

PERFORMANCE EVALUATION OF Wf/W PFM DEVELOPED AT SWIP FOR APPLICATION IN NUCLEAR FUSION REACTORS

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October 2025 · Chengdu, China

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02 Manufacturing of Wf/W at SWIP

Introduction and background

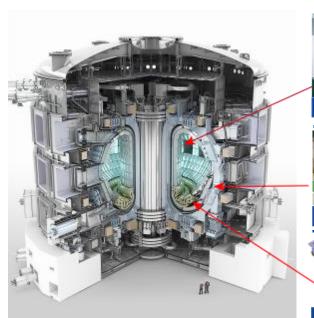
O3 Performance of Wf/W

04 Summary



1.1 Materials development platform at SWIP

ITER In-Vessel Components





60 High-Heat-Flux First Wall **Panels**



220 units of Shield Block (SB)



Integrated assembly of 58 cassettes

Material and component testing platform









The 1st ISO standard in the nuclear fusion



Liquid metal research platform with 4 Tesla **Magnetic field**



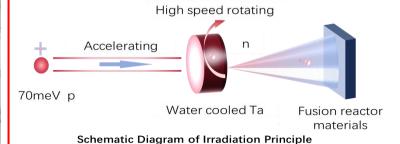
Hydrogen Sinte. Furn. (2200°C) Fatigue machine (1600°C)

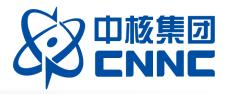


Tensile Machine (2300°C)

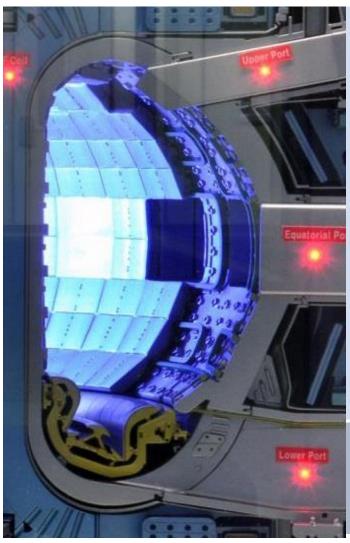


Recrystallization test furn. (2000°C)





1.2 Introduction materials research at SWIP



Blanket:

Plasma facing Material:

- Tungsten plate
- CVD Tungsten

Structure Material:

- CLF-1(TMT-CLF)
- ODS steel
- vanadium alloy/liquid metal

Functional Material

- Tritium Breeding: Li₄SiO₄, Li₂TiO₃-Li₄SiO₄

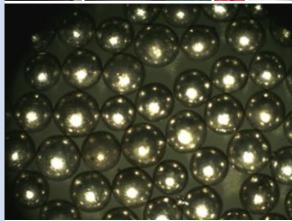
- Neutron multiplier: Be₁₂Ti

- Barrier coatings: Al₂O₃, HEA coatings

Divertor:

- Advanced Tungsten,
- ODS Cu/Cu alloy





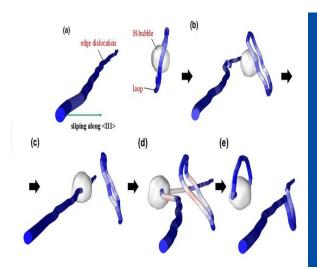




1.2 Introduction materials research at SWIP

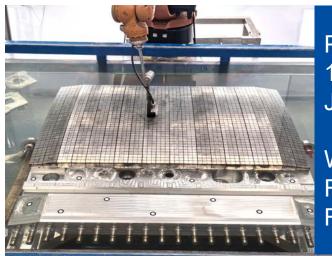


Posters 5, 8:30-12:20 17 Oct, B-55, G.Yang, Progress on the engineering qualification of CN-RAFM steel

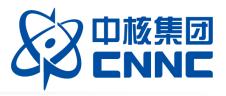


Posters 5, 8:30-12:20 17 Oct, B-68, Y.X. Wei, The interaction between the edge dislocation and the dislocation loop bubble complex under shear stress in BCC iron

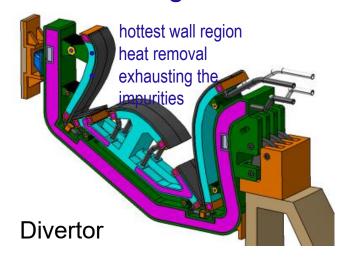


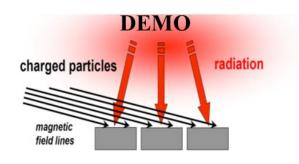


Posters 7, 8:30-10:40 18 Oct, B-36, Jiming Chen, R&D on W First Wall for ITER and Future Fusion Reactors



1.3 Advance Tungsten PFMs research at SWIP





HHF: $\sim 20 \text{MW/m}^2$

life: > 2fpy

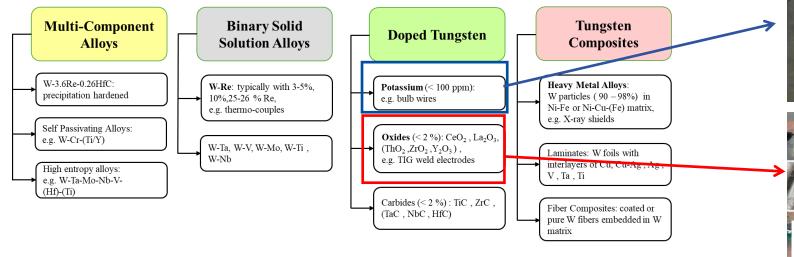
erosion: ~2mm/fpy

 $n \sim 2MW/m^2(3-4dpa/fpy)$



High DBTT, Embri. due to Recry. &Neutr.

Strategy for advanced Tungsten

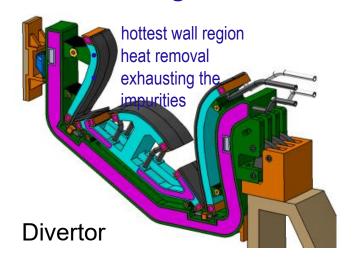


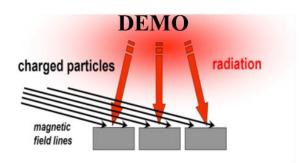






1.3 Advance Tungsten PFMs research at SWIP





HHF: $\sim 20 MW/m^2$

life: > 2fpy

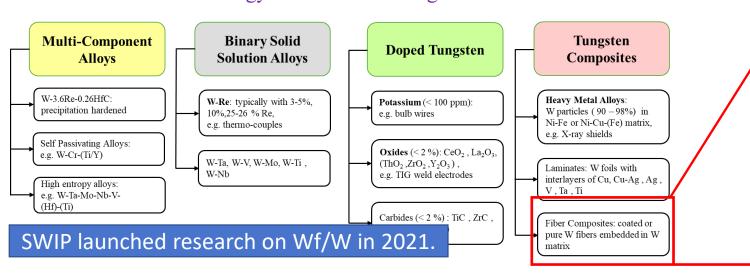
erosion: ~2mm/fpy

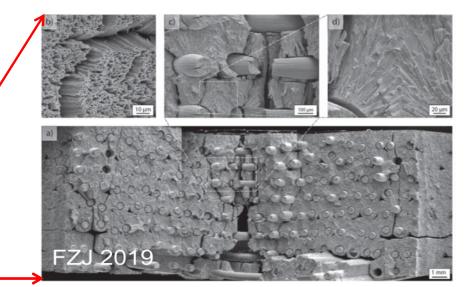
 $n \sim 2MW/m^2(3-4dpa/fpy)$



High DBTT, Embri. due to Recry. &Neutr.

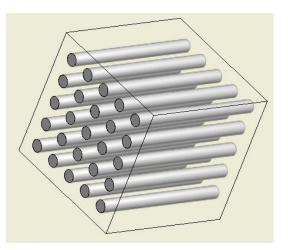
Strategy for advanced Tungsten

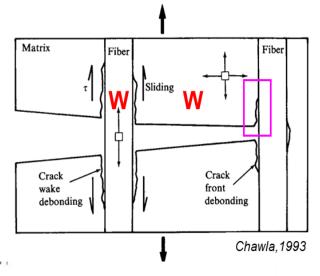


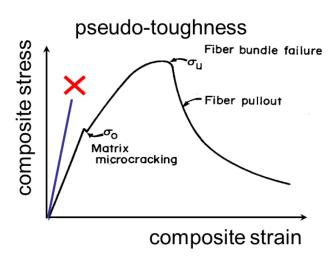




1.4 Tungsten fiber reinforced tungsten matrix composite(Wf/W)





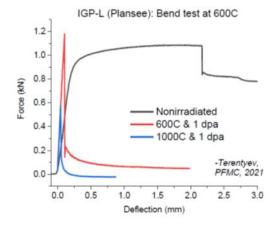


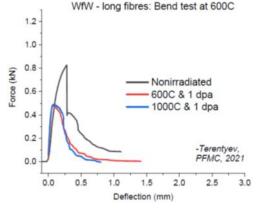
Benefit by the energy dissipation:

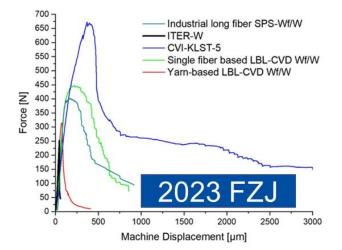
- interface debonding
- fiber sliding

W_f/W_m composite shows excellent ductility, which is called—

Pseudo Toughness.





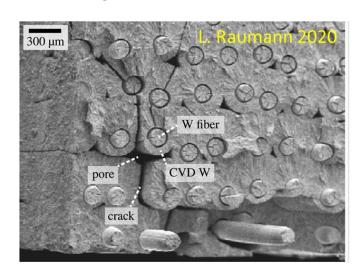


W_f/W material is one of the most promising plasma facing materials for the future fusion reactor.

T. Hoeschen 2020



1.5 Key points of Wf/W fabrication



Density

Therm. Conduct.: $\lambda = \alpha \cdot C_p \cdot \rho$, W/m·k

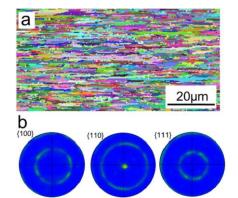
a —therm. Expa. Coeffi., m^2/s

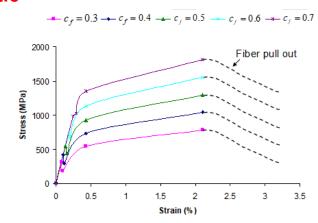
 c_p —spec. heat capa., J/kg·K

 ρ —density, kg/m³

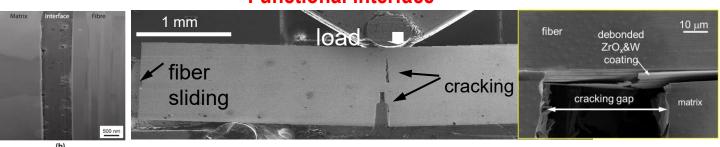
Fiber volume ratio







Functional interface



Fiber volume, Functional interface, Density, 3D!

02

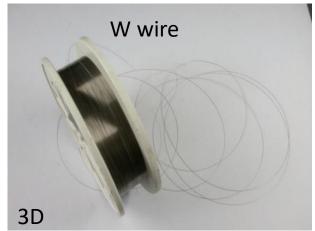
Manufacturing of Wf/W at SWIP

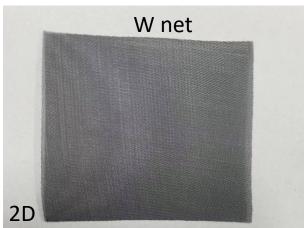


2.1 Fiber with functional interface

Fiber

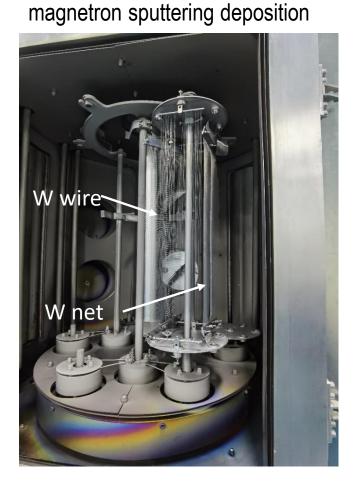
commercial W wire and W net





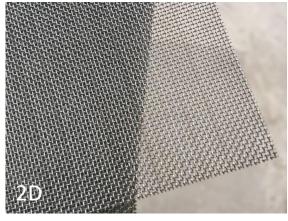


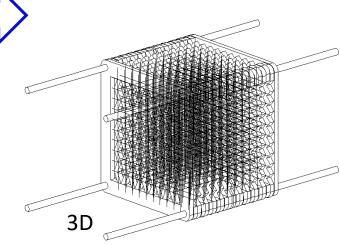
Interface



Reinforcement

fiber with interface

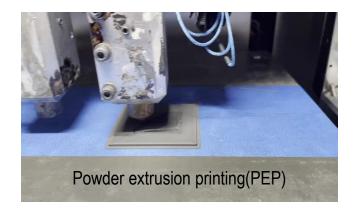


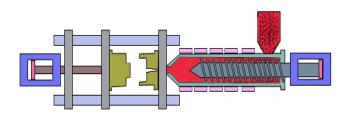




2.2 Composite fabrication-green part

Green part fabrication



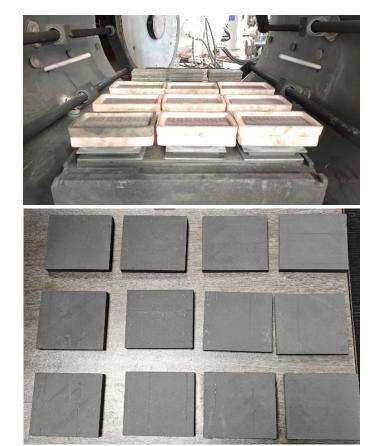


Injection molding

Green part before debinding



Green part after debinding





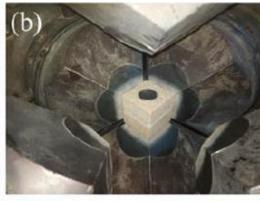
2.3 Composite fabrication-final densification

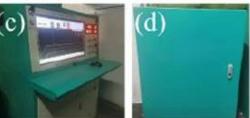
Green part



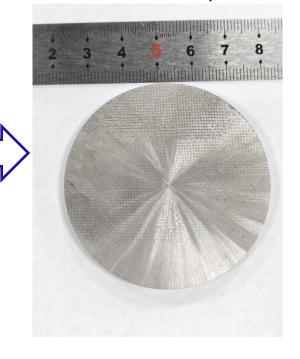
High pressure high temperature sintering device







Finished sample



86.6×86.8×10.2mm³ Relative density:50%

Isotropic pressure: 3~5GPa

Sintering temperature: 1400~1600°C

Sintering duration: 30min

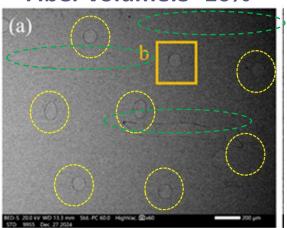
Relative density:98~99%



2.4 Properties of Wf/W

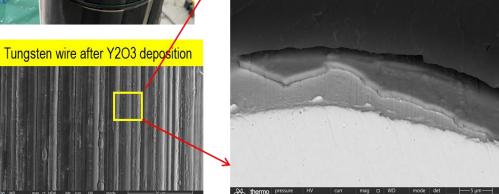
Relative density 150 98.8±0.5 99.0±0.7 99.1±0.4 1200 1400 Sintering Temperature(°C)

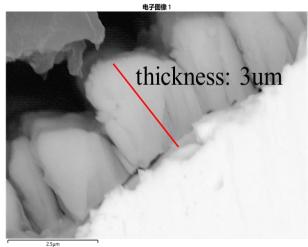
Fiber volume:3~10%

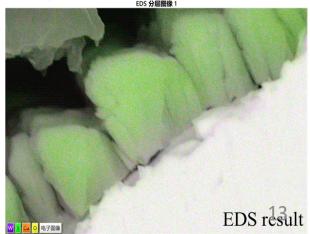


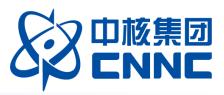
Functional interface



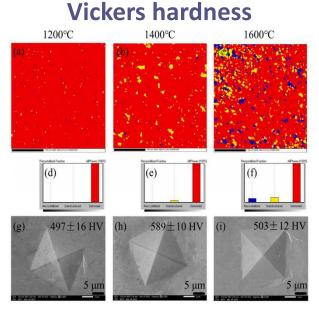




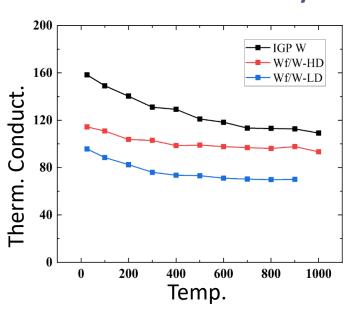




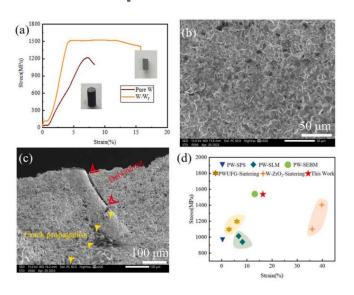
2.4 Properties of Wf/W



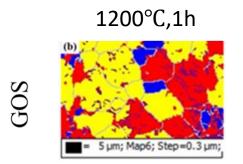
Thermal conductivity

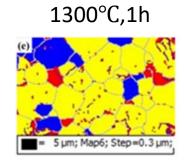


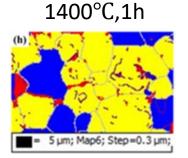
Compression test

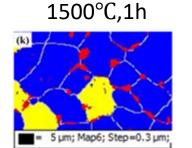


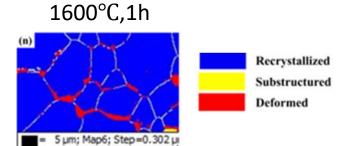
Heat treatment test: RCT of Wf/W is around 1600°C.









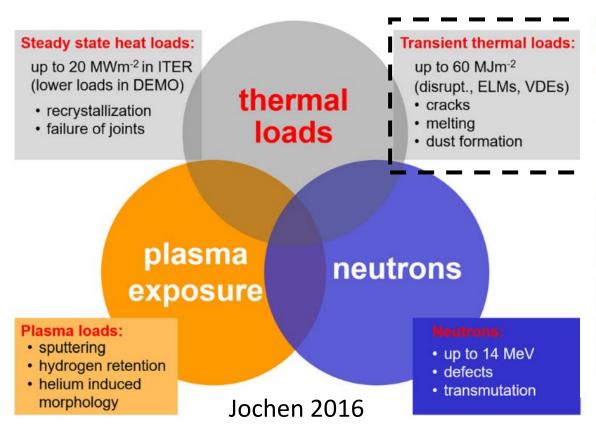


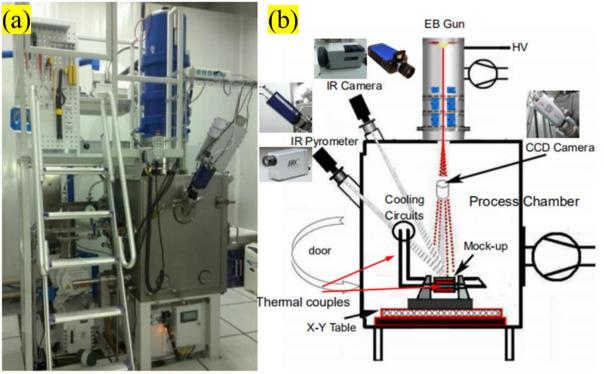
03

Performance of Wf/W



3.1 Performance Wf/W under transient thermal loads



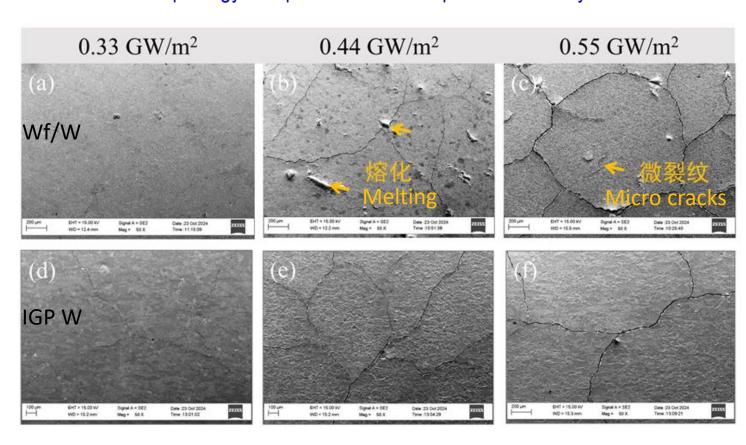


EMS-60 at SWIP, HHF via electron beam



3.1 Performance Wf/W under transient thermal loads

Morphology of top surface the samples after 100 cycles



Cracking threshold comparison

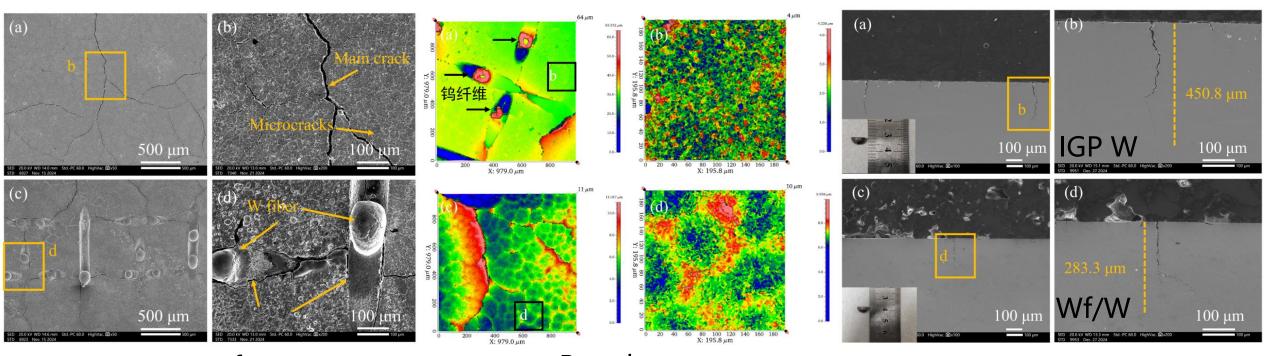
	Pulse	Cracking
Tungsten grades	duration	threshold
	(s)	(GW/m ²)
IGP W	1	0.22-0.33
W-K via SPS	5	0.37-0.50
W via SPS	5	< 0.37
W Forged	1	0.19-0.38
W-Y2O3 Forged	1	0.22-0.33
W-ZrC via SPS	1	0.22-0.33
W-ZrC Rolled	1	0.22-0.33
W-TaC Rolled	1	0.33-0.44
Wf/W(this work)	1	0.33-0.44

Cracking threshold is higher than IGP W and comparable to most advanced W!



3.1 Performance Wf/W under transient thermal loads

Morphology of top surface the samples after 1000 cycles @ 0.55 GW/m²

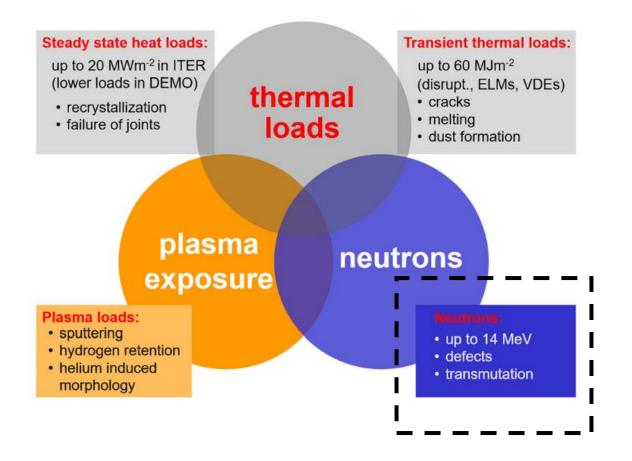


Top surface Roughness Cross section image

Crack depth is reduced by 40% due to the pseudo toughness!



3.2 Performance Wf/W under displacement damage



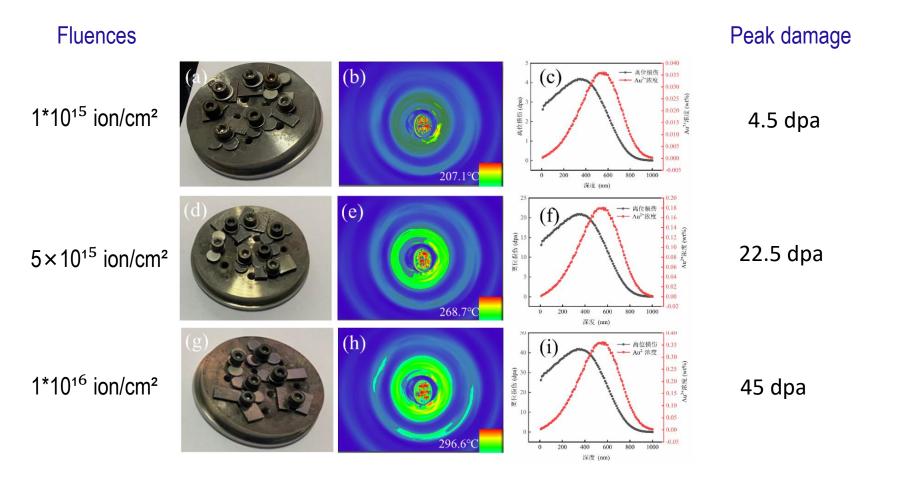


HVE-3MV Tandetron Accelerator at SCU

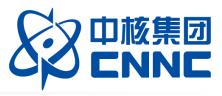


3.2 Performance Wf/W under displacement damage

7.5 MeV Au²⁺, total accumulated fluences were 1*10¹⁵ ion/cm², 5×10¹⁵ ion/cm² and 1*10¹⁶ ion/cm²



Via SRIM calculation, the damaged zone extends to around 800 nm and the peak damage is located at around 400 nm.



3.2 Performance Wf/W under displacement damage

7.5 MeV Au²⁺, total accumulated fluences were 1*10¹⁵ ion/cm²、 5×10¹⁵ ion/cm² and 1*10¹⁶ ion/cm²

Damage evolution in matrix (a) (a) (a) (d) (d) (g) (g) 22.5 dpa 45 dpa 4.5 dpa

The damage accumulate more due to the increase of dpa dose.

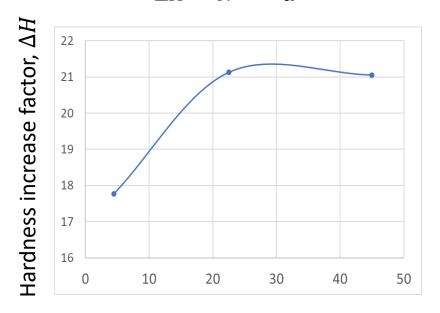


3.2 Performance Wf/W under displacement damage

Dislocation summary

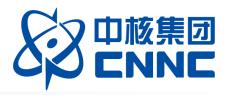
Disp. Dama(dpa)	Average size of the dislocation loop d (nm)	Density of dislocation loop N (/cm³)
4.5	4.03	4.41×10^{16}
22.5	8.95	2.36×10^{16}
45	10.63	1.98×10 ¹⁶

$$\Delta H \propto N^{1/2} \cdot d^{1/2}$$



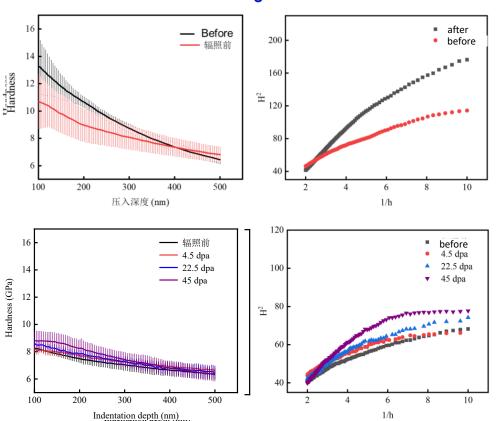
Displacement damage, dpa

• The damage accumulate more due to the increase of dpa, with a saturation phenomenon after high dose of damage.



3.2 Performance Wf/W under displacement damage

Irradiation hardening via Nano-indentation



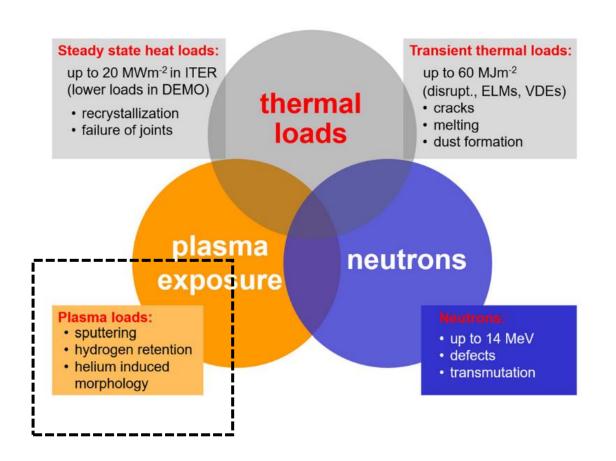
Hardness value before and after displacement damage

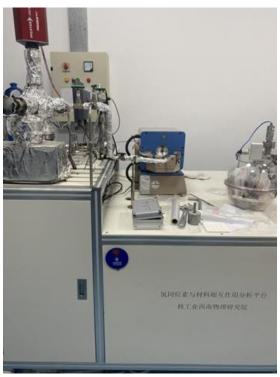
Sample	Damage (dpa)	Temp. (℃)	Hard. before (GPa)	Hard. after (GPa)	ΔH (GPa)
W fiber	45	RT	8.07	8.39	0.32
W matix	45	RT	6.95	8.25	1.3
W matix	22.5	RT	6.95	7.97	1.02
W matix	4.5	RT	6.95	7.44	0.49
W-K	0.45	127	-	-	0.7
Pure W	7	500	-	-	1.1
W-ZrO	100	RT	-	-	4.52
W-K	11.5	650	-	-	1

Tungsten fiber has better irradiation resistance than matrix suggests promising radiation tolerance in Wf/W.

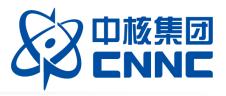


3.3 Plasma exposure

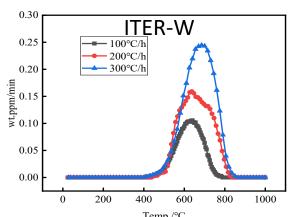


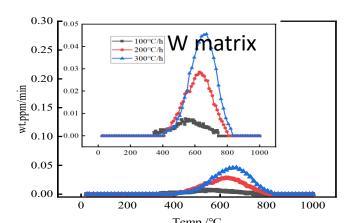


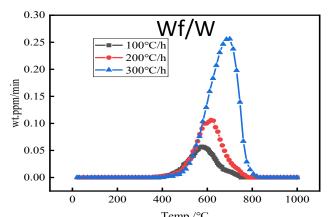
Analysis platform for the hydrogen isotopes



3.3 Plasma exposure-Hydrogen retention







• HTHP W has H retention of 1 ppm, which is 1/8 of ITER-W's. Wf/Wm has comparable H retention than ITER-W.

3.4 Plasma exposure- Helium ion irradiation

ITER W Wf/W



50keV He²⁺, total accumulated fluences 2*10¹⁷ ion/cm²

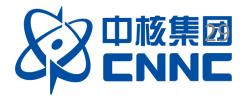
Sample	Fluences (ion/cm²)	He bubble size (nm)	He bubble density (/cm³)
Wf/W	2*10 ¹⁷	1.45	1.72*10 ¹³
ITER W	2*10 ¹⁷	1.21	1.45*10 ¹³

The helium bubble size and density of Wf/W samples are around 85% of ITER-grade tungsten.

4. Summary



- ➢ Injection moulding combined with high-pressure high-temperature sintering demonstrates potential for fabricating high-density three-dimensional Wf/W.
- The composite exhibits enhanced transient HHF resistance, attributed to its pseudo-ductile behavior enabled by interfacial debonding and sliding mechanisms.
- The superior irradiation resistance of tungsten fibers compared to the tungsten matrix suggests promising radiation tolerance in Wf/W under fusion-relevant conditions.
- ➤ Wf/W displays favorable hydrogen retention properties and outperforms ITER W in He²⁺ irradiation resistance, positioning it as a candidate material for plasma-facing components in future fusion reactors.



Thanks for listening!