaments)
- Profiles at fixed  $\alpha_{t:}$
- Steeper ne in pedestal/ separatrix/nearSOL
- In far-SOL differences largely disappear!
- Density shoulder formation similar
- #103332 (OSP:VT) vs #103334 (OSP:HT) 1.05-1.15 (VT) 1.30-1.50 (HT) 0.95
- → OSP geometry NOT found to impact radial ne transport in far-SOL

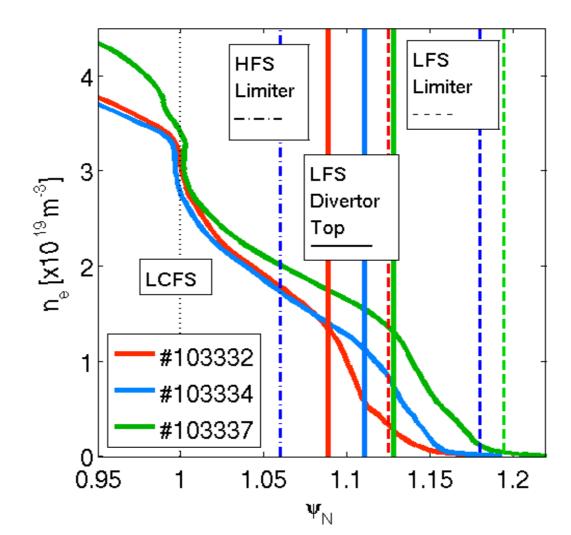
## 9. Density decay length in far-SOL





•  $\alpha_t$  unifies dataset! Linear correlation both in flux- and real- space.  $\rightarrow \alpha_t$  good descriptor to quantify far-SOL density flattening on JET

## 10. Outer inflection radius 11. MAIN CONCLUSIONS



- Pulses run with higher than normal clearance to outer wall
- Shoulder inflection NOT set by intersection with limiters but LFS divertor top
- → route to reduce MC W source?

- SOL ne profiles carefully evaluated for very diverse set of discharges
- $\bullet \alpha_t$  unifies measured ne radial decay lengths in far-SOL:

 $\lambda_{n,far}[\psi_N] = 0.123(\pm 25\%) * \alpha_t$  $\lambda_{n,far}$  [mm]=70(±25%) \*  $\alpha_t$ 

- Inner inflection radius (near-/far-SOL transition):  $\psi_N \sim 1.01-1.02$
- *Outer* shoulder inflection radius can be moved away from LFS limiters!







This scientific paper has been published as part of the international project co-financed by the Polish Ministry of Science and Higher Education within the programme called 'PMW' for 2023-2025. This work has been carried out within the framework of the Eurofusion Consortium, funded by the European Union via the Euratom Research and Training Programme (Grant Agreement No 101052200-EUROfusion). Views and opinions expressed are however those of the author(s) only and do not necessarily reflect those of the European Union or the European Commission. Neither the European Union nor the European Commission can be held responsible for them.