



Tungsten limiter start-up experiments in different boronization states in support of ITER

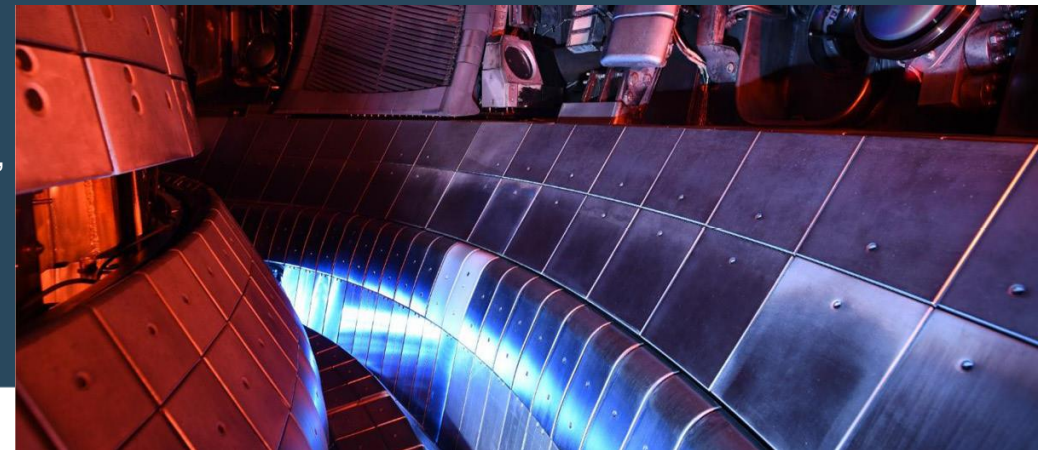


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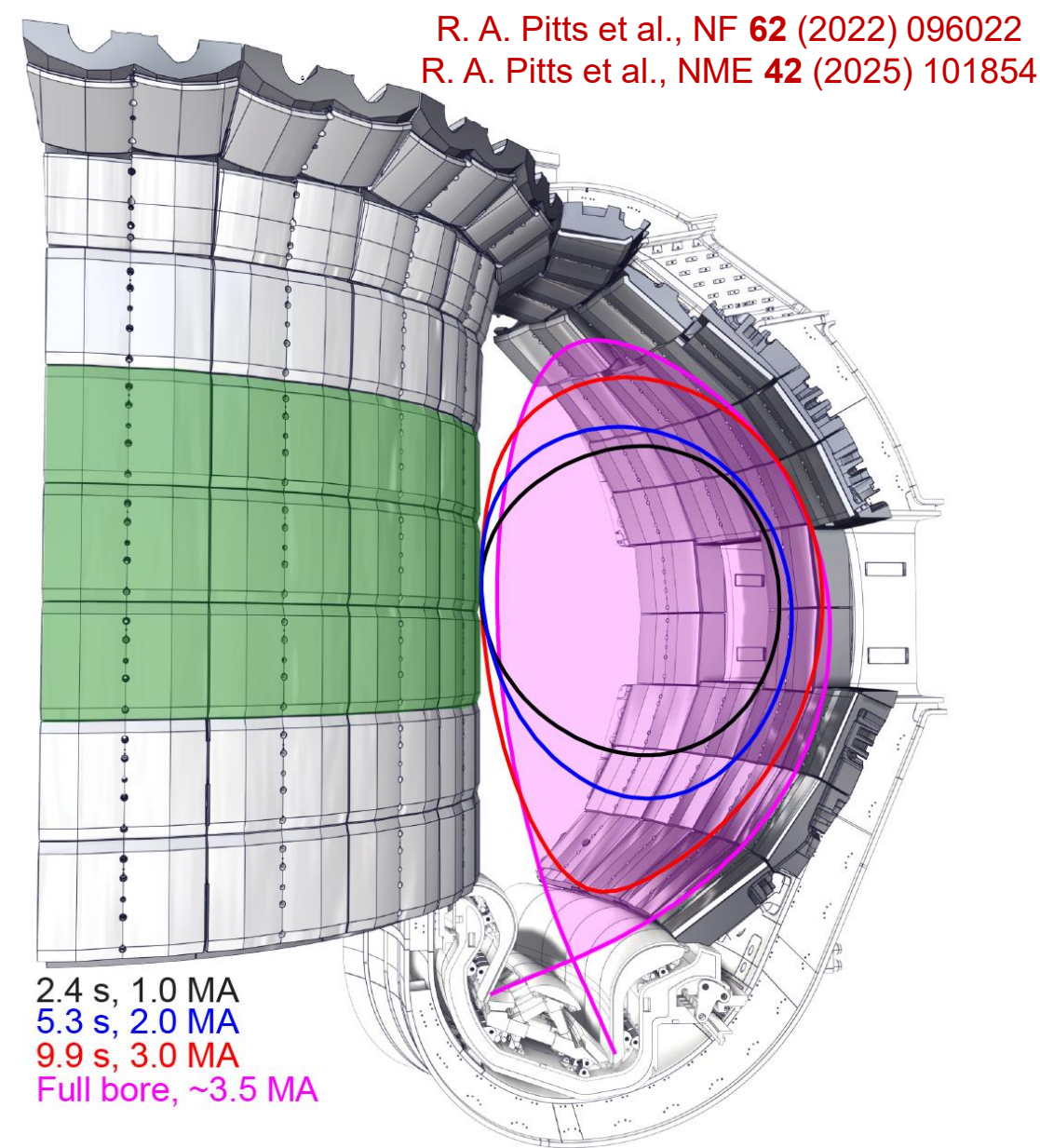


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- Introduction
- Startup on outboard limiter, unboronized wall, ASDEX Upgrade
- Startup on inboard limiter, unboronized and transition to non-uniform boronized wall, WEST
- W sources in outboard limiter plasmas on ASDEX Upgrade
- SOLPS-ITER calculations for ASDEX Upgrade
- Summary and Conclusion

Limiter start-up on W – an issue or not?

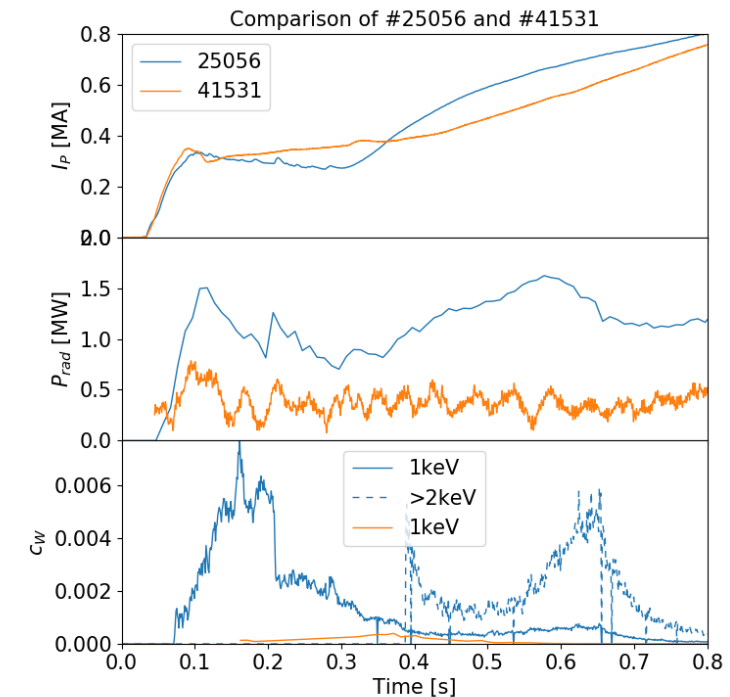
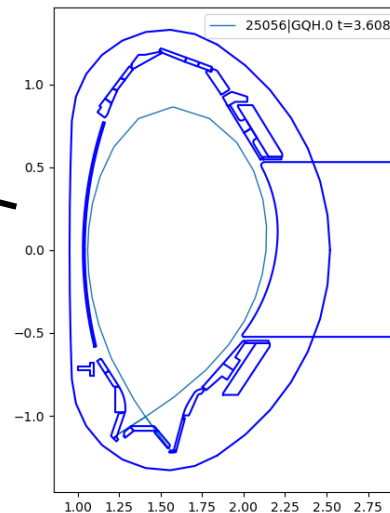
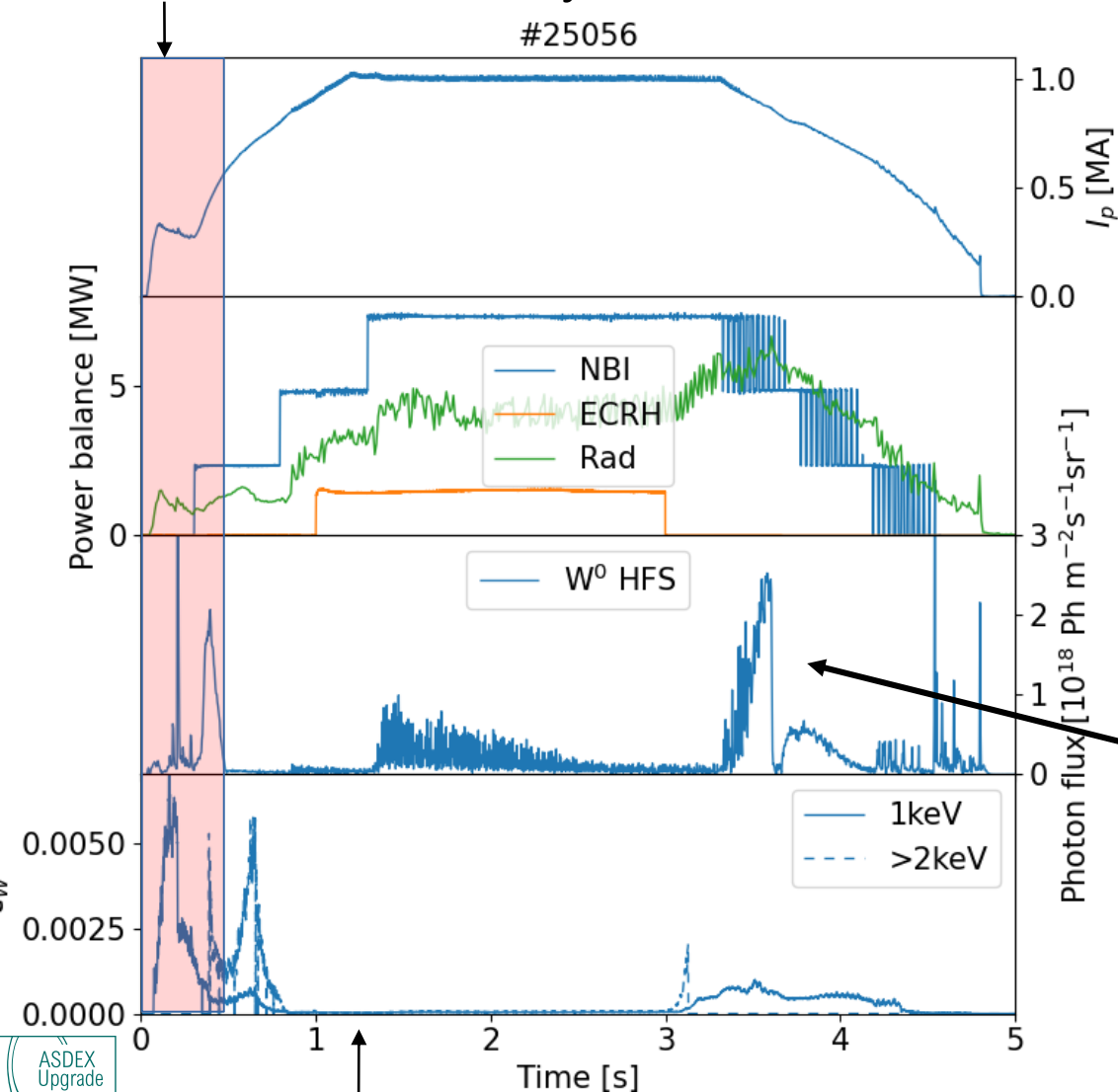
- Like many tokamaks, ITER plasmas will start up on the central column
- Switch from Be → W can have strong potential impact ($P_{\text{RAD,W}} \gg P_{\text{RAD,Be}}$)
- Limiter phase is rather long (~10 s) on ITER cf. current devices
- Very little detailed attention paid to this early phase
 - New expts. on ASDEX Upgrade, WEST, and EAST
 - New modelling for ITER
 - Related: Is machine conditioning (boronization) necessary to start the machine?



W radiation in AUG *inboard* limiter plasmas higher than in divertor plasmas

Limiter 2009, 2nd day after summer break, non boronised

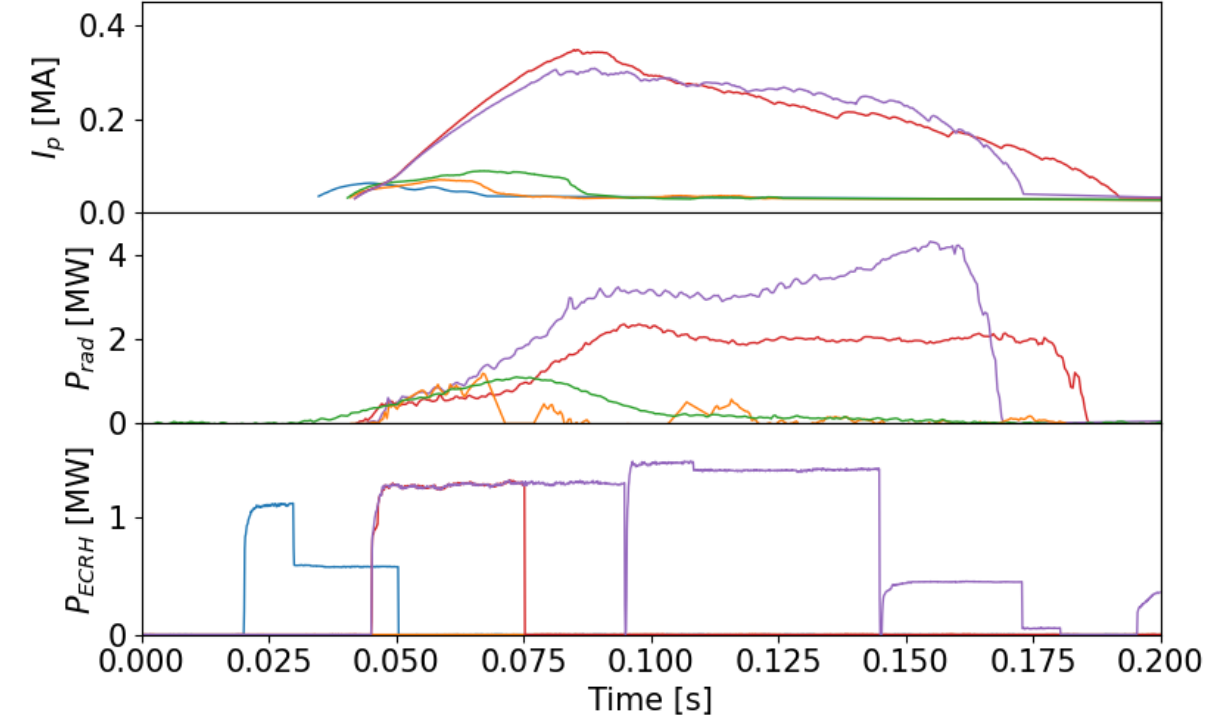
- W source from inner heat shield higher than in divertor H-mode
- Radiation losses can lead to very low electron temperatures
- Boronised and optimised pulses have a **factor 10** lower concentrations



Non boronized restart on *outboard* limiter not successful



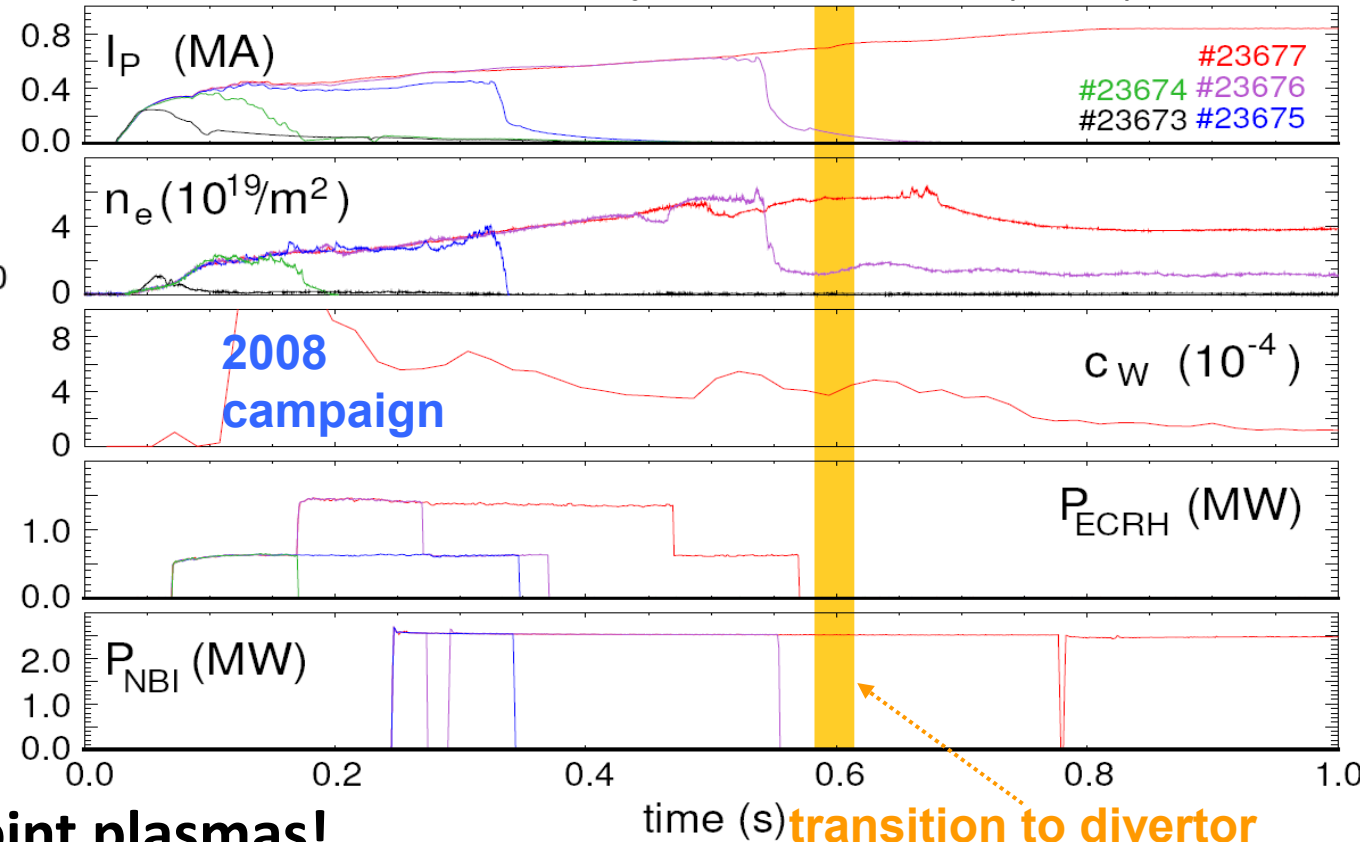
#41683 (blue), #41699 (orange), #41711 (green), #41729 (red), #41733 (purple)



- ECRH pre-ionisation successful but not long enough
- Later (and well aimed) ECRH allows plasma current ramp up
- After ECRH switch off plasma can't be sustained
→ too high (low Z, O) radiation

- Quite a few technical problems delaying progress (2 days too late prefill, no DCN → **very short ECRH (40ms)**, real time equilibrium wrong, right plasma position for ECRH had to be found)

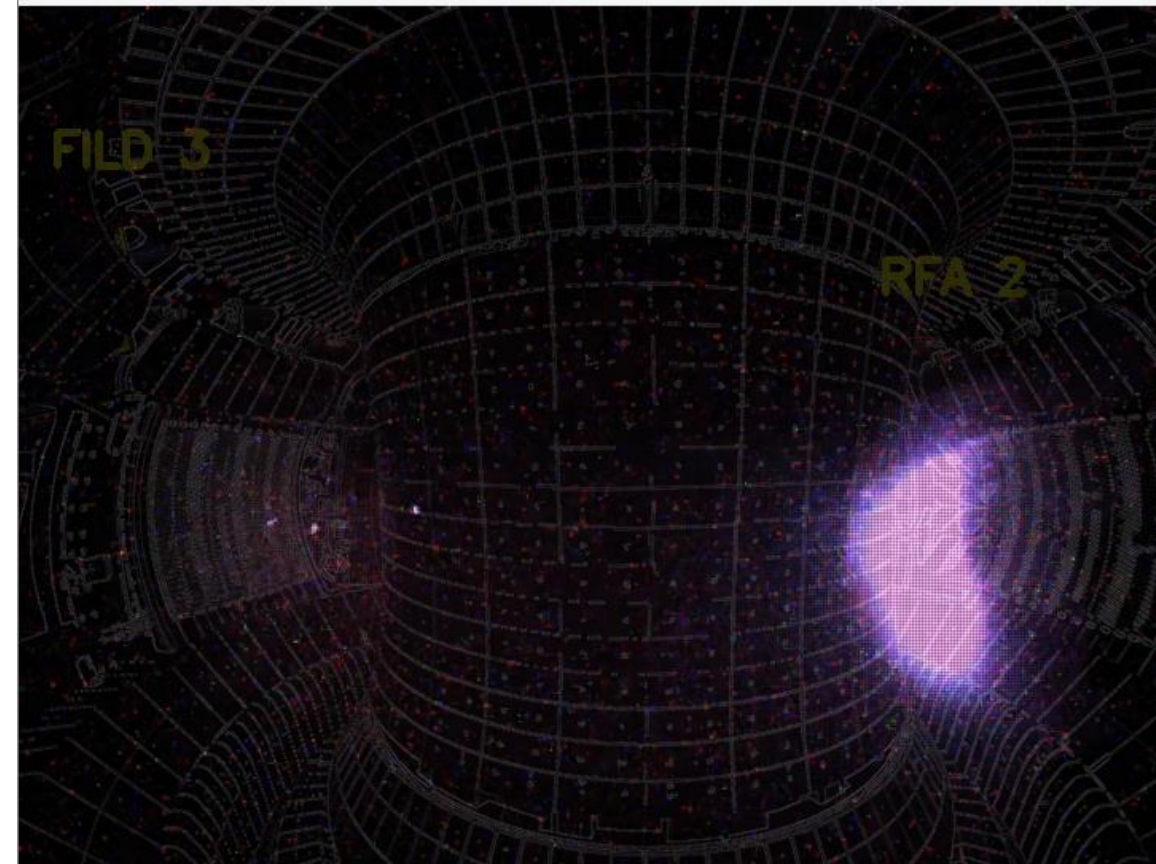
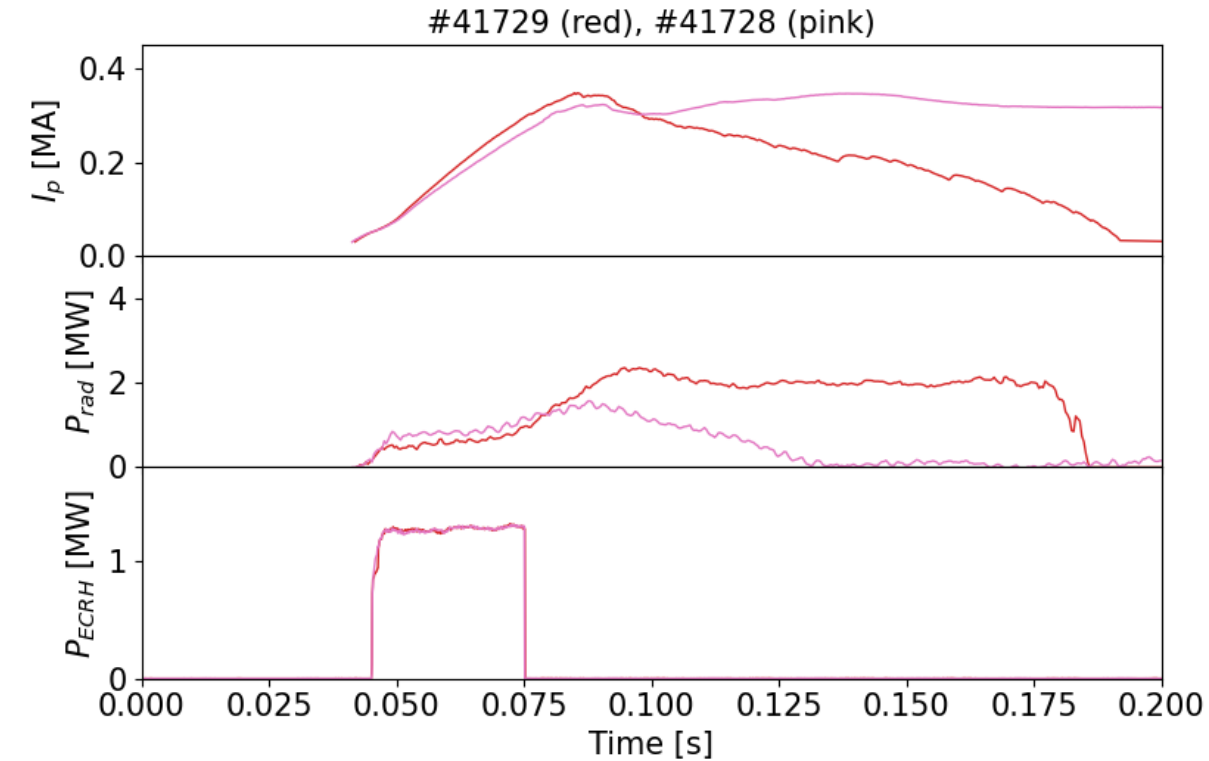
R. Neu *et al*, Phys. Scr., 014038, (2009)



Last slide with X-point plasmas!



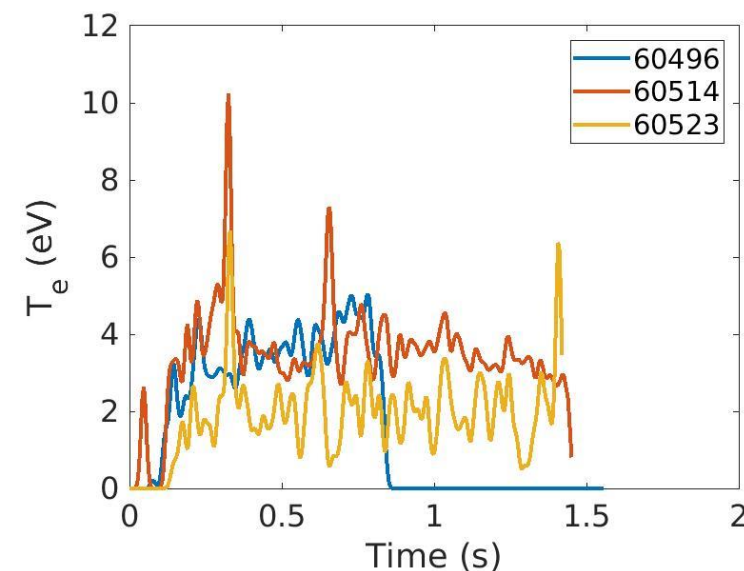
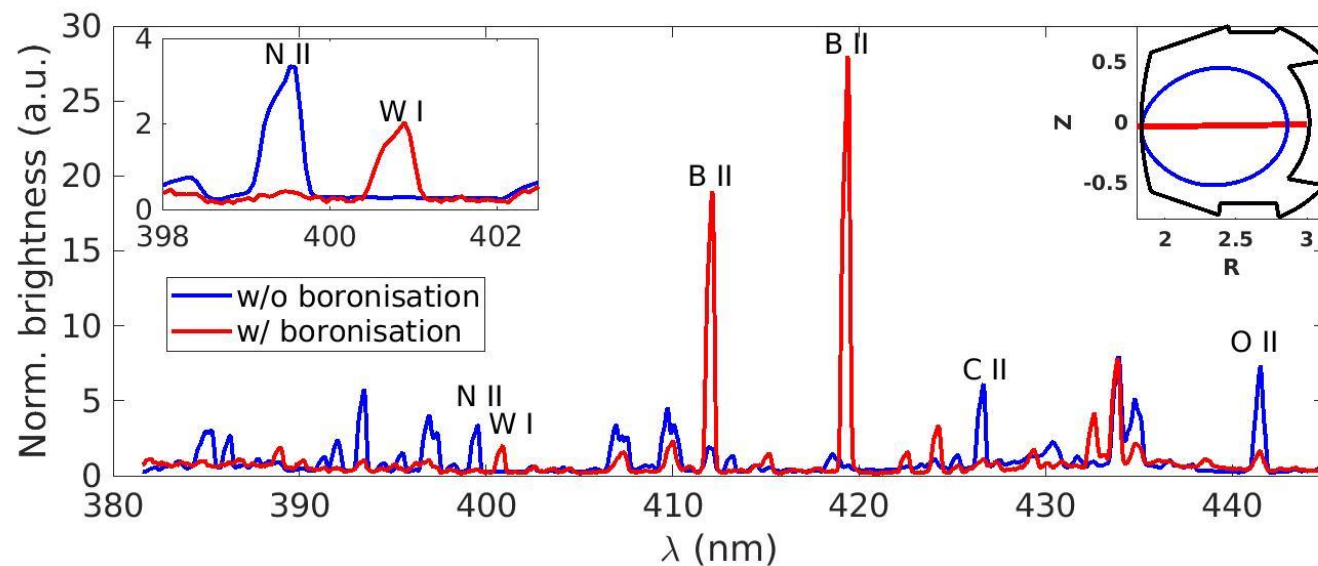
Non boronized restart can lead to runaways



- High loop voltage and low density can lead to sustained runaway beams

WEST plasma restart on *inboard* limiter without boronisation

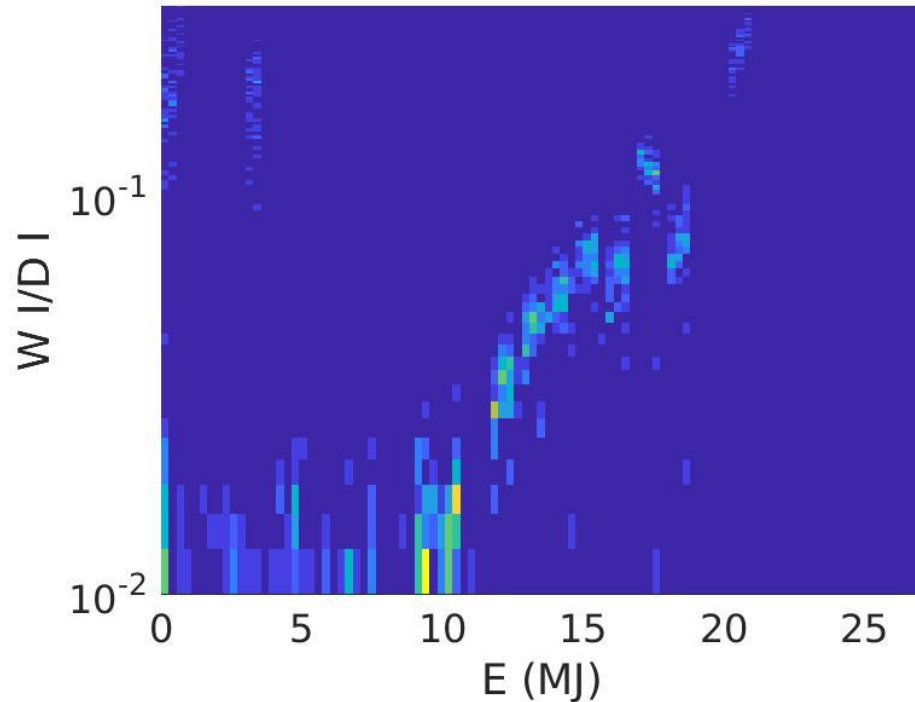
- Startup on bulk W inner limiter
- Density uncontrolled (strong outgassing from wall) but progressive conditioning
- RGA shows dominant mass 3 (HD) over 4 (D2) and isotopic ratio increasing during discharge (no D2 injection)
- Most plasmas cold and dense and do not attach on inner limiter (increase in density and $P_{\text{rad}} < 0.8$ s)
- Several “long pulses” obtained (~ 1.5 s) and up to 600 kA!
- Sawteeth observed and low loop voltages (~ 1 V)
- Visible spectroscopy shows dominant presence of oxygen carbon, nitrogen (also from UV) but no tungsten
- Newly installed Langmuir probes on inner limiter providing data (Tantalum probes embedded in tungsten tiles in the lower half of the limiter) with $T_e < 10$ eV



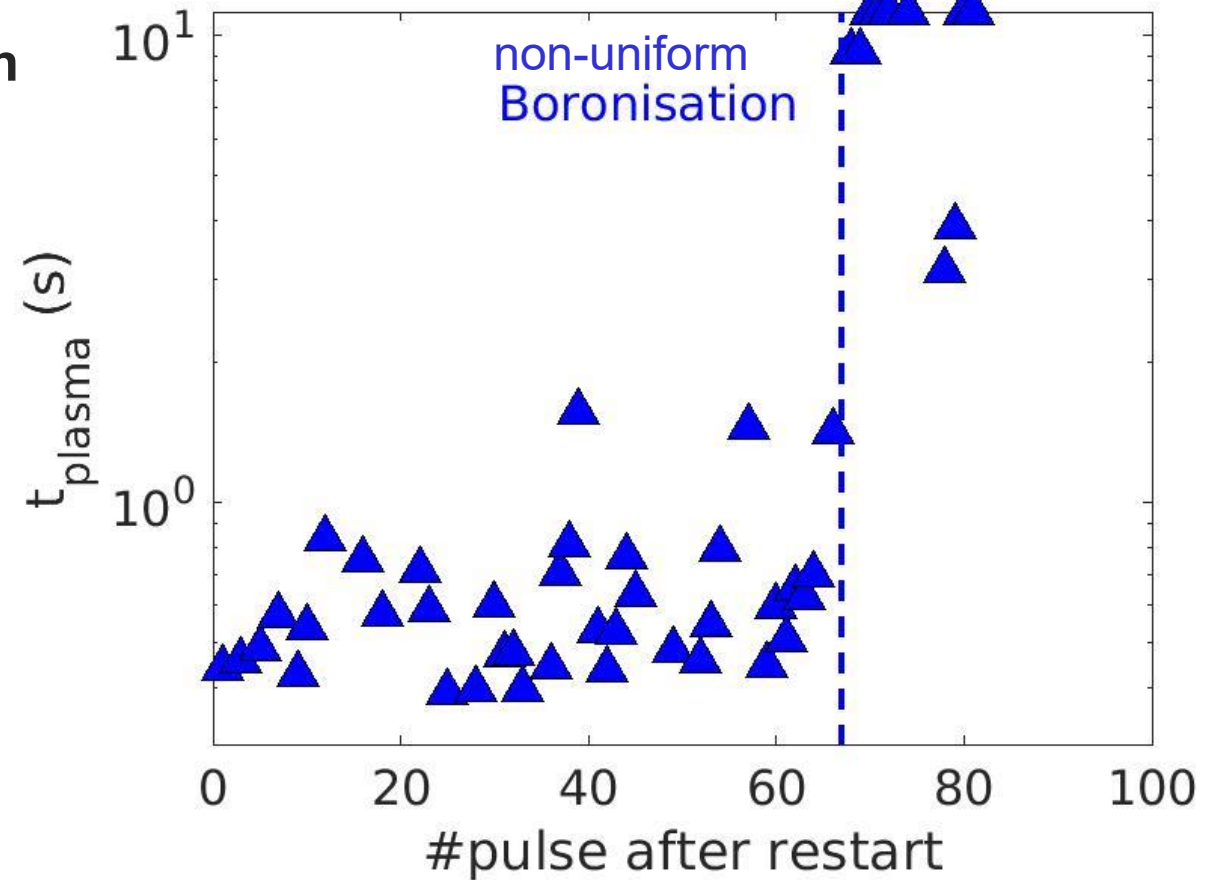
WEST plasma restart after non-uniform boronisation

■ First plasmas after non-uniform boronisation

- Startup on W inner limiter (ohmic pulses)



- First two shots with natural density ($n \sim 0.5 \times 10^{19} \text{m}^{-3}$)
- Increasing intensity of tungsten visible line during the first two shots without gas injection

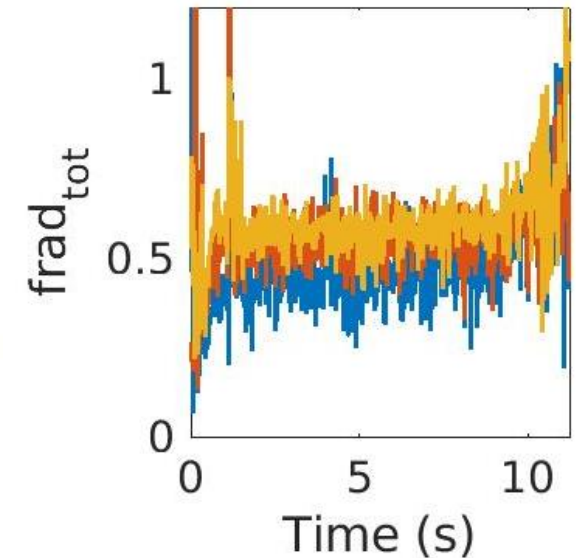
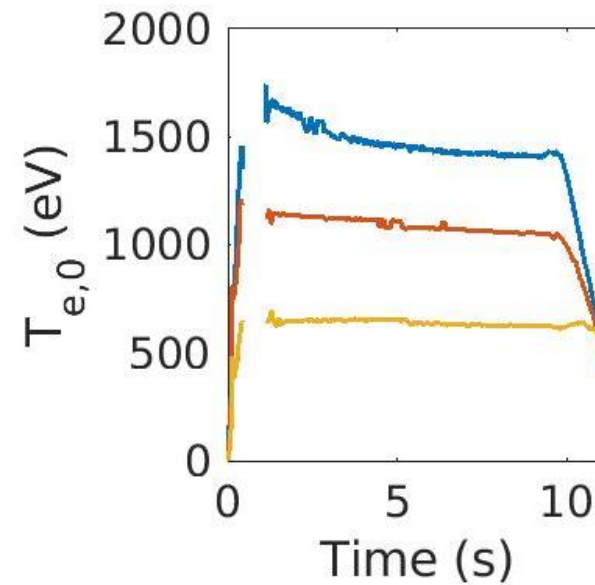
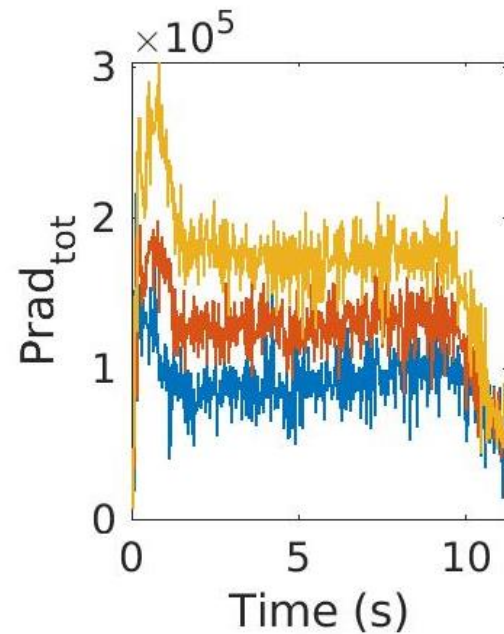
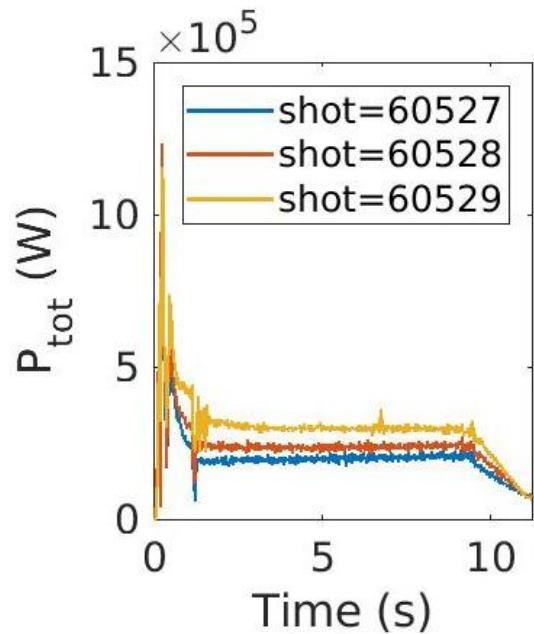


- Long pulses only after boronization
- Small conditioning effect visible

3 consecutive shots with increasing core W radiation

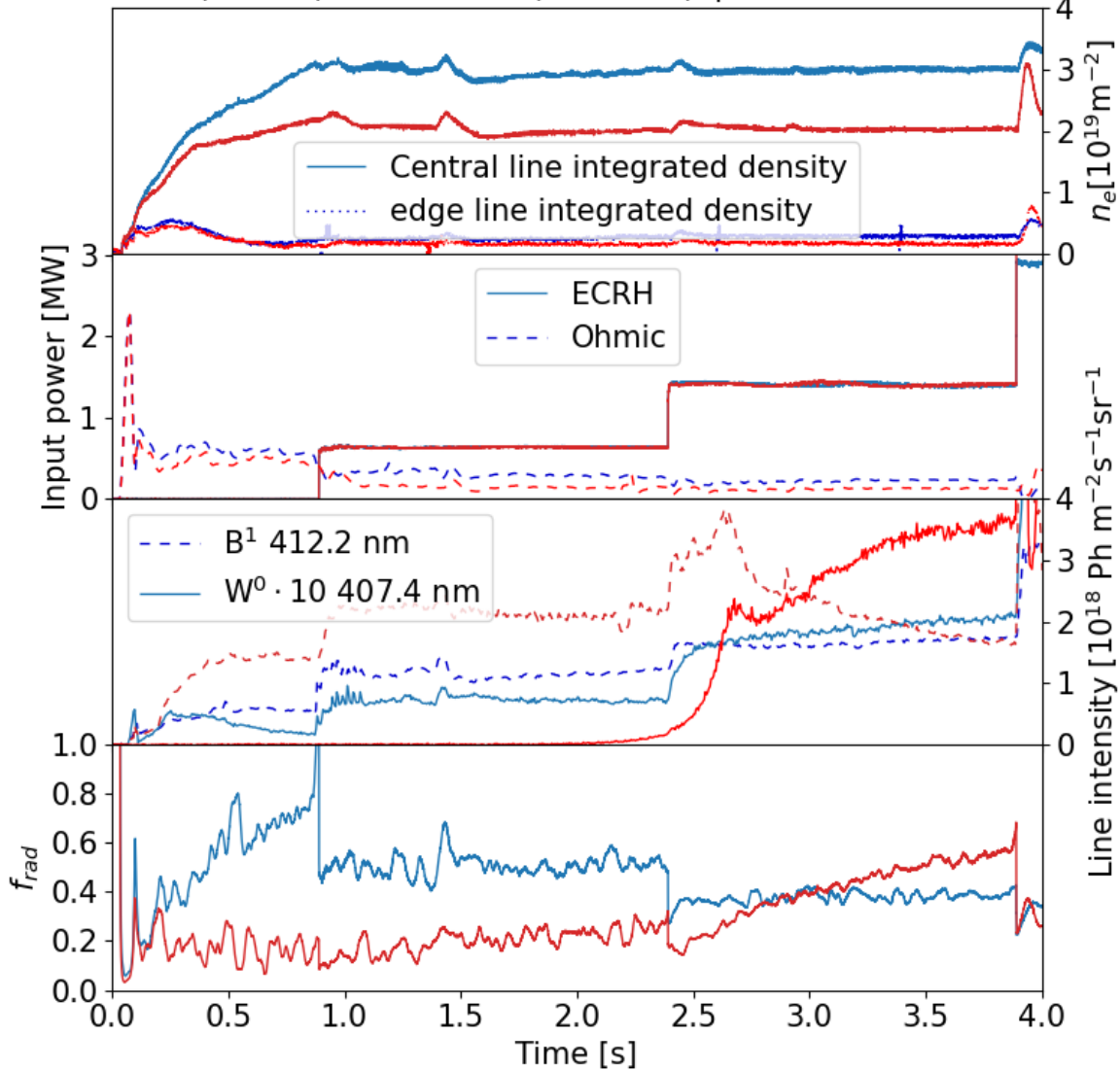
First cases with consecutive ohmic shots after boronisation:

- ❑ Central temperature decreasing from shot to shot
- ❑ Radiated power increasing together with ohmic power
- ❑ Lower frad for 60527 but similar between 60528 and 60529



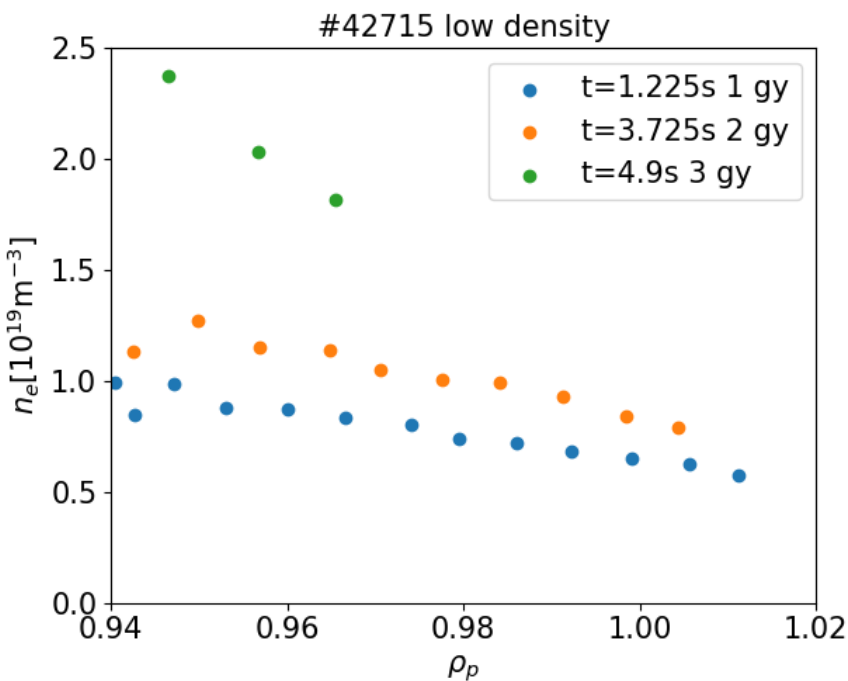
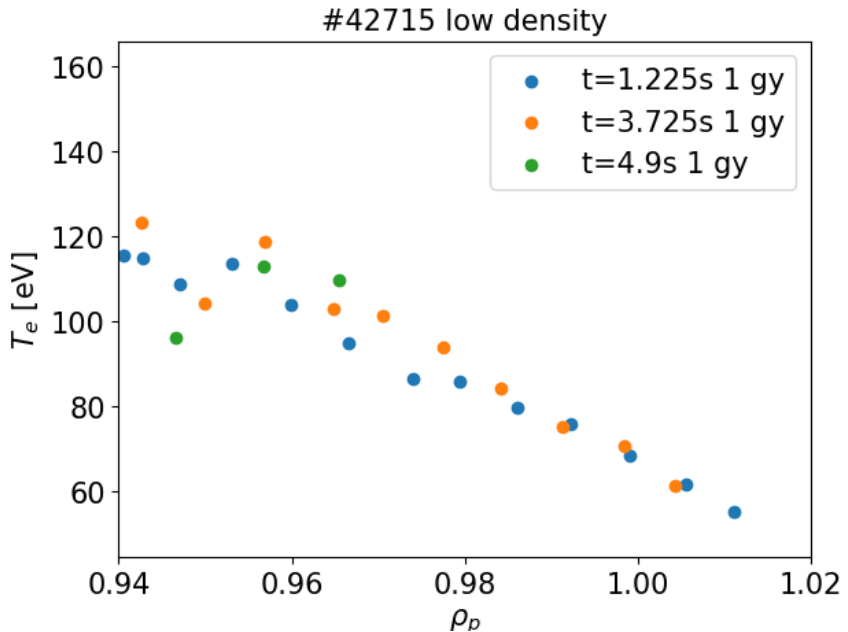
Pulses directly after boronization work well

#41876 (red, 1st) and #41877 (blue, 2nd), pulses after boronization

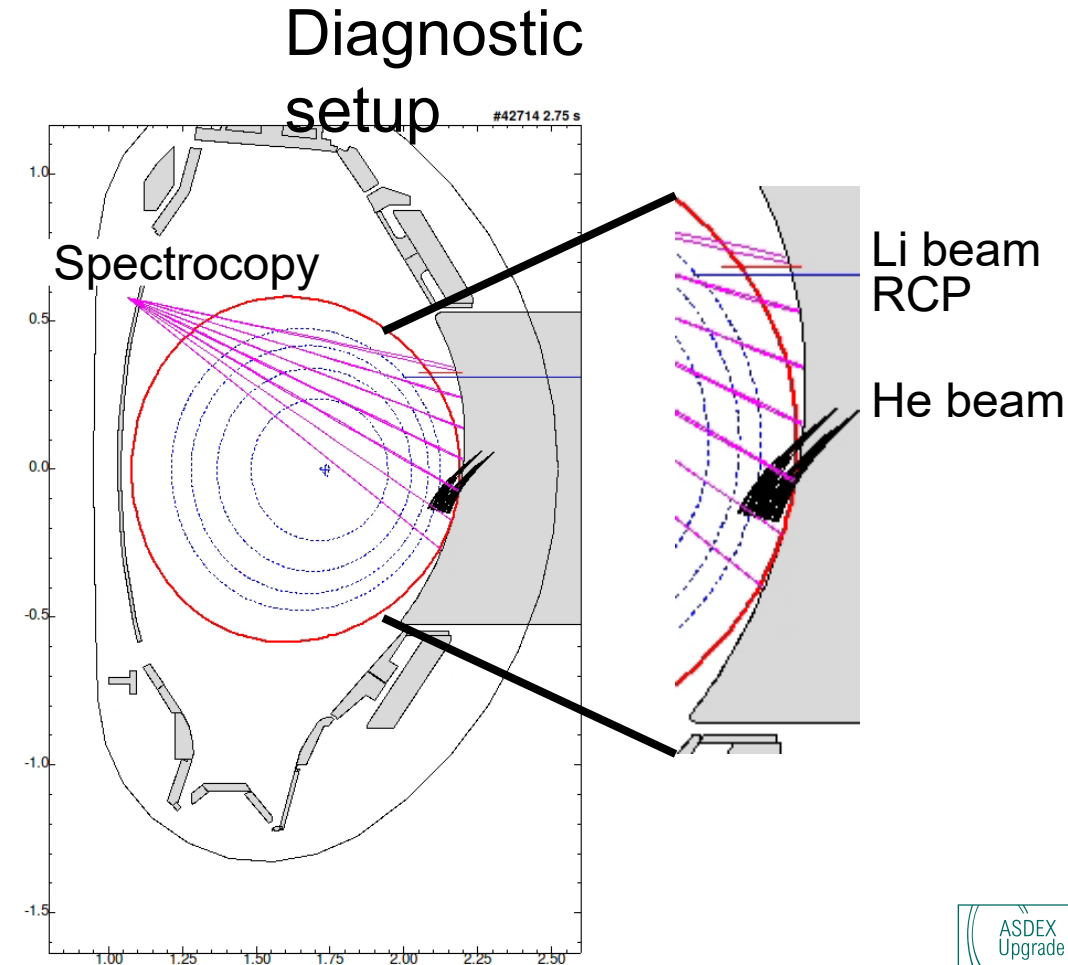


- 2 core densities on outboard limiter
- ECRH power scan 0.8MW and 1.5MW
- High heating power (5MW/3MW in 41876/41877) phase leads to melting and unstable plasmas
- Higher density pulse very stable in both heating phases
- W source and f_{rad} are increasing with time, even though background plasma stable. Max $f_{\text{rad}} \sim 57\%$, stronger visible at low density
- No W source in #41876 before 2 gyrotron phase → indication of B layer lifetime
- **Operation on outboard limiter not dependent on limiter surface B layer health**

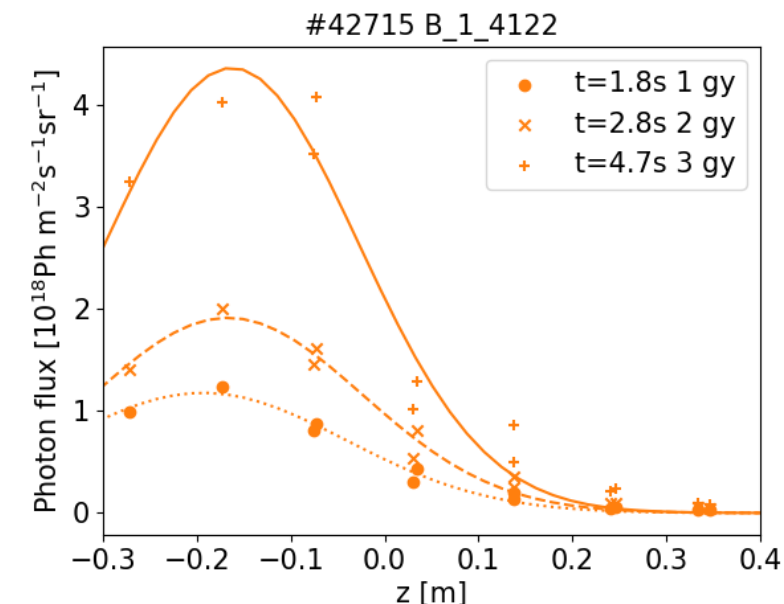
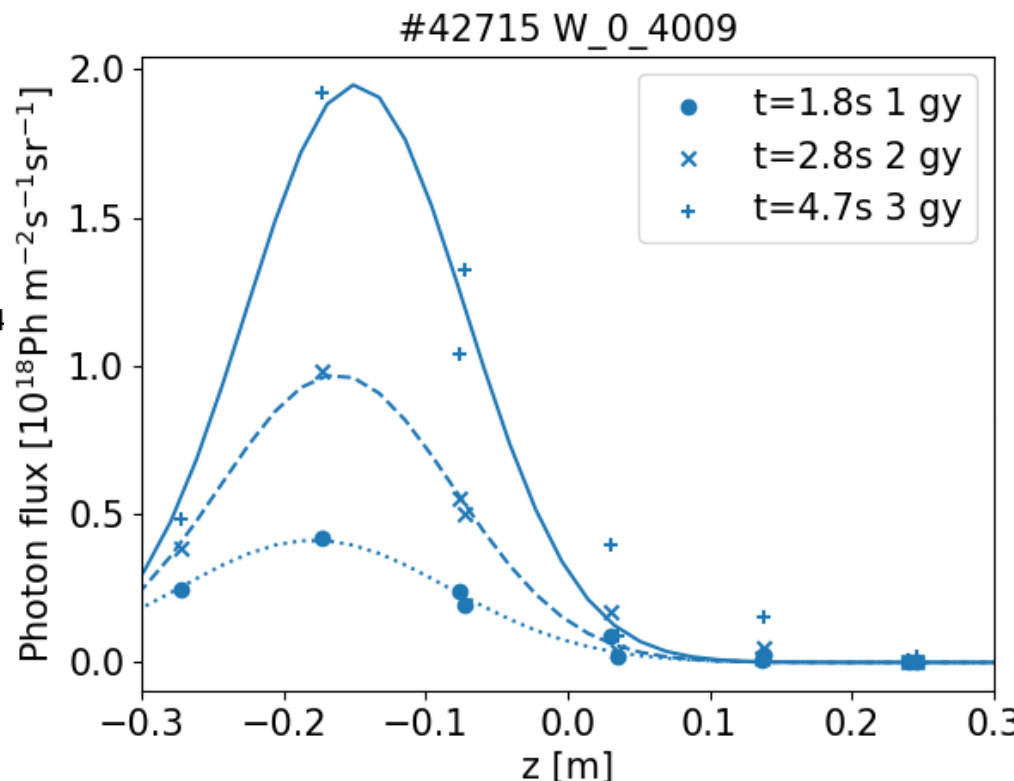
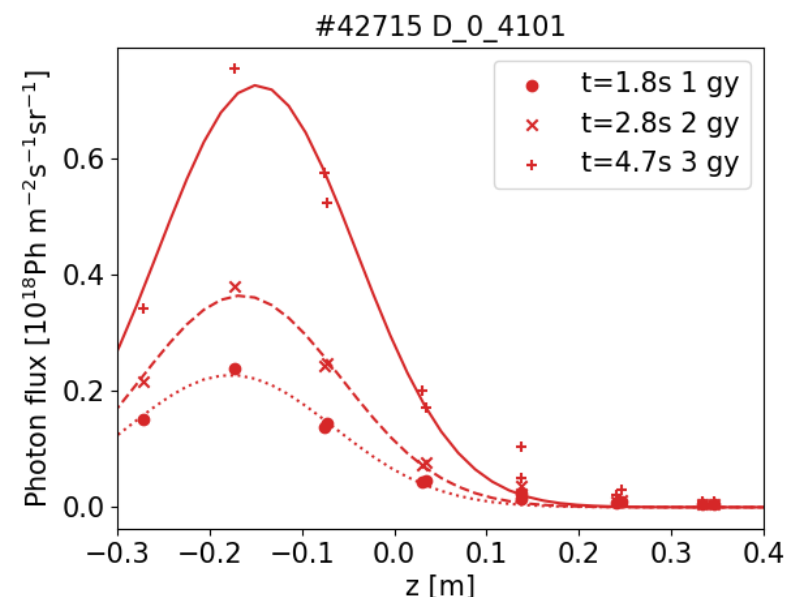
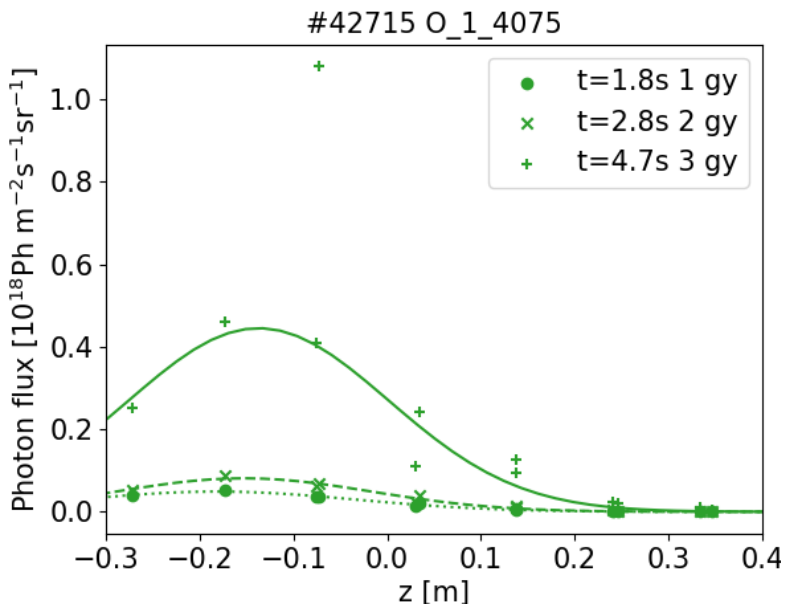
Profiles do not change with heating power, diagnostics agree



- Pulse with low density and 1.7MW ECRH
- $n_e^{LCFS} \sim 5-12 \cdot 10^{18} m^{-3}$
- $T_e^{LCFS} \sim 40-60 eV$
- LCFS electron temperature constant with input power
- LCFS density increasing



At constant density impurity sources increase with input power

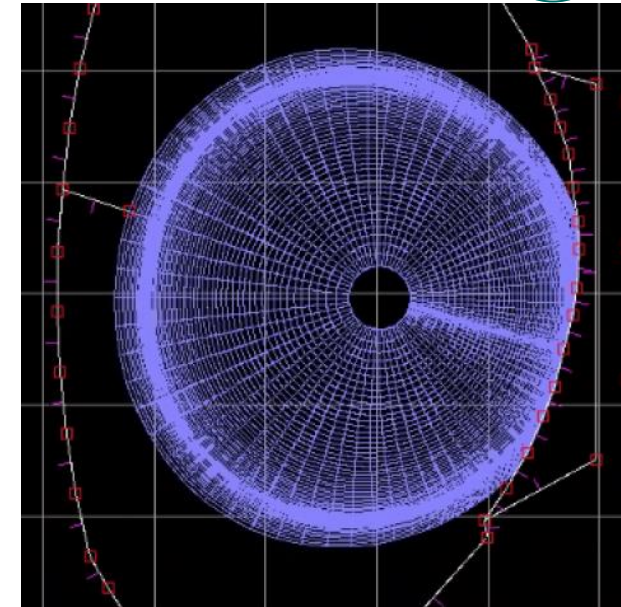
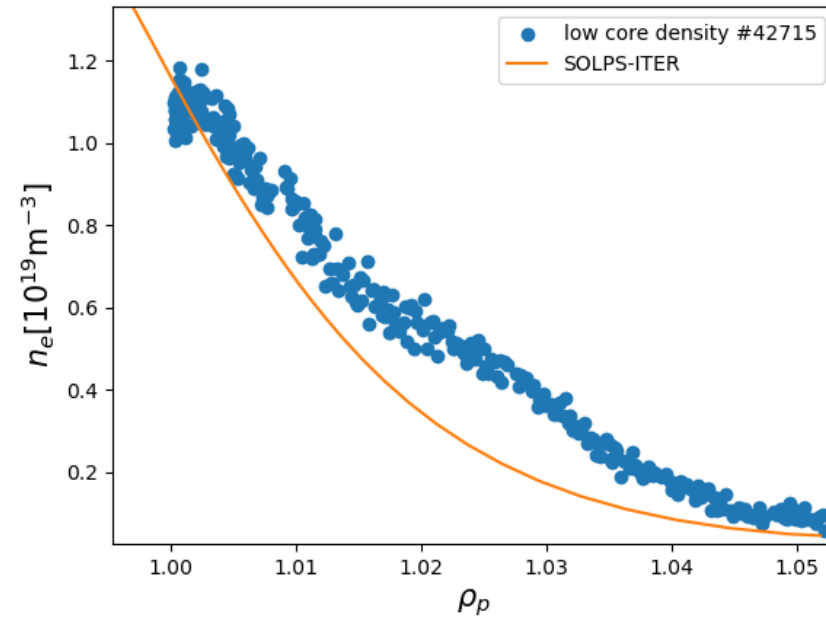
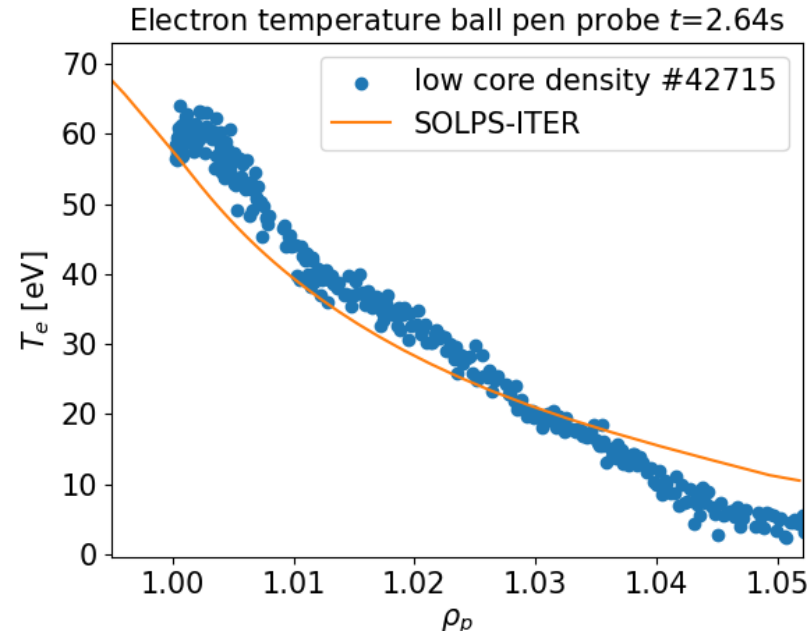


- W source profile narrow
- Other profiles wider
- W source proportional to input power
- Oxygen only increases at highest power

W source profile for 3 different heating phases, similar pulse to shown before, low density

SOLPS-ITER run matches experimental profiles well

Courtesy A. Pshenov



- SOLPS has been run to match the ASDEX Upgrade parameters.
- Uniform transport assumed
- W self consistently calculated (source $\leftrightarrow$ radiation balance)
- Impurities (B,O) have little impact on result!
- W source in SOLPS mainly by W self sputtering
- Main “tuning” parameter r_{prompt} roughly consistent with publications* and ERO calculations for ITER

Used parameters:

$D=0.5\text{m}^2/\text{s}$

$\chi_{e,i}=1\text{m}^2/\text{s}$

$v_{\text{pinch}}=2\text{m/s}$ (only W)

$P_{\text{heat}}=1.7\text{MW}$ 2:1 e:i

$c_B=2\%$

$c_O=0.5\%$

W up to $W40+$

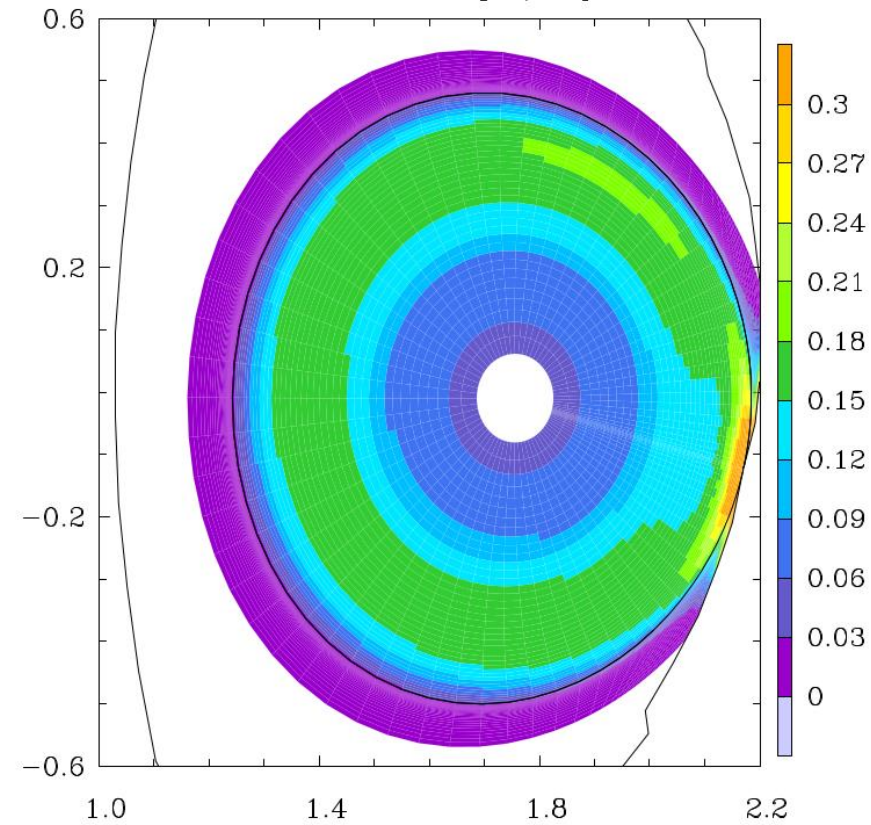
$r_{\text{prompt}}=0.6$

$n_{e,\text{sep}}=6 \cdot 10^{18}\text{m}^{-3}$

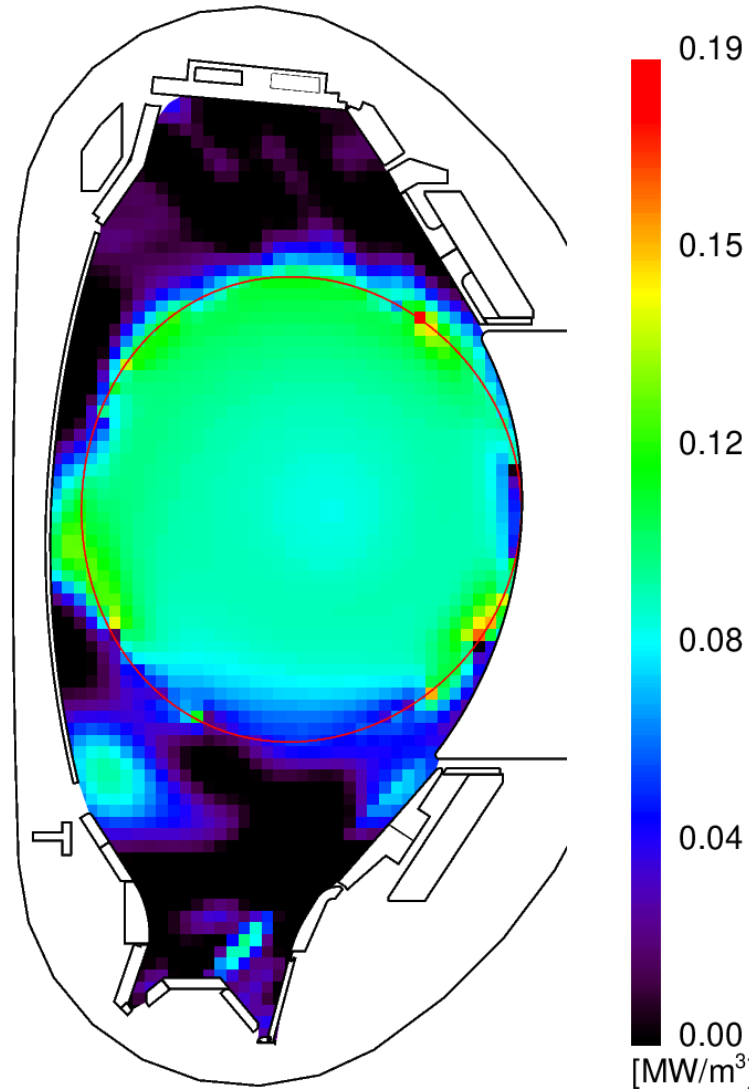
SOLPS-ITER calculation gives a very similar radiation pattern

SOLPS-ITER

Total radiation [MW/m³]

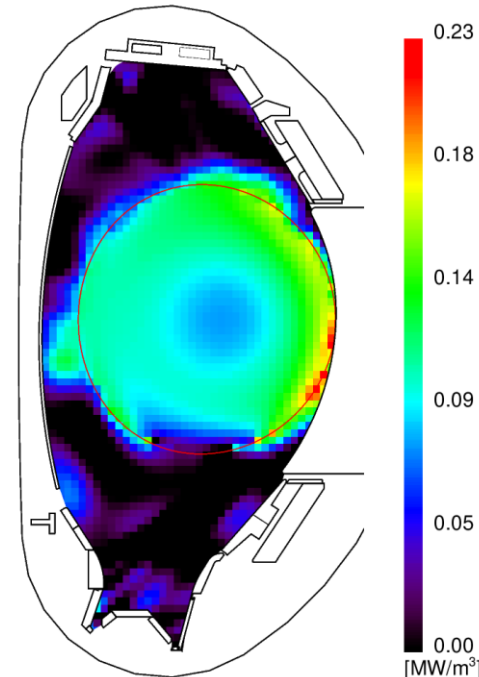


#42715 @3.80s



- Radiation pattern from SOLPS-ITER very similar to experimental one
- Profile very flat – no peaking
- Higher radiation at interaction point

#42715 @4.20s



Summary and conclusions

- “Multi-machine” experiment conducted (AUG, EAST, WEST) in support of ITER
- AUG and WEST have tried a non boronized startup
- Both experiments show difficulties to control the (low Z) radiation and to produce long flat-top plasmas without boronization
- more runtime and/or heating power could have allowed longer pulses
- Non-uniform boronization allows “normal” startup
- Detailed impurity source and plasma data has been collected
- Preliminary SOLPS-ITER runs show very good agreement with AUG data
- These AUG and WEST experiments clearly indicate that ITER's decision to include a diborane boronization system is an important risk mitigation measure!

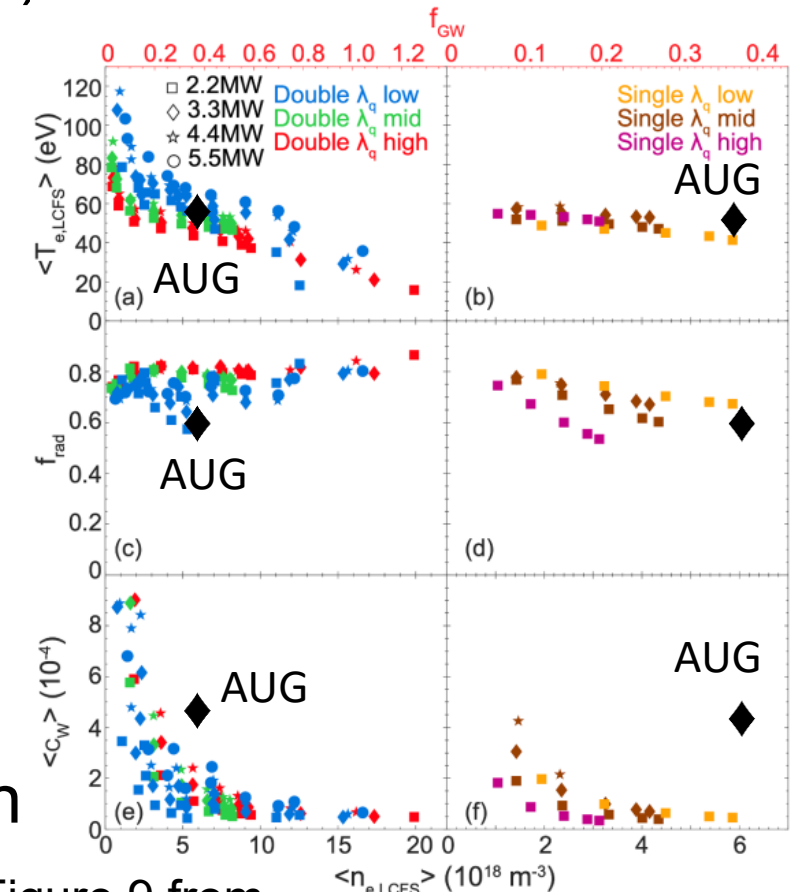


Figure 9 from
Y. Zhang, NF **65** (2025), 056035