Theory-based integrated modelling of tungsten transport: validation in present-day devices and predictions for ITER





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Abstract

- Impurities play crucial roles in the core-edge integration of fusion devices:
 - Core high-Z impurity accumulation must be avoided
 - Impurity seeding will be essential for a tolerable heat exhaust
- We have developed an entirely theory-based integrated modelling workflow [1] that self-consistently describes impurity transport and radiation coupled to the plasma
- It has been successfully validated against AUG, JET and C-Mod experiments [1-4]
- This provides confidence on its extrapolation for predictions of ITER plasmas [2,3]
- [1] Fajardo 2024 *NF* **64** 046021 [2] Fajardo 2024 *NF* **64** 104001
- [3] Fajardo 2025 *PPCF* **67** 015020

[4] Fajardo subm. to NF

- [5] Staebler 2021 *NF* **61** 116007
- [6] Fajardo 2023 *PPCF* **65** 035021
- [7] Fable 2022 *NF* **62** 024001 [8] Giroud this conference
- [9] Perks subm. to NF
 - [10] Muraca 2025 *EU-US TTF*

Conclusions

A validated modelling workflow can be applied to

auxiliary heating and plasma performance

In a reactor core, turbulent transport will dominate

over neoclassical transport for high-Z impurities,

This leads to flat W densities, regardless of ECRH

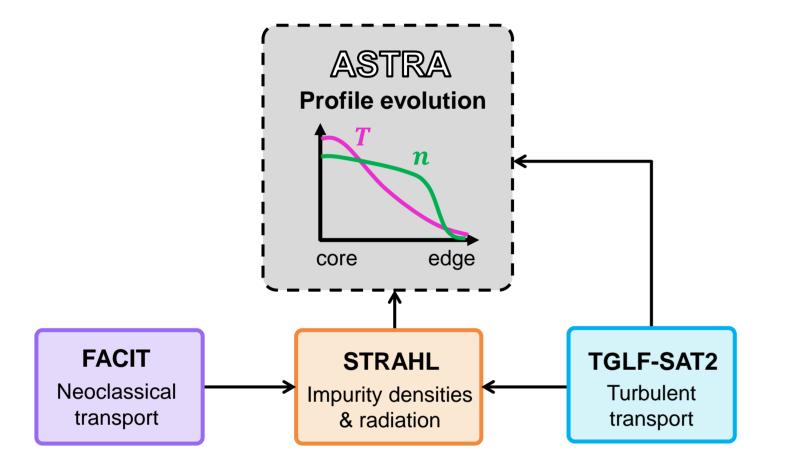
power: W content limited by H-mode sustainment

predict the interplay between impurities,

in contrast to present-day devices

Modelling workflow

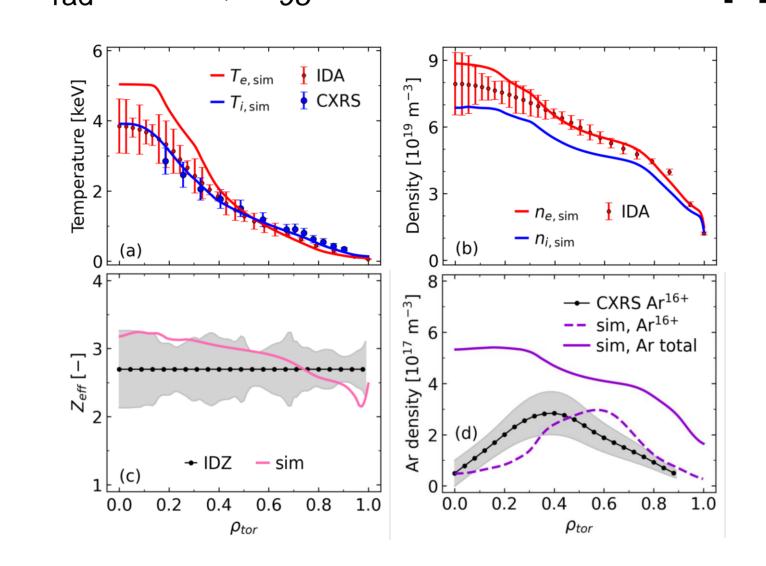
- Couples the impurity code STRAHL to the ASTRA transport modelling suite
- Uses state-of-the-art theory-based transport models TGLF-SAT2 [5] and FACIT [6]
- FACIT was extended to include the effects of **rotation** on high-Z neoclassical transport



Simulations up to the separatrix for L-modes or H-modes with weak pedestals, or up to the pedestal top in H-mode

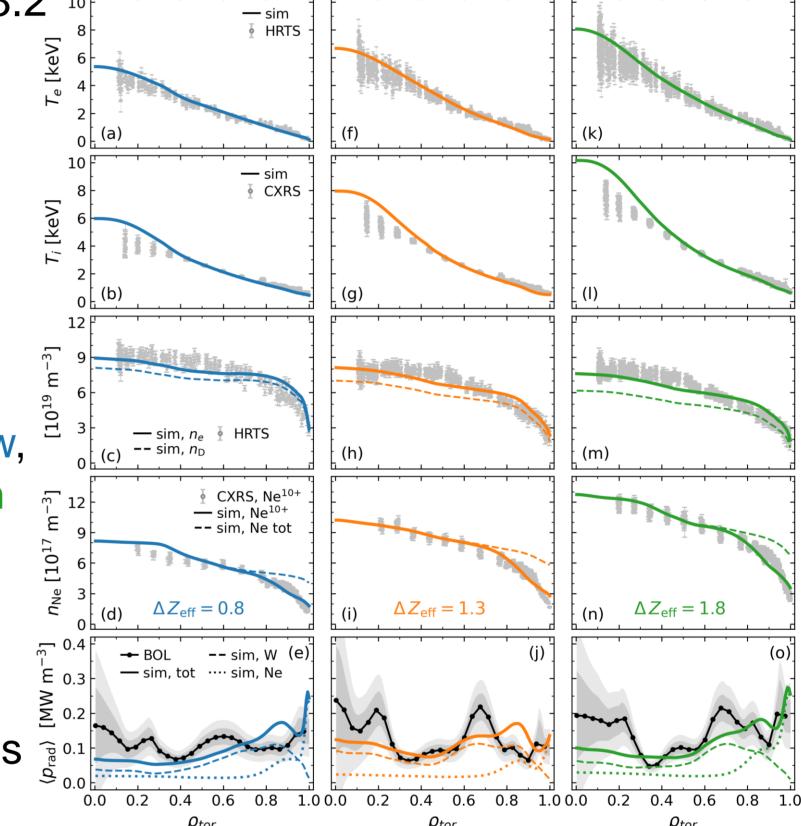
Full-radius modelling of high-power seeded L-modes in AUG and JET

- AUG #37041 with $P_{aux} \sim 8$ MW but $P_{sep} < P_{LH}$: feedback with **Ar** seeding
- $f_{\text{rad}} \approx 70\%$, $H_{98} \approx 0.95$ without ELMs [7]



- Simulation of main plasma with three impurities [1]: Ar, W and B
- High confinement is reproduced
- Simple model for X-point radiation was included

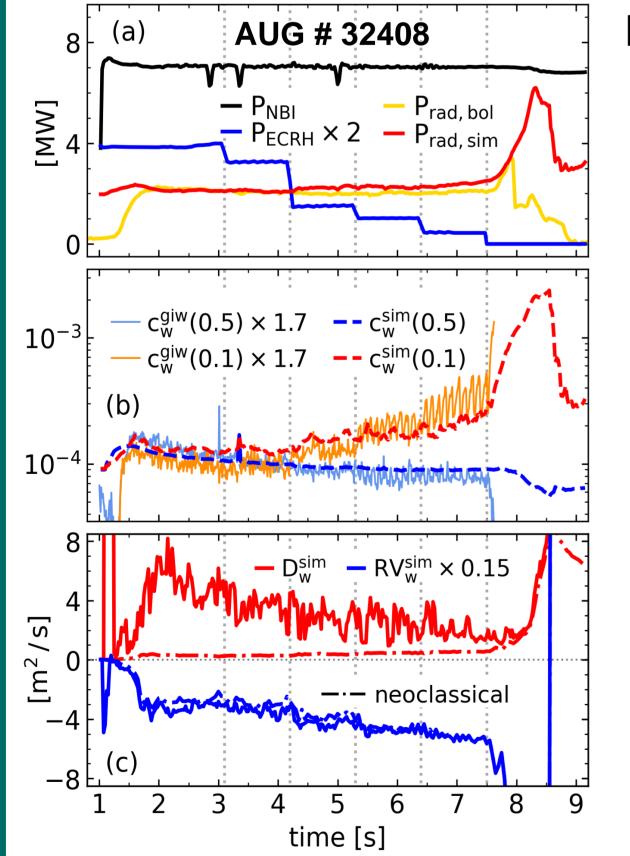
- High current $I_D = 3.2$ MA at $B_t = 3.45 \text{ T}$ $(q_{95} \sim 3.3)$ in deuterium [8]
- 28 MW of NBI, 4 MW of ICRH
- Neon seeding: low, intermediate, high
- Excellent agreement with experiment across all transport channels



 Including neon is necessary to reproduce the main profiles and global confinement in the simulations

Control of W accumulation with ECRH in AUG

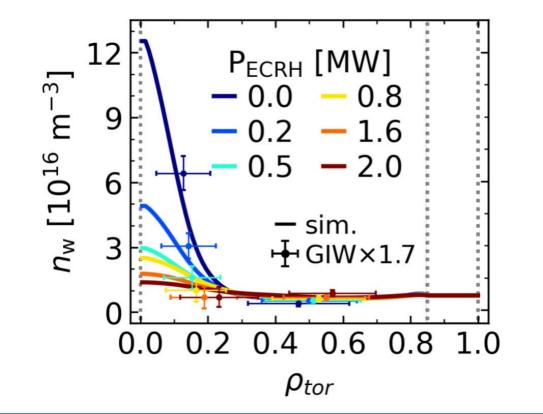
 Quantitative predictions of tungsten accumulation control with central wave heating in NBI-heated H-modes [2]



ECRH modifies the competition between neoclassical and turbulent W transport

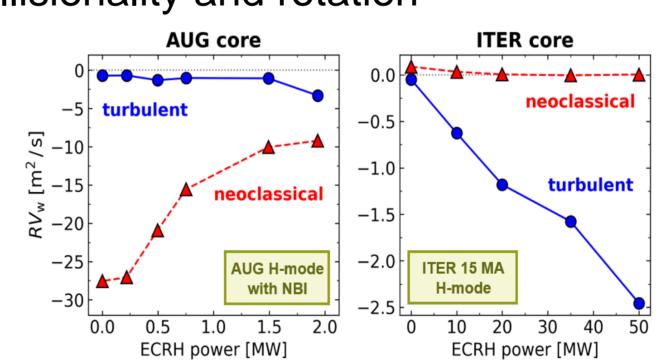
increases turbulent diffusion & reduces neoclassical pinch

flattening of central W density reproduced by the modelling:

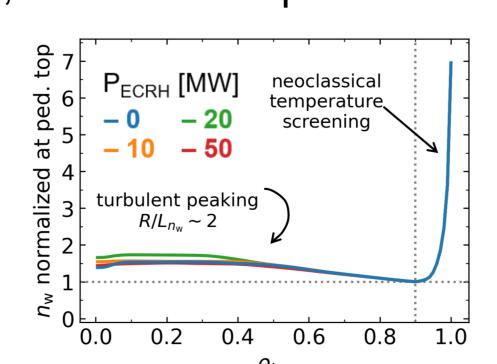


Predictions of tungsten transport in ITER

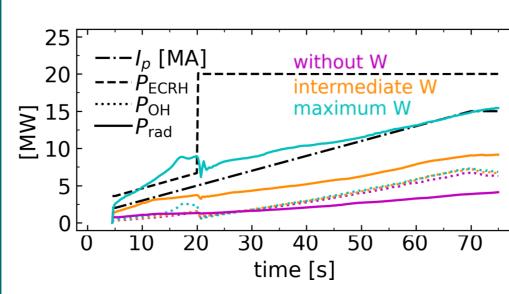
 In the core of ITER, high-Z neoclassical transport is much lower than turbulent transport due to the much lower collisionality and rotation



- Turbulent transport does not lead to central accumulation
- W densities are flat regardless of ECRH power, in contrast to present devices [2]

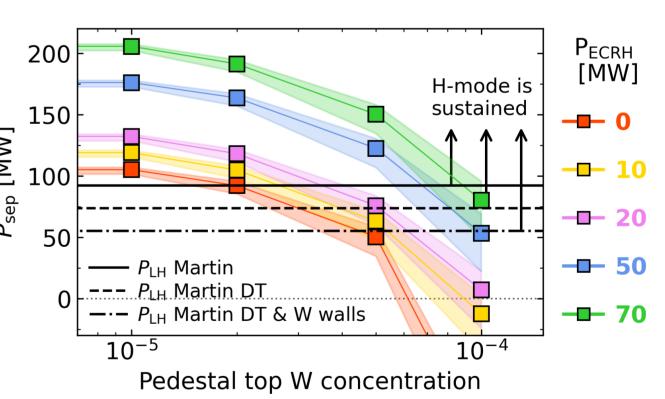


• Max. allowed c_w for survival in the I_p ramp

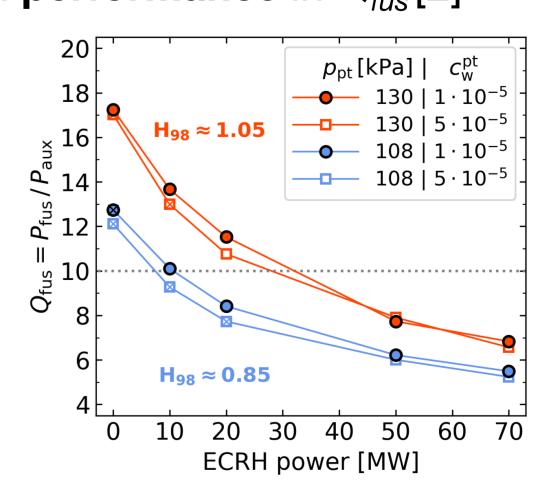


up, and for Hmode access in electron heated! L-modes, have also been identified [3].

 In the absence of core accumulation, the maximum tolerable W concentration is limited by the sustainment of H-mode operation [2]



Global radiative losses due to W must be sufficiently compensated with auxiliary heating, at a penalty on **performance** in $Q_{fus}[2]$



- $c_{w} \sim 3 5 \times 10^{-5}$ are marginal
- A better confinement also provides a higher margin for H-mode sustainment due to increased alpha heating [3]

Core high-Z transport in future fusion reactors

A transition in high-Z impurity transport regime takes place, from dominant neoclassical to dominant turbulent convection, as reactor core conditions are approached

