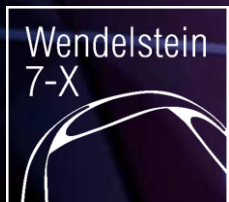




Towards a Stellarator Fusion Reactor: Achievements of the European Stellarator Program



EUROfusion

Felix Warmer for SPPS Team & Colleagues

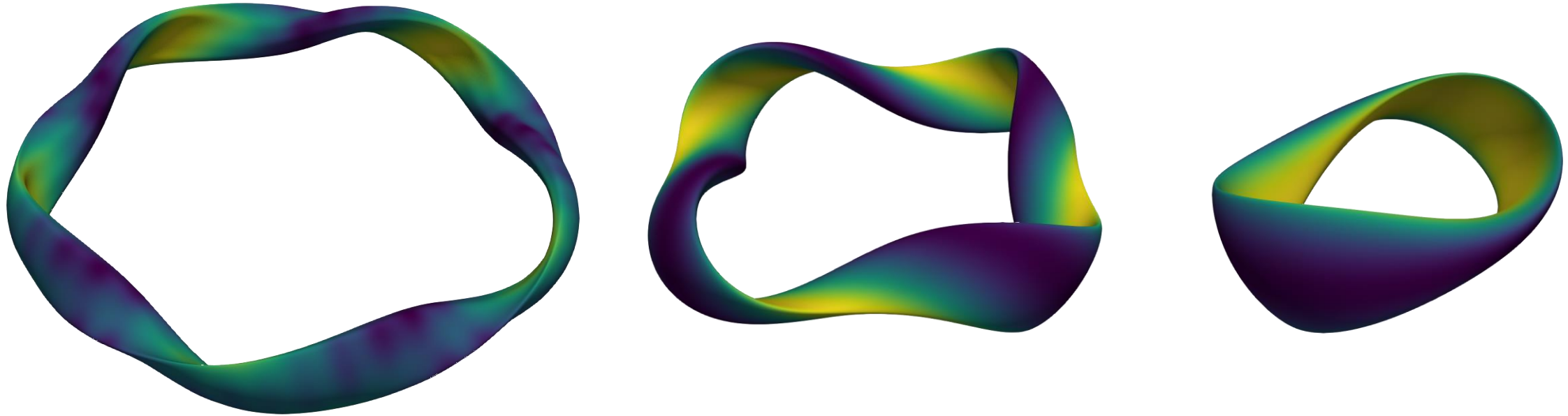
IAEA FEC, October 2025



This work has been carried out within the framework of the EUROfusion Consortium, funded by the European Union via the Euratom Research and Training Programme (Grant Agreement No 101052200 — EUROfusion). Views and opinions expressed are however those of the author(s) only and do not necessarily reflect those of the European Union or the European Commission. Neither the European Union nor the European Commission can be held responsible for them.



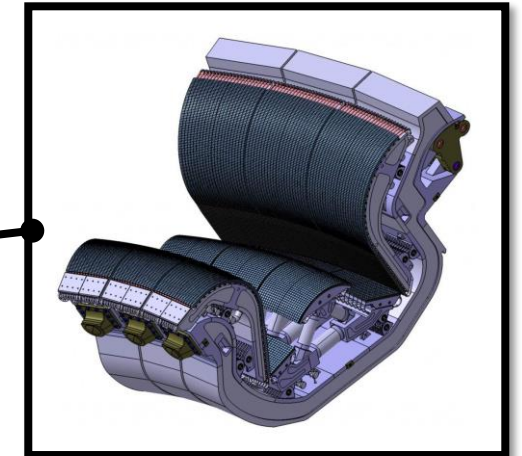
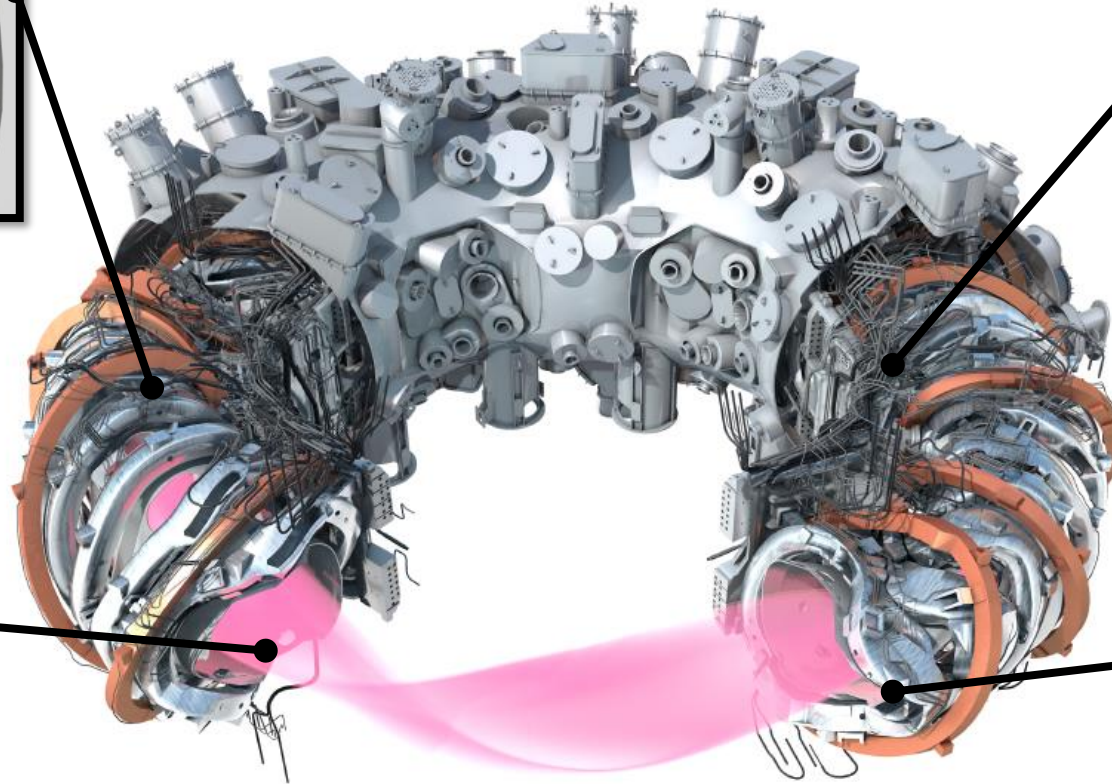
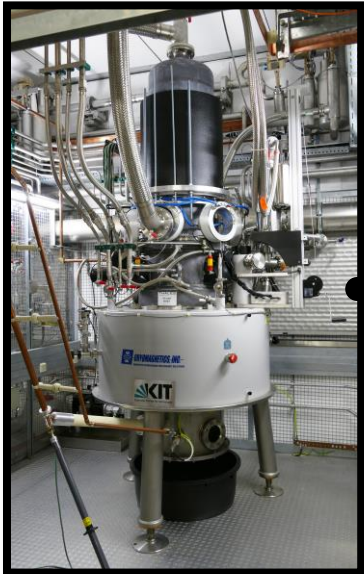
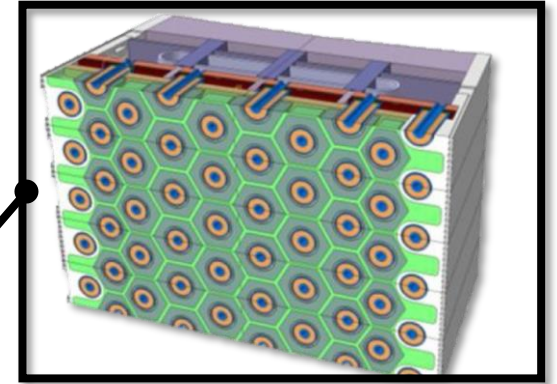
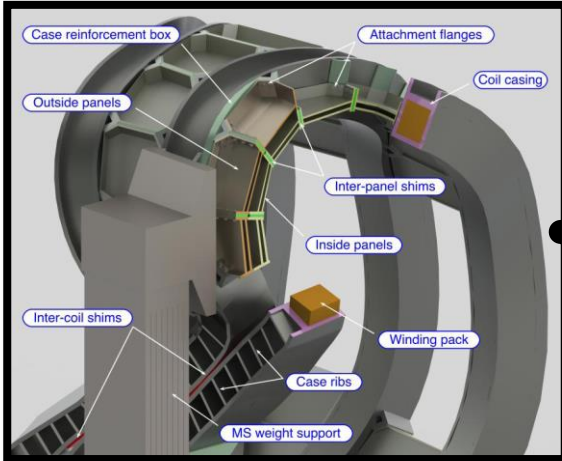
Advance in Stellarator Optimisation



Big steps in Stellarator Optimisation with new configurations



Reactor Engineering



Various technology options & integration

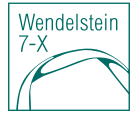
Goal: Automating the Assessment of new Stell. Configurations



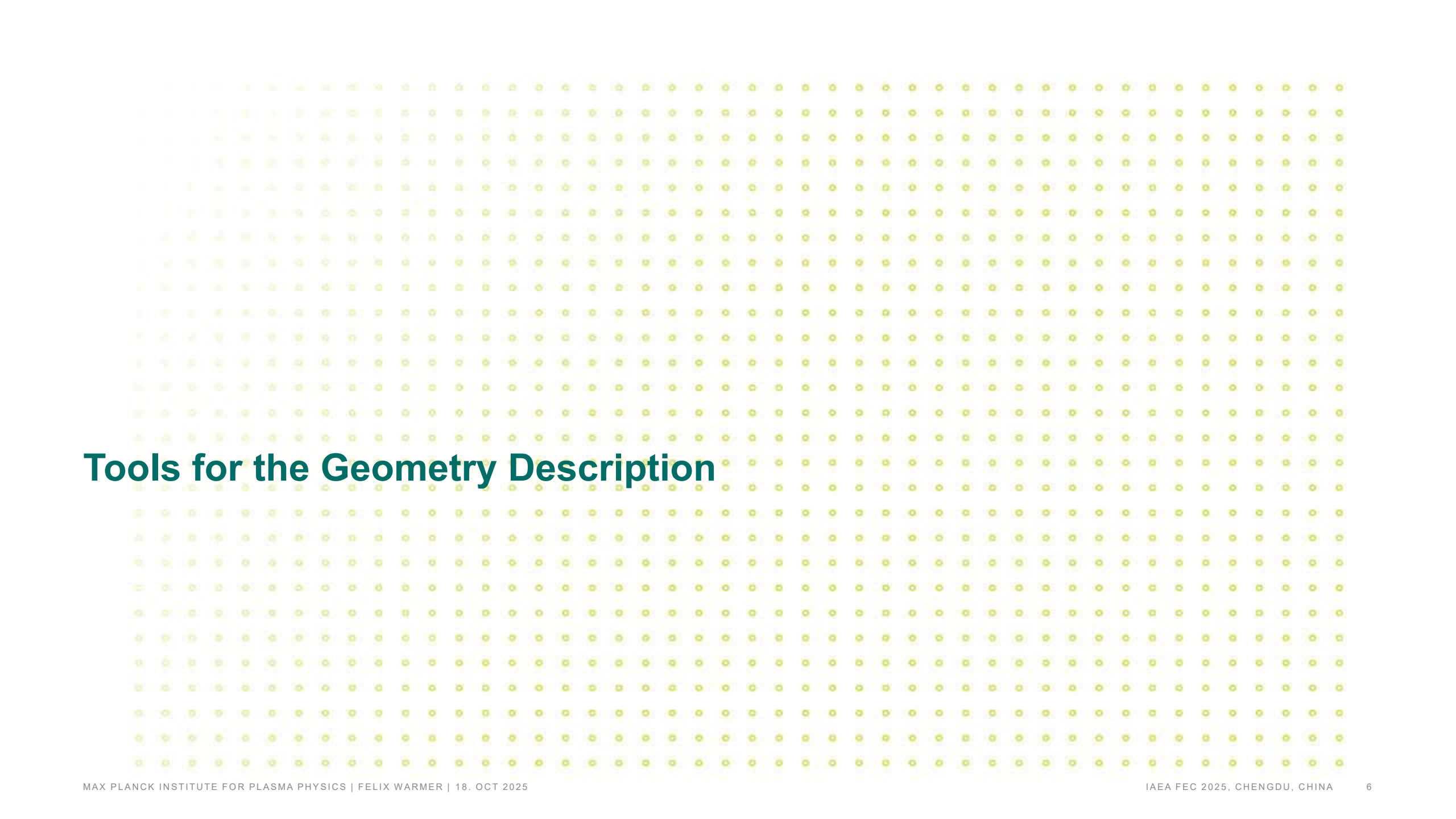
- Potentially countless Stellarator shape variations
- Different technology options (HTS vs. LTS, blanket materials, etc.)
- High entanglement between components (& plasma) – challenge of integration

Importance of having an (automated) workflow for design assessment
(while working with limited resources)

Content

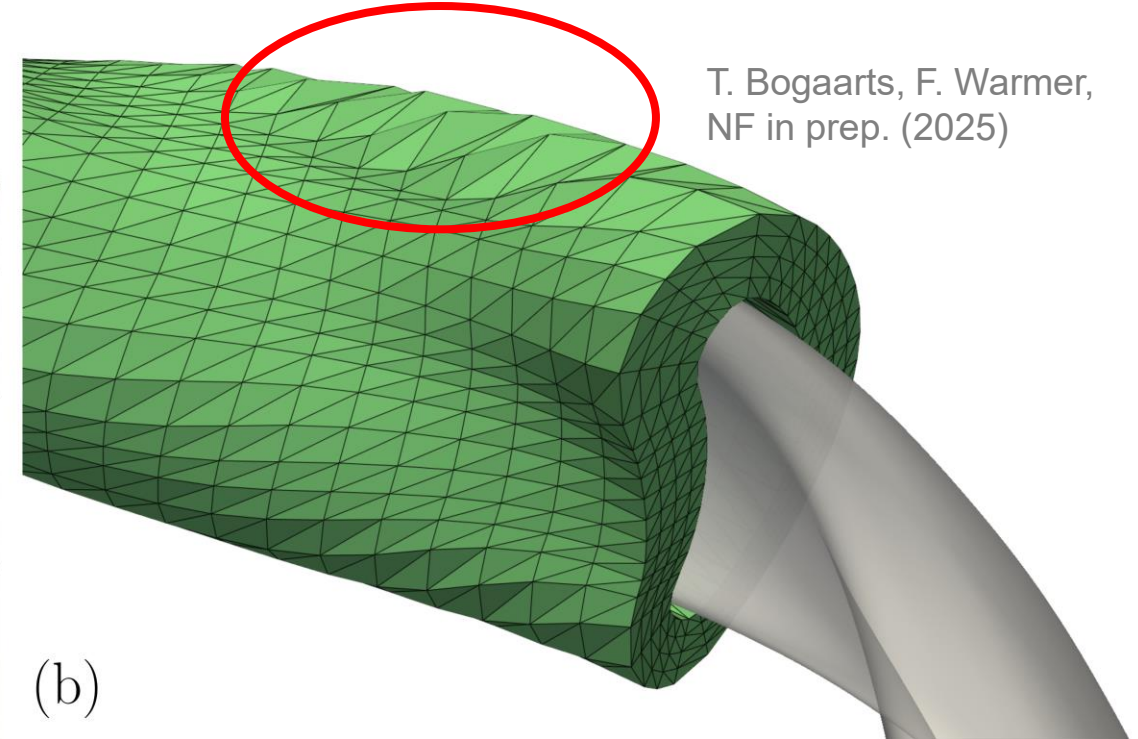
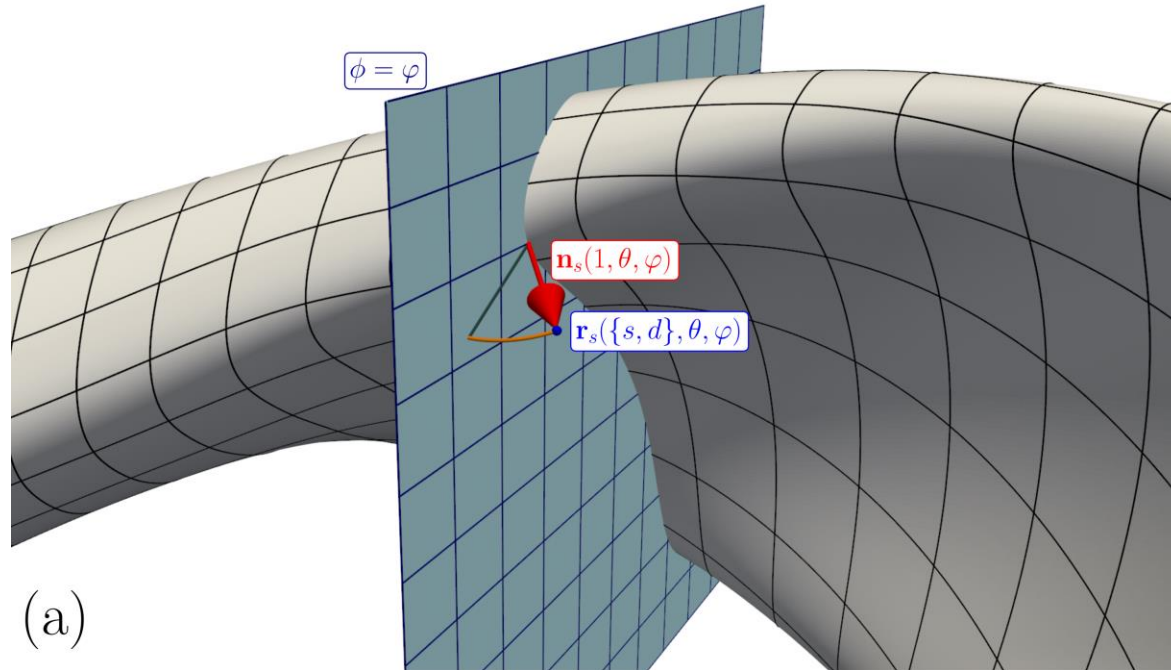


- Tools for the Geometry description
- Parametric workflow for neutronic analysis
- Parametric workflow for magnet assessment
- Summary



Tools for the Geometry Description

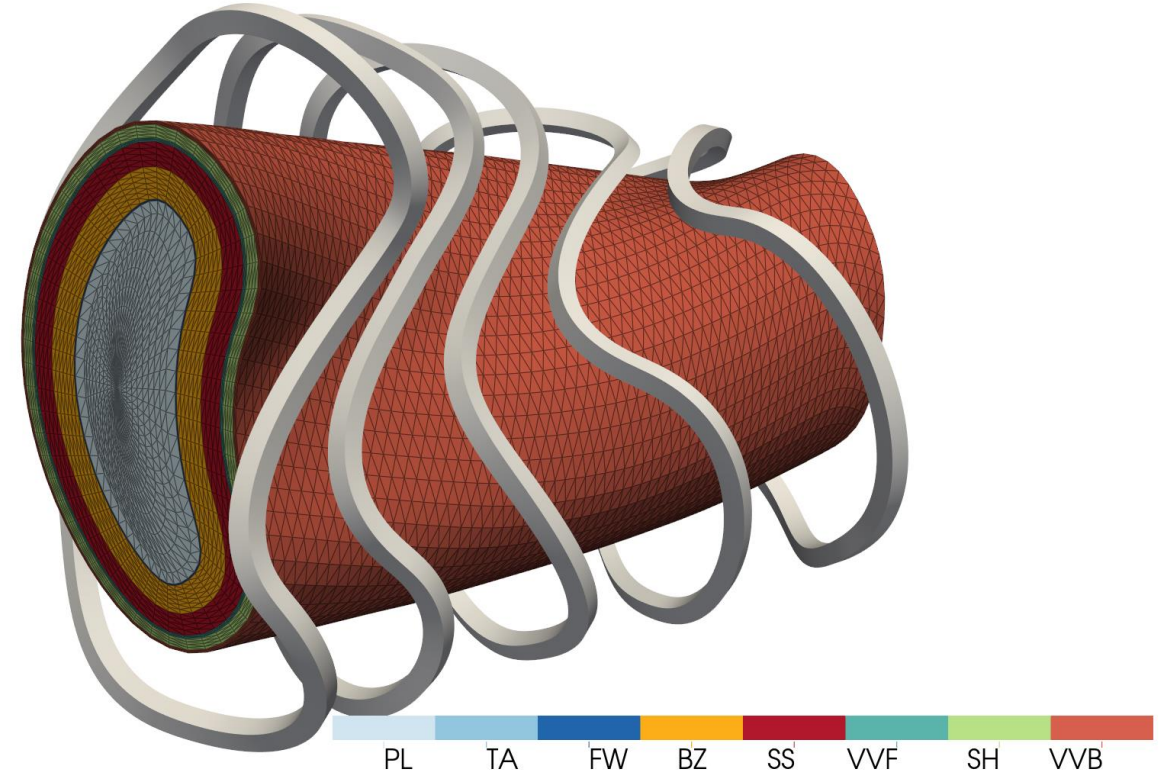
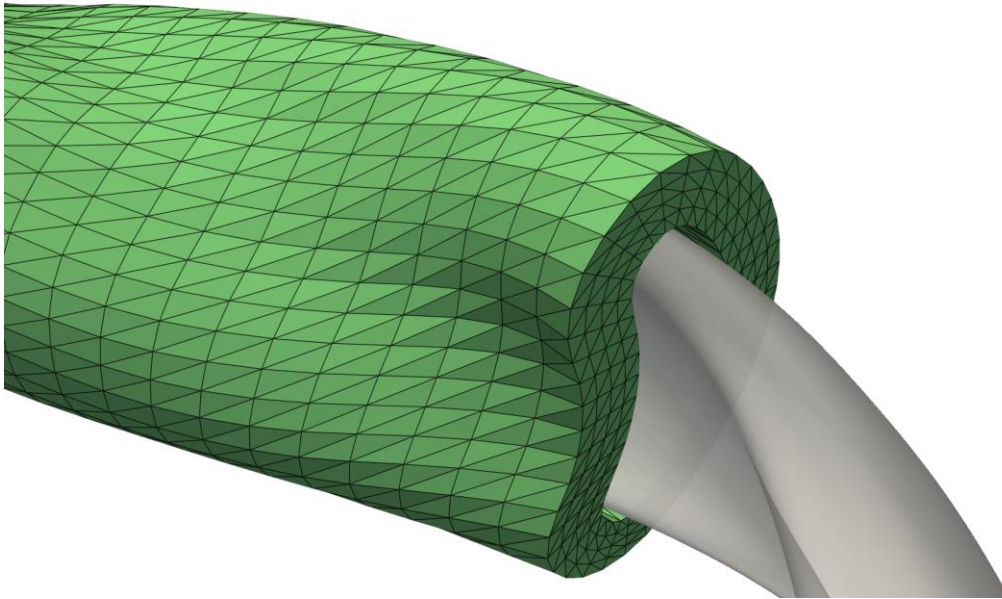
Creating a 3D (equidistant) wall surface / blanket layers



T. Bogaarts, F. Warmer,
NF in prep. (2025)

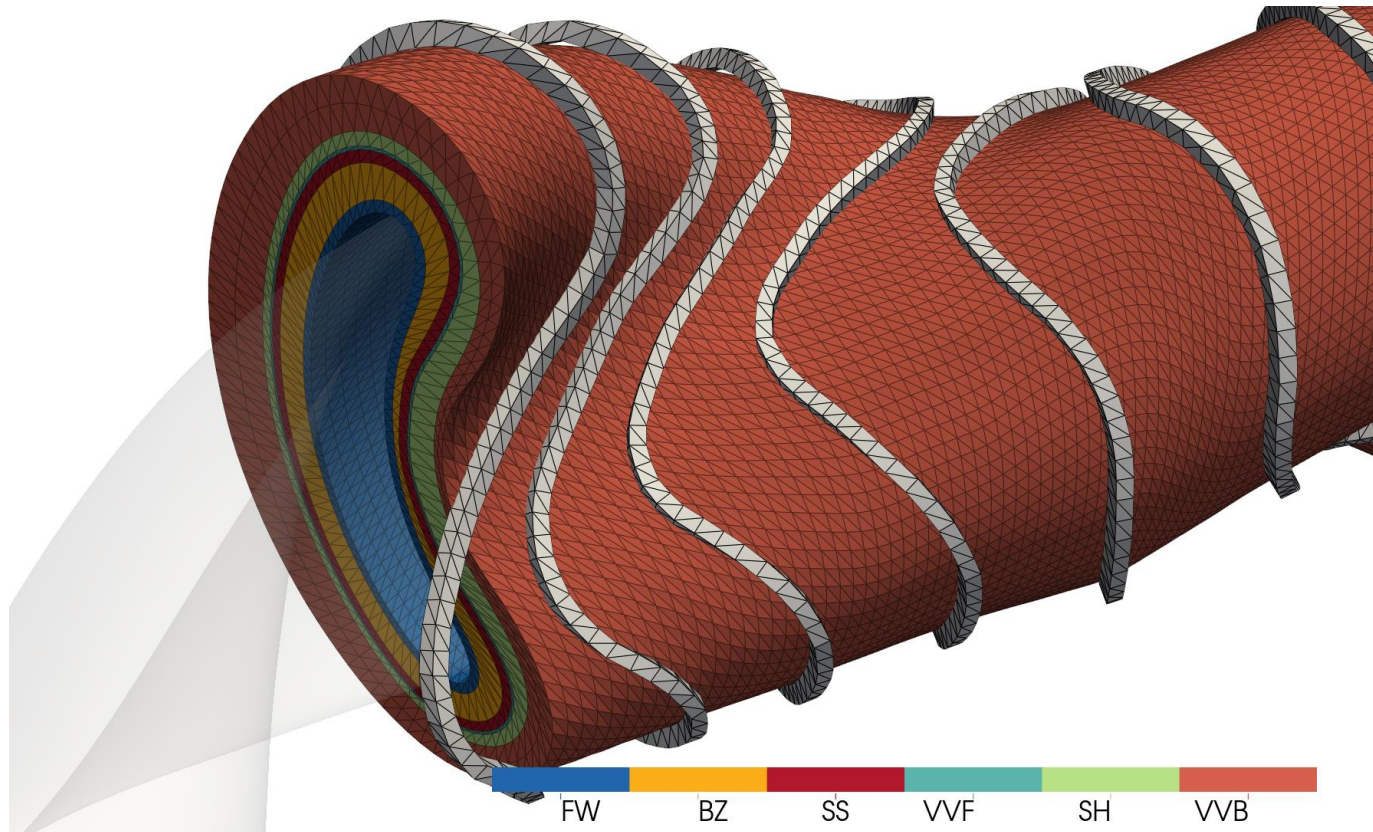
- Normal vectors have a toroidal component $\rightarrow$ elements no longer lie on a poloidal cut
- Curvature leads to unequally-spaced elements

Solution: Fourier Transform of wall & blanket surfaces



- Apply Fourier Transform and re-order poloidal coordinate for a constant poloidal arc length
→ better mesh (also reduces Fourier harmonics)
- Multiple surfaces can be combined resulting in layered blanket

Fitting “arbitrary” surfaces, i.e. coil surface



- Find ‘distances’ and ‘angles’ from arbitrary points using a combination of a grid search and Newton iterations
- Apply Fourier Transform
- Particularly useful for the ‘coil surface’

Coil Geometry



- Centroid
- RMF
- Frenet
- Surface

Typical input:

- Discrete filaments OR Fourier-based filaments

Several ways to define a coordinate system along a coil to get finite size coils:

- *The Frenet-Serret frame* (leads to irregular shapes)
- *The centroid frame* (vector to its center of mass)
- *The rotation-minimized frame* (reduces twisting)
- (any user-defined normal's)

Likely to be custom defined depending on detailed engineering requirements (e.g. HTS tape orientation)





Parametric workflow for neutronic analysis



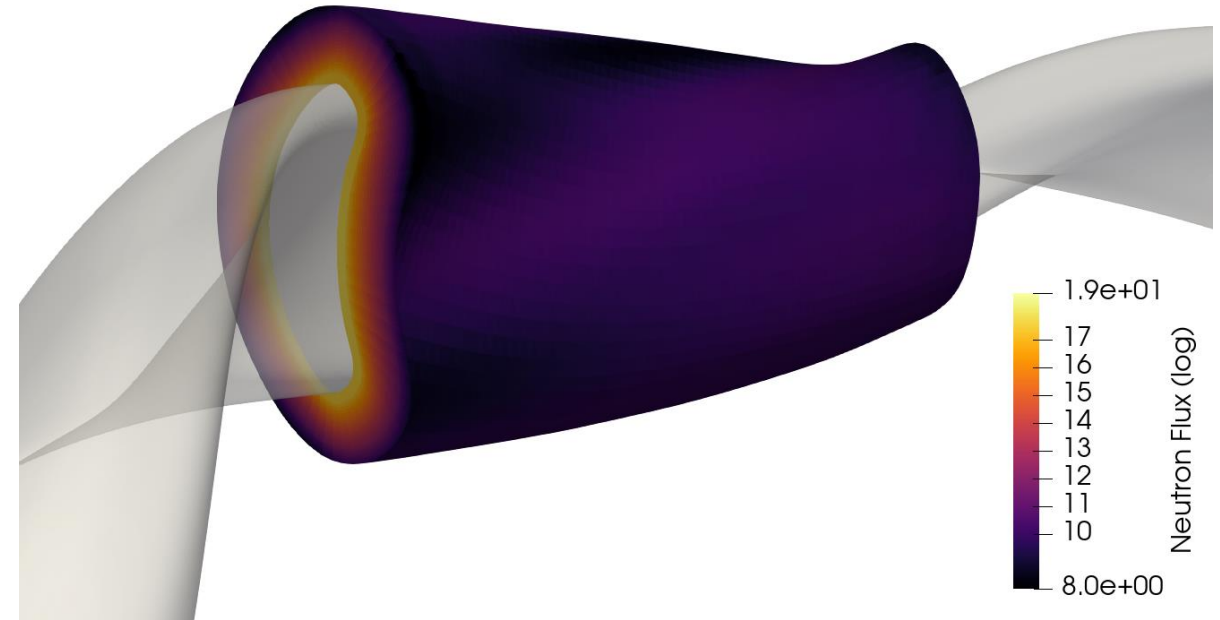
Neutron Transport & Blanket

Neutrons are a key aspect that separates experiments from a Fusion Reactor

Desired quantities:

- Tritium breeding
- Nuclear heating
- Neutron damage
- Shielding / Safety

→ Solve neutron transport



$$\underbrace{\hat{\Omega} \cdot \nabla \psi(\mathbf{r}, \hat{\Omega}, E)}_{\text{Streaming}} + \underbrace{\sigma(\mathbf{r}, E) \psi(\mathbf{r}, \hat{\Omega}, E)}_{\text{Collisions}} = \underbrace{q(\mathbf{r}, \hat{\Omega}, E)}_{\text{Sources}}$$



Neutron Transport: Traditional MC approach

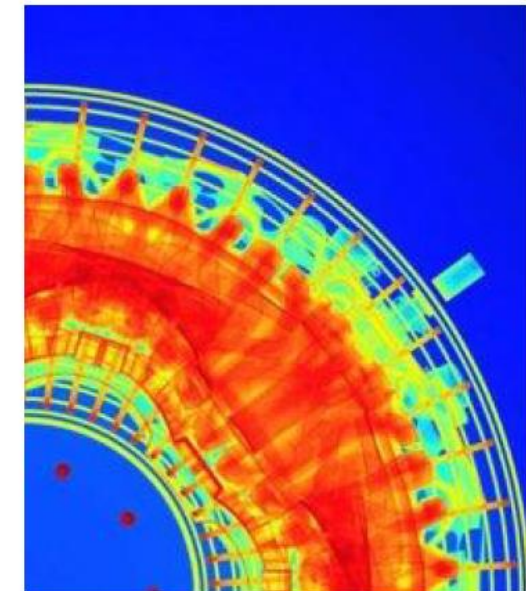
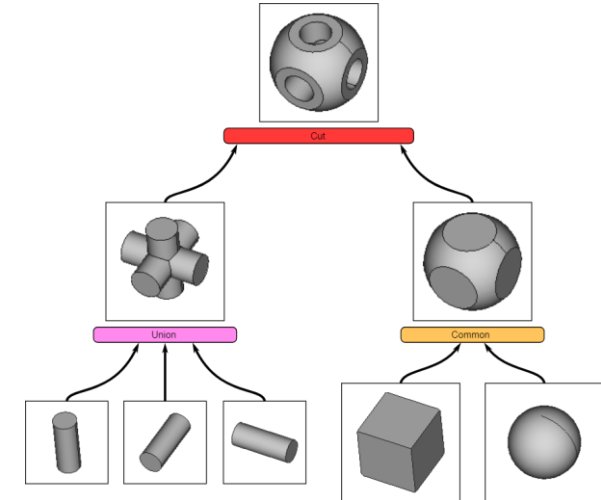
- Monte Carlo method
- Requires 10^9 samples
- Poor statistics far from source
- Not suited for complex shapes (CSG format)

→ Has been successfully applied to stellarators in EUROfusion: MCNP5/6, Serpent2, OpenMC

→ But impractical for design exploration

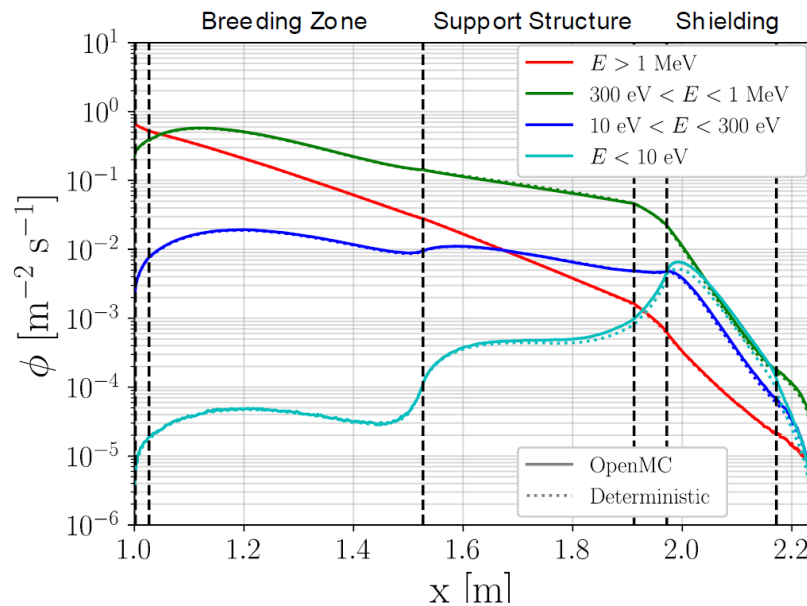
A. Häussler, et al. FED 136 (2018)
I. Palermo, et al. NF 61 (2021)
T. Lyytinen, et al. NF 64 (2024)
etc.

I. Palermo, this IAEA FEC,
Friday, 14:40, TEC/4-3



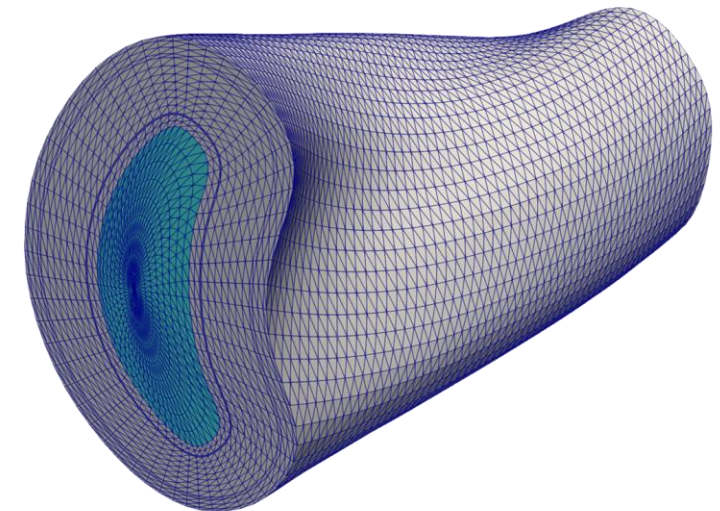
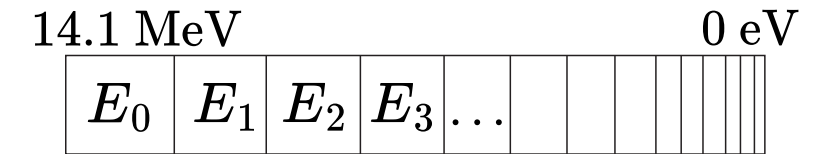
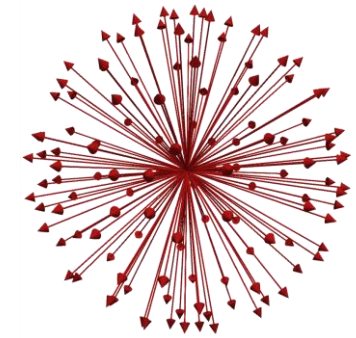
Modelling Innovations: Deterministic Neutronics

- Directly solve neutron transport equation
- E : Multigroup, $\hat{\Omega}$: discrete ordinates, **Tetrahedral mesh**
- **Discontinuous Galerkin**, sweep algorithm
- Exact neutron conservation

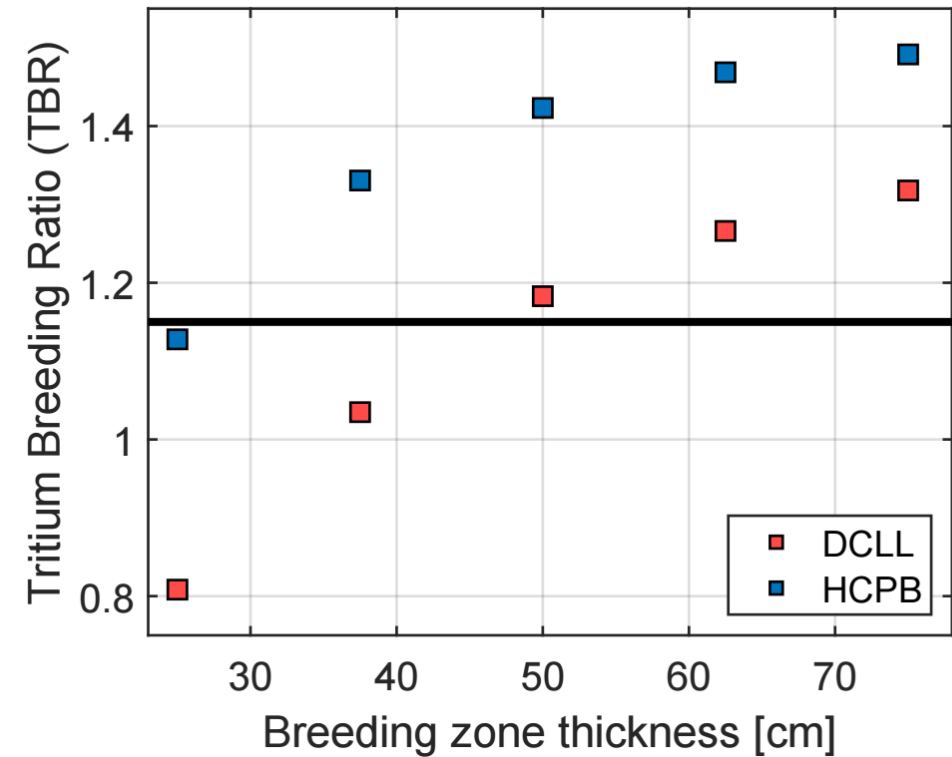
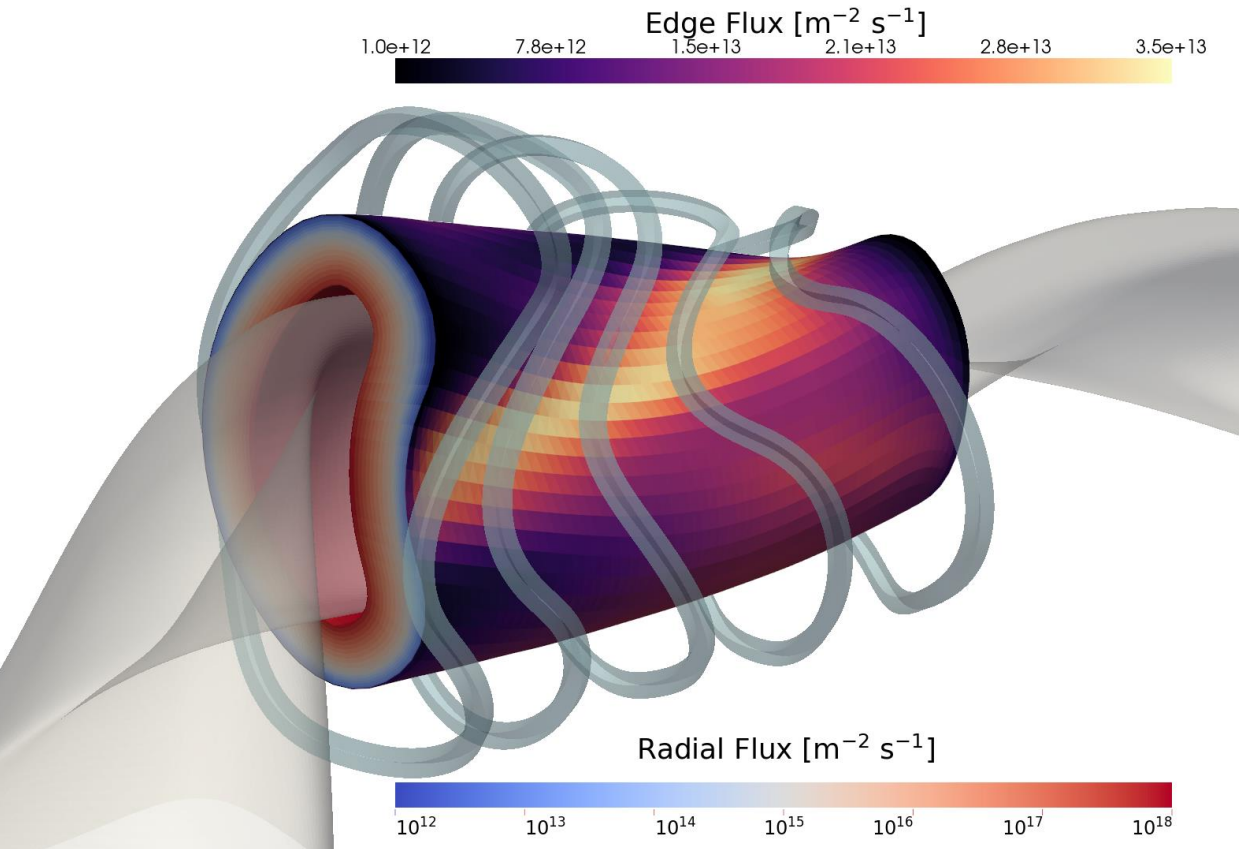


- Finished development
- Fully 3D capable
- Benchmarked
- 10⁵ faster than MC

T. Bogaarts, F. Warmer, NF 65 (2025)

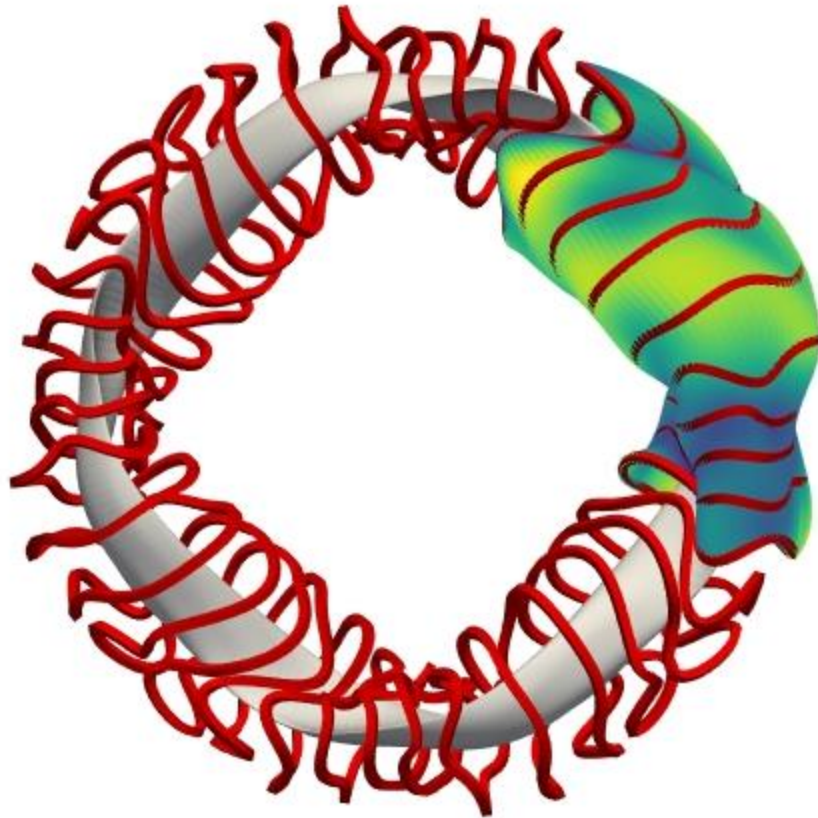


Deterministic Neutronics: 3D solution & optimisation

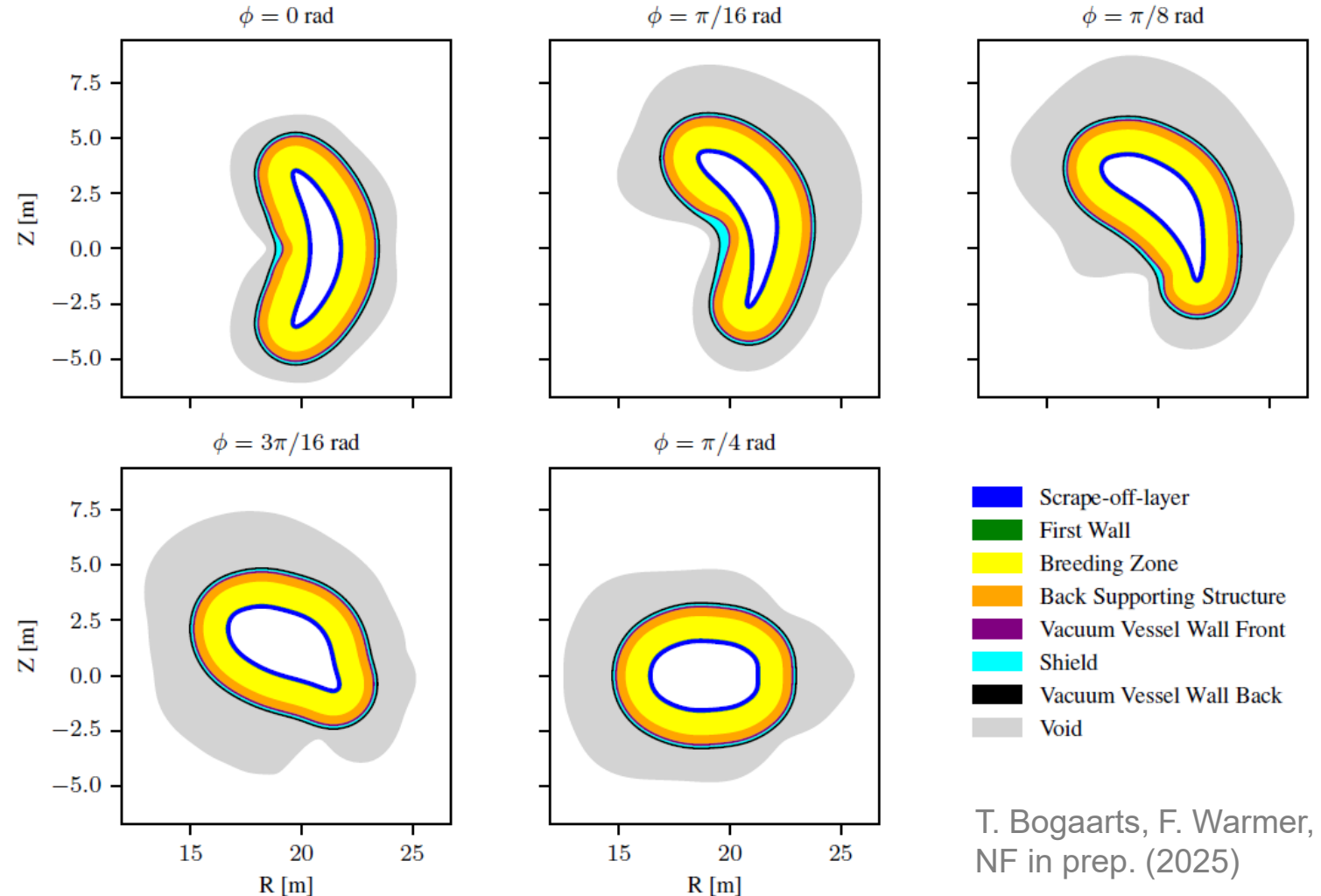


- **Neutrons are a key design driver → new tool allows fast design iteration and optimisation**

Method Application to a New Configuration

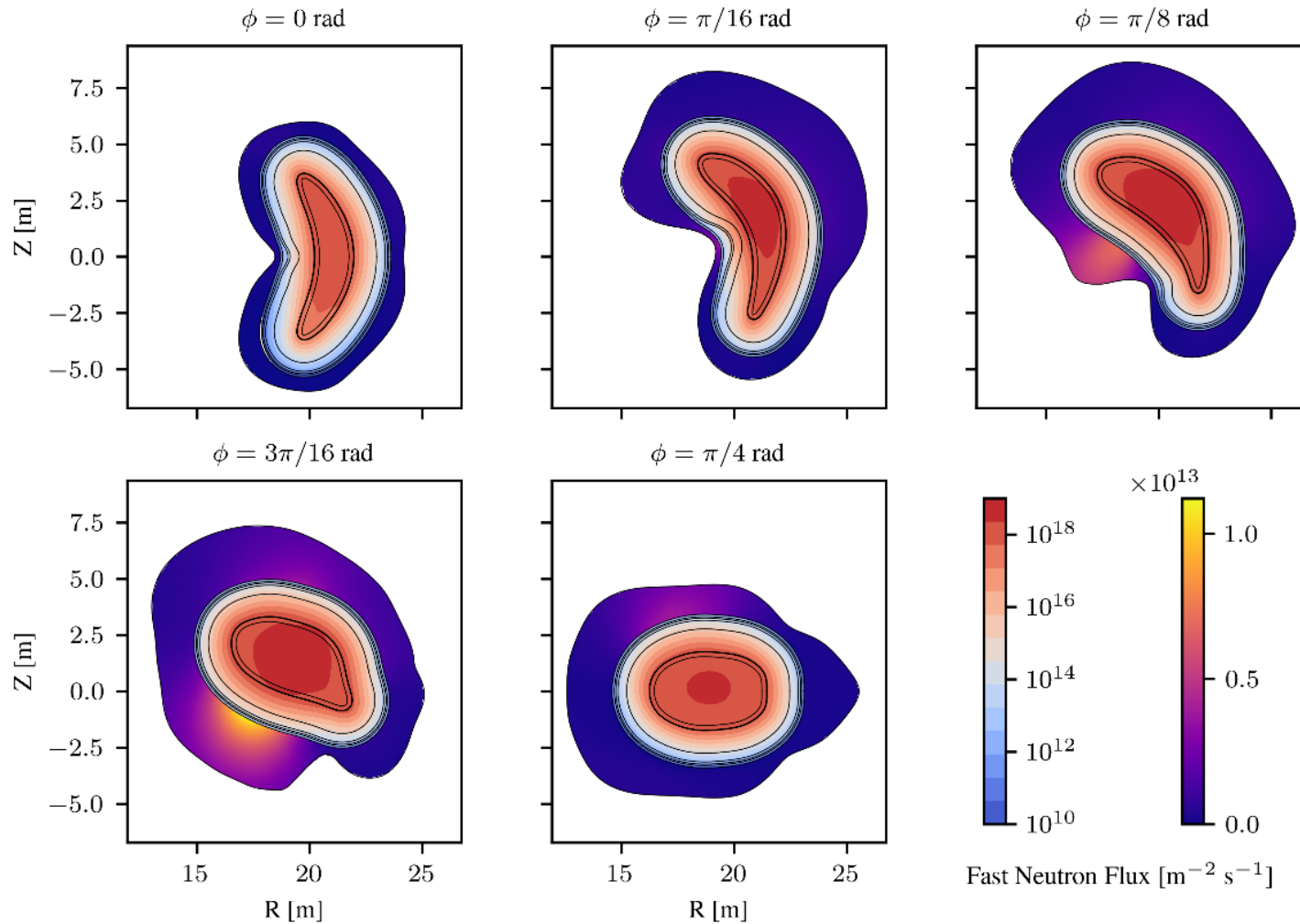


A. Goodman, et al., PRX Energy 3.2 (2024)
A. Goodman, et al., JPP, in prep. (2025)



T. Bogaarts, F. Warmer,
NF in prep. (2025)

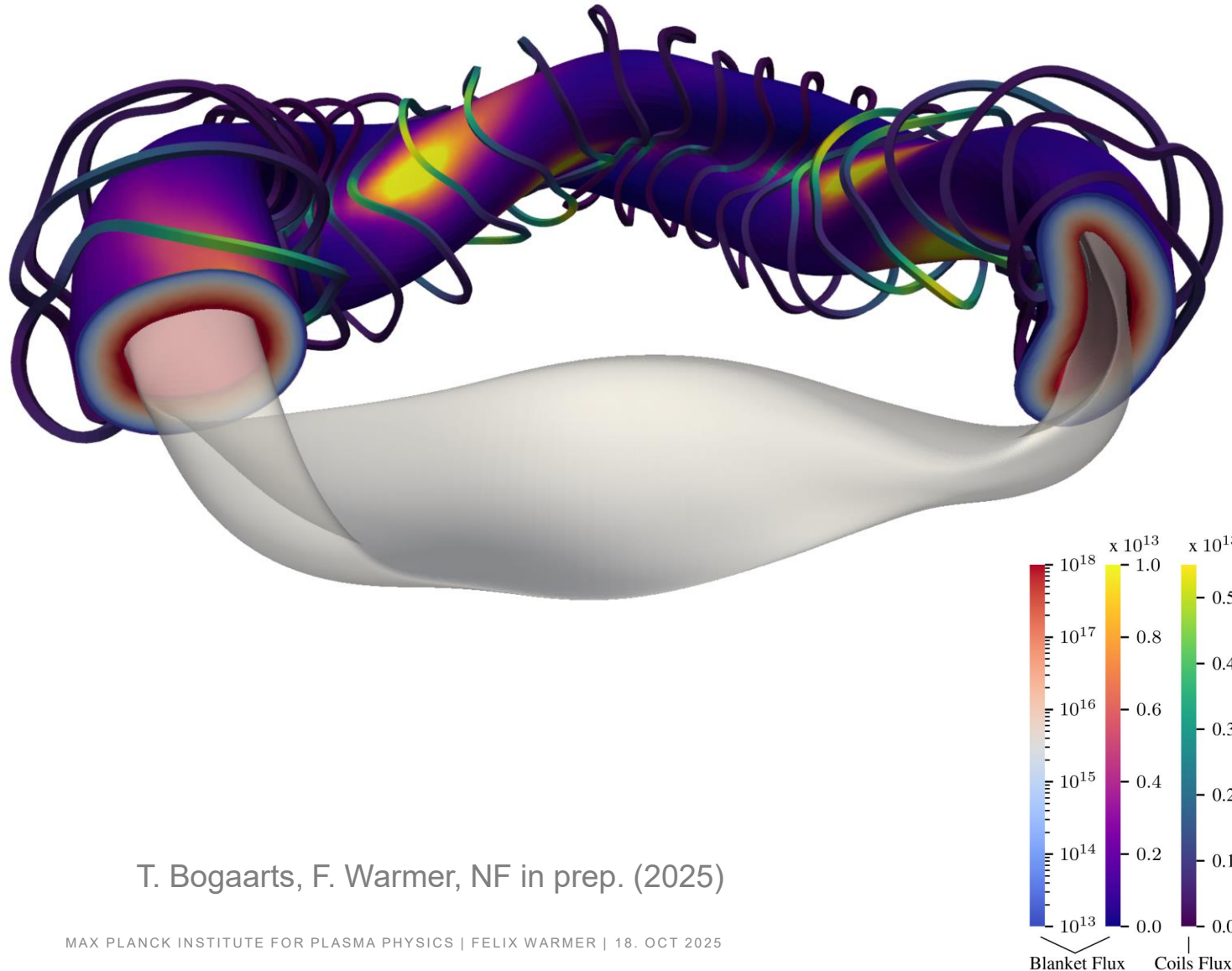
Insights from the New Configuration + Coil set



- Highly varying coil-plasma distance
- Typical narrow regions on the inboard side
- Shielding needed to be adjusted to satisfy the fast flux constraint on the coils ($< 10^{13} \text{ m}^{-2} \text{s}^{-1}$)
- **Conclusion:** coils need iteration, but new method key for quick assessment

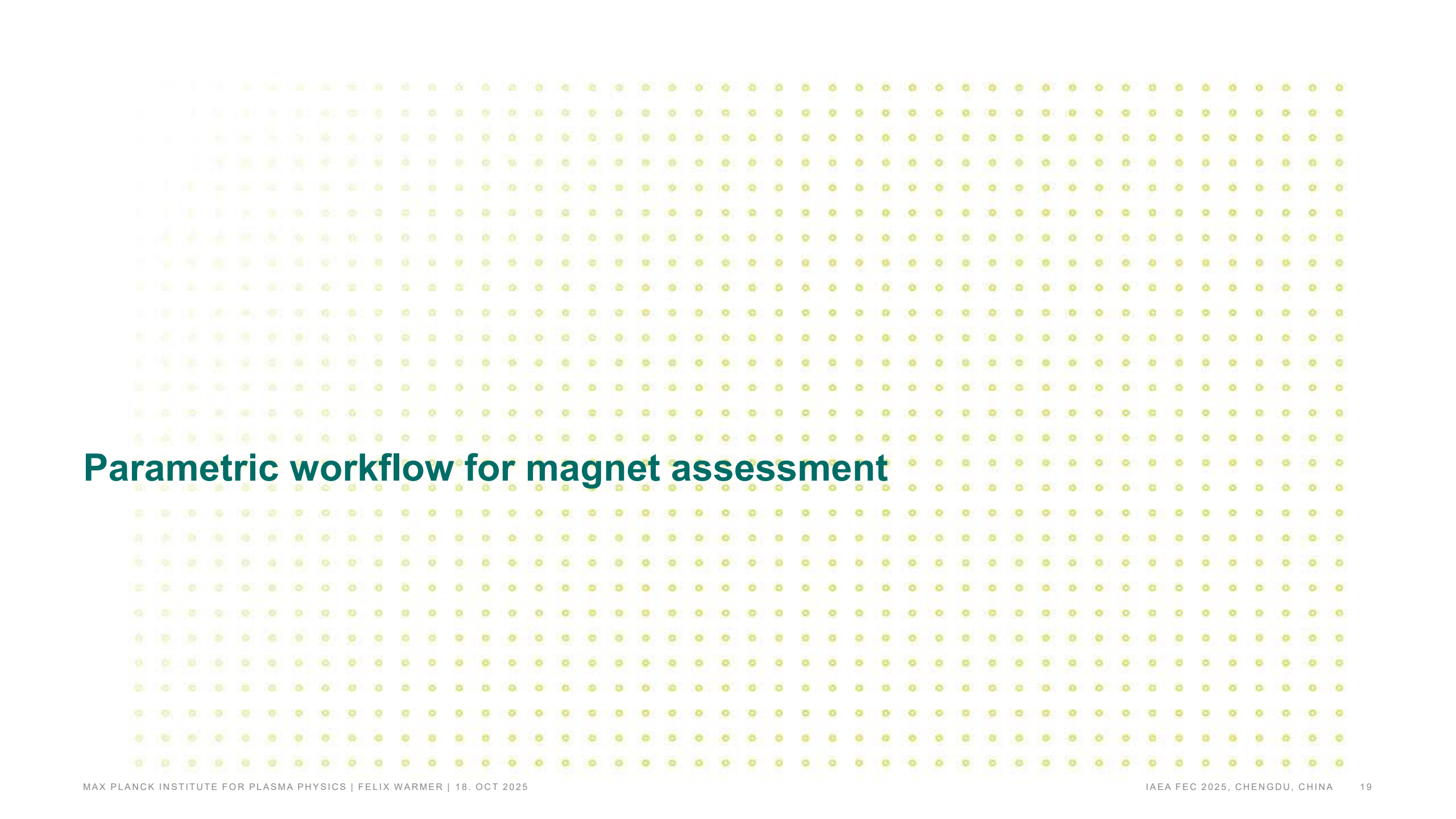
T. Bogaarts, F. Warmer, NF in prep. (2025)

Insights from the New Configuration + Coil set



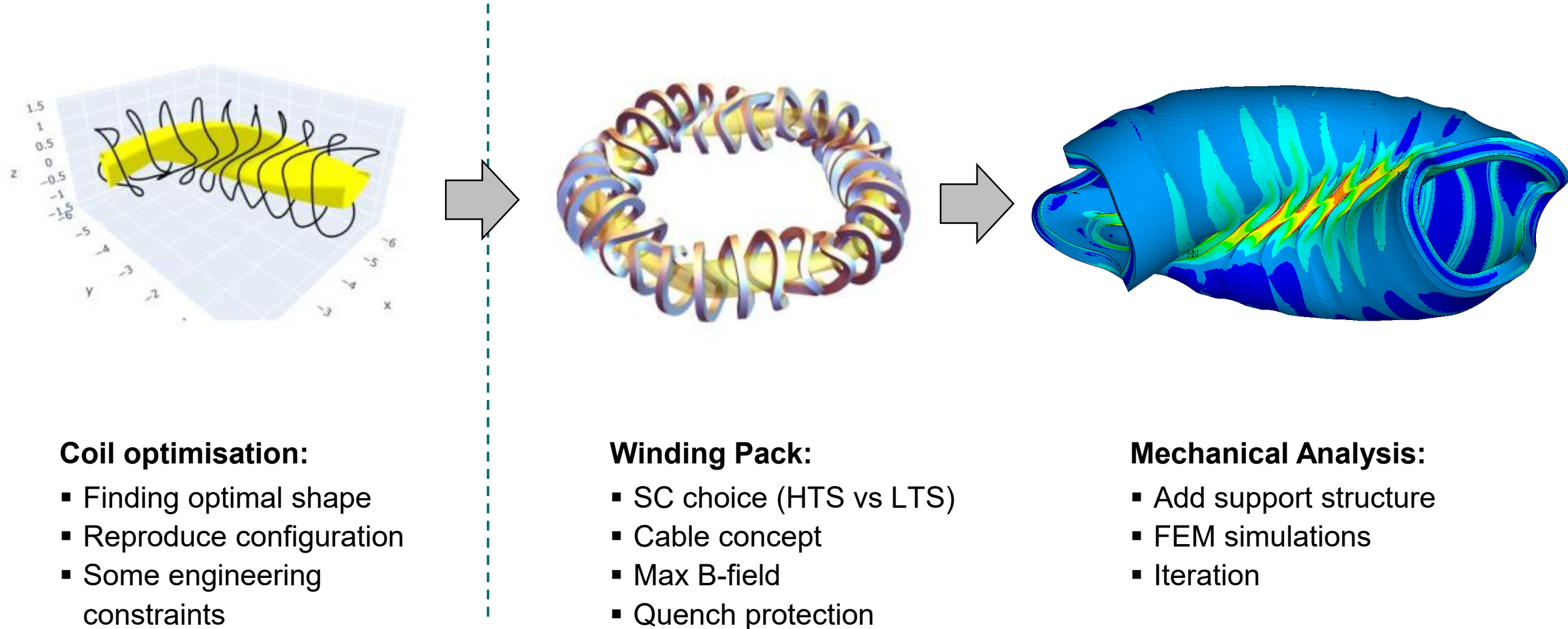
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T. Bogaarts, F. Warmer, NF in prep. (2025)



Parametric workflow for magnet assessment

Towards a realistic & feasible coil set



1) Bayesian Optimisation of Stellarator Coils

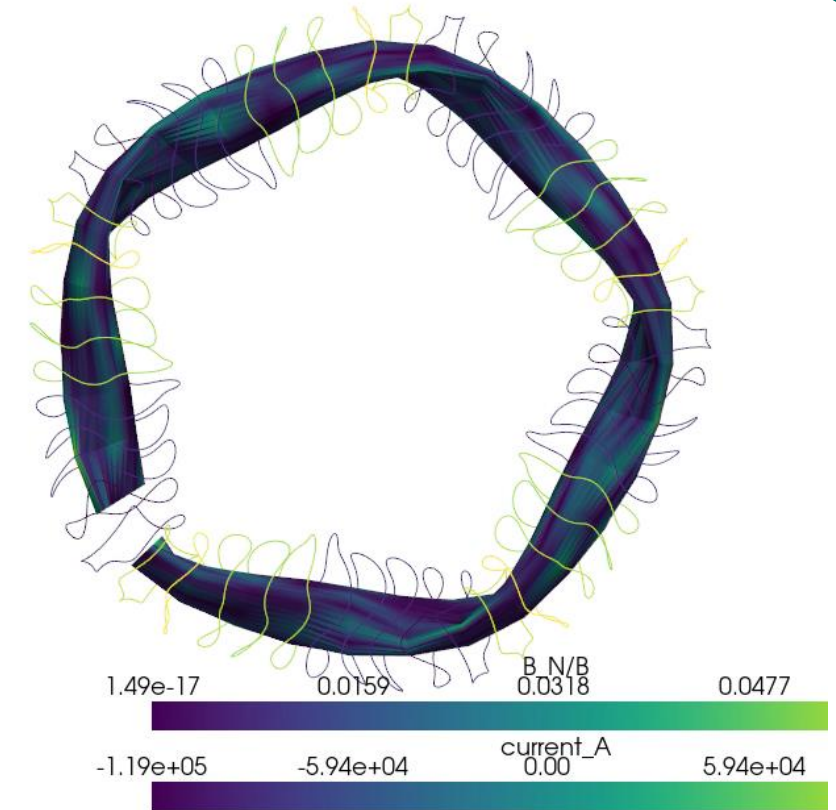
Coil design is a heavily constrained optimisation problem that needs modern solutions

Automated and intelligent design space exploration:

- Many engineering constraints → many weights
- Typically requires manual adjustment or studies
- Reformulate as a Bayesian optimization problem
- Prototype optimization implemented

Ongoing improvements:

- more effective initial guesses
- employ Gaussian processes to model the coils
- ...

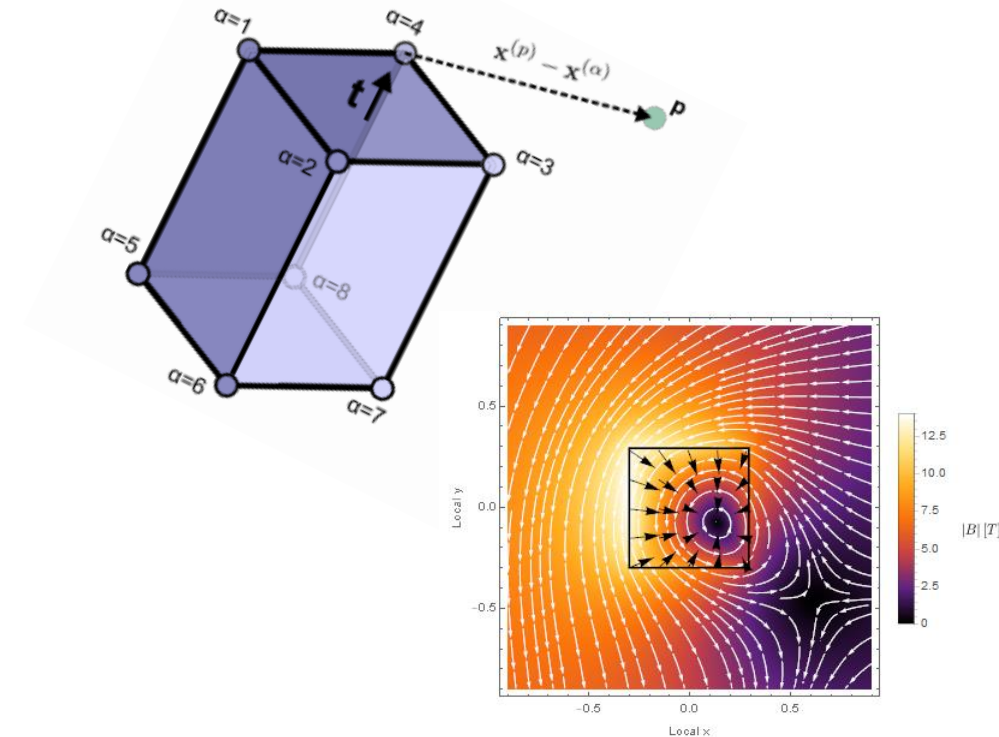


Quantity	W7-X	Bayesian
$\langle B \cdot n \rangle / \langle B \rangle \times 10^{-3}$	29	14
$\max \langle B \cdot n \rangle / \langle B \rangle \times 10^{-2}$	11	6.4

2) Winding Pack modelling

J. Lion, F. Warmer, et al NF 61 (2021)

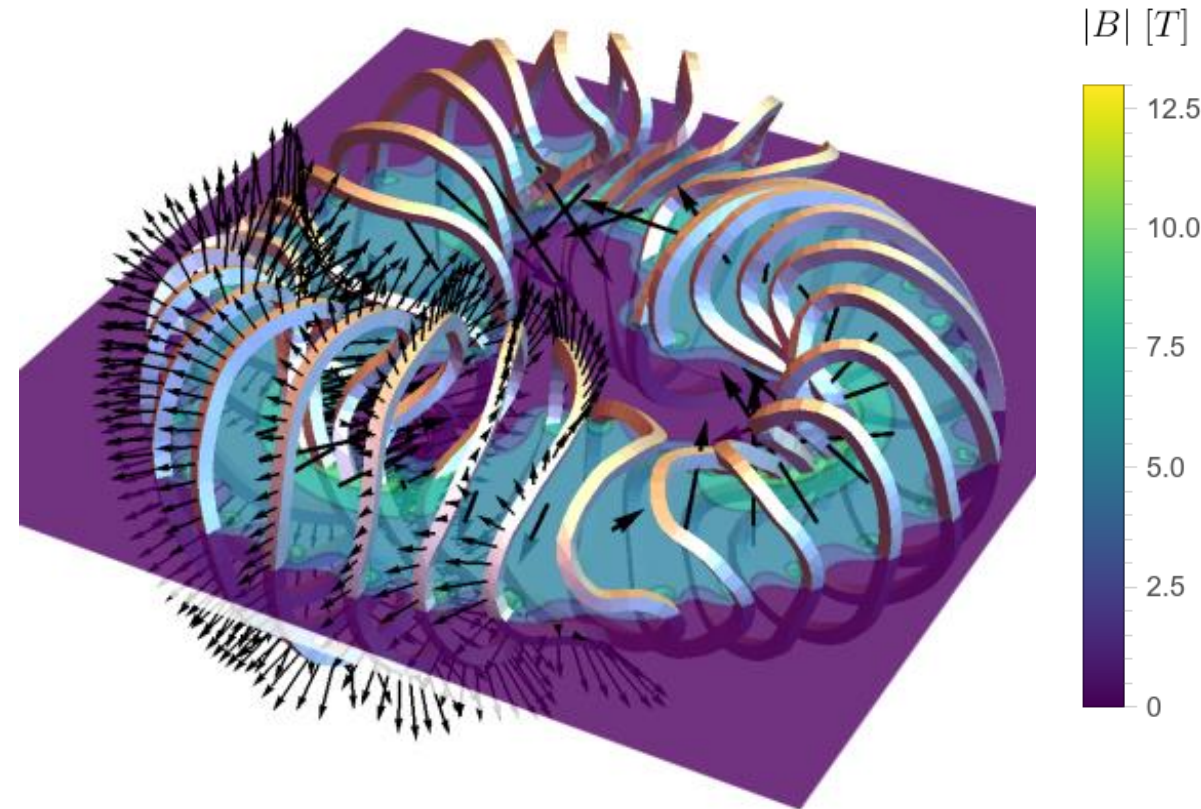
- **Flexible model that considers engineering constraints self-consistently:**
 - Superconductor properties (j_{crit} , B_{max} , T_c)
 - B-Field inside the coils (Biot-Savart)
 - Coil quench protection (Cu fraction)
 - Coil-coil and coil-plasma distance
 - Lateral and radial forces
 - Bending radius
- **Under development:**
 - superconductor strain limits
 - structure stress
 - explore alternative approaches



- Coils approximated by cuboids with constant current density
- Analytic solution to Biot-Savart

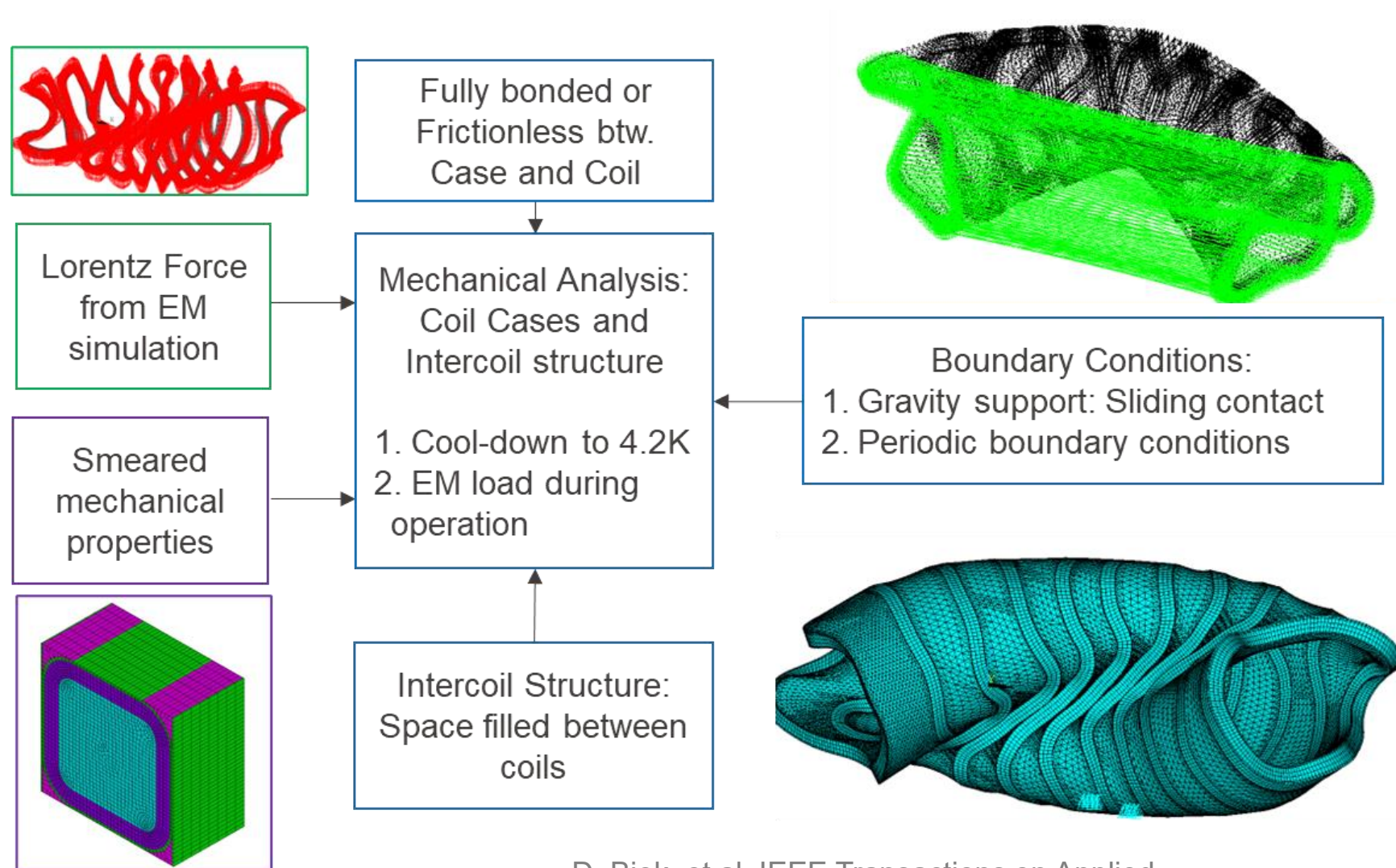
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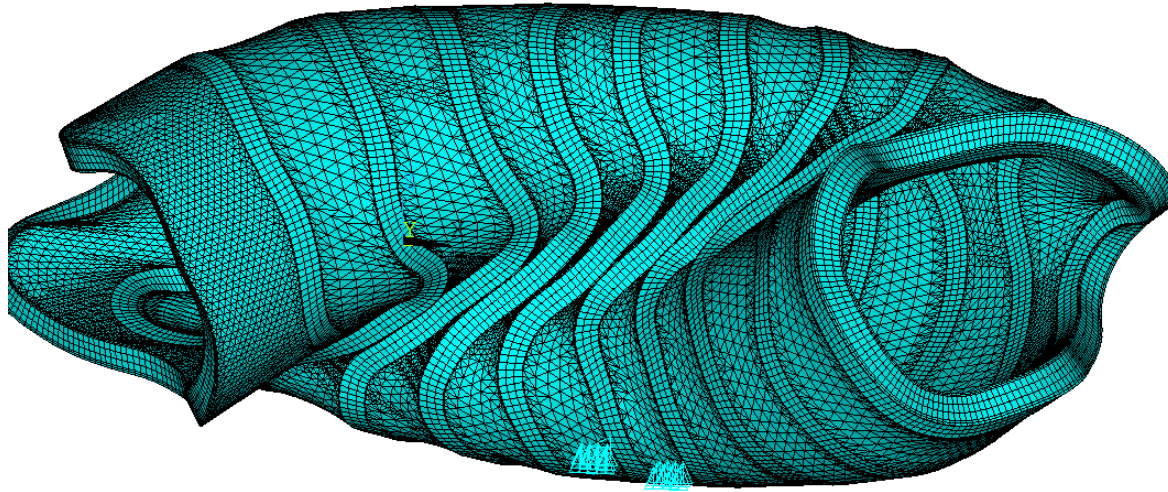
J. Lion, F. Warmer, et al NF 61 (2021)

3) Mechanical Analysis workflow



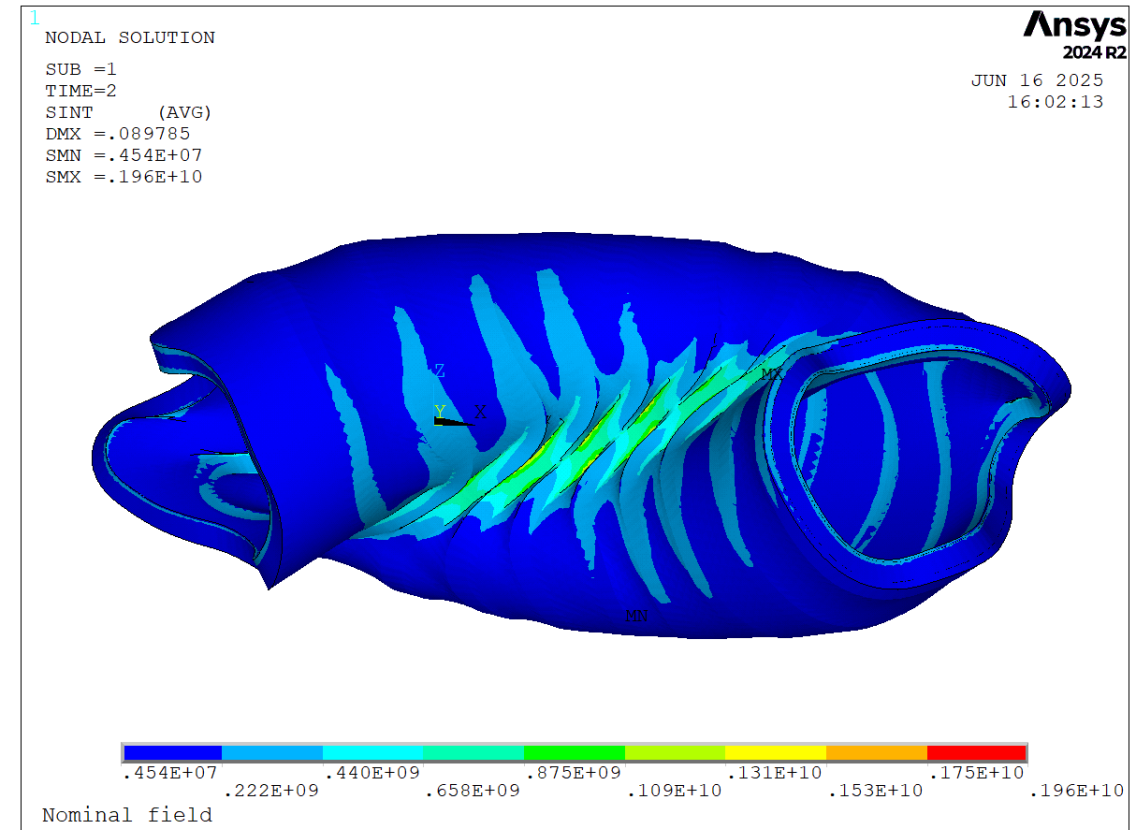
D. Biek, et al, IEEE Transactions on Applied Superconductivity, submitted (2025)

3) Mechanical Analysis



- First prototype workflow (old config.)
- Automate, apply to new configurations
- Include ports / access / inner ring(?)

D. Biek, et al, IEEE Transactions on Applied Superconductivity, submitted (2025)





Coherent Blanket design for Stellarators

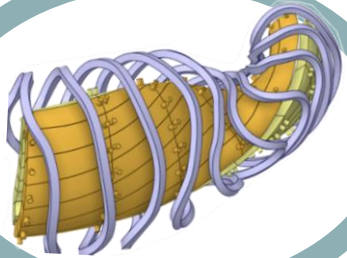


More detailed engineering activities within EUROfusion

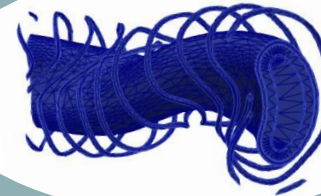
I. Palermo, this IAEA FEC,
Friday, 14:40, TEC/4-3



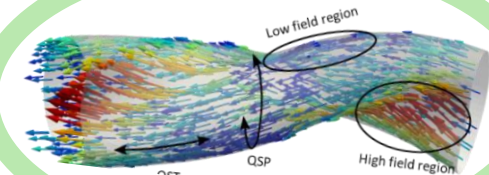
Segmentation



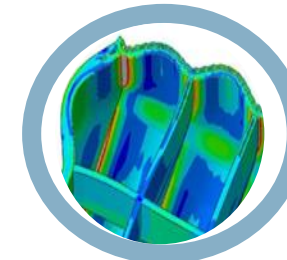
**Neutronics
tools for MC**



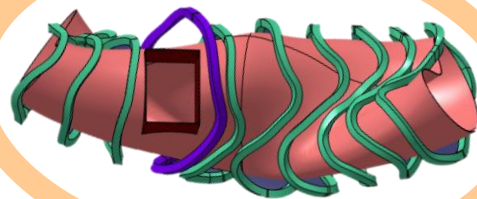
MHD tools



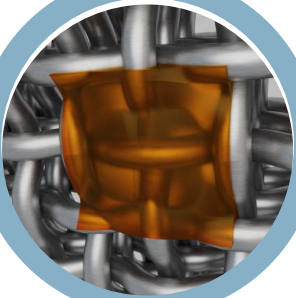
**Thermal-hydraulics +
Thermo-mechanics
Multi-scale approach**



RH approaches



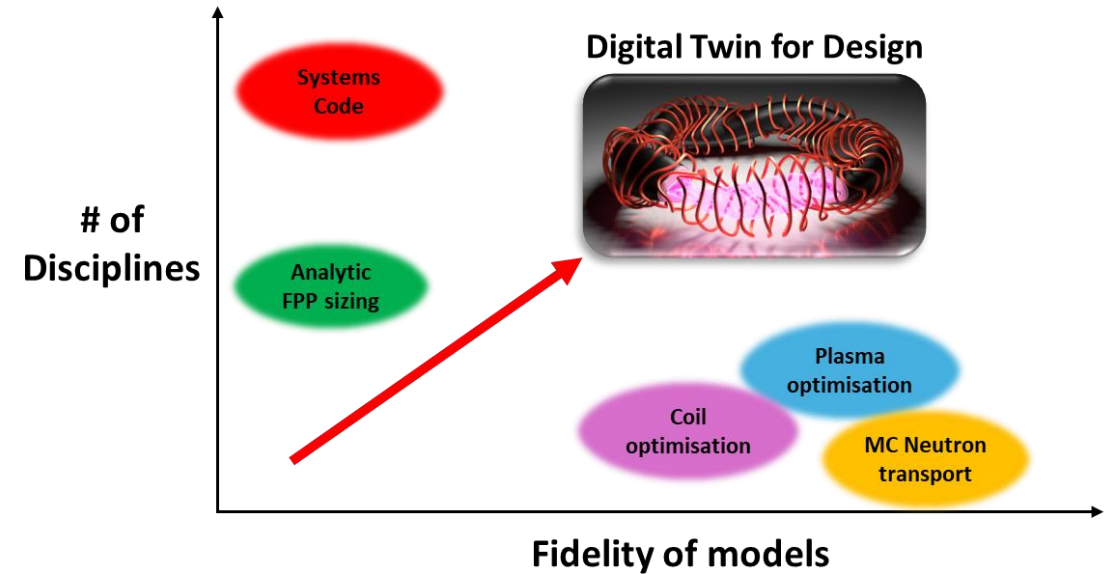
FW design



Dedicated Stellarator Reactor group founded at IPP Greifswald

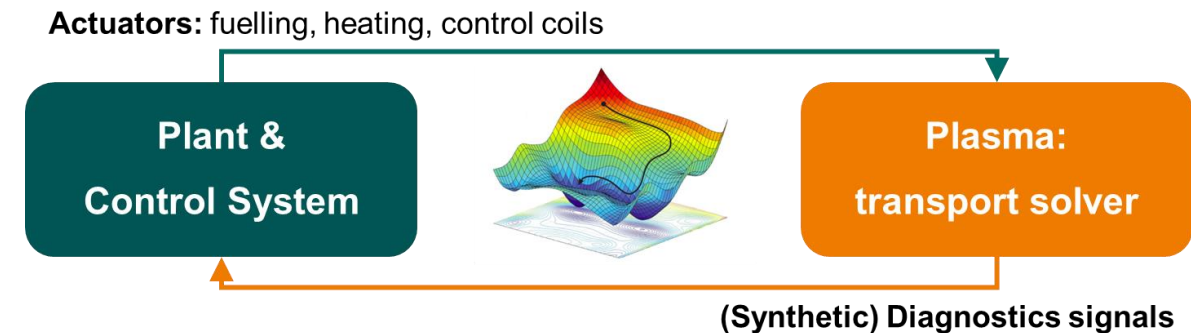
‘Digital Twin’ for Stellarator Design

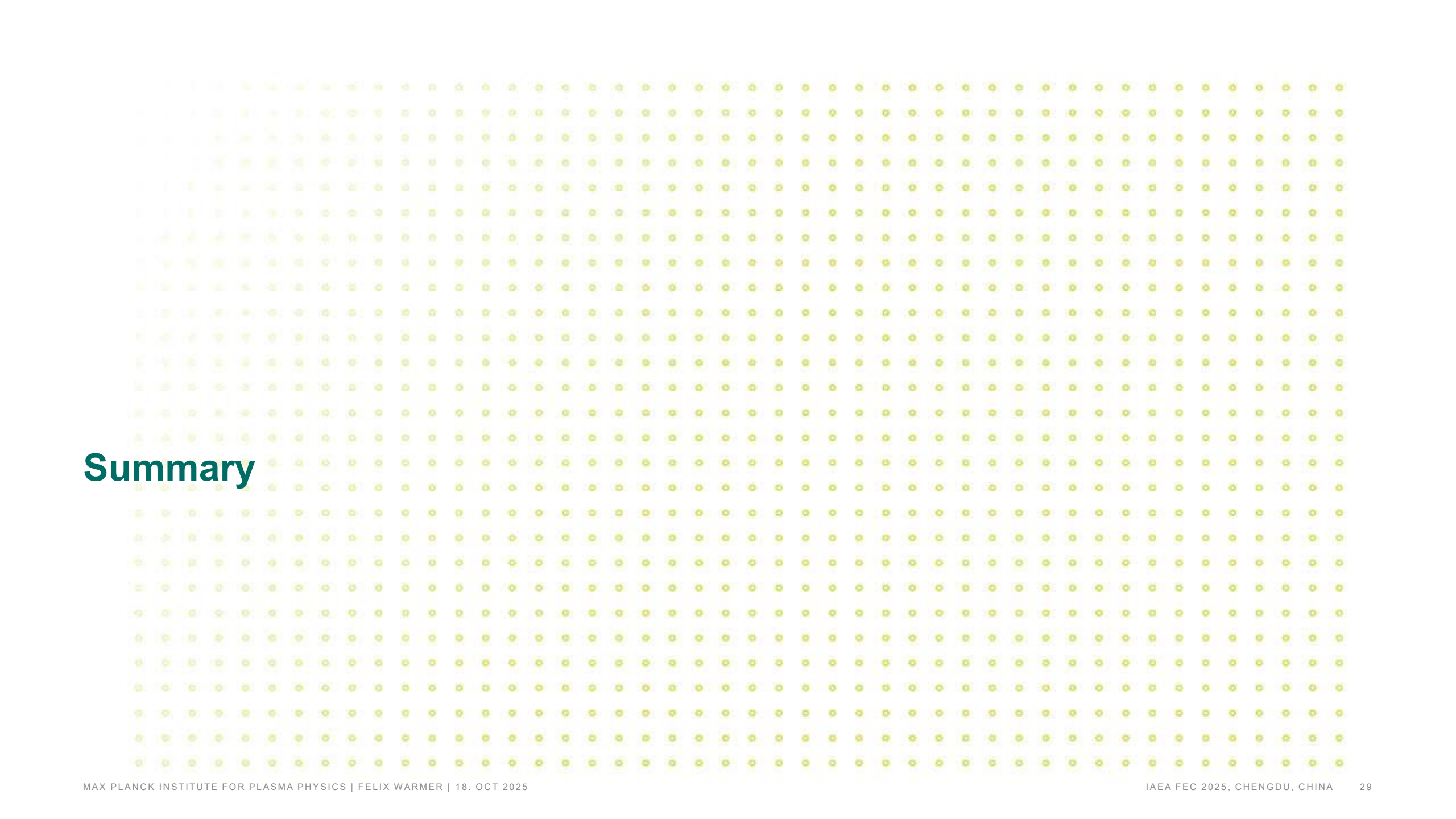
- innovative modelling
- model integration
- intrinsic multidisciplinary



‘Flight Simulator’ for Stellarator Operation

- reactor operation & control
- model predictive control
- diagnostic requirements





Summary



Acknowledgements

C. Albert⁷, J. Alguacil¹, D. Biek², T. Bogaarts³, G. Bongiovi⁴, V. Bykov⁵, J.P. Catalán¹, R. Duligal³, I. Fernandez⁶, S. Giambrone⁴, C. Hume¹⁰, M. Hrecinuc¹⁰, J. Lion⁵, T. Lyytinen^{8/9}, J.A. Nogueron⁶, I. Palermo⁶, V. Queral⁶, D. Rapisarda⁶, L. Sanchis⁸, X. Sarasola², K. Sedlak², A. Snicker^{8/9}, D. Sosa⁶, F.R. Ugorri⁶, F. Warmer³

3-4ppy/y



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³Eindhoven University of Technology

⁴Università degli Studi di Palermo

⁵Max Planck Institute for Plasma Physics

⁶Centro de Investigaciones Energéticas, Medioambientales y Tecnológicas

⁷Technical University Graz

⁸Aalto University

⁹VTT Finland

¹⁰UK Atomic Energy Authority





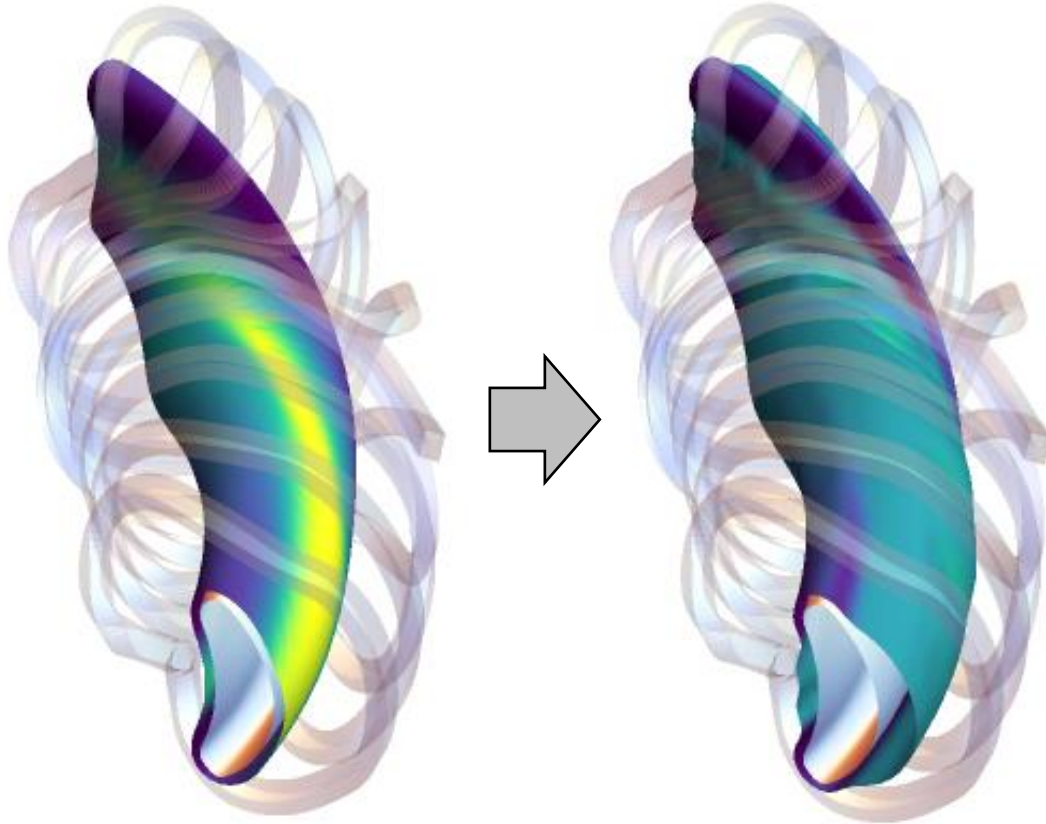
Summary

- **Geometry tools developed for blanket & coil representation (tetrahedral mesh)**
- **New, fast deterministic neutronics method that works directly on this geometry**
- **Application to a new promising configuration reveals the importance of coil design**
- **Consequently, advances are made in automating magnet system design workflow**
- **New methods for coil optimisation emerging (+ adding engineering constraints)**
- **First steps towards FEM structural mechanical analysis**
- **EU filling gaps in Stellarator engineering that were desperately needed**
- **contact: felix.warmer@ipp.mpg.de**



Backup slides

Such Models allow unprecedented Design Optimisation

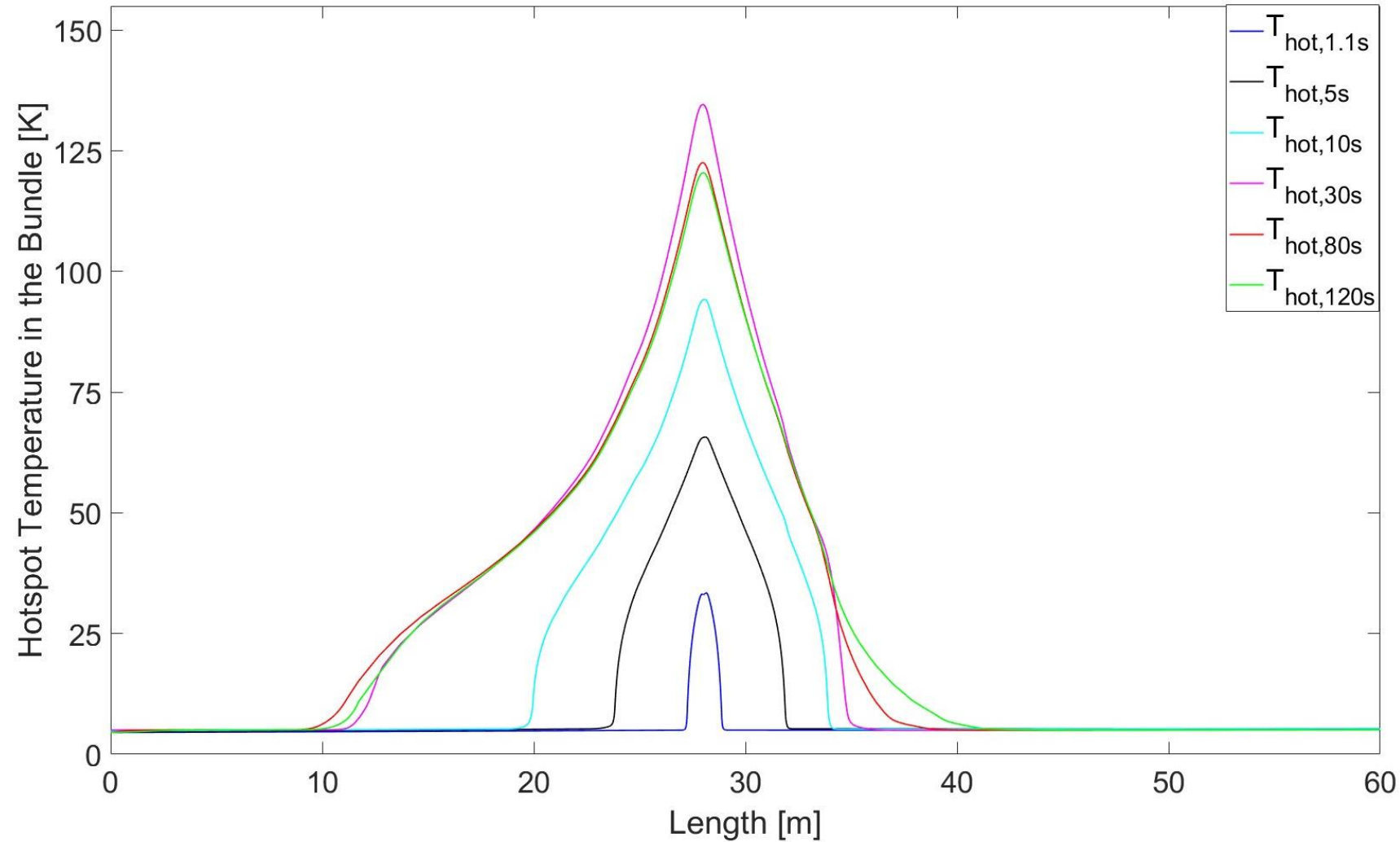


- Allows 3D blanket optimisation
- Potentially increased life-time
- **Next step: integrate with coil optimization**

**Virtually impossible
with previous methods**

J. Lion, F. Warmer, et al NF 62 (2022)

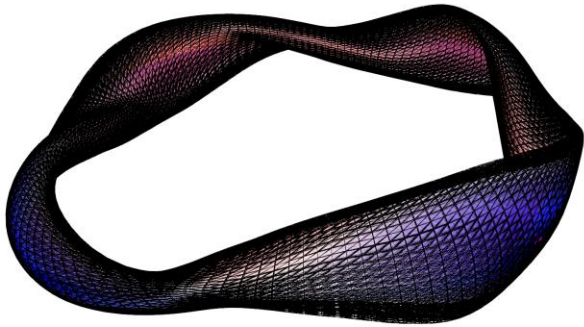
3) Quench Modelling



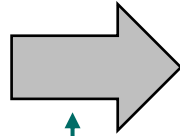
Neutron Transport: Traditional MC approach applied to Stellarators



automatic CAD model

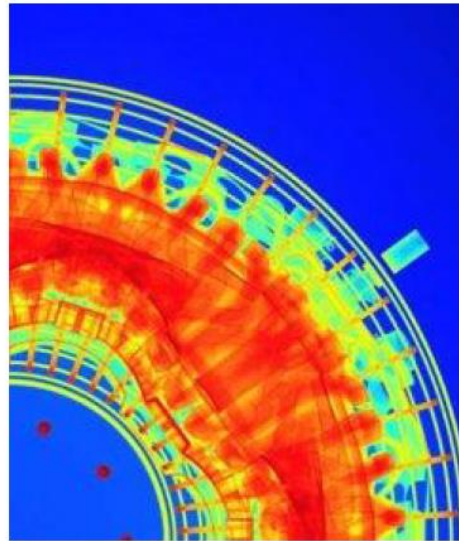


Tetrahedral / Meshed

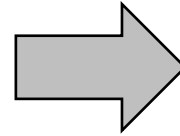


Conversion tools /
directly on mesh

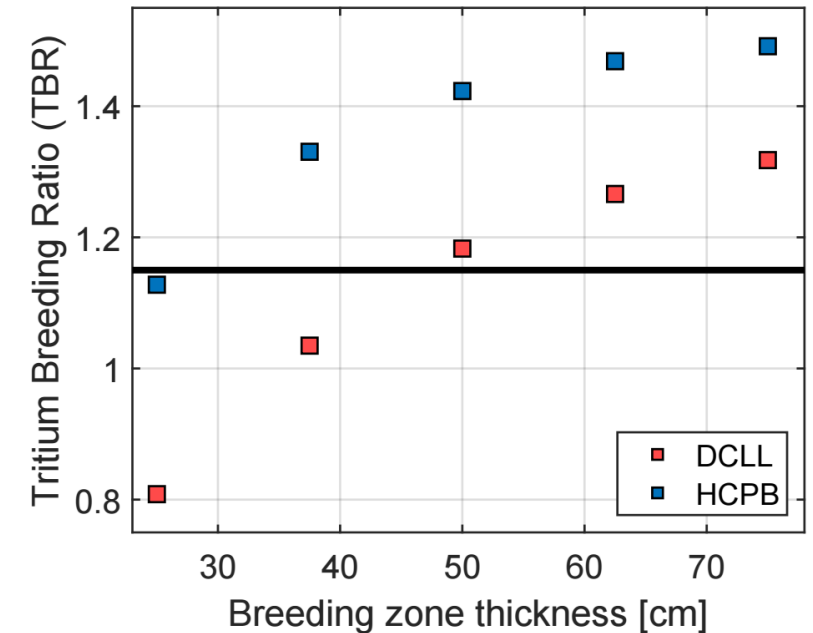
3D nuclear response



e.g. MCNP6, Serpent2



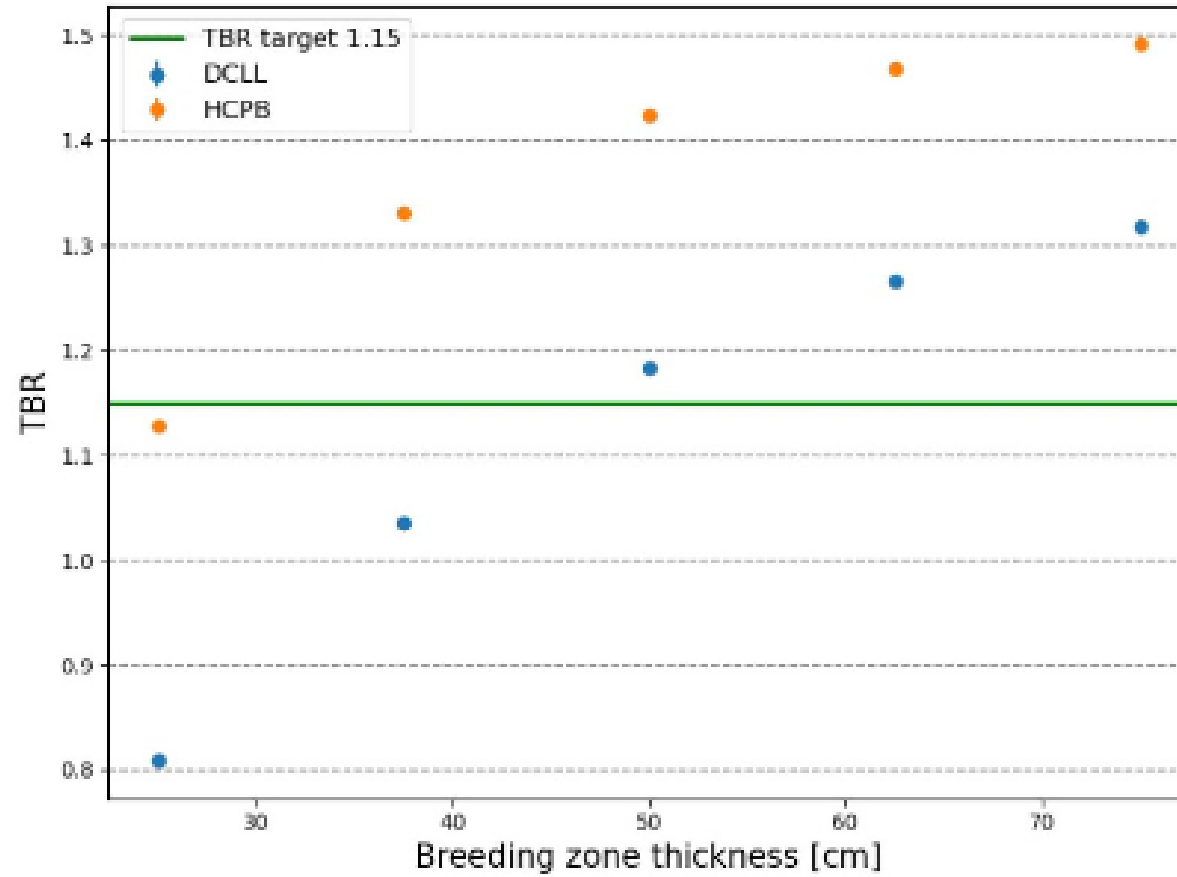
Parametric Studies



Full 3D Stellarator geometry

EUROfusion
collaboration

TBR deterministic neutronics



(a)



State-of-the-art: Systems Codes such as “PROCESS”

Simplified, yet comprehensive model of an entire fusion reactor (0D or 1D!)

- Holistic framework
- Modular(?), multidisciplinary
- Fast for Design Point exploration

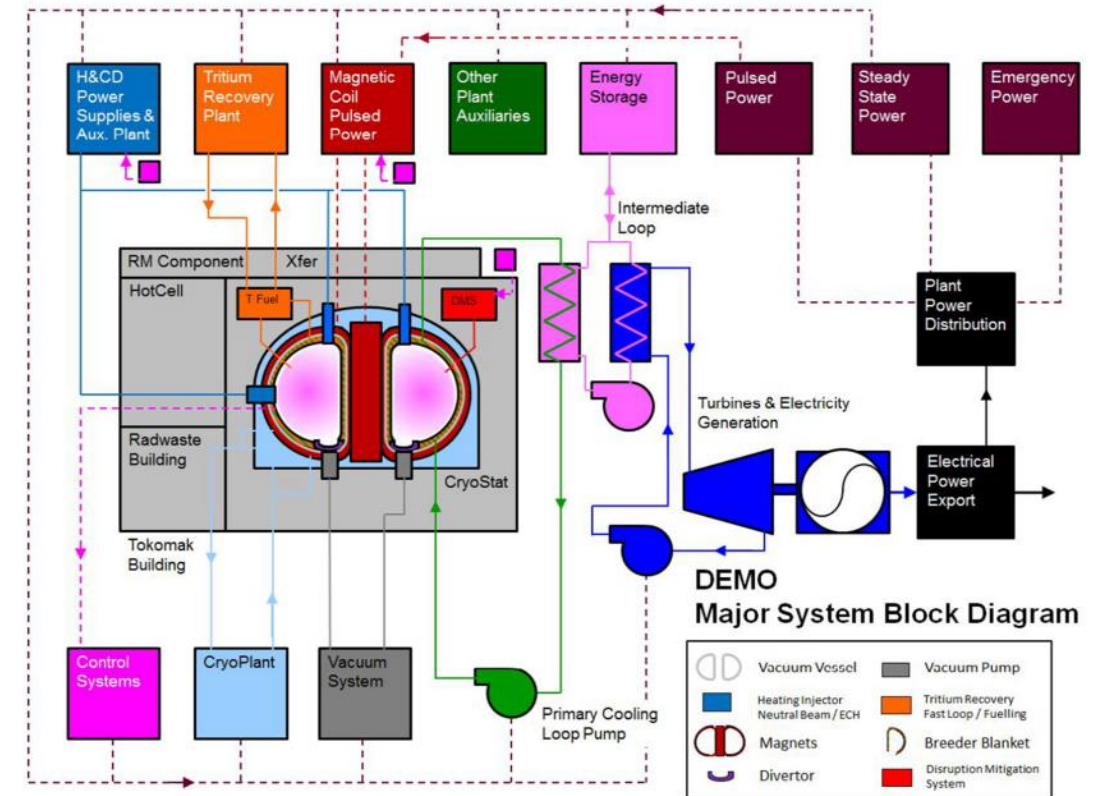


Figure 1. Schematic of a DEMO power plant.

Limits of Systems Codes

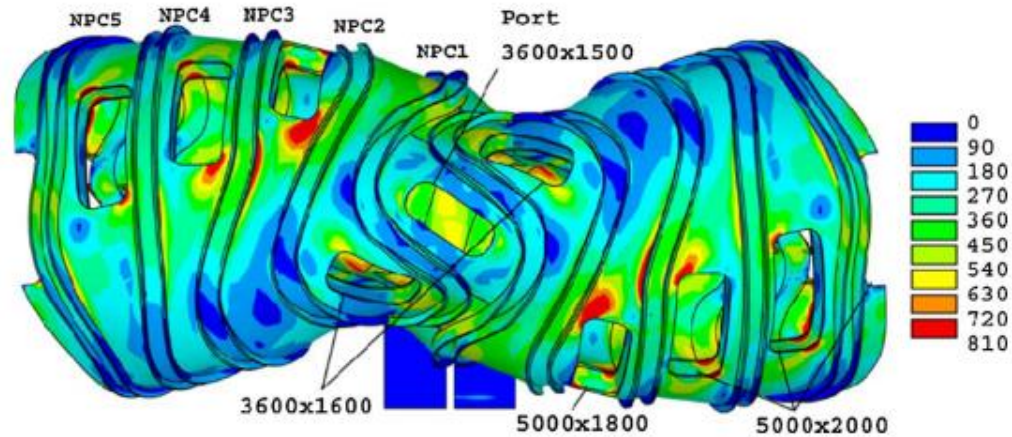
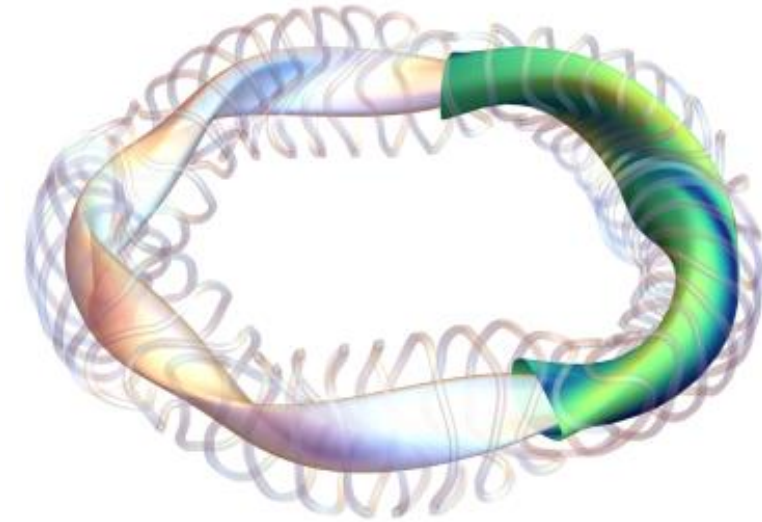
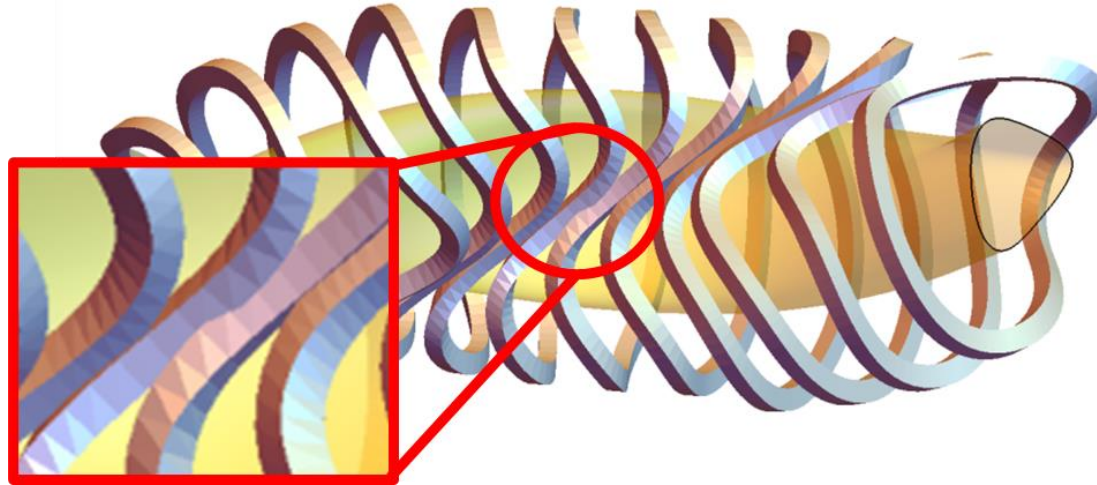
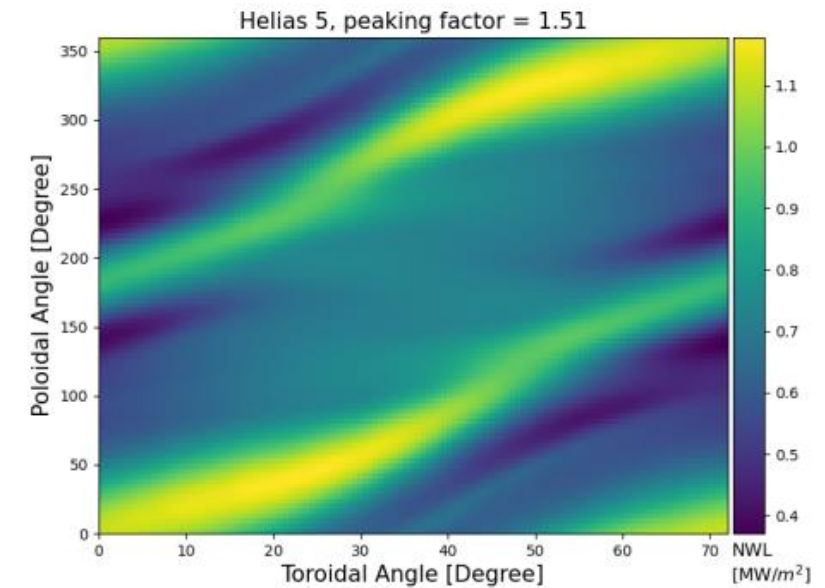
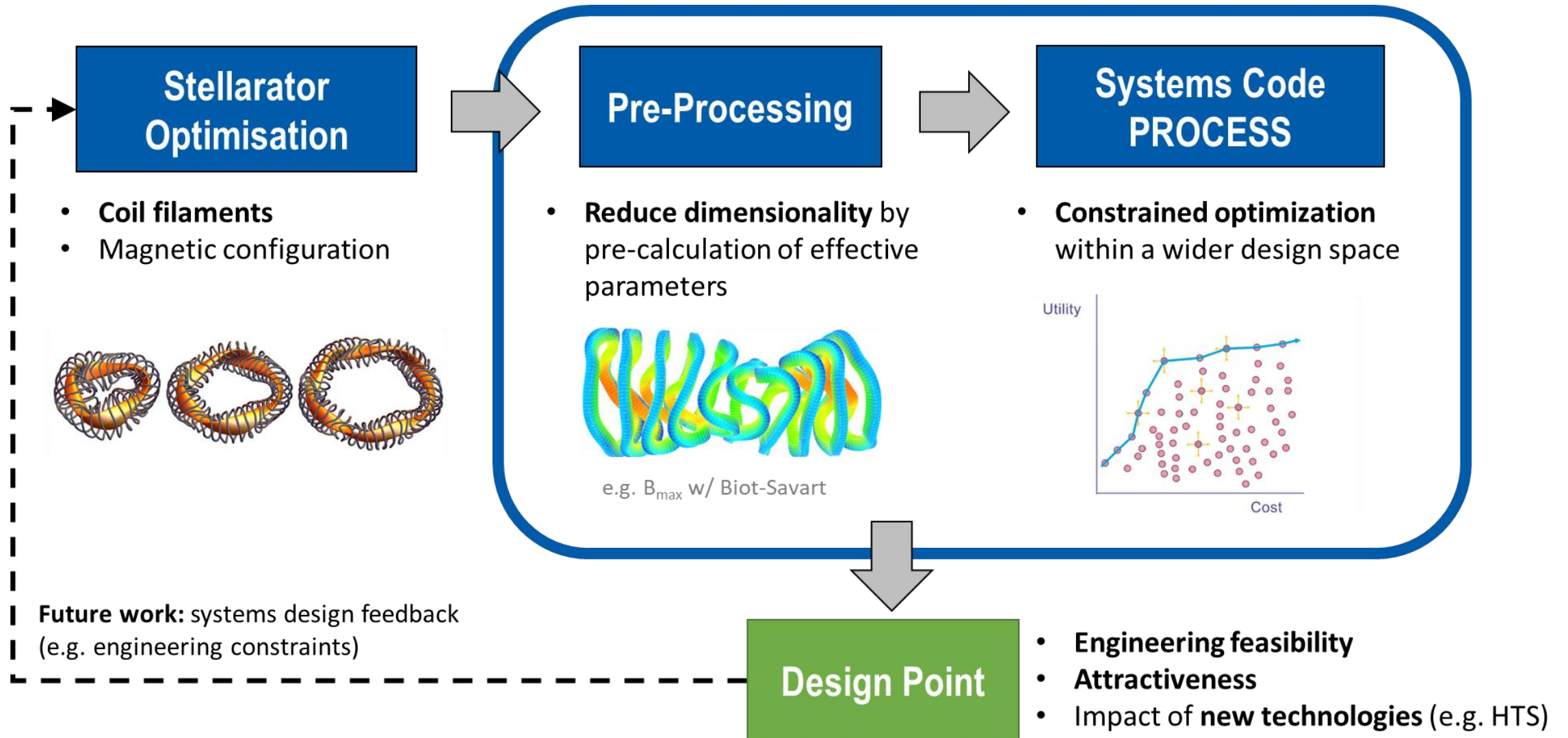


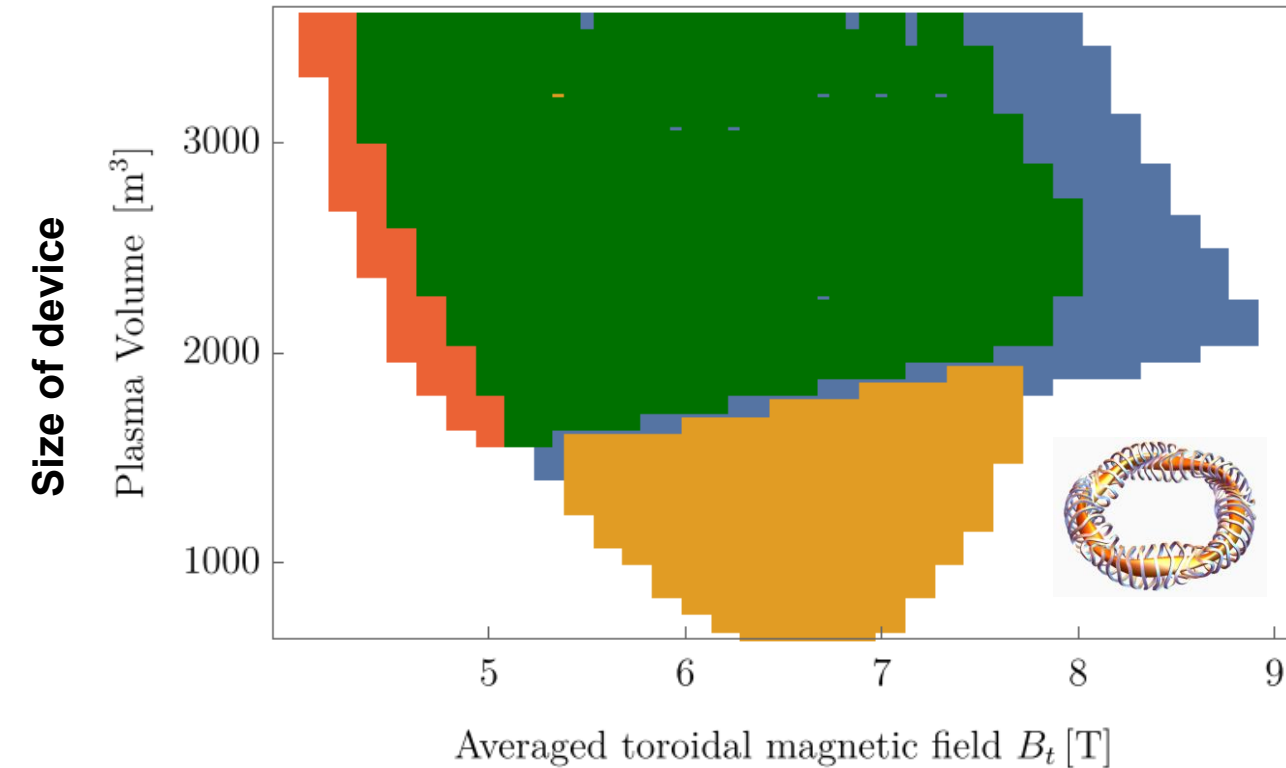
Fig. 4. Stress intensity (MPa) in magnet system and maximally possible port windows (mm); structure not yet optimized.



State-of-the-art: Systems Codes such as “PROCESS”



State-of-the-art: Systems Codes such as “PROCESS”



Successfully used in the past

- **Identify design space boundaries**
- **Assess engineering limits**
- **Study impact of new technologies**



ECRH design for new reactor-relevant Stellarator configurations

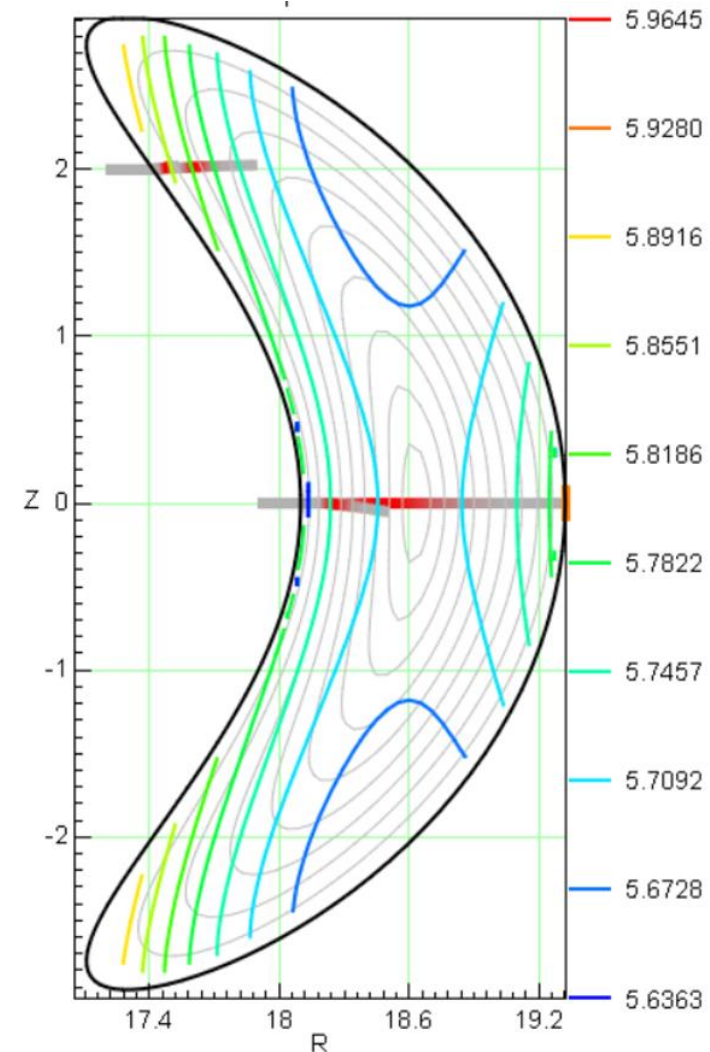
Experiments today use X2 or O2, but high field & density requires X1 or O1

Extensive TRAVIS simulations done:

- SQUID: High mirror ratio / saddle-points
- Injection location (pol./tor.) & angle
- Scenario depends on freq. and density
- Several solutions found for a combination of O1 (startup) and X1 (overtake)

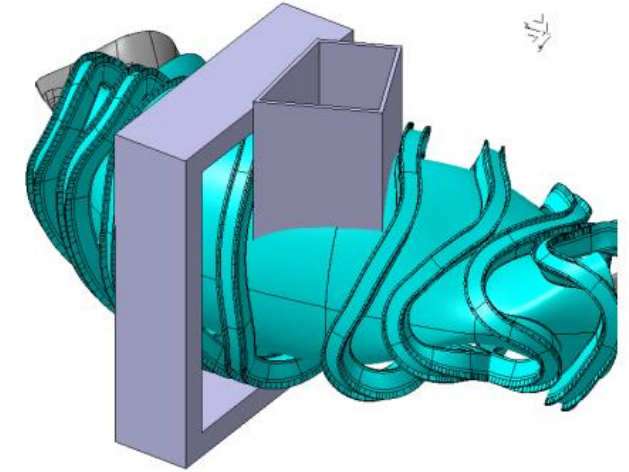
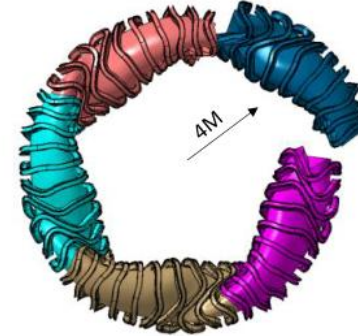
Complexities identified:

- Sensitive to magnetic geometry
- Heating of trapped electrons
- Relativistic effects



Remote Maintenance

- 1) Vertical Ports only
- 2) Vertical + Horizontal Ports
- 3) Enlarged Vertical Ports
- 4) Sector Splitting

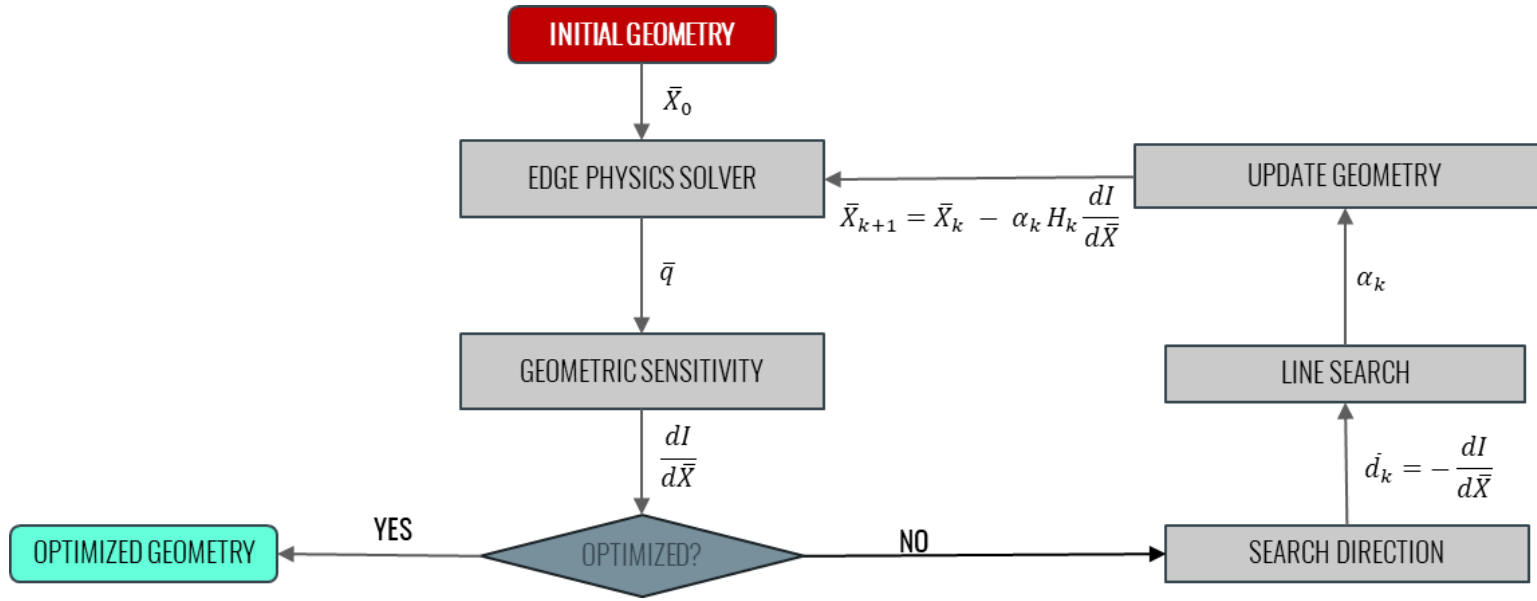


(Baseline)

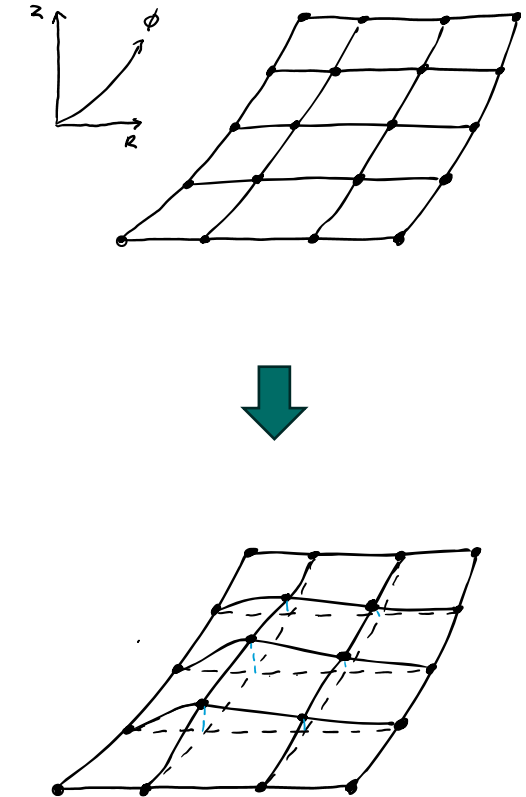
Consideration	Approach 1	Approach 2	Approach 3	Approach 4
Blanket handling	0	+1	+1	+2
Divertor handling	0	-1	0	+1*
Failure scenarios	0	+1	+1	+1
Inspectability	0	+1	+1	+1**
Hardware costs	0	0	0	-2
Radiation & CC	0	-1	-1	-1
RM Durations	0	0	0	0
Wider plant implications	0	-1	-1	-2
Total:	0	0	+1	0

+2 Much better than
+1 Better than
0 Same as baseline
-1 Worse than
-2 Much worse than

Divertor Development: a) Automatic Target Optimisation



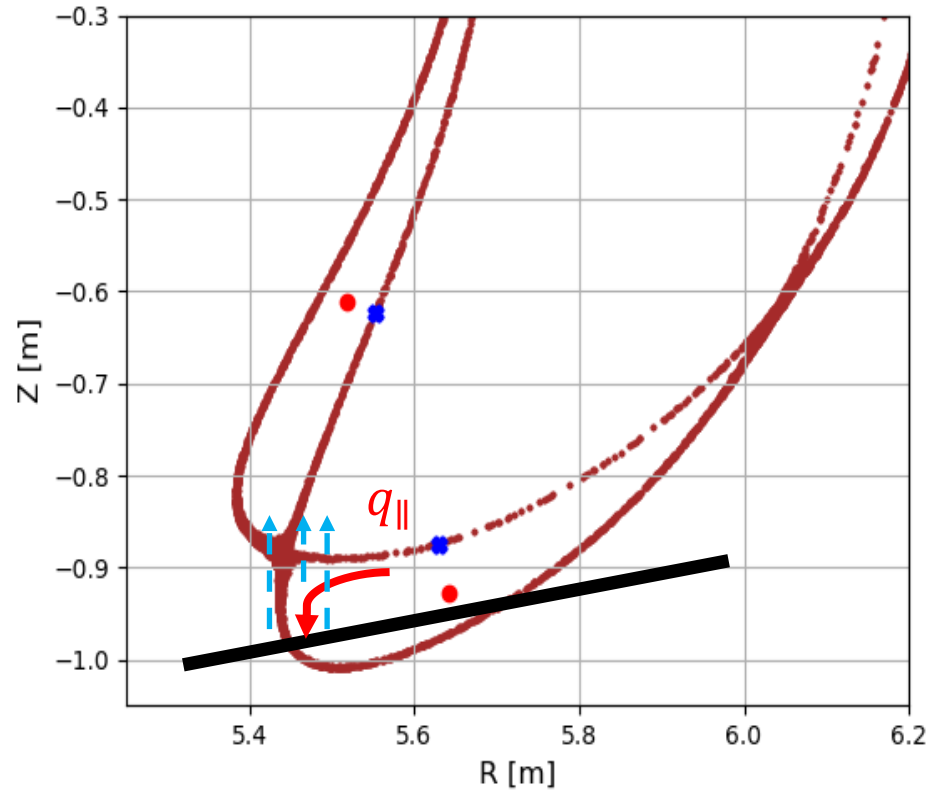
see Poster Nr. 28 by
Reeve K. Duligal



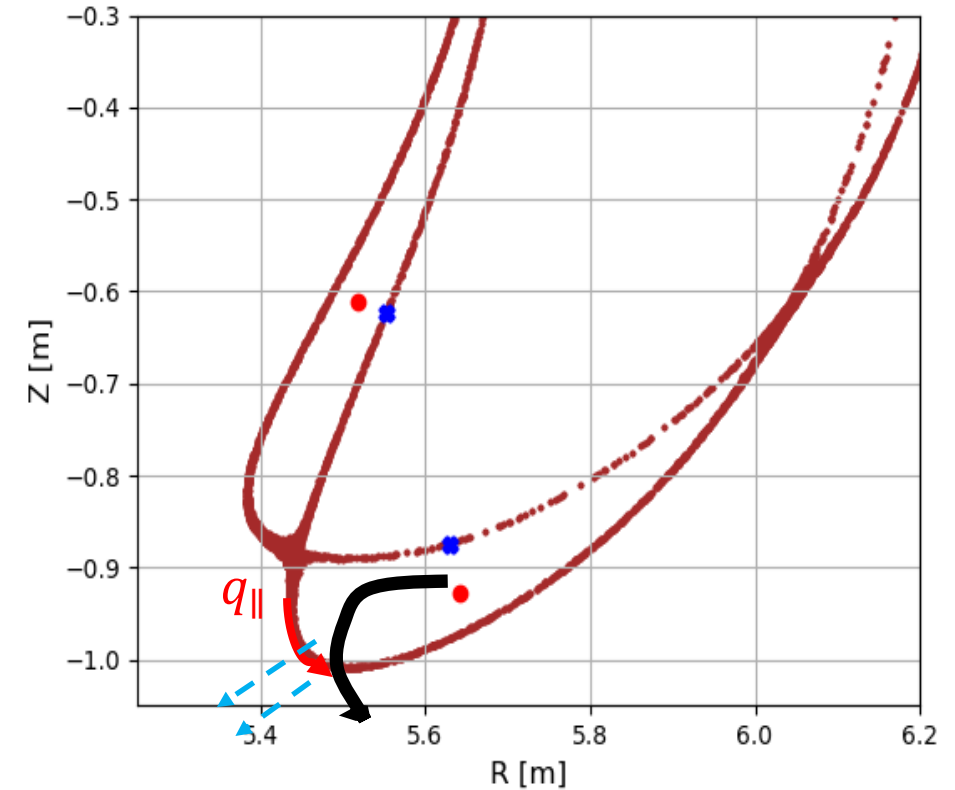


Divertor Development: b) Closed Divertor

Open (W7-X now)



Closed (LHD/Tokamaks)



see also presentation N. Maaziz

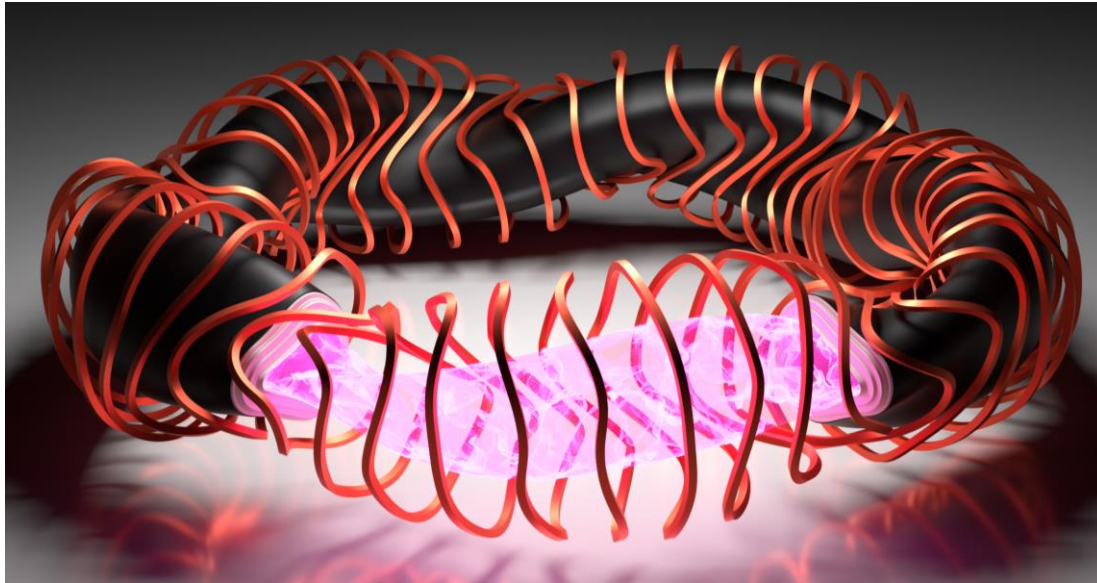


Limits of Systems Codes

- Naturally evolved over years: old code, no standardised interfaces
- not as modular as required, difficult to make changes
- Mostly 0D or 1D models
- Addressing 3D features important
- **Stellarator version is diverging from main PROCESS**

→ Develop new stellarator design platform: a Digital Twin

New Project: an integrated 3D design platform



A virtual stellarator reactor

- Configuration flexibility
- 3D engineering models
- Enables fast design iteration
- Allows data-driven decisions
- Link to stellarator optimization

But: What do we actually need to model?



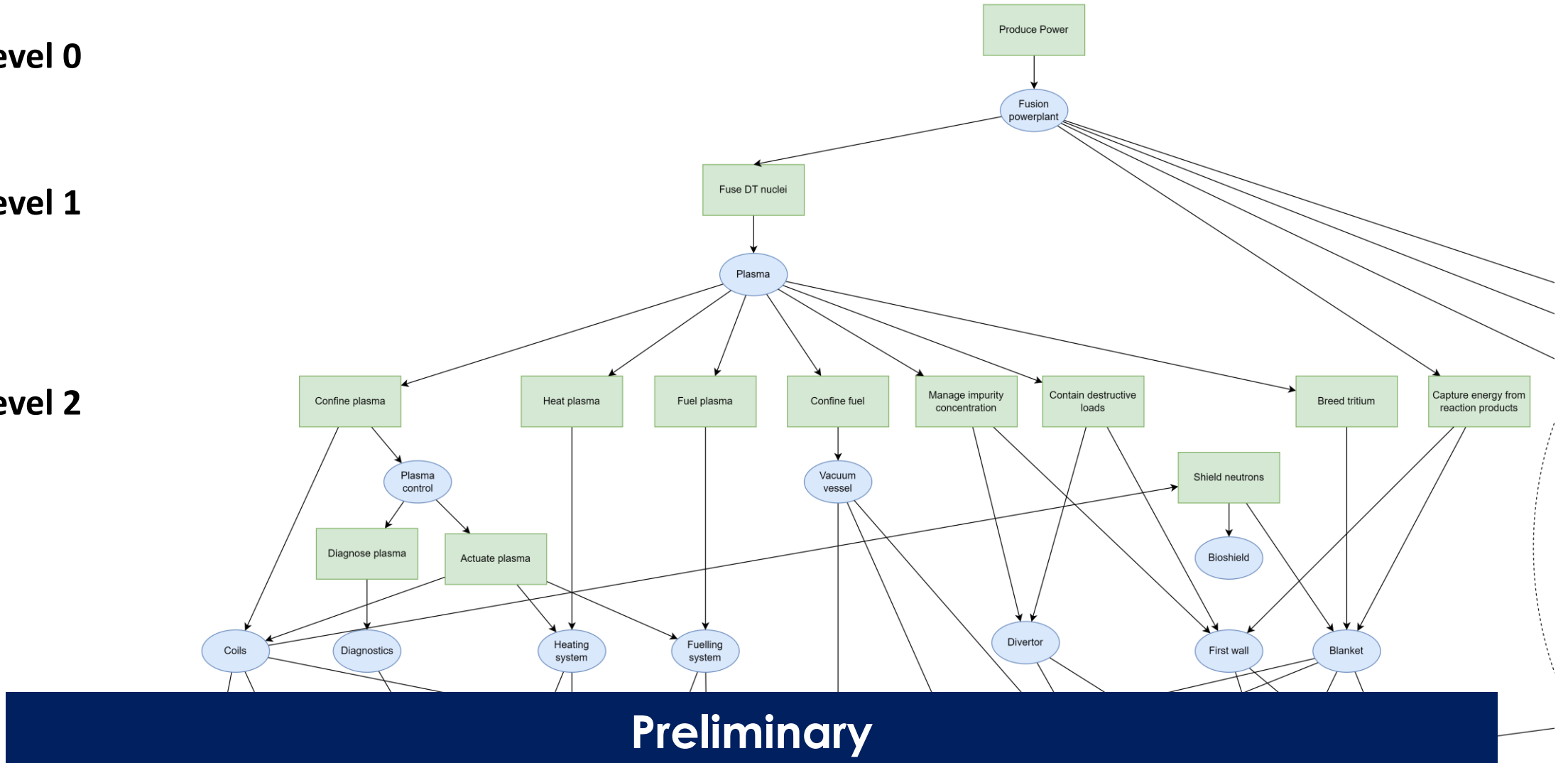
Structured System / Functional Decomposition

Level 0

Level 1

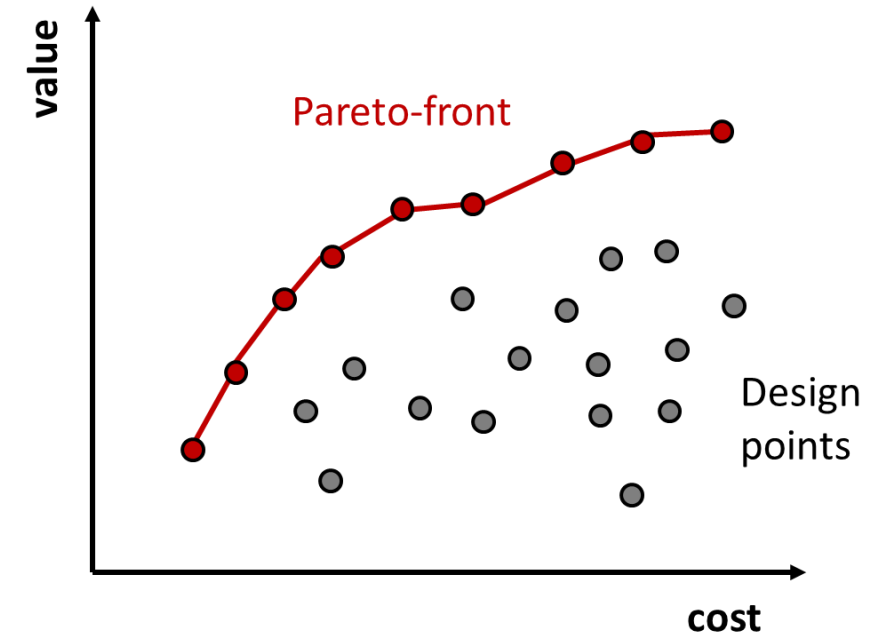
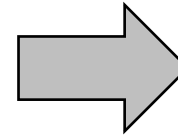
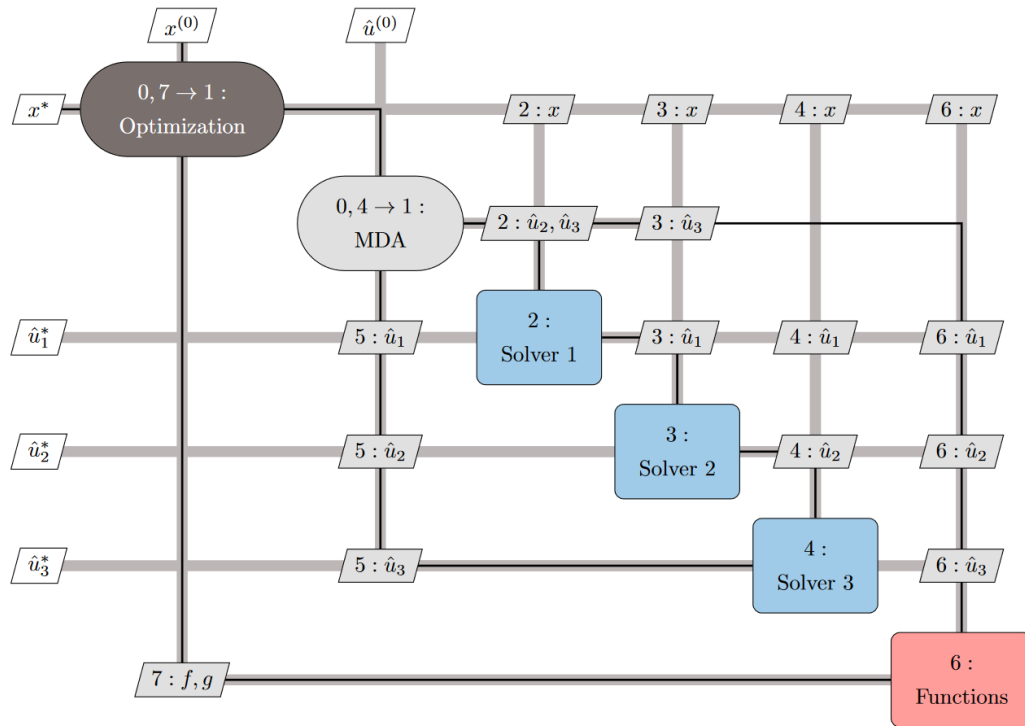
Level 2

...





Optimisation Architecture in a multidisciplinary setting



Quantifying trade-offs for data-driven design decisions