

# Establishment and Progress of Korean Fusion Reactor Design Activities: A Coordinated National Approach

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and *KFE researchers and domestic and international collaborators*  
*in Korea's fusion energy development.*



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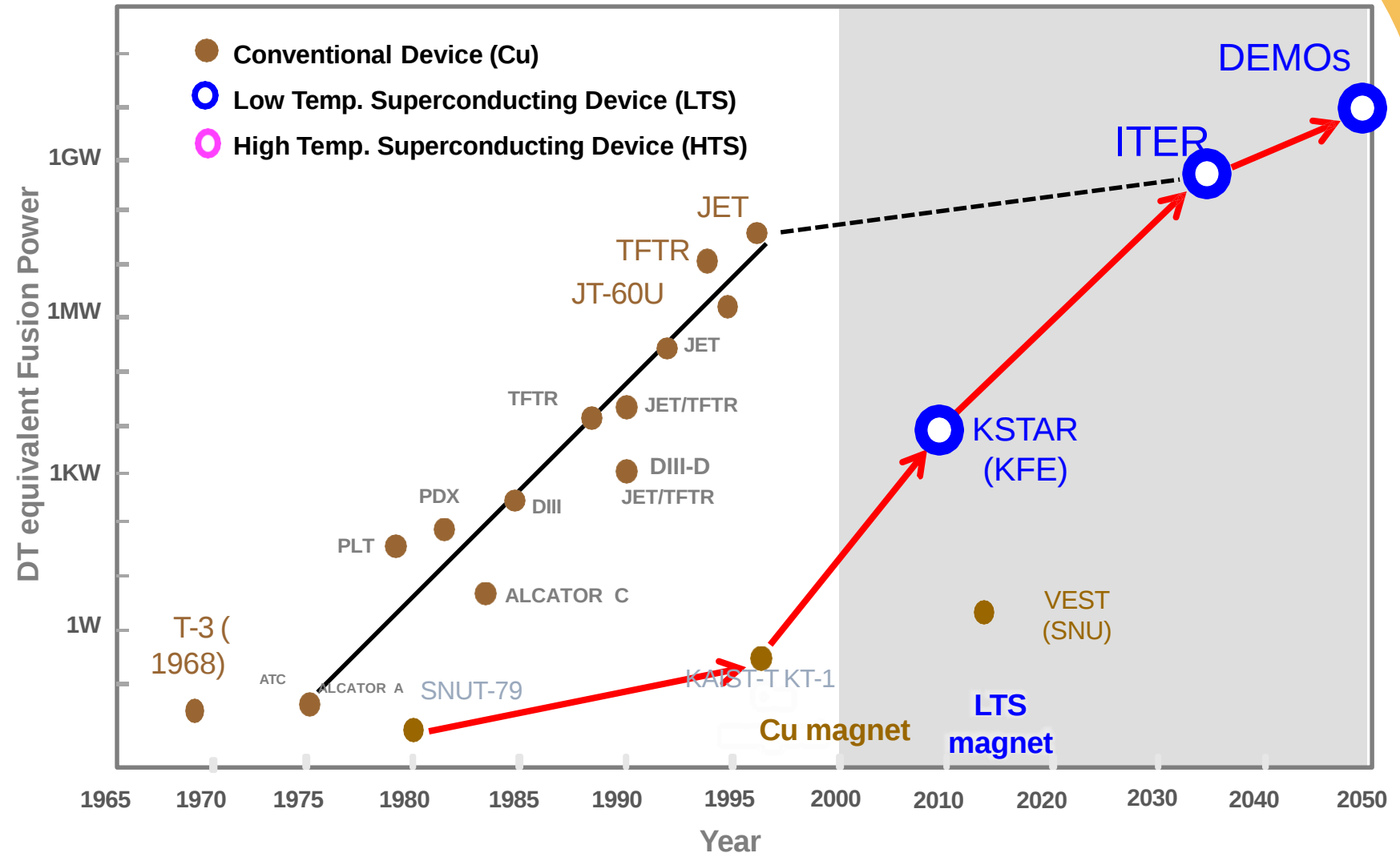
# I. Introduction of Korea National Fusion R&D

# Korean Fusion R&D has been making steady progresses since the start of KSTAR

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- Fusion R&D is progressing under “Fusion Energy Promotion Act”.
- Korean government establishes and updates the master plan for the Promotion of Fusion Energy Development for every 4 years.
- We are now in the middle of the period for the 4<sup>th</sup> master plan: 2022 ~ 2026.



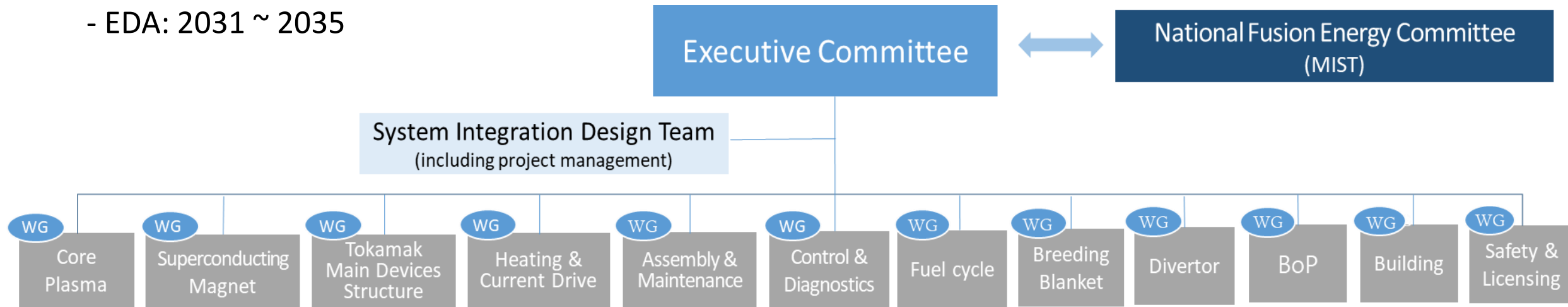
Global Fusion R&D and Korea's mid-entry strategy (1995)

# DEMO Design Task Force was established in 2023

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- Categorized into 12 working groups, the System Integration Design Team organizes and integrates them.
- TF members are from National Labs, Universities, and Private Companies.
- Design Roadmap
  - Pre-CDA: 2023 ~ 2026
  - CDA: 2026 ~ 2030
  - EDA: 2031 ~ 2035



*Structure of DEMO Design Task Force*

## **II. Strategy & Design of the K-DEMO**

## ■ The Basis for K-DEMO Design

### [Top Tier Requirements]

- ① Max. Electrical Power  $> 500MW_e$  ( $\sim 1,500MW_{th}$ )
- ② TBR (Tritium Breeding Ratio)  $\geq 1$
- ③ Verification of Intrinsic Safety for Fusion
- ④ Acquisition of data for Economic Evaluation

### [Major Design Requirements]

- ① Major radius (R)  $< 7m$
- ② Reactor operation rate  $> 60\%$
- ③ Reactor life  $> 40$  years
- ④ Seismic Safety up to 7.0 Richter scale (0.3g)

| Parameters                              | Designed     | Remarks                                       |
|-----------------------------------------|--------------|-----------------------------------------------|
| Major radius, $R_0$                     | $\sim 6.8$ m | 6.8 m as reference                            |
| Minor radius, a                         | $\sim 2.2$ m |                                               |
| Elongation, $\kappa$                    | $\sim 2.0$   | $\kappa_{95}$                                 |
| Triangularity, $\delta$                 | $\sim 0.6$   | $\delta_{95}$                                 |
| Plasma shape                            | SN           | Backup: DN                                    |
| Density ( $\langle n_e \rangle / n_G$ ) | $\sim 1.2$   | $n_G \sim 0.82 \times 10^{20} \text{ m}^{-3}$ |
| Temperature( $\langle T \rangle$ )      | $> 13$ keV   |                                               |
| Plasma current, $I_p$                   | $\sim 13$ MA |                                               |
| Toroidal field, $B_0$                   | $\sim 6.5T$  | High field TF (Nb3Sn)                         |
| $\beta_N$                               | $\sim 3.5$   | High beta operation                           |
| Fusion power                            | 1500 MW      |                                               |
| Blanket                                 | HCCP         |                                               |
| Heating/CD                              | $> 60$ MW    |                                               |

*Key parameters for a preliminary K-DEMO system design*

# Extensive System Analyses were performed for K-DEMO Scoping

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- Ranges of system parameters were scanned with engineering and physics constraints:

- External power injection:  $P_{inj} < 100$  MW
- Neutron wall loading:  $\Gamma_N < 2.0$  MW/m<sup>2</sup>
- Divertor FOM:  $P_{div}/R_0 < 25.0$  MW/m
- TF current density limit:  $J_{TF} < 2.5 \times 10^7$  A/m<sup>2</sup>
- TF coil case stress limit:  $s_{TFcase} < 560$  Mpa
- Shielding requirements for TF coil
  - dose to the insulators  $< 1 \times 10^9$  rad
  - displacement damage on Cu  $< 5 \times 10^{-5}$  dpa
  - fast neutron fluence  $< 10^{18}$  n/cm<sup>2</sup>
  - nuclear heating rate  $< 5 \times 10^{-5}$  W/cm<sup>3</sup>

- Additional filters were applied to narrow down feasible system designs:

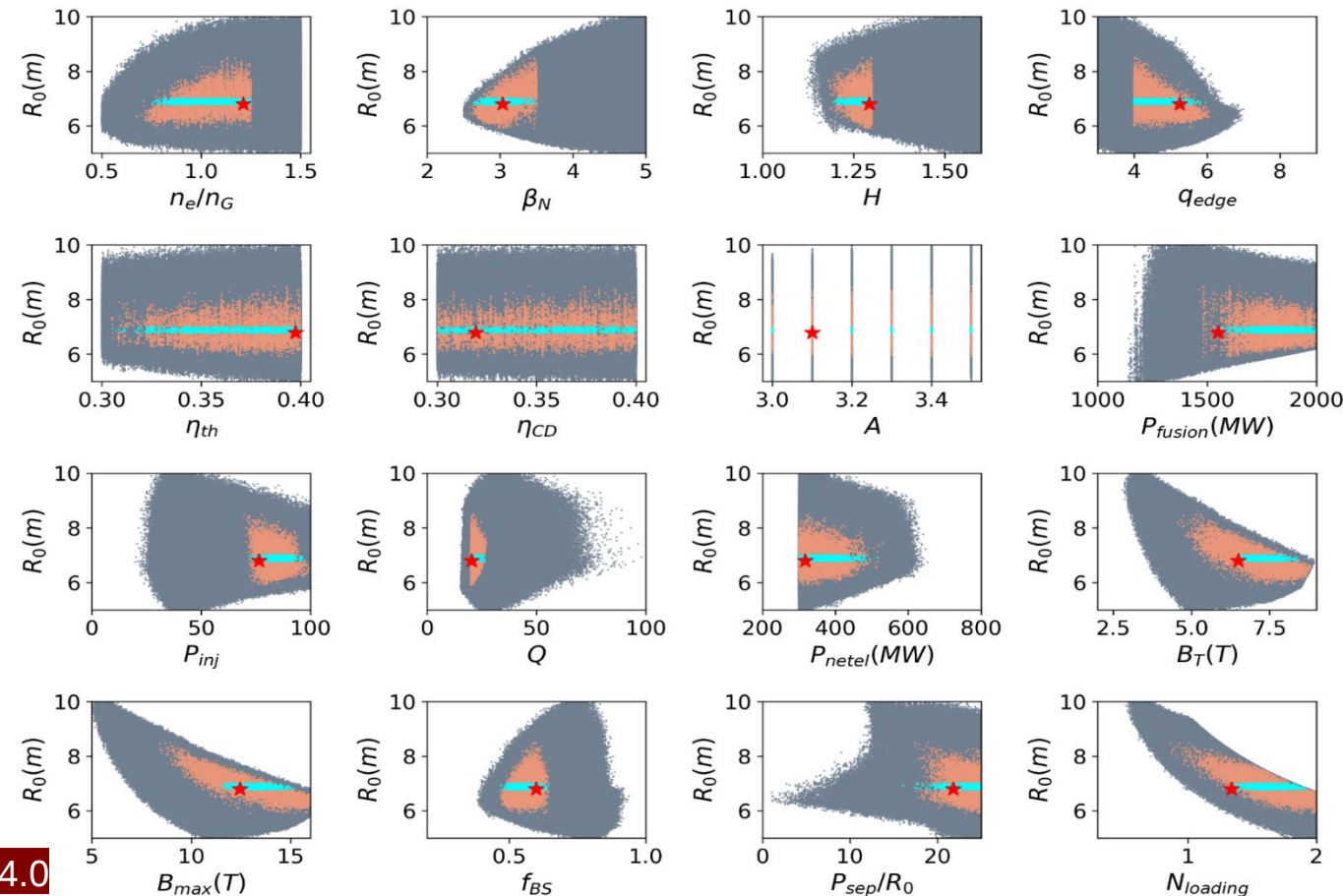
$R_0 < 10.0$  m,  $P_{netel} > 300$  MW, Direct cost  $< 7.5$  B\$

above filters +  $Q > 20.0$ ,  $n_e/n_G < 1.25$ ,  $\beta_N < 3.5$ ,  $H < 1.3$ , and  $q_{edge} > 4.0$

above filters +  $6.8$  m  $< R_0 < 7.0$  m,  $A = 3.1$ ,  $6.5$  T  $\leq B_T$

Design point (★):  $R_0 \sim 6.8$  m,  $P_{fusion} < 1,600$  MW

Scanning of K-DEMO system parameters with the IPB98(y,2) scaling

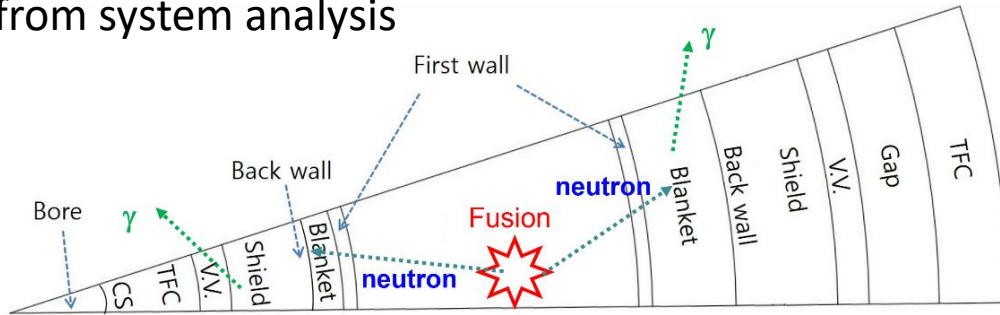


# Neutronics calculations were coupled to the system analyses

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- Assume simplified cylindrical geometry based on 1D build from system analysis



- ANISN radiation transport calculation including anisotropic scatterings of neutrons and gamma rays with detailed material compositions (FENDL-3.3 C.X. library)
- Tritium Breeding Ratio based on Li C.X. ENDF/B-VII.1 library

| Component           | Material (Volume%)                           |
|---------------------|----------------------------------------------|
| First wall          | FMS (50), He (33), W (17)                    |
| Breeding blanket    | Coolant (10), Breeder (80), FMS (10)         |
| Back wall/Manifold  | FMS (40), He (60)                            |
| Shield              | FMS (5), WC (85), H <sub>2</sub> O (10)      |
| Vacuum vessel       | Borated steel (60), H <sub>2</sub> O (40)    |
| Toroidal field coil | Nb <sub>3</sub> Sn, Cu, Epoxy, SUS316, L. He |

*Materials for major components*

*K-DEMO Reference Design with  $R_0 = 6.8\text{m}$  &  $B_T = 6.5\text{T}$*

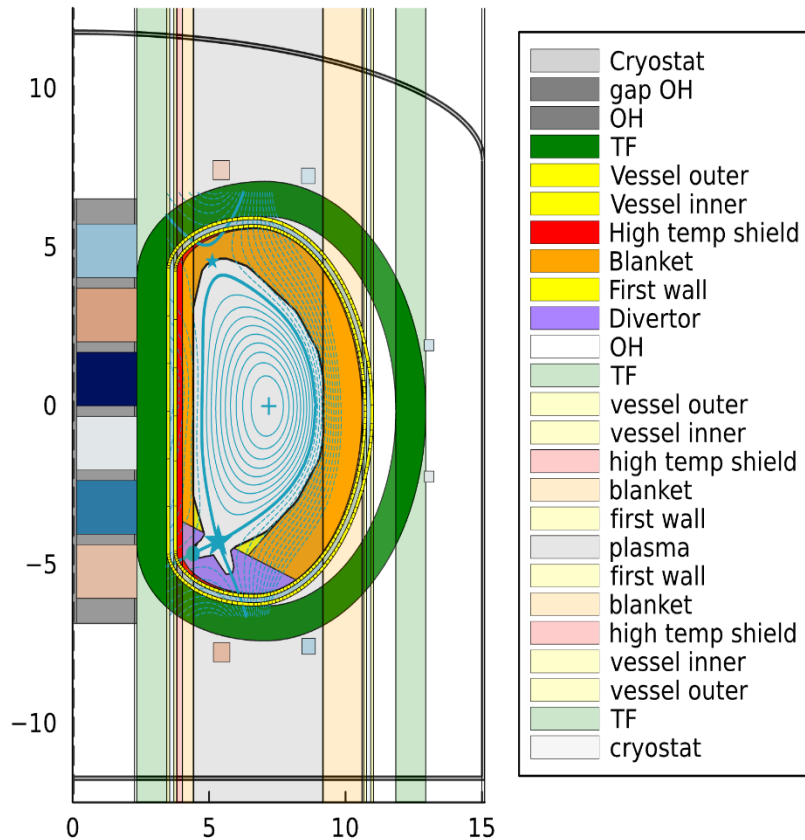
| Key System Parameters                                     |       |
|-----------------------------------------------------------|-------|
| Major radius, $R_0$ (m)                                   | 6.80  |
| Aspect ratio, $A$                                         | 3.1   |
| Magnetic field at plasma center, $B_T$ (T)                | 6.55  |
| Peak magnetic field, $B_{peak}$ (T)                       | 12.5  |
| Stored energy of plasma (MJ)                              | 750.3 |
| Auxiliary heating, $P_{inj}$ (MW)                         | 73.7  |
| Fusion gain, $Q$                                          | 20.4  |
| Divertor power handling index, $P_{sep}/R_0$ (MW/m)       | 20.7  |
| Neutron wall loading $\Gamma_N$ (MW/m <sup>2</sup> )      | 1.3   |
| Net electric power, $P_{netel}$ (MW)                      | 329   |
| Thermodynamic efficiency, $\eta_{th}$                     | 0.39  |
| Wall-plug efficiency of current drive system, $\eta_{CD}$ | 0.38  |
| Thermal fusion power, $P_{fusion}$                        | 1502  |
| Inboard blanket thickness (m)                             | 0.38  |
| Inboard shield thickness (m)                              | 0.40  |
| TF coil thickness (m)                                     | 1.14  |
| CS thickness (m)                                          | 0.64  |
| CS bore radius (m)                                        | 1.67  |
| Outboard (shield + blanket) thickness (m)                 | 1.2   |
| Direct cost (B\$)                                         | 6.76  |

# 2D Build and Reference Scenarios were generated

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- The FUSE Code (O. Meneghini, GA) was used to generate 2D build and self-consistent plasma equilibria → Consistent with the 1D build from the system analyses



K-DEMO 2D Build generated by FUSE

## Key Plasma Parameters for K-DEMO

|                                                                             |      |
|-----------------------------------------------------------------------------|------|
| Elongation, $\kappa$                                                        | 2.0  |
| Plasma current, $I_p$ (MA)                                                  | 13.6 |
| Electron density, $\langle n_e \rangle$ ( $\times 10^{20} \text{ m}^{-3}$ ) | 0.82 |
| Electron Temperature, $\langle T_e \rangle$ (keV)                           | 13.5 |
| Bootstrap current fraction, $f_{BS}$                                        | 0.57 |
| Edge safety factor, $q_{edge}$                                              | 5.05 |
| Greenwald density fraction, $n_e/n_G$                                       | 1.07 |
| Normalized beta, $\beta_N$                                                  | 2.97 |
| Confinement enhancement factor, $H$                                         | 1.27 |

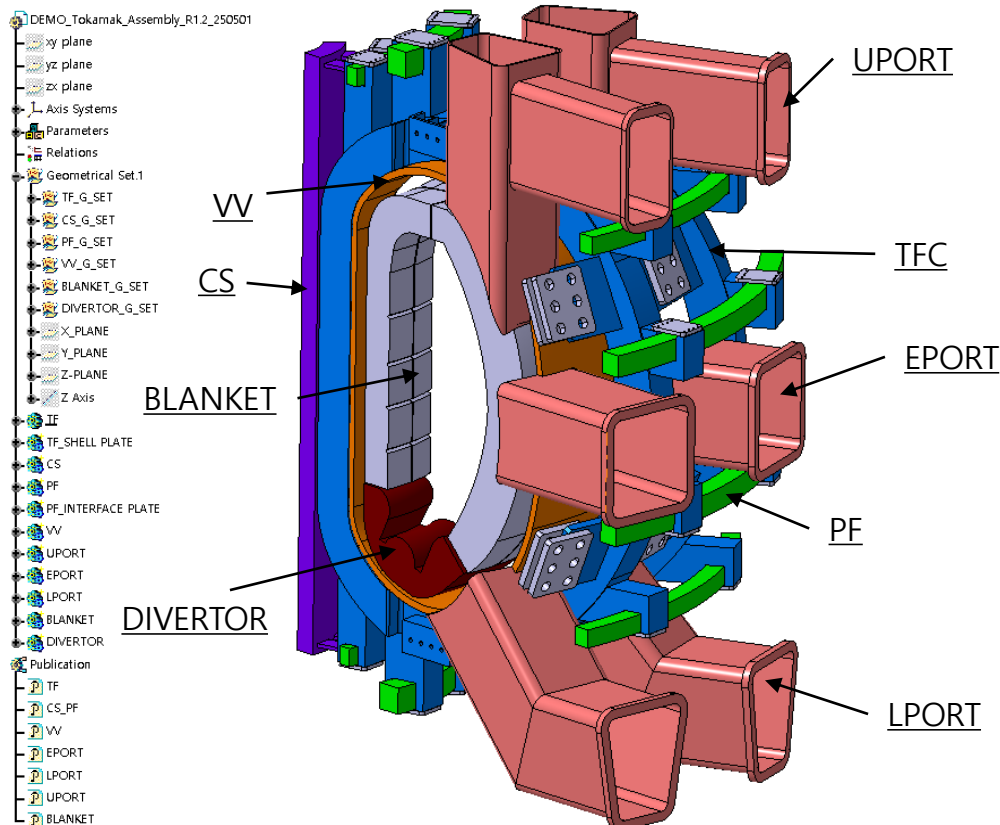
- K-DEMO requires plasma scenarios with significantly improved performance compared to standard H-mode → *hybrid or high-li (e.g. recent dedicated experiments on KSTAR)*

# Preliminary 3D Build was generated with Site Layout

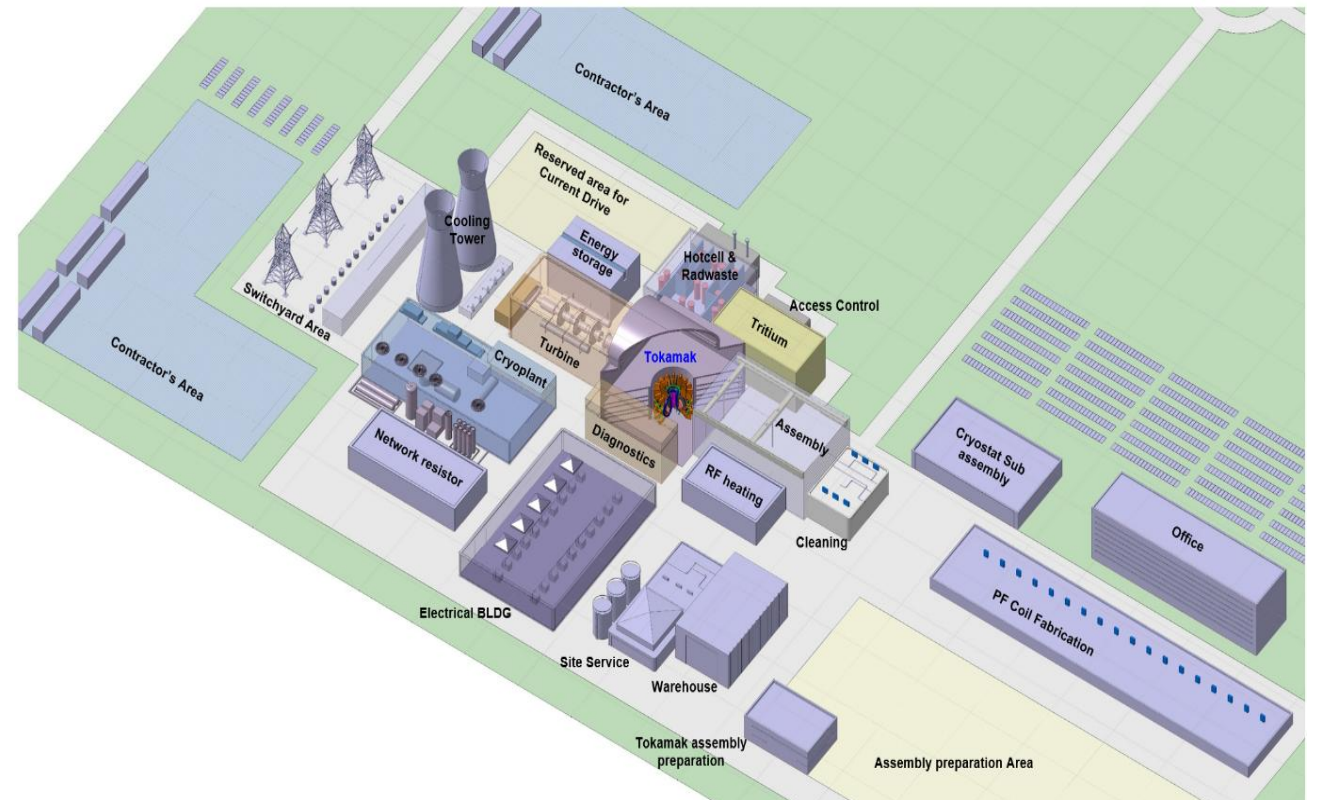
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- Tools and integration platform are prepared with Plant Breakdown Structure (PBS).
- 3D build was constructed with 2D build and consideration for remote maintenance.
- Site layout was made including buildings for main reactor and all necessary components.



K-DEMO 3D Build



K-DEMO Site Layout

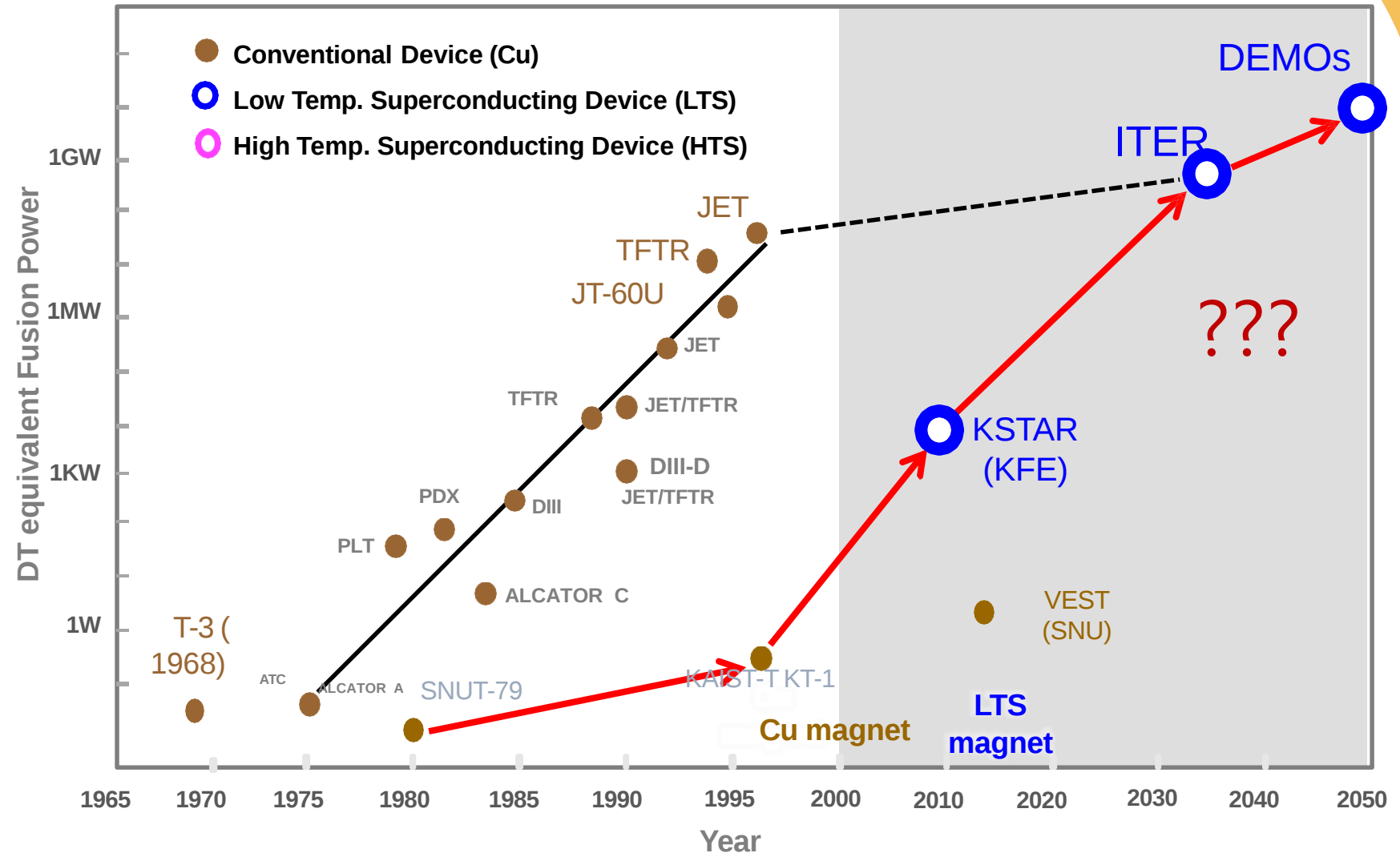
## **III. Strategy for Innovative Compact Pilot Device (CPD)**

# Growing Interests on Innovative Fusion Tech in Korean Fusion Community

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- Recently, a community-wise consensus was made to embrace these innovative technologies (e.g. HTS) to expand Korean Fusion R&D.

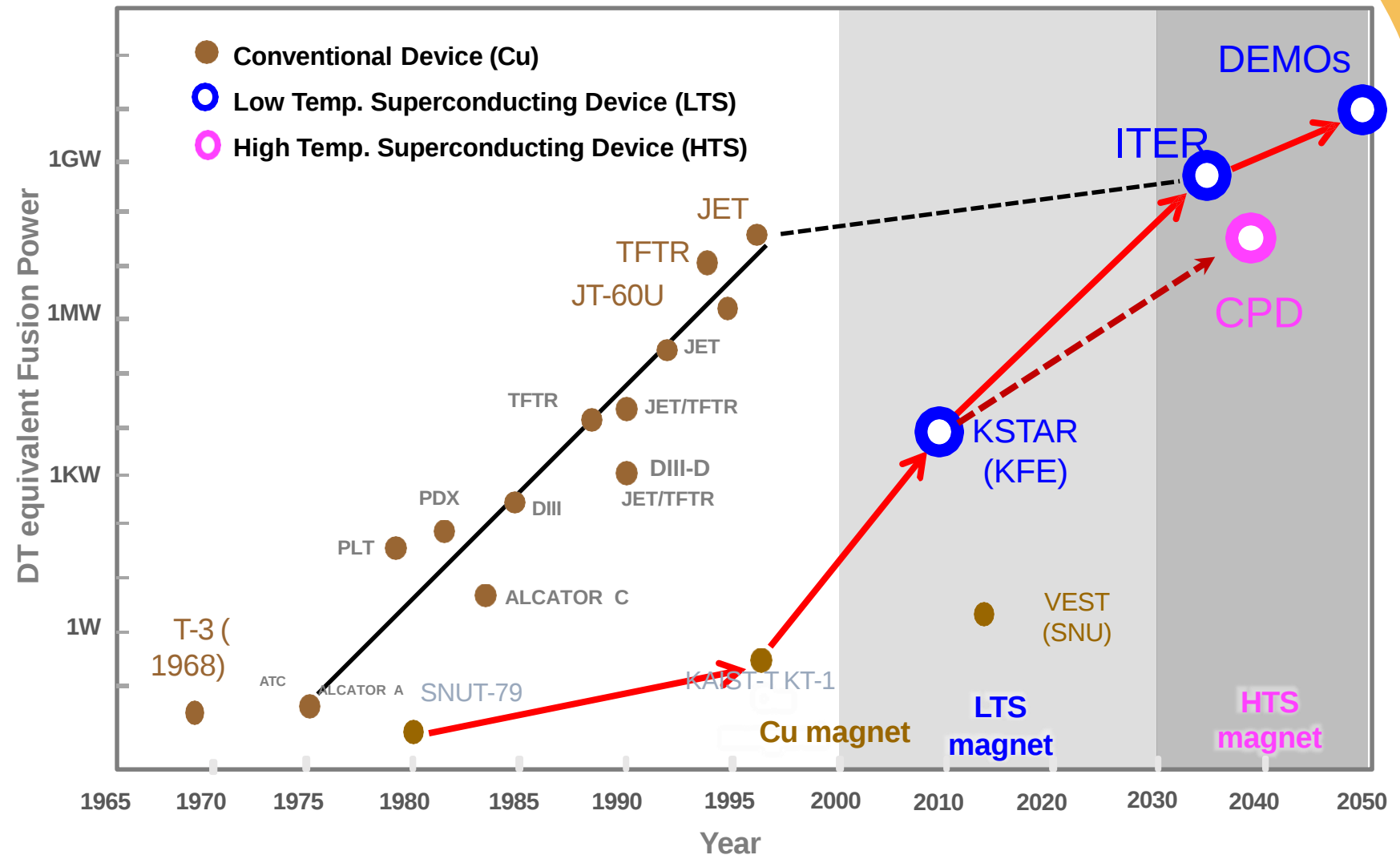


# Expansion of Korean National Fusion R&D Path *(not approved yet)*

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- Recently, a community-wise consensus was made to embrace these innovative technologies (e.g. HTS) to expand Korean Fusion R&D.
- It is proposed to expand national fusion R&D projects including **compact pilot device (CPD)** employing HTS technology along with the conventional **KSTAR→ITER→K-DEMO** path.

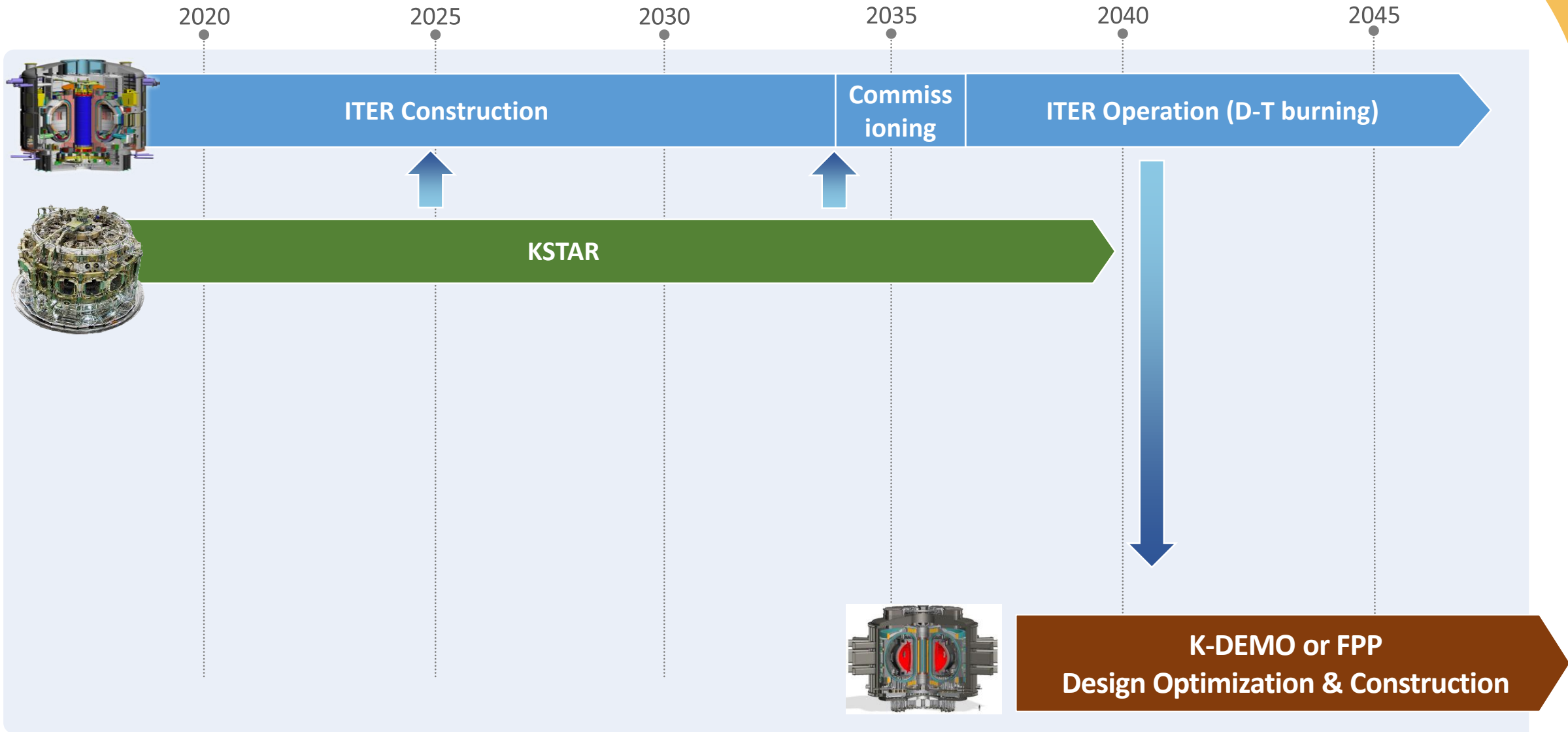


Newly Proposed Korean Fusion R&D including CPD based on HTS Technology

# Proposed Timeline of Key Fusion Programs

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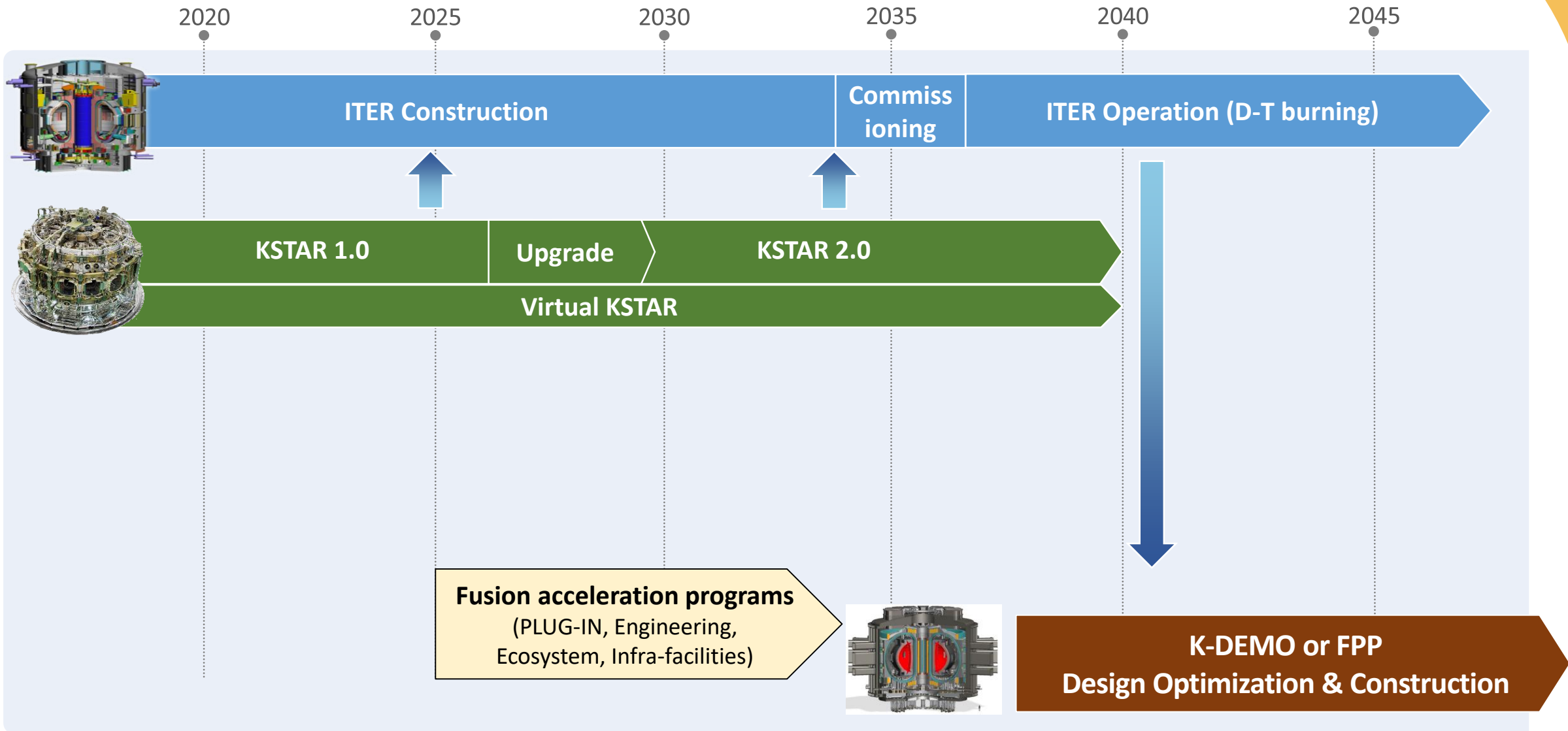
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# Proposed Timeline of Key Fusion Programs

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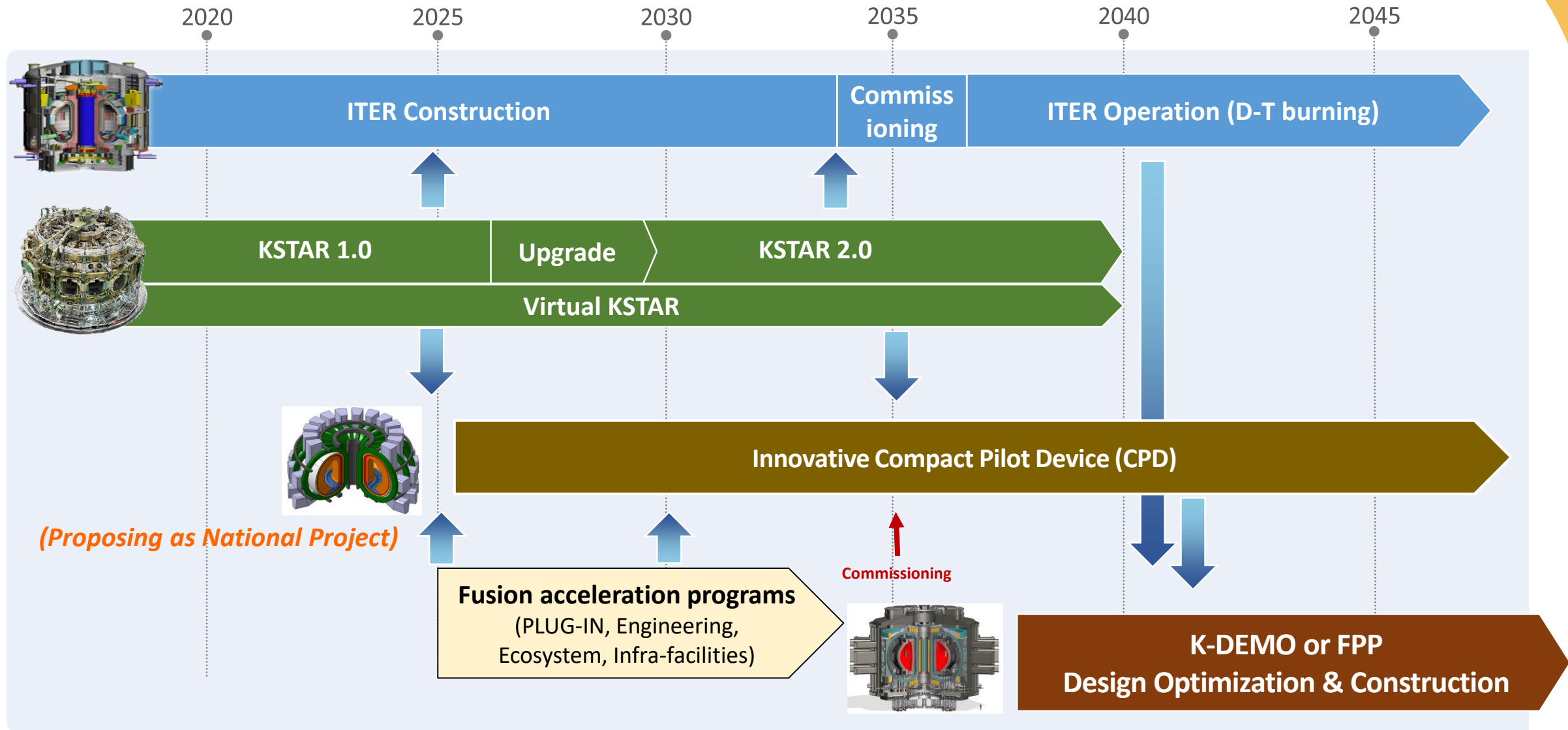
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# Proposed Timeline of Key Fusion Programs

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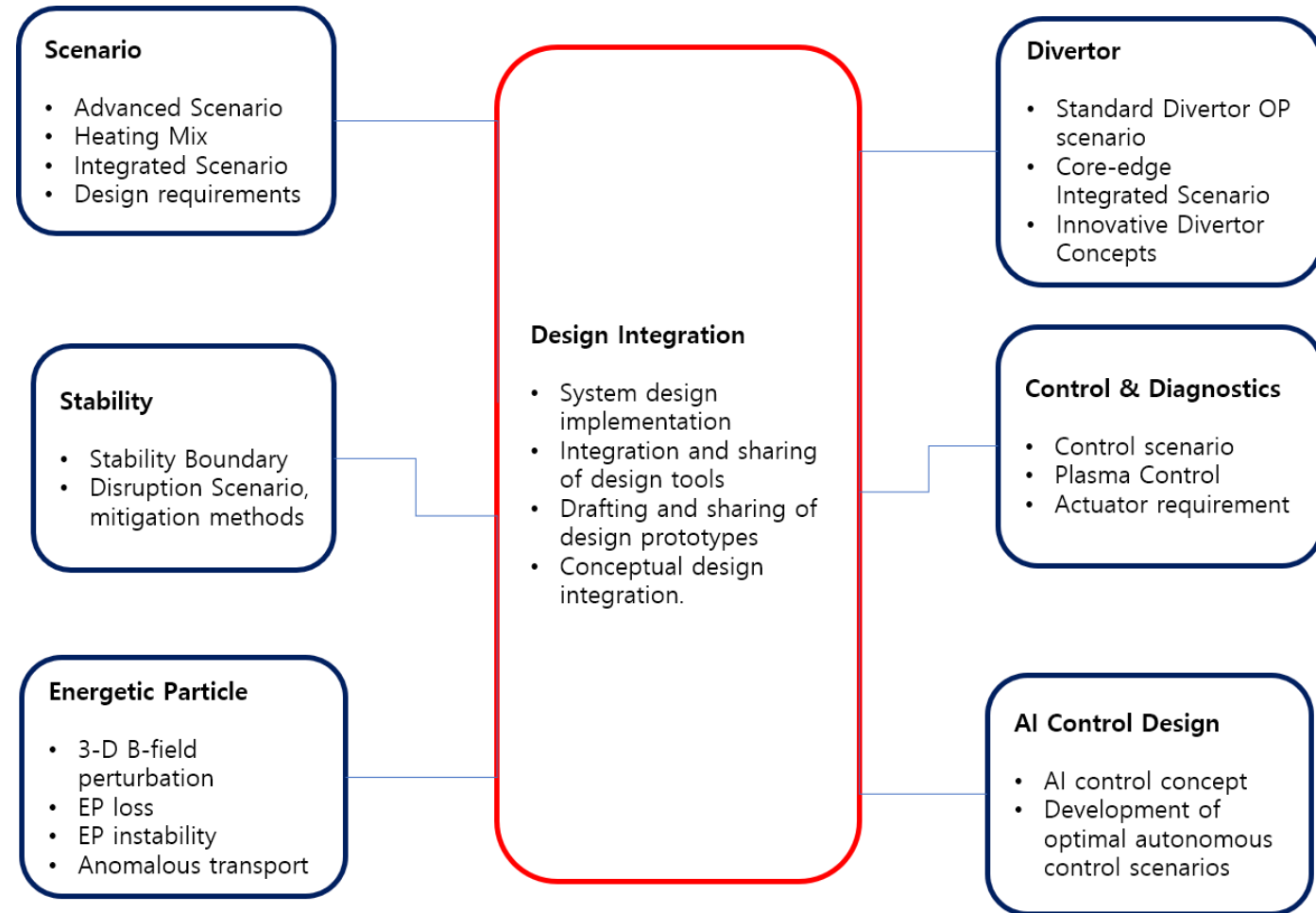
## **IV. Preliminary Design Study for the CPD**

# K-DEMO design TF is moving toward the design of CPD

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- A new sub-design TF is formed to conduct:
  - Development of advanced design tools for rapid proto-typing and design changes
  - System scoping for optimum radial build and physics parameters



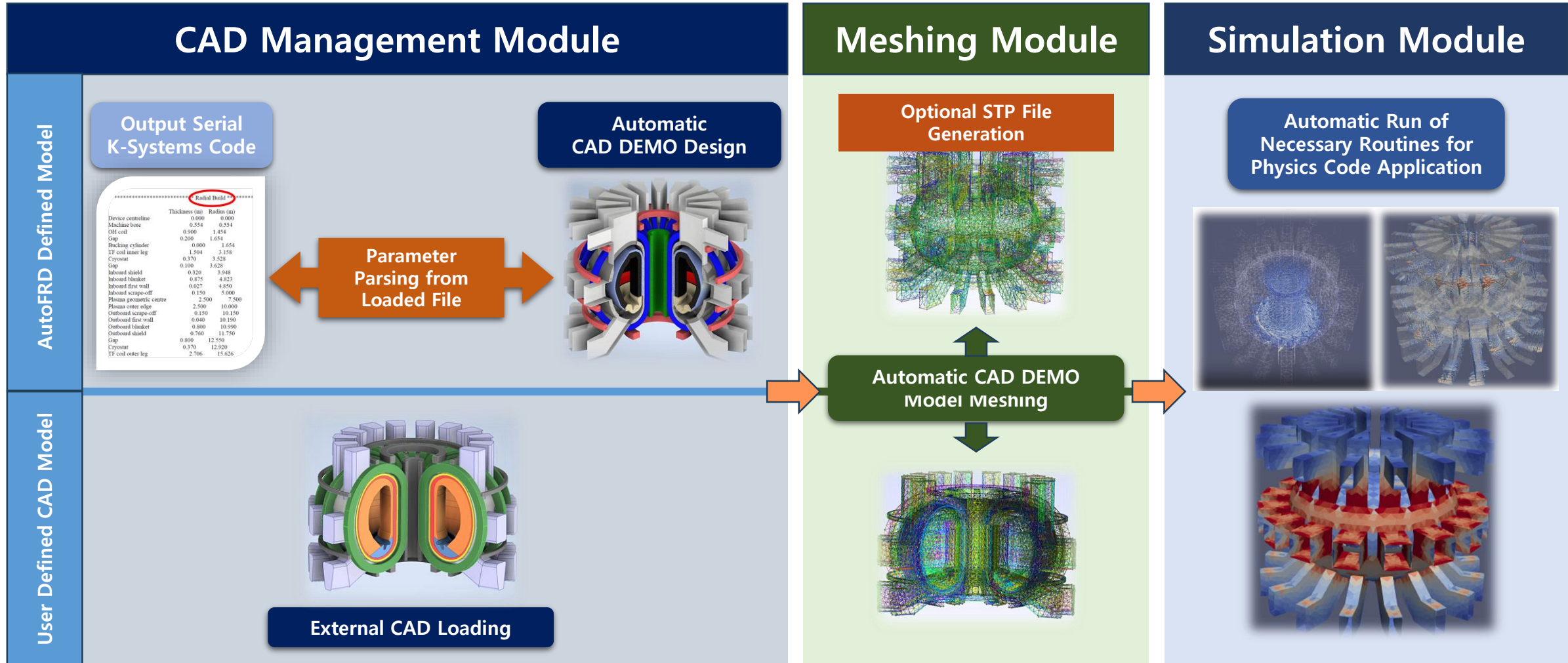
*Sub-design TF to explore the CPD concept*

# Advanced Design Tools are being developed for CPD

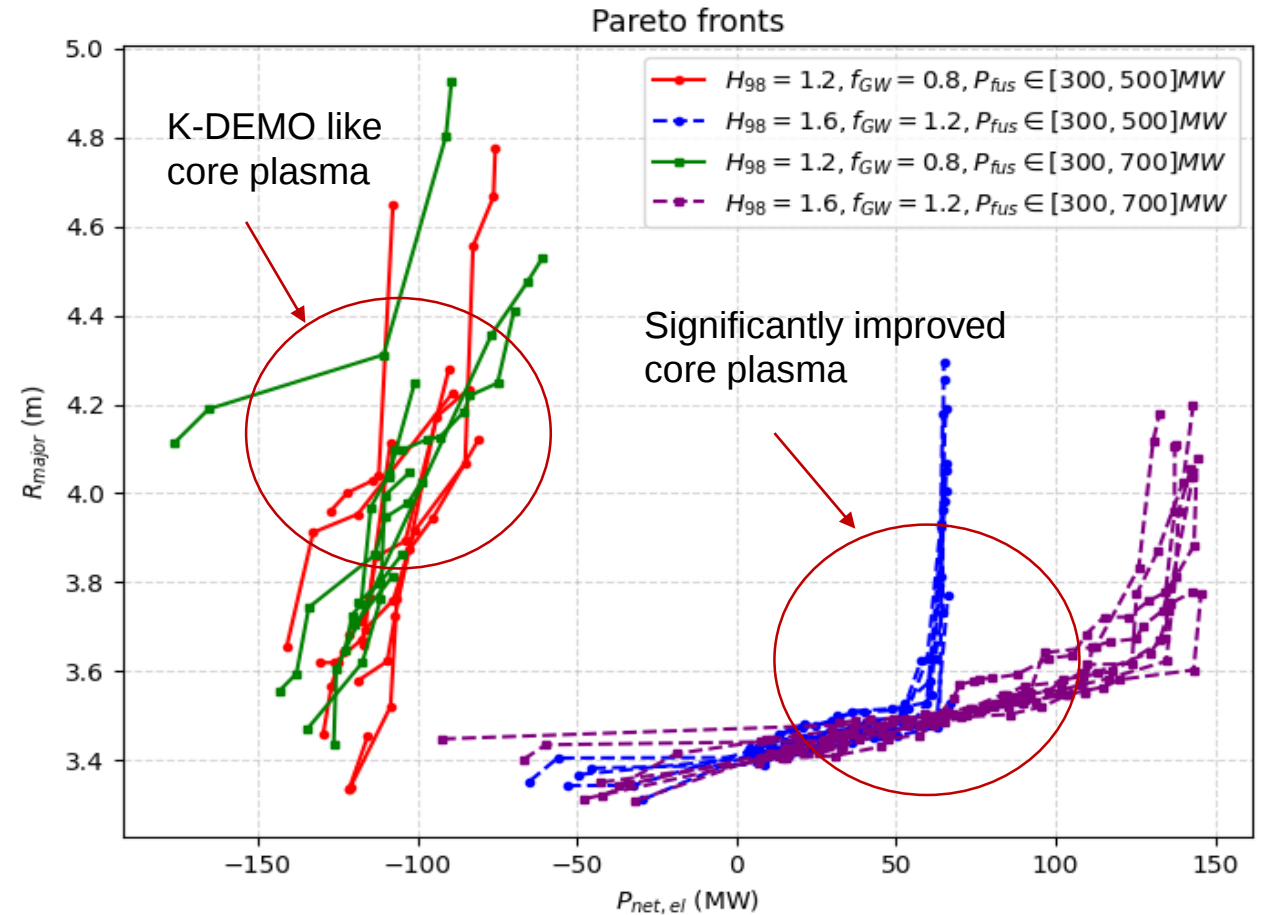
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- Automated 3D CAD generation and mesh generation for physics simulation and engineering analysis
- Aiming Bayesian optimization based design accelerations for as many design elements as possible



- Multi-objective Bayesian optimizations of CPD were performed to set major parameters with different core plasma performances and maximum allowed fusion powers (500, 700 MW)
- Compared to K-DEMO, significantly improved plasma performance is required to achieve net electricity with  $R_0 < 4m$ 
  - $H_{98} = 1.2 \rightarrow 1.6$ ,
  - $n_e/n_{GW} = 1.0 \rightarrow 1.2$
  - $f_{BS} = 0.6 \rightarrow 0.8$  etc.
- 60 ~ 150 MWe net electricity are expected for machines with major radius in [3.5m, 4m]



Optimization of Machine Size for CPD

# A preliminary CPD Reference ( $R_0 = 3.5m$ & $B_T = 8.2T$ ) is selected

- Many similarities with previously proposed small reactor concepts employing HTS, allowing high  $B_T$

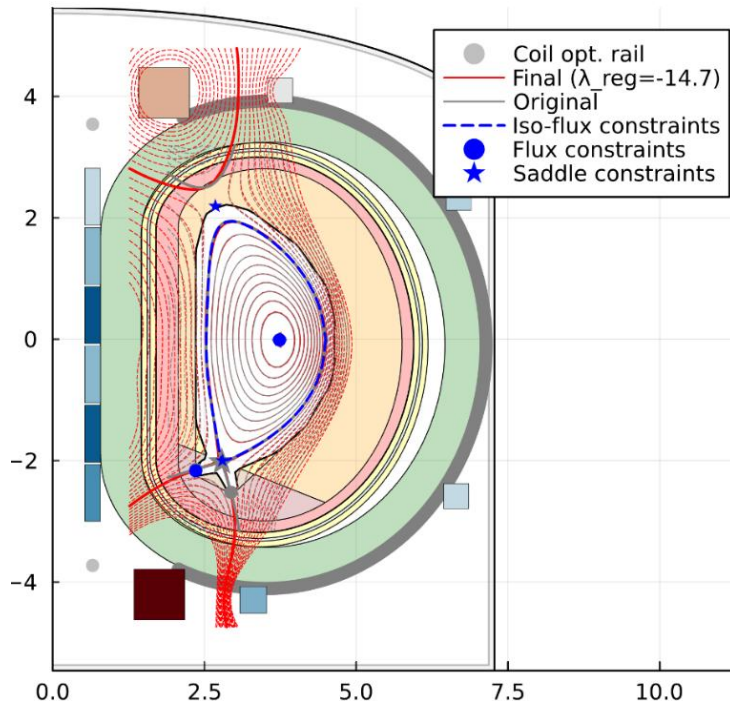
|                                             | ARC (FED 2015)                                                        | GA CTPP (FST 2021)                                     | COMPACT (FED 2023)                                                                 | CPD with $R_0 = 3.5m$                                                                                          |
|---------------------------------------------|-----------------------------------------------------------------------|--------------------------------------------------------|------------------------------------------------------------------------------------|----------------------------------------------------------------------------------------------------------------|
| CS role, $\tau_{pulse}$ (h)                 | $I_p$ start-up, S-S                                                   | $I_p$ ramp-up & burn, 1~2                              | 10% flux for $I_p$ ramp-up, S-S                                                    | 50% flux for $I_p$ ramp-up, S-S                                                                                |
| $B_{maxTF}$                                 | 23T, REBCO                                                            | 23T, REBCO                                             | 23T, REBCO                                                                         | 23T, REBCO                                                                                                     |
| $J_{TF}/J_{CS}$ (MA/m <sup>2</sup> )        | 44.0                                                                  | 39.1/64.6                                              | 40.0/40.0                                                                          | 40.0/60.0                                                                                                      |
| $\Delta_{TF}/\Delta_{CS}/\Delta_{plug}$ (m) | 0.64/0.25/0.45                                                        | 0.3/0.3/0.98                                           | 0.88/0.13/>0.2                                                                     | 0.69/0.28/0.56                                                                                                 |
| $\Delta_{SHI}/\Delta_{BLi}$ (m)             | 0.55/0.31                                                             | 0.45/0.3                                               | 0.5/0.18                                                                           | 0.37/0.30                                                                                                      |
| Shield                                      | TiH <sub>2</sub> ,<br>n fluence: $3 \times 10^{18}$ n/cm <sup>2</sup> | WC?<br>n fluence: $1 \times 10^{19}$ n/cm <sup>2</sup> | WC, n fluence: $1 \times 10^{18}$ n/cm <sup>2</sup><br>40 yrs and 75% availability | W <sub>2</sub> B <sub>5</sub> , n fluence: $3 \times 10^{18}$ n/cm <sup>2</sup><br>30 yrs and 50% availability |
| Blanket                                     | FLiBe                                                                 | HCLL?                                                  | HCLL w. 90 % Li-6 (ref.)                                                           | HCLL/HCSB w. 90 % Li-6 (ref.)                                                                                  |
| Thermal efficiency, $\eta_{th}$             | 0.4                                                                   | 0.4                                                    | 0.4                                                                                | 0.4                                                                                                            |
| Major Device Parameters                     | $R_0$ (m)/A/ $\kappa$                                                 | 3.3/3/1.84                                             | 3.63/3.2/2.0                                                                       | 3.50/3.5/2.0                                                                                                   |
|                                             | $B_T/B_{TF}$ (T)                                                      | 9.2/23                                                 | 8.3/19.4                                                                           | 8.2/18.3                                                                                                       |
|                                             | $P_{fusion}/P_{CD}/P_{netel}$ (MW)                                    | 525/38.6/190                                           | 664/31.6/102                                                                       | 490/30.0/80                                                                                                    |
|                                             | Direct capital cost (B\$)                                             | ~5.6 B\$ (w.o. BOP)<br>from V, m & unit prices         | ~4.2 B\$ (w. BOP)<br>from Sheffield [2016, 1986]                                   | ~4.2 B\$ (w. BOP)<br>from Sheffield [2016, 1986]                                                               |

# CPD 2D/3D Builds and scenarios are being developed

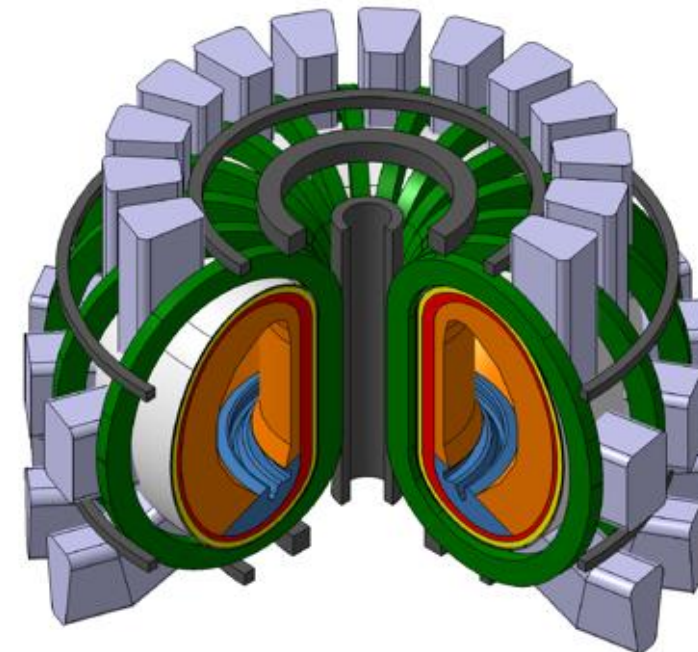
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- FUSE was utilized to generate 2D build, which is consistent with the 1D build from the system analysis.
- 3D build was generated by the AutoFRD with 16 TF coils – 0.5% ripple without ferritic inserts.
- Scenario developments are on-going:
  - Improved inductive scenarios based on conventional ones (e.g. hybrid) for pulse operation
  - Advanced Tokamak-like scenario (e.g. high- $\beta_p$ ) for steady state operation, especially with innovative actuators



2D Build by FUSE: REBCO for TF&CD and  $Nb_3Sn$  for PF



3D Build by AutoFRD with Parametric Model Variation and Optimization

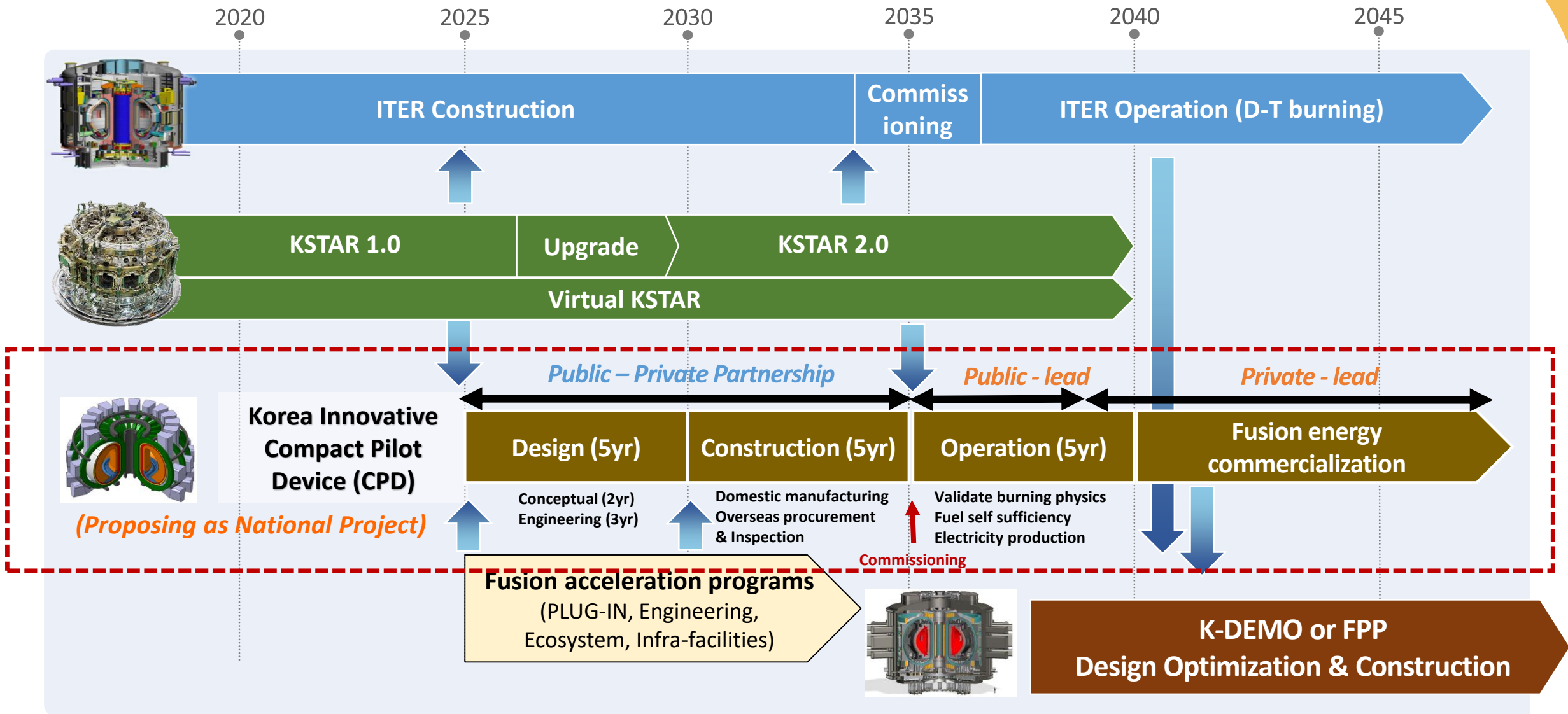
# **V. Summary and Future Perspective**

- The Korea National Fusion R&D is making steady progress:
  - DEMO design TF launched in 2023 aiming Pre-CDA until 2026
  - Government approved the top design requirements of K-DEMO
  - Preliminary 3D build and site layout were produced
  
- Compact Pilot Device (CPD) is being proposed with growing consensus in Korean Fusion Community to embrace innovative technologies for the acceleration of fusion R&D:
  - Developments of AI-based advanced design tools
  - System scoping study to set reference design points and 2D, 3D builds with rapid proto-typing
  - On-going discussion for alignment with other national fusion R&D projects to develop key enabling technologies including innovative actuators

# CDP Design & Construction Schedule is being proposed

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PPP : Public-Private Partnership, CPD : Compact Pilot Device, K-DEMO : Korean Demonstration Fusion Reactor, FPP (Fusion Power Plant)