

Thermal Quench Dynamics and Heat Flux Distribution during Massive-Impurity-Injection Triggered Disruption in EAST

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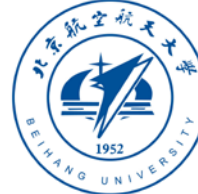
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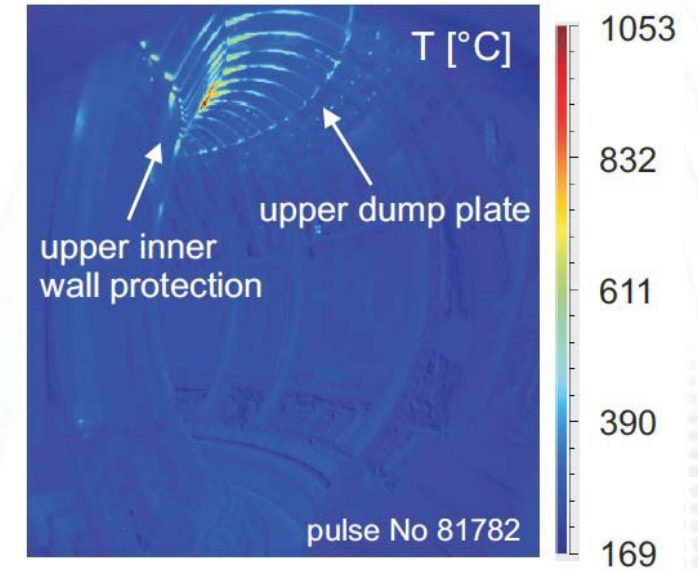
*See Hoelzl et al 2024 (<https://doi.org/10.1088/1741-4326/ad5a21>) for the JOEKE Team



Thermal quench poses serious threat to PFCs

- Thermal quench (TQ) process can cause unacceptable heat load on PFCs in ITER.
 - The stored thermal energy at TQ
 - The duration of TQ and deposition area on PFCs

Massive impurity injection could decrease the PFC energy deposition



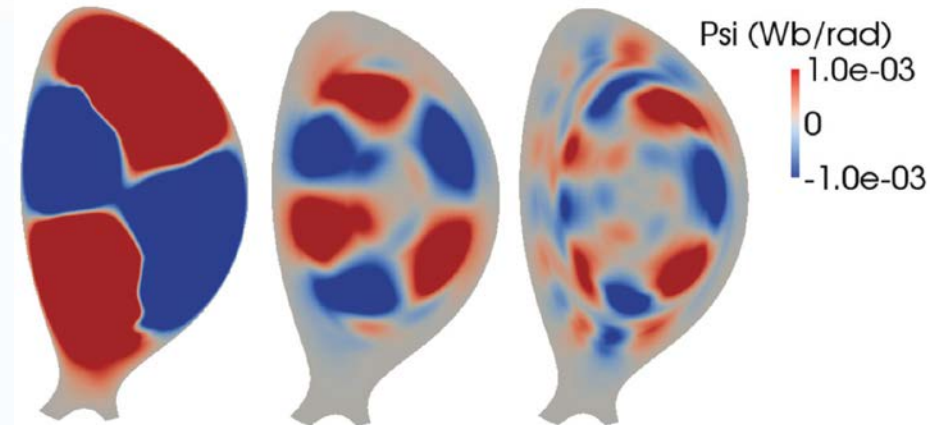
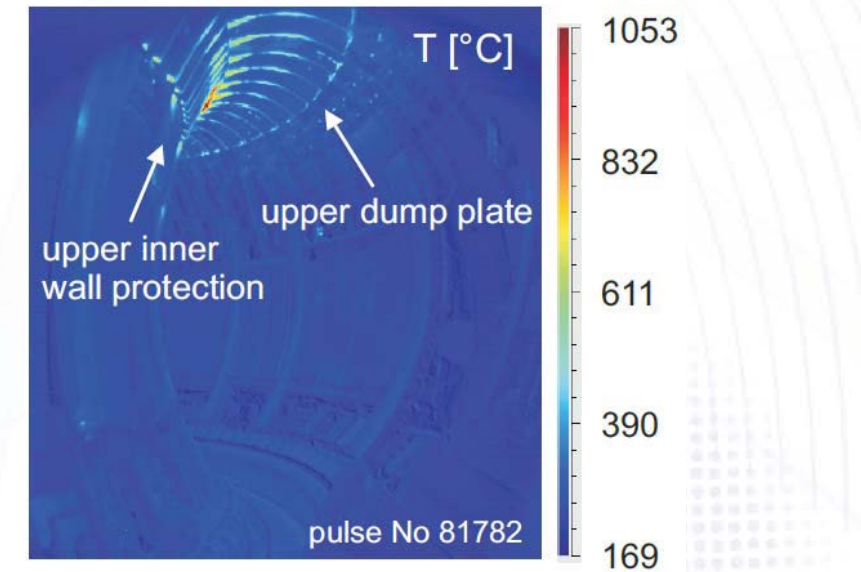
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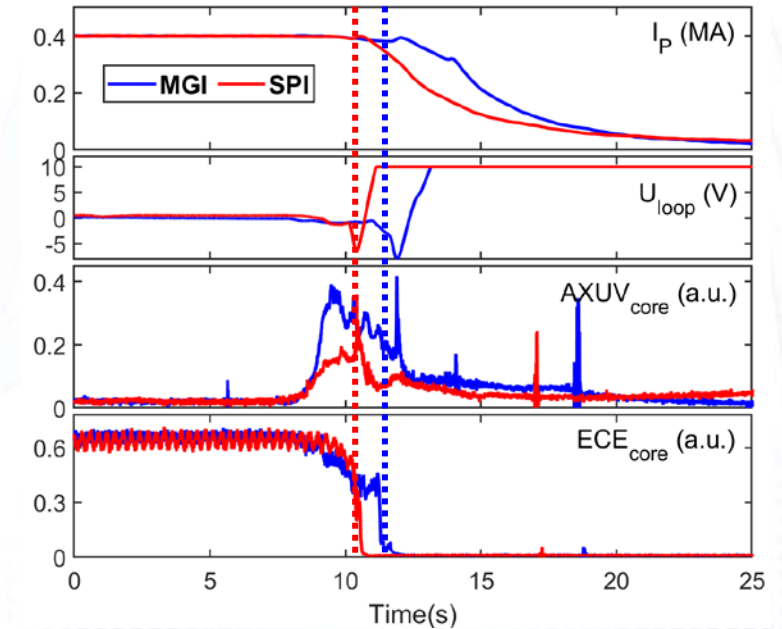
- Talk presents results for:
 - Impurity transport during massive-impurity-injection triggered disruptions
 - TQ process on EAST and its interpretive MHD modelling with JOEUK



Characteristics of disruption mitigation via SPI and MGI



- Both shattered pellet injection (SPI) and massive gas injection (MGI) technologies have been developed for disruption mitigation on EAST.
 - **SPI**: Ne, H₂+Ne; Pellet speed < 400 m/s.
 - **MGI**: Ar, Ne, He, D₂; Response time < 1 ms.

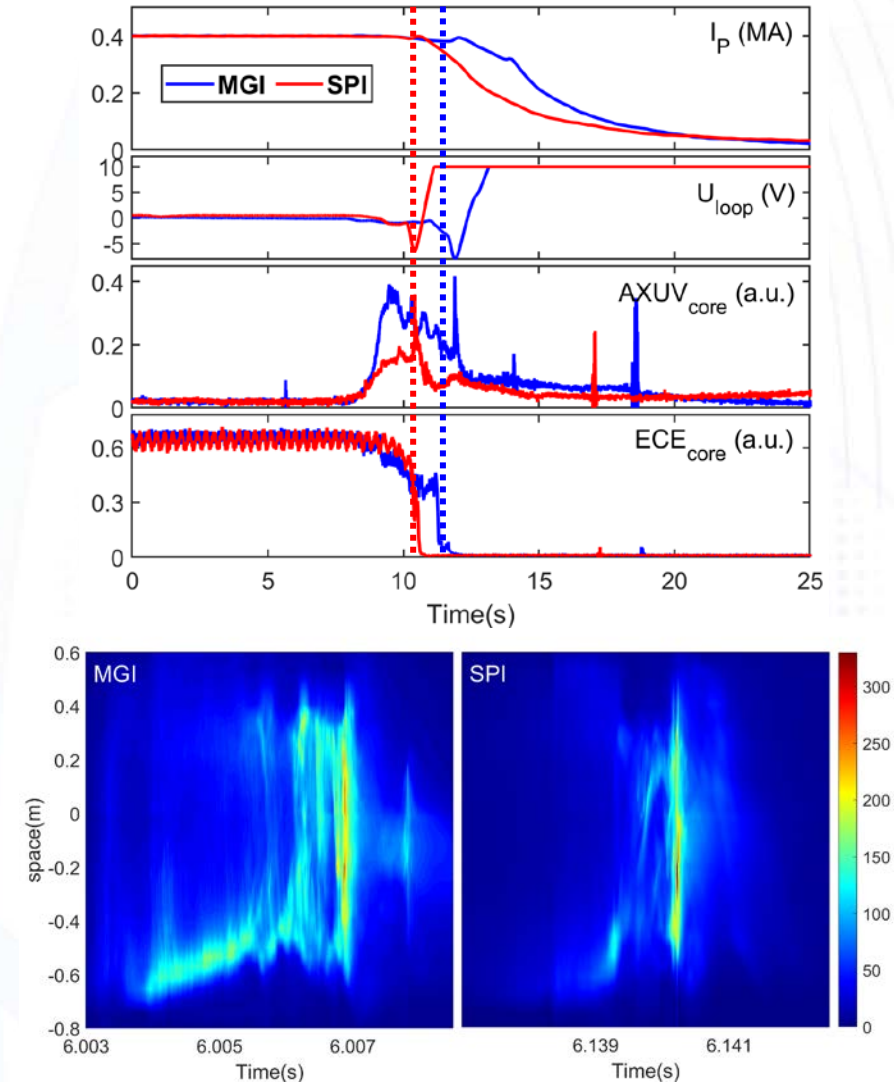


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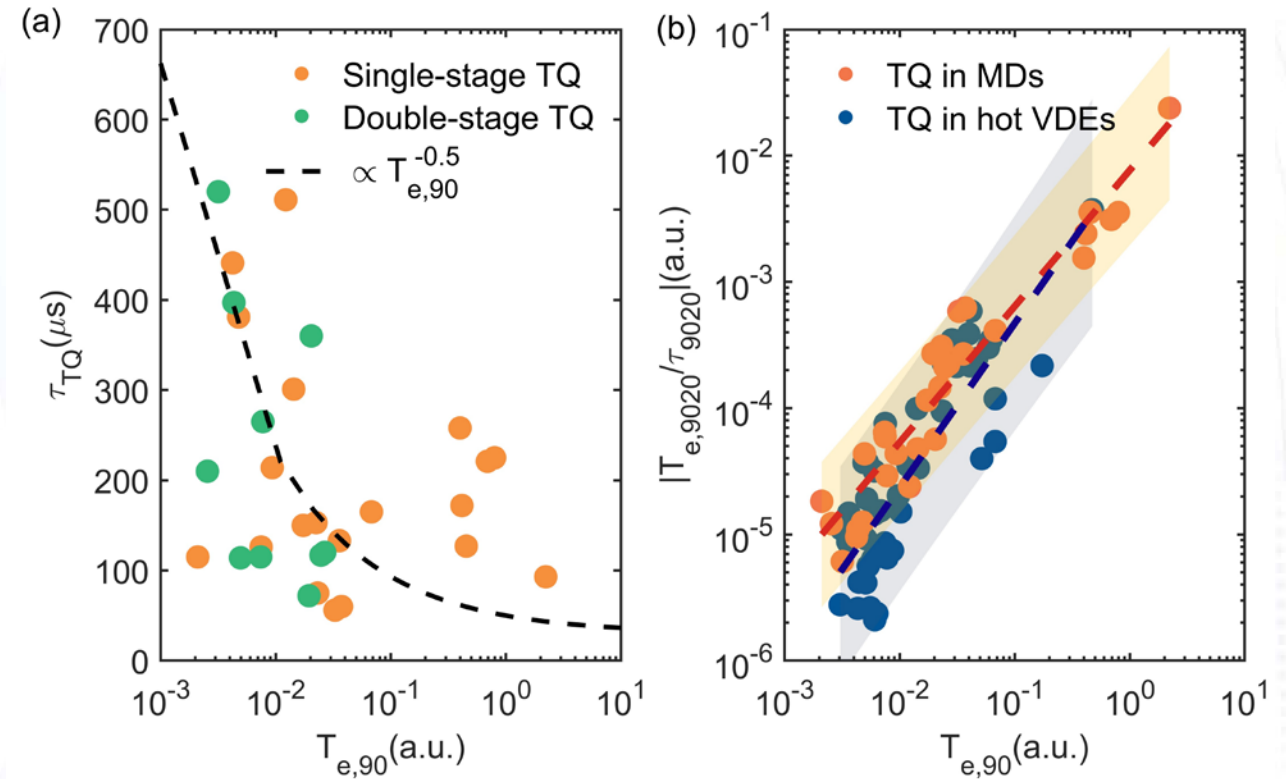
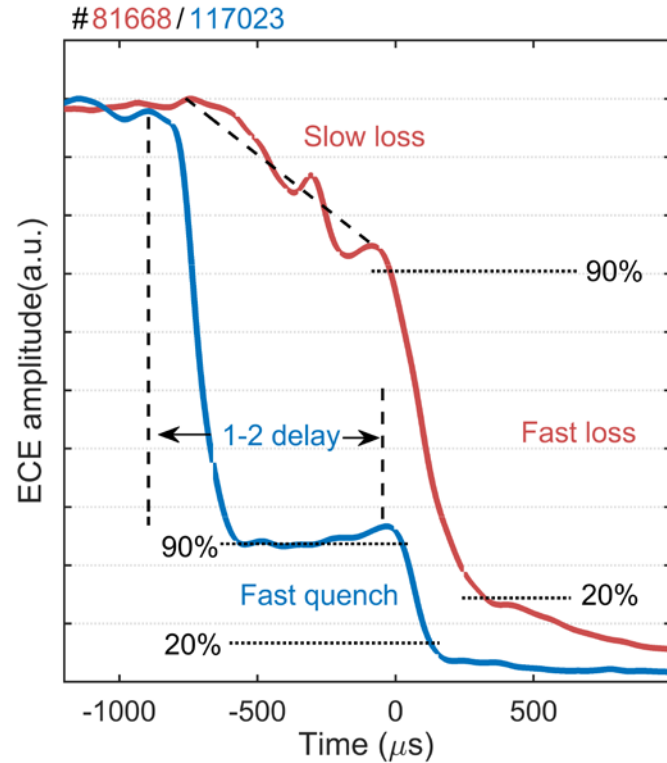


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 - SPI: Ne, H₂+Ne; Pellet speed < 400 m/s.
 - MGI: Ar, Ne, He, D₂; Response time < 1 ms.
- Intended disruption triggered by SPI and MGI
 - While MGI tends to deposit more impurities at the edge, SPI penetrates deeper into the plasma and results in a shorter overall disruption duration.
 - The impurity trajectory is highly asymmetric in both the toroidal and poloidal directions.

No clear difference in TQ process found



Typical TQ process in the EAST disruptions



- Single-stage TQ and double-stage TQ during disruptions.

- A large range at the low temperature and short TQ durations at high T_e , not fully consistent with $T_e^{-0.5}$.

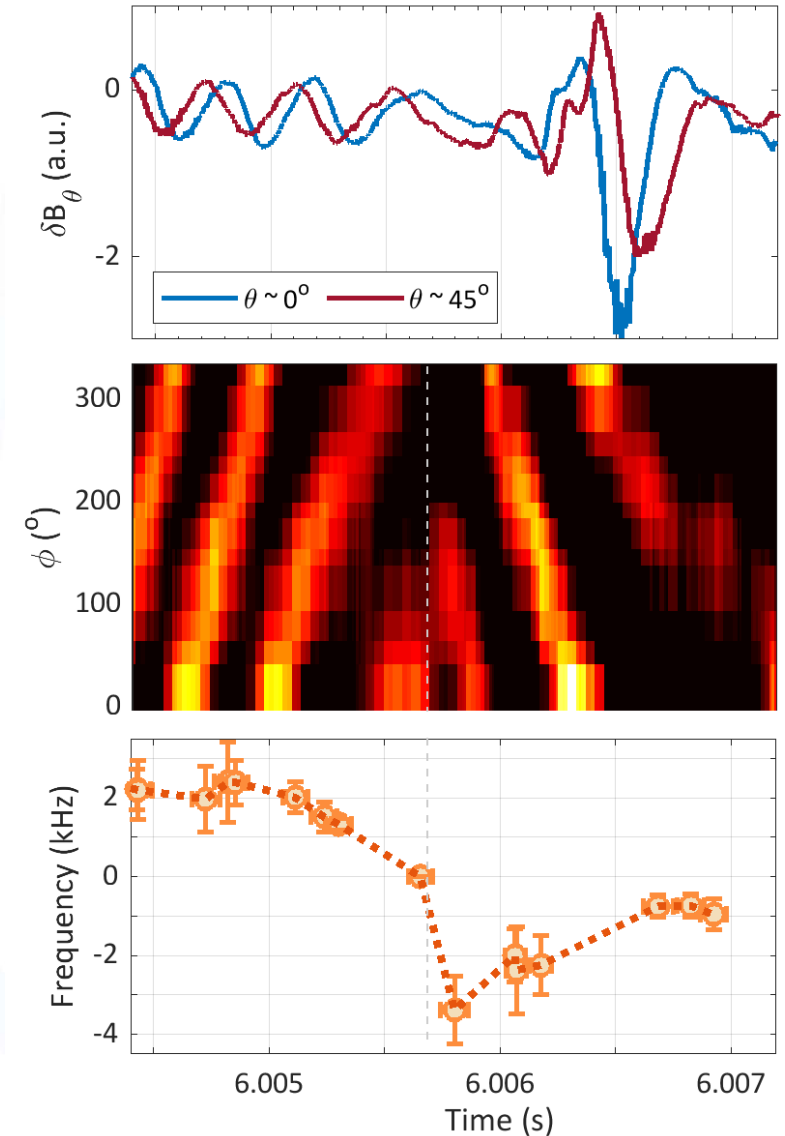
*TQ duration: The time intervals between 90%-20%

Impurity transport during the pre-TQ phase

Rotation direction of magnetic island observed to reverse in the pre-TQ phase



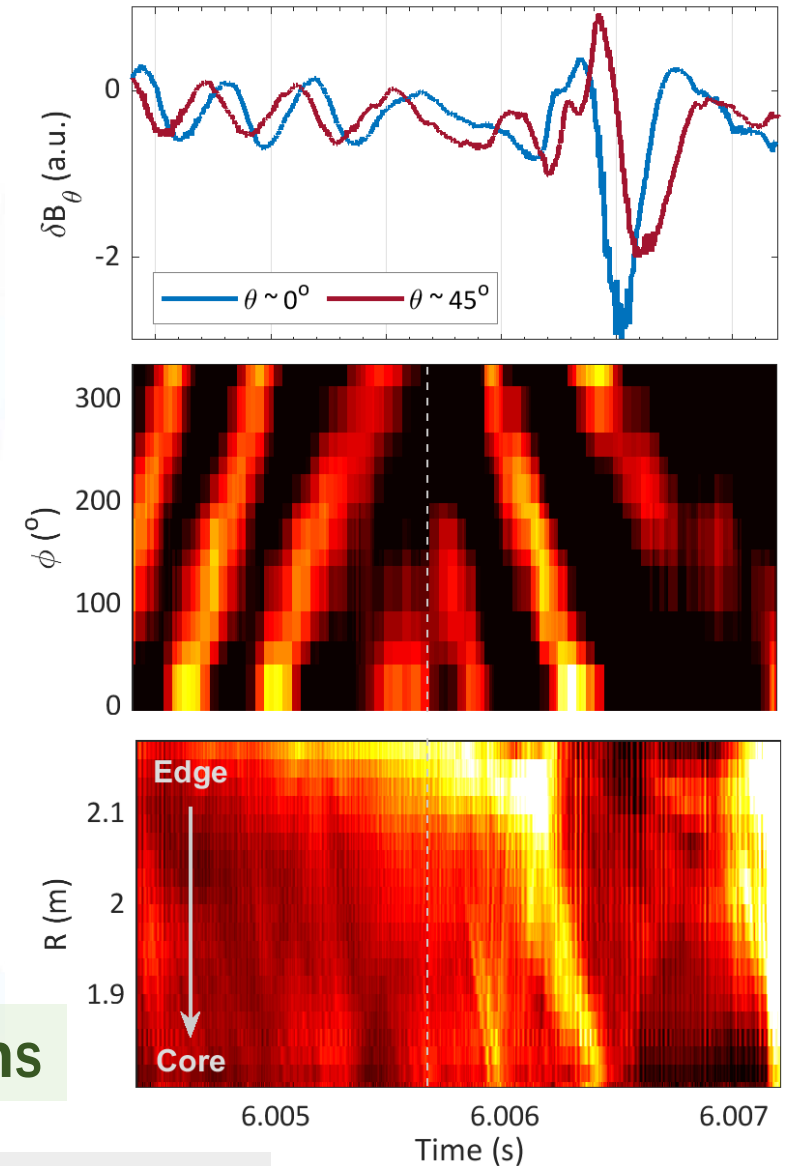
- The 2/1 mode's rotation slows down and then reverses from the ion diamagnetic to the electron diamagnetic direction after neon injection into co-current NBI-heating plasmas.
 - The mode frequency changes from **2 kHz to -3 kHz** in **< 0.5 ms**.



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 - The mode frequency changes from **2 kHz to -3 kHz** in **< 0.5 ms**.
- The radiation in the core region significantly increases following the rotation direction of the mode reversed.



Observed in both SPI and MGI triggered disruptions

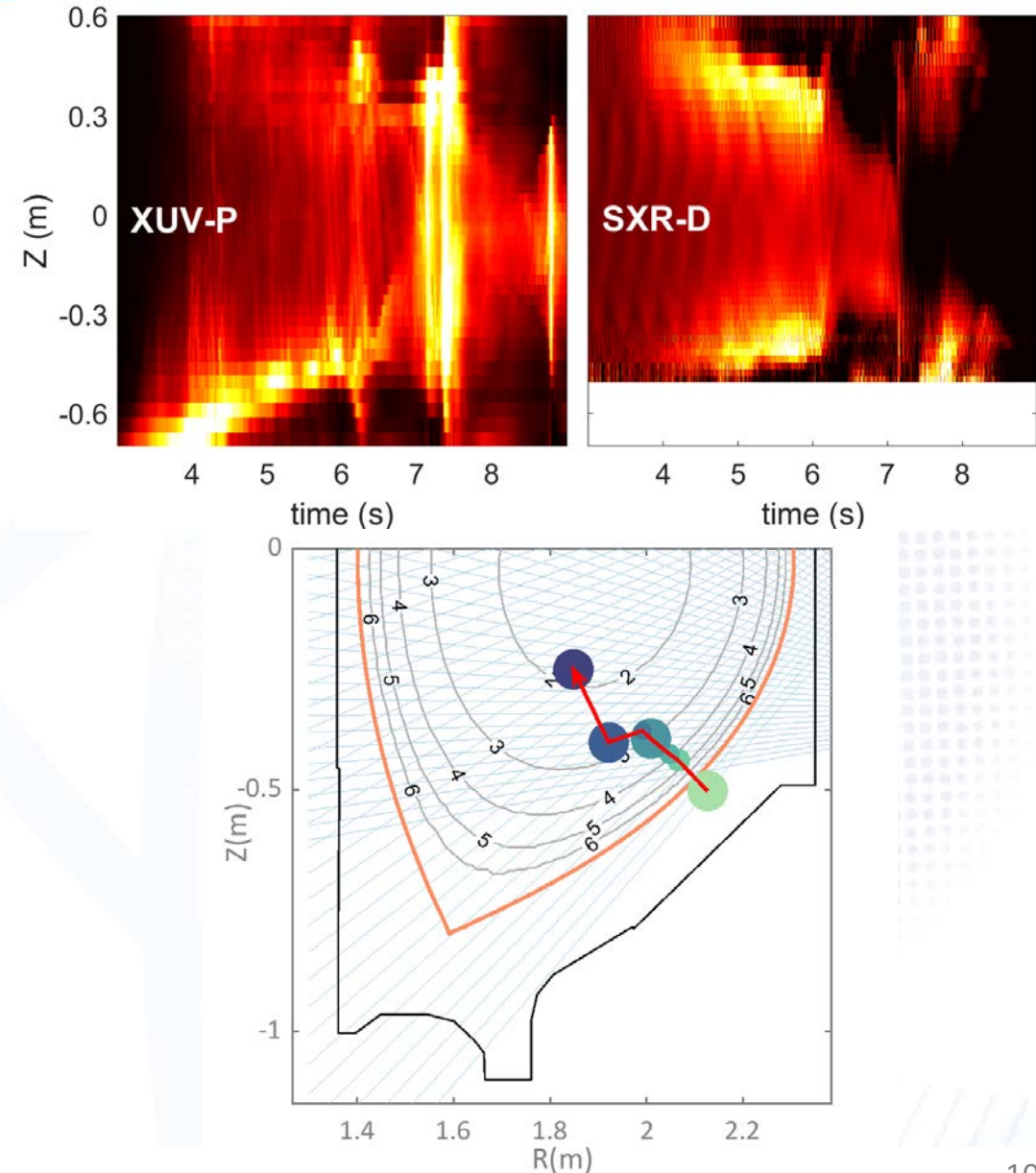
The core radiation enhanced following the rotation direction of the mode reversed



- The impurity influx divided into two stages according to the XUV and SXR detectors.
 - **STAGE 1:** impurities drift along ion diamagnetic direction; Outside of $q=2$ surface. *Inward pinch speed ~ 60 m/s.*

Rotation direction of the mode reversed

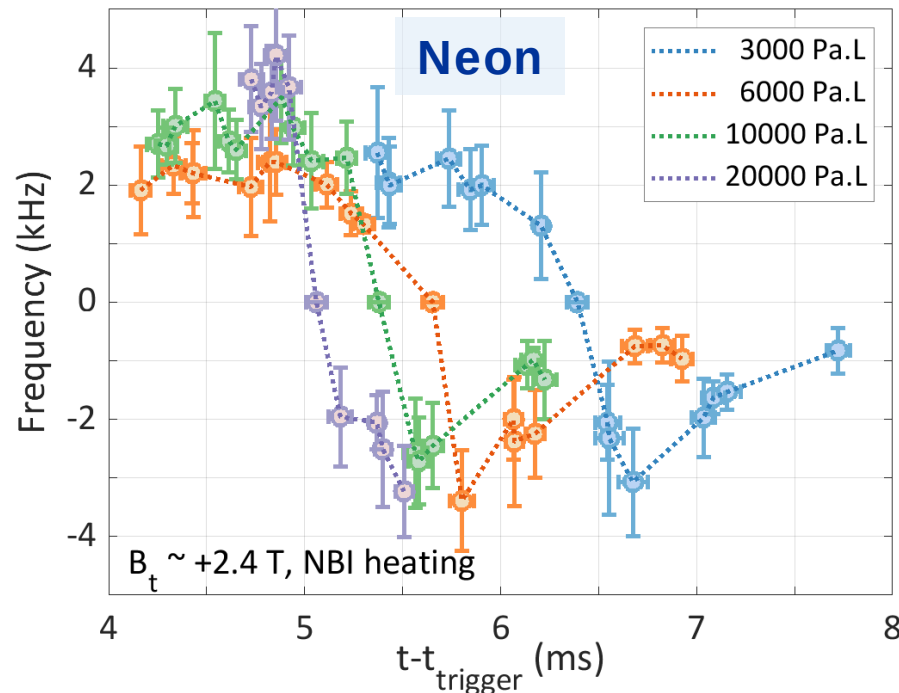
- **STAGE 2:** impurities drift along electron diamagnetic direction; Across $q=2$ surface. *Inward pinch speed ~ 300 m/s.*



Parameter dependence of mode-frequency change



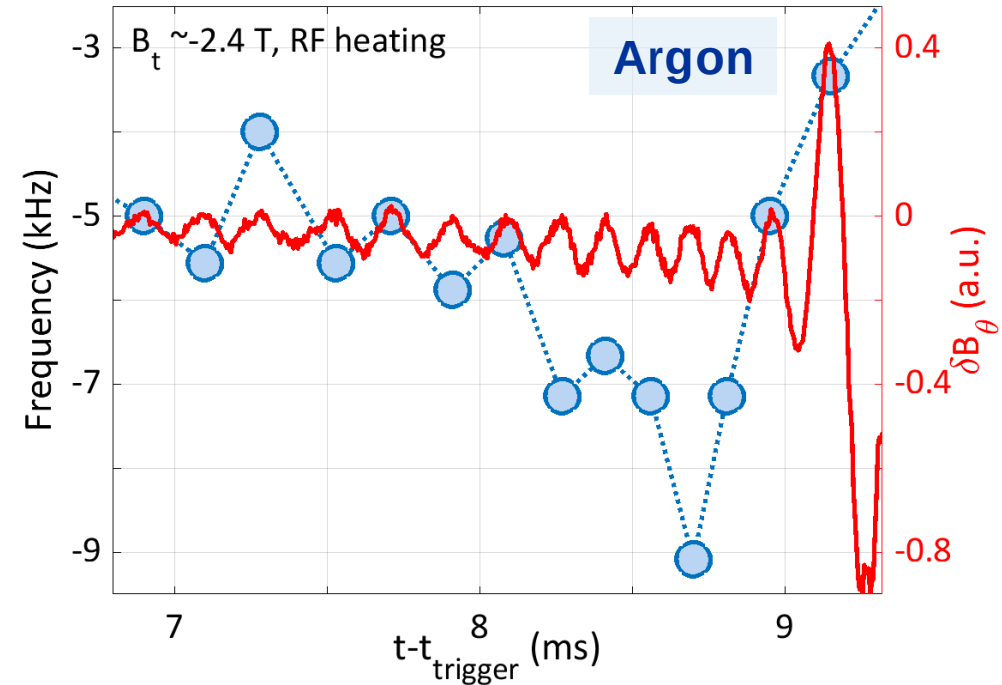
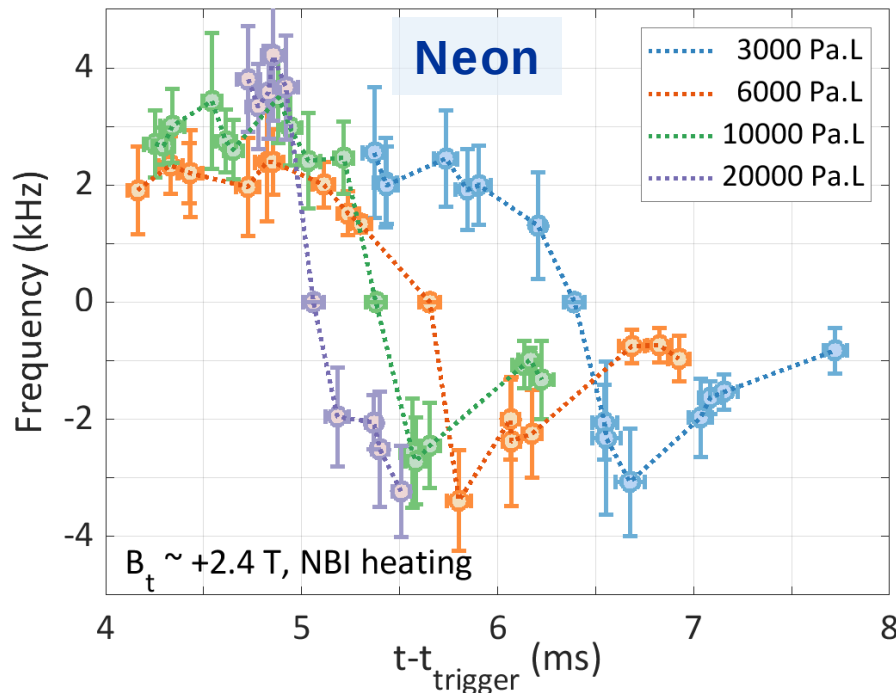
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 - *Amount scan: Earlier and faster reversion of rotation direction.*



Parameter dependence of mode-frequency change



- The phenomenon has been observed by scanning species and numbers of impurities, and heating methods.
 - Amount scan: Earlier and faster reversion of rotation direction.
 - The mode frequency changes from -5 kHz to -9 kHz in ~ 0.5 ms after argon injection into RF-heating plasmas.
 - The mode frequency after argon injection seems lower than after neon injection.



Change of mode frequency is mainly caused by poloidal angular frequency



- Frequency of (2,1) tearing mode is written as

$$2\pi \cdot f_{\text{MHD}} = \Omega_{E \times B} + \Omega_{*e} + 0.71\Omega_{*Te}$$

- $\Omega_{E \times B}$ can be determined by ion radial force balance equation.

$$\Omega_{E \times B} = U \cdot \nabla \zeta - q U \cdot \nabla \theta - \Omega_{*i}$$

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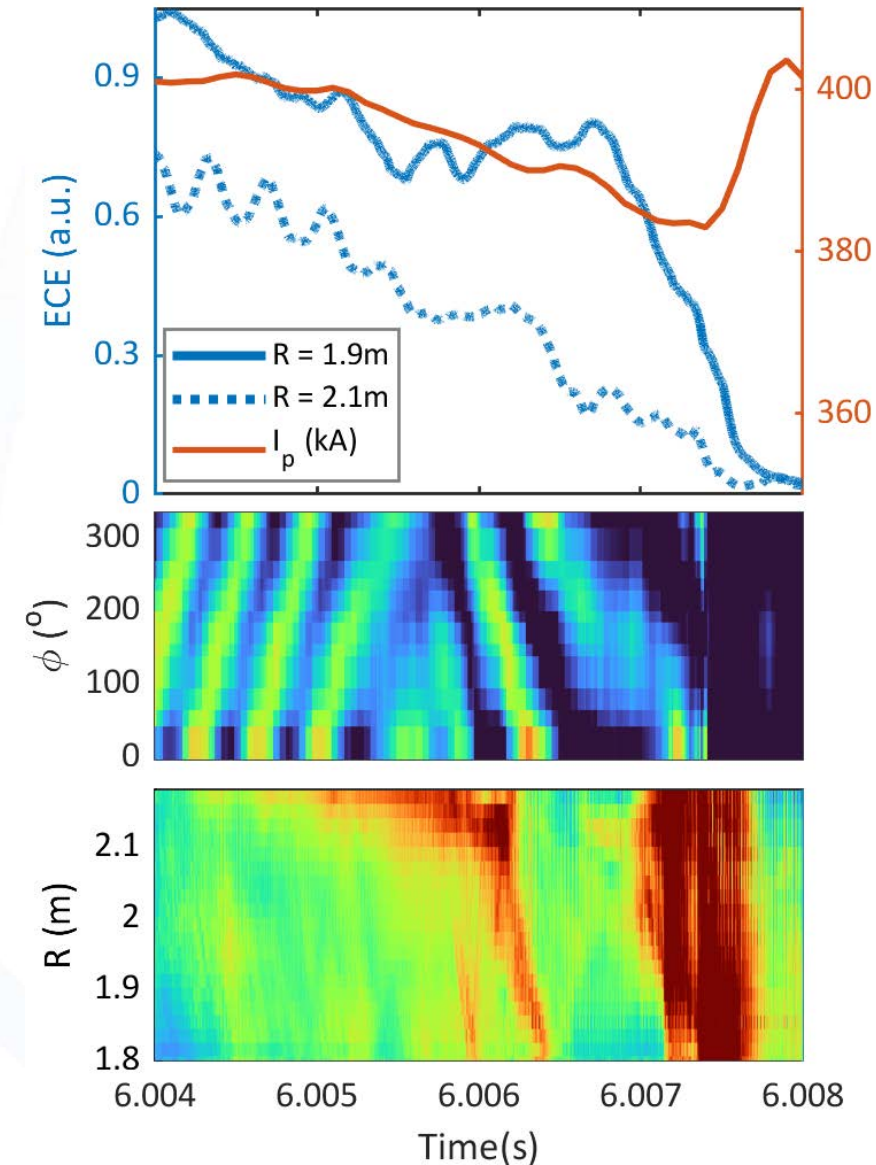
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$$\Omega_{E \times B} = U \cdot \nabla \zeta - q U \cdot \nabla \theta - \Omega_{*i}$$

- The reversed time scale (< 0.5 ms) is much less than energy confinement time and momentum transport time, so the change of mode frequency is mainly caused by poloidal angular frequency.

Poloidal rotation velocity

$$\delta U_{\theta} = -\frac{B_T}{B^2} \delta E_r$$



The neoclassical theory is applied to understand the phenomenon



- Time scale of frequency change (< 0.5 ms) is much less than τ_E (50-100 ms) on EAST.
 - *The neoclassical relaxation time in the collisional regime $\tau_\theta < 0.5$ ms ($\tau_\theta \simeq q^2 R^2 \nu_i / v_{Ti}^2$).*

The neoclassical theory ☺

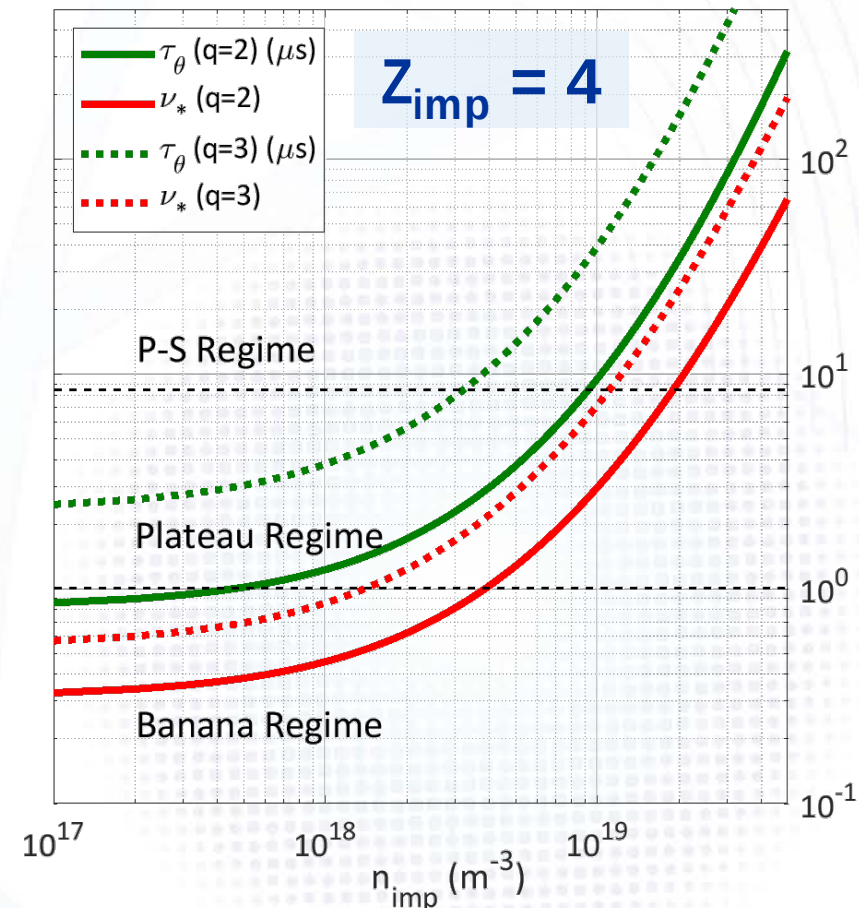
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The neoclassical theory ☺

- The poloidal rotation can be driven by the viscosity associated with the collisionality and impurity.
 - *In standard neoclassical theory, the poloidal component of the perpendicular $E \times B$ drift is damped by the poloidal (parallel) viscosity.*



A model for poloidal rotation during the transient state



- The parallel momentum and heat flow balance equations for the impurity and ion species in the form of matrix equations are as followed.
 - particle and heat flow of impurity, radial electric field, and ion heat flow included.*

$$\begin{bmatrix} \frac{n_I m_I \tau_{ii}}{n_i m_i \tau_{II}} \mu_{00}^I + \alpha & \frac{n_I m_I \tau_{ii}}{n_i m_i \tau_{II}} \mu_{01}^I & \alpha \frac{B_T}{B_p} & -1.5\alpha \\ \frac{n_I m_I \tau_{ii}}{n_i m_i \tau_{II}} \mu_{10}^I & \frac{n_I m_I \tau_{ii}}{n_i m_i \tau_{II}} \mu_{11}^I + \alpha \frac{T_i}{T_I} (7.5 + \sqrt{2} \alpha x^{-1}) & 0 & -6.75 \alpha x^2 \\ -\alpha & 0 & -\mu_{00}^i \frac{B_T}{B_p} - \alpha \frac{B_T}{B_p} & \mu_{01}^i + 1.5\alpha \\ -1.5\alpha & -6.75 \frac{T_i}{T_I} \alpha x^2 & -\mu_{10}^i \frac{B_T}{B_p} - 1.5\alpha \frac{B_T}{B_p} & \mu_{11}^i + (\sqrt{2} + 3.25\alpha) \end{bmatrix} \begin{bmatrix} u_{0\theta}^I B^2 \\ u_{1\theta}^I B^2 \\ E_r \\ u_{1\theta}^i B^2 \end{bmatrix}$$

$\alpha \equiv n_I Z_I^2 / n_i Z_i^2$
(Impurity strength parameter)

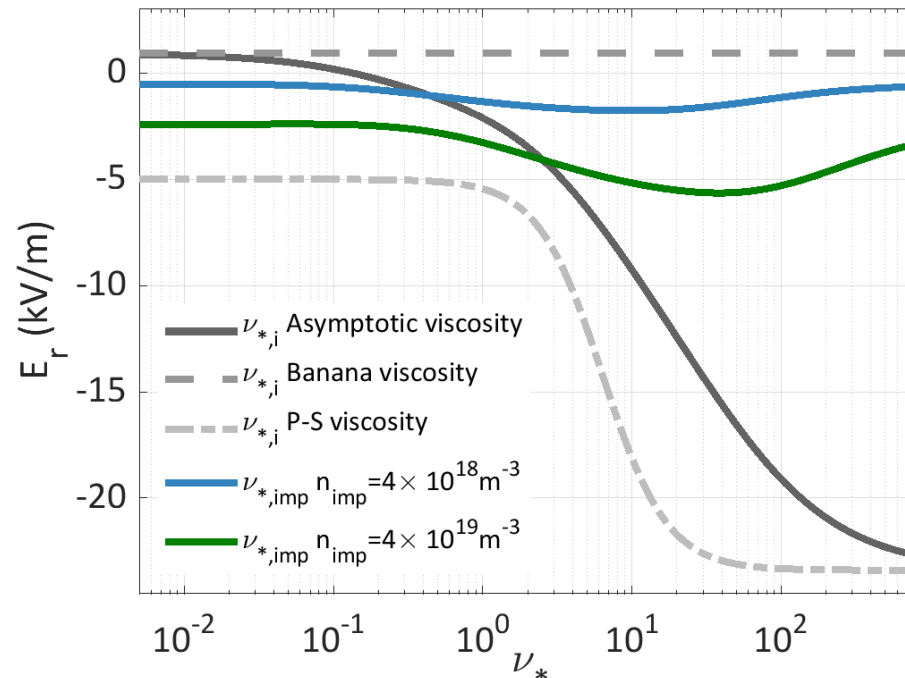
$x \equiv V_{th,I} / V_{th,i} \ll 1$

$$= \begin{bmatrix} \alpha \frac{B_T T_I}{e_I B_p L_{pI}} + \alpha U_{0\parallel}^i B + 1.5\alpha \frac{B_T T_i}{B_p e_i L_{Ti}} \\ \alpha \frac{T_i}{T_I} (7.5 + \sqrt{2} \alpha x^{-1}) \frac{B_T T_I}{B_p e_i L_{Ti}} + 6.75 \alpha x^2 \frac{B_T T_i}{B_p e_i L_{Ti}} \\ -\mu_{00}^i U_{0\parallel}^i B - \mu_{00}^i \frac{B_T T_i}{e_i B_p L_{pi}} - \alpha U_{0\parallel}^i B - 1.5\alpha \frac{B_T T_i}{B_p e_i L_{Ti}} - \alpha \frac{B_T T_I}{B_p e_i L_{Ti}} \\ -\mu_{10}^i U_{0\parallel}^i B - \mu_{10}^i \frac{B_T T_i}{B_p e_i L_{pi}} - 1.5\alpha U_{0\parallel}^i B - (\sqrt{2} + 3.25\alpha) \frac{B_T T_i}{B_p e_i L_{Ti}} - 1.5\alpha \frac{B_T T_I}{e_I B_p L_{pI}} + 6.75 \frac{T_i}{T_I} \alpha x^2 \frac{B_T T_I}{B_p e_i L_{Ti}} \end{bmatrix}$$

Effect of collisionality and impurity on radial electric field



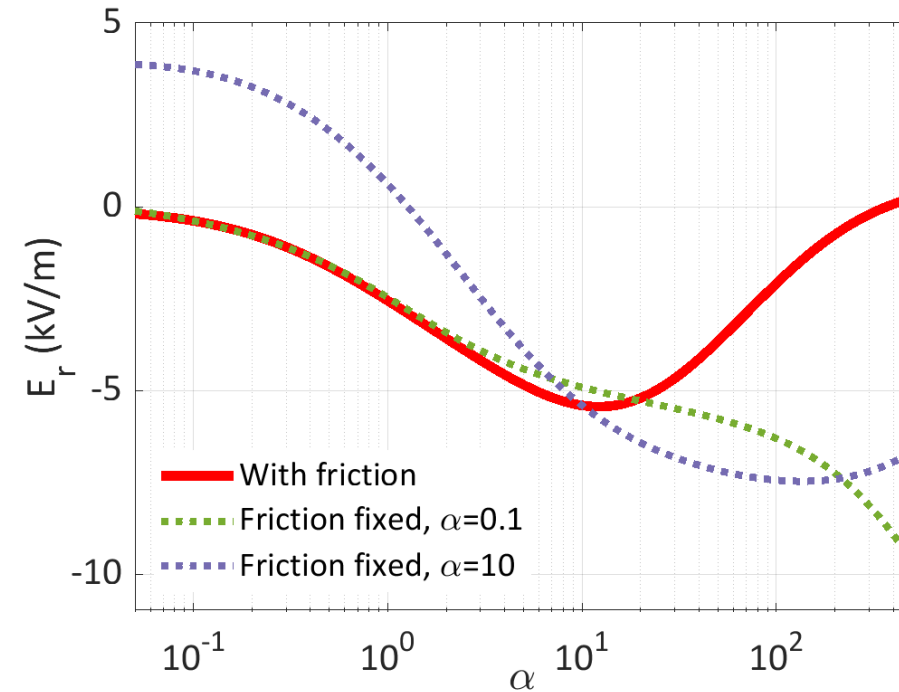
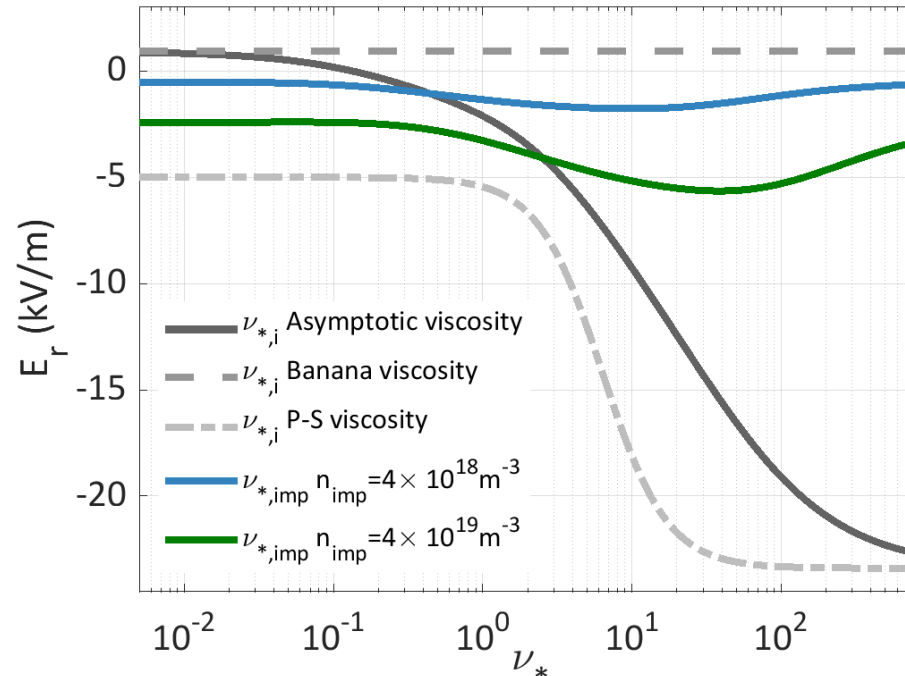
- A clear change in radial electric field is observed as collisionality transitions from the banana to the P-S regime.
 - As the ion collisionality increases, the radial electric field decrease nonlinearly.



Effect of collisionality and impurity on radial electric field



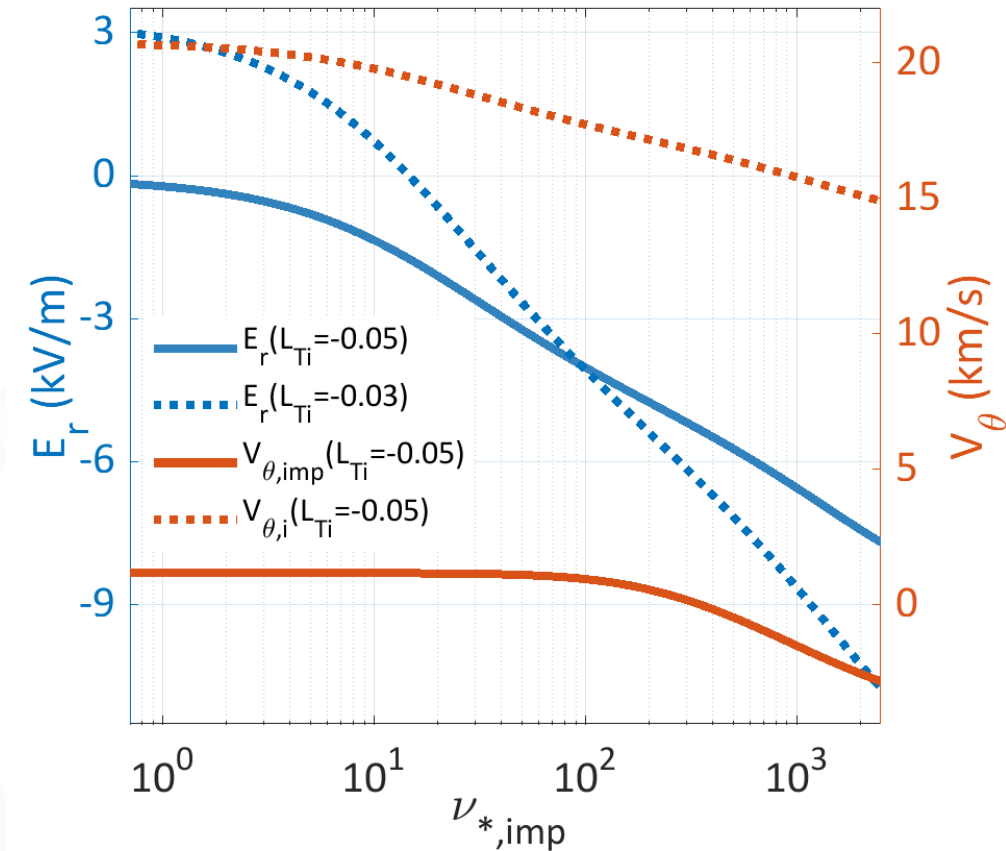
- A clear change in radial electric field is observed as collisionality transitions from the banana to the P-S regime.
 - As the ion collisionality increases, the radial electric field decrease nonlinearly.
 - The viscosity is sensitive to impurity strength: impurity orbits perturb ion orbits and thereby alter the effective viscosity.



E_r estimated from the model is consistent with the experimental value



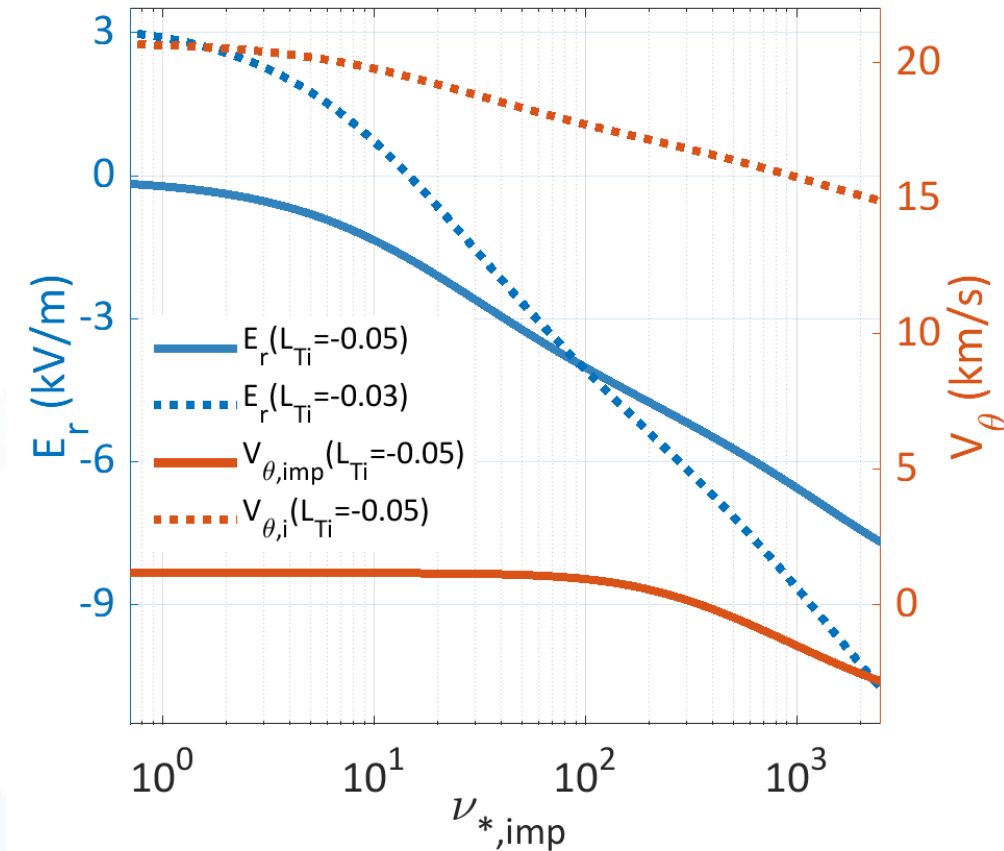
- Global effects with massive impurity injection show that:
 - The ion poloidal velocity is directed toward the ion-diamagnetic side, whereas the impurity poloidal velocity is directed toward the electron-diamagnetic side.
 - Variations in the ion poloidal flow are governed by the $E \times B$ drift.
 - The magnitude of the radial electric field increases as the ion-temperature gradient steepens.



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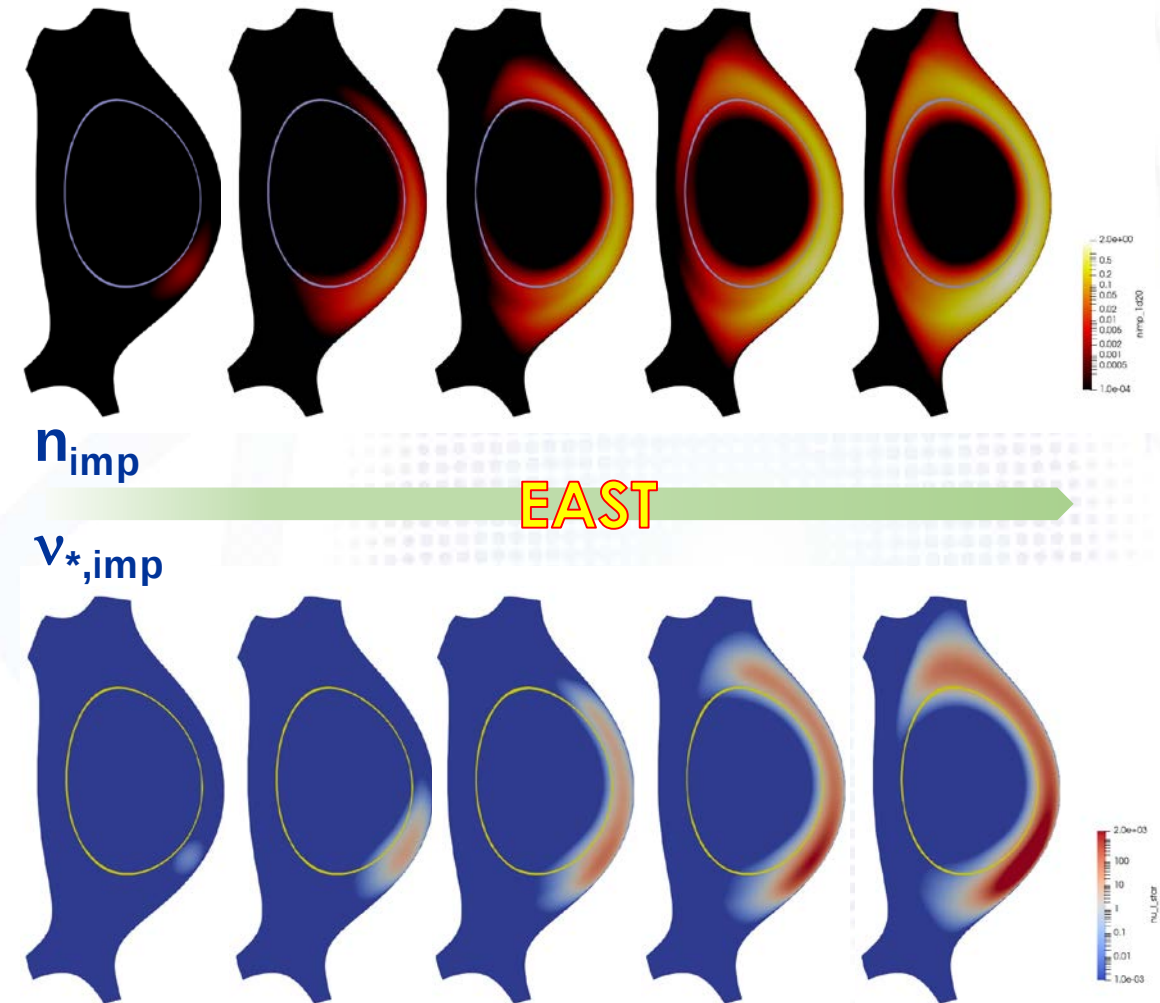
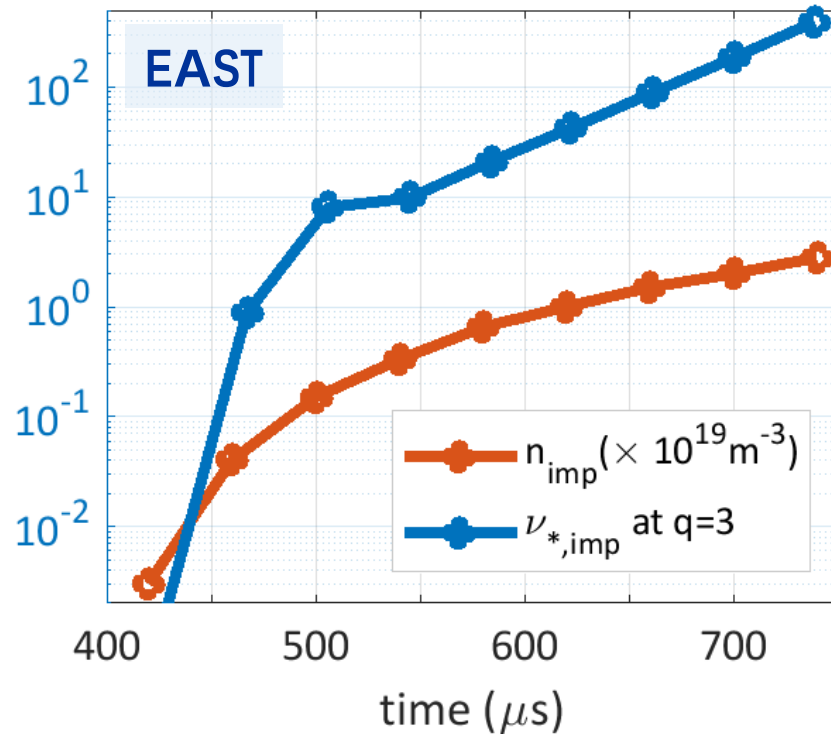


E_r is estimated in the range from -12 kV/m to -8 kV/m according to the typical parameters, consistent with the experimental value of ~ -10 kV/m.

Evolution of plasma parameters presents clear change in collisionality and amount of impurity



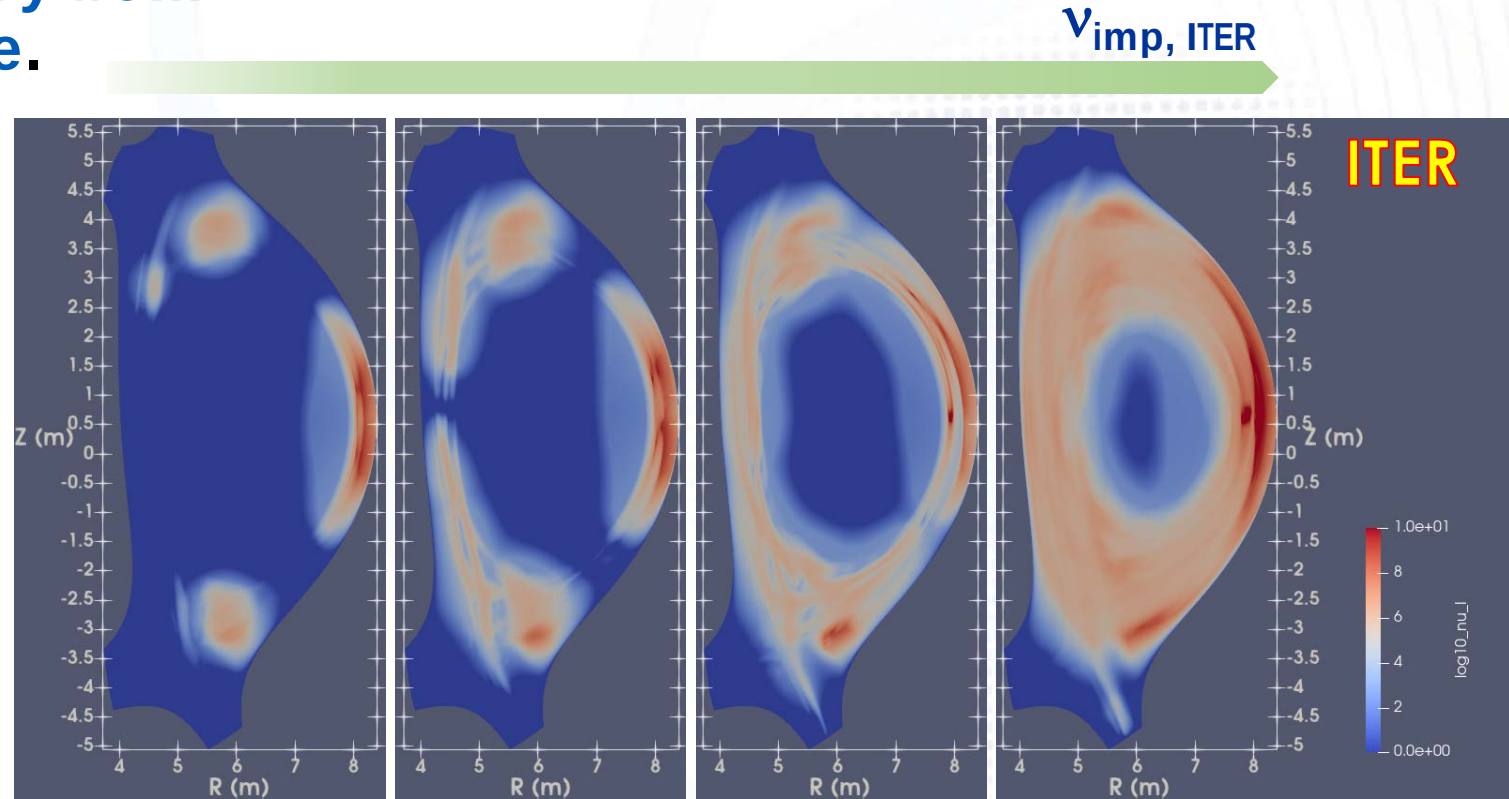
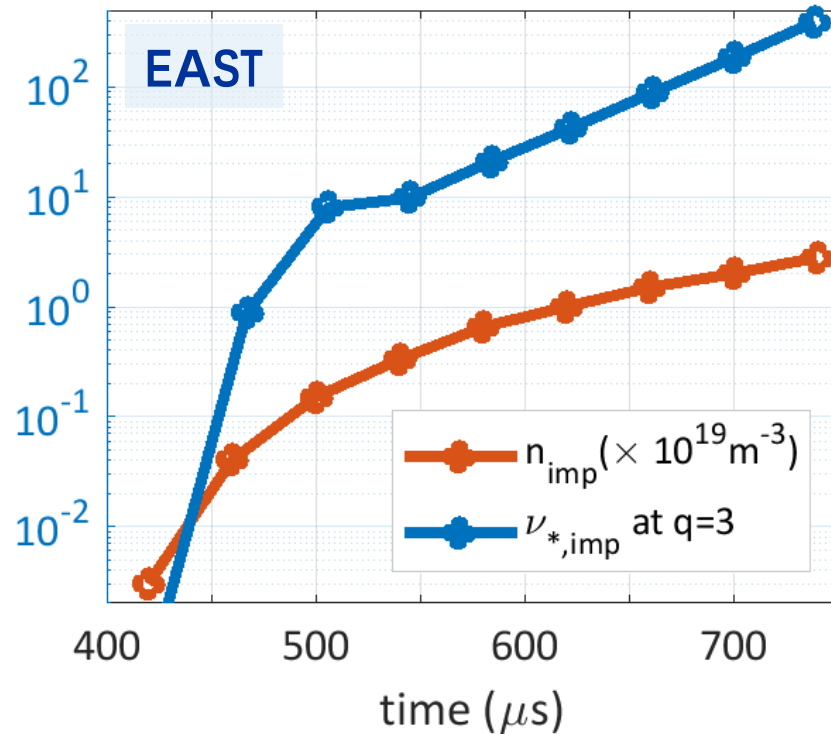
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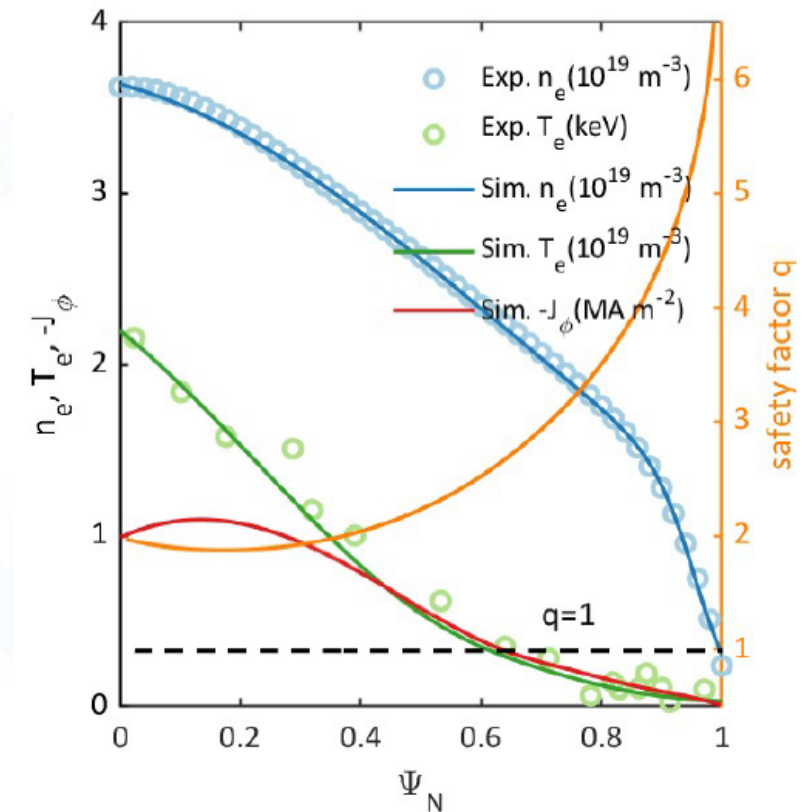


Interpretive JOREK simulation of TQ triggered by MGI

Simulation setup for disruptions by JOREK



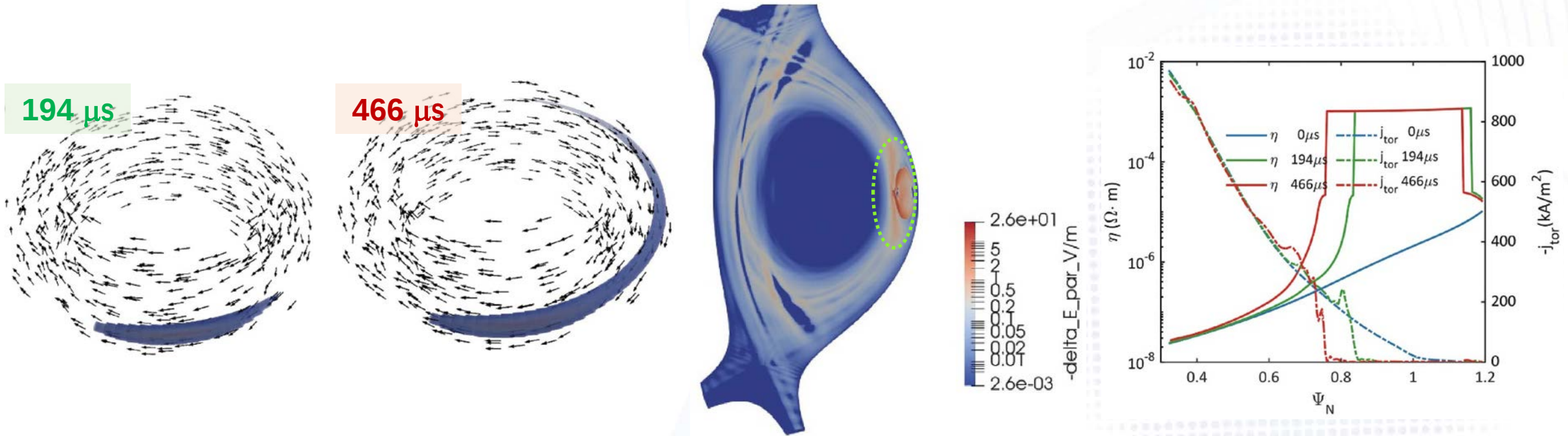
- Models: **single-fluid reduced nonlinear MHD model** with **non-equilibrium impurity treatment**.
 - The perpendicular diffusivity are chosen to keep the profile approximately constant in the simulation.
- Simulation setup: A typical L-mode plasma with $I_p \sim 0.4$ MA and $q_{\min} \sim 1.6$.
 - The initial core T_e is ~ 2.2 keV, electron and ion temperatures are assumed to be equal, and the core n_e is $\sim 3.6 \times 10^{19} \text{ m}^{-3}$.
 - MGI triggered.



Impurity trajectory in toroidal direction



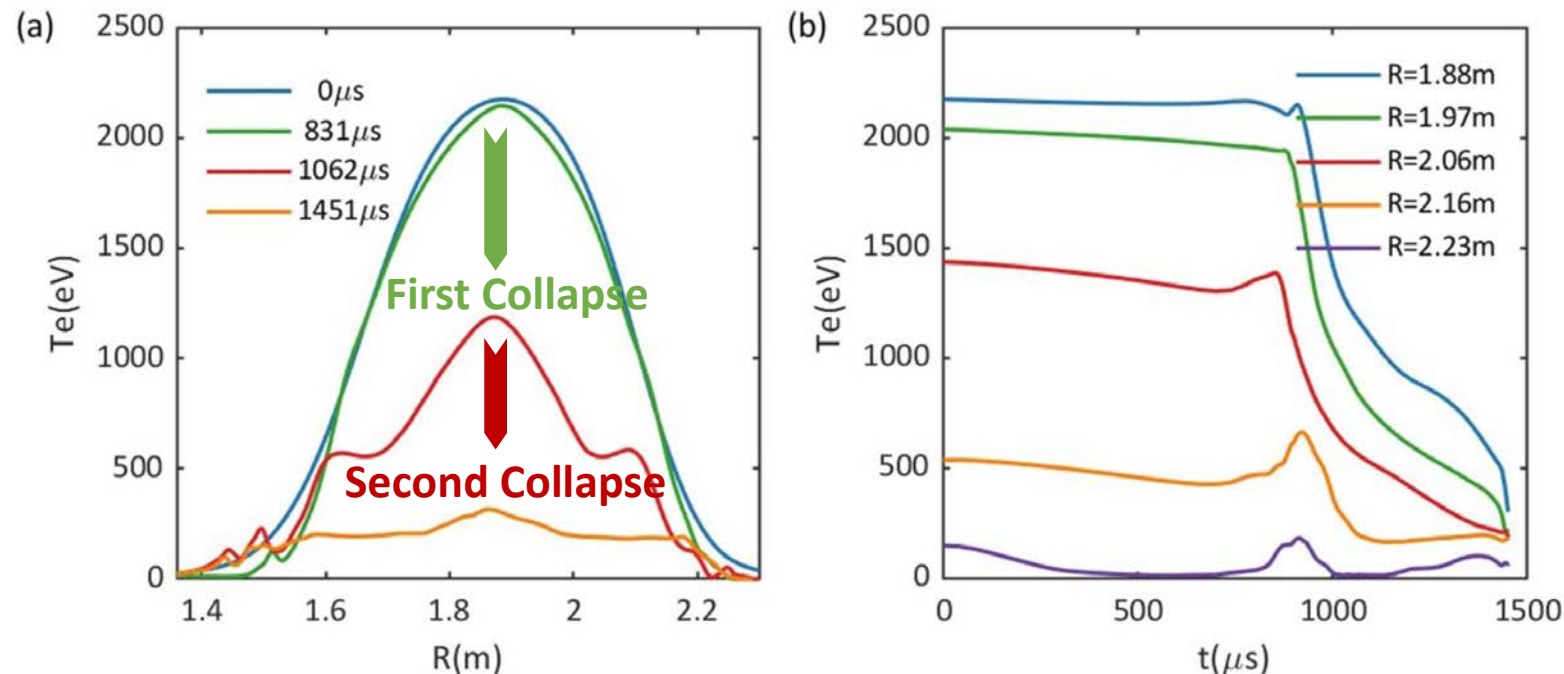
- Motion asymmetrically in toroidal direction
 - It is driven by a *parallel electric field* $E_{||}$ of approximately 30 V/m.



Current relaxation time > Resistivity increase time

Te evolution during typical double-stage TQ

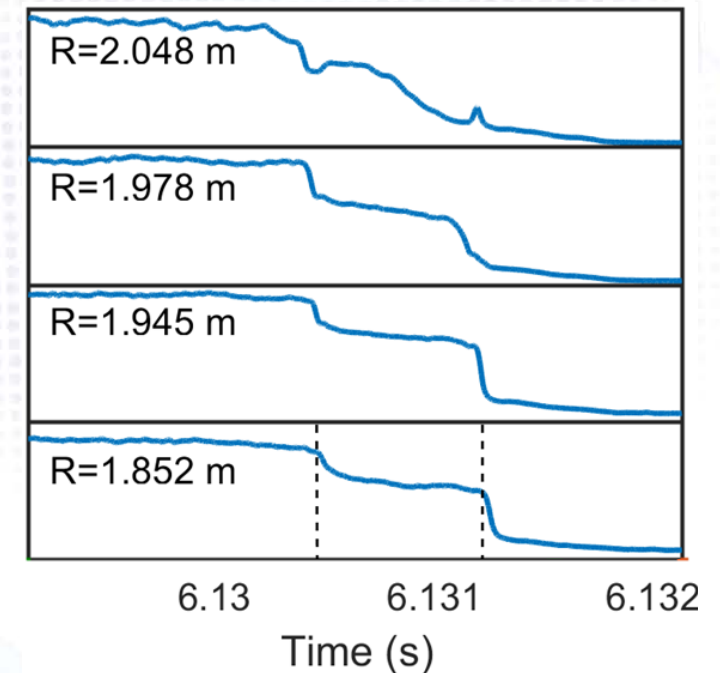
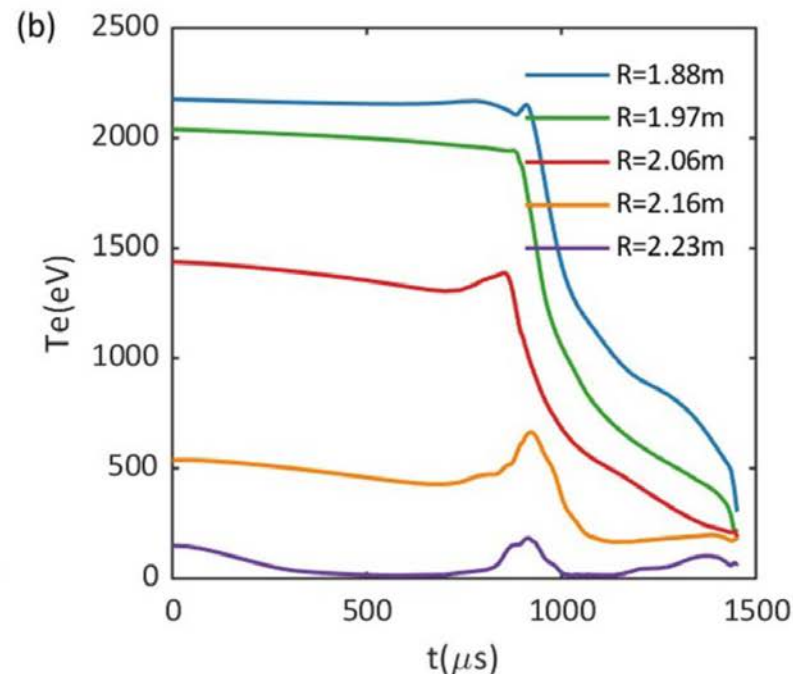
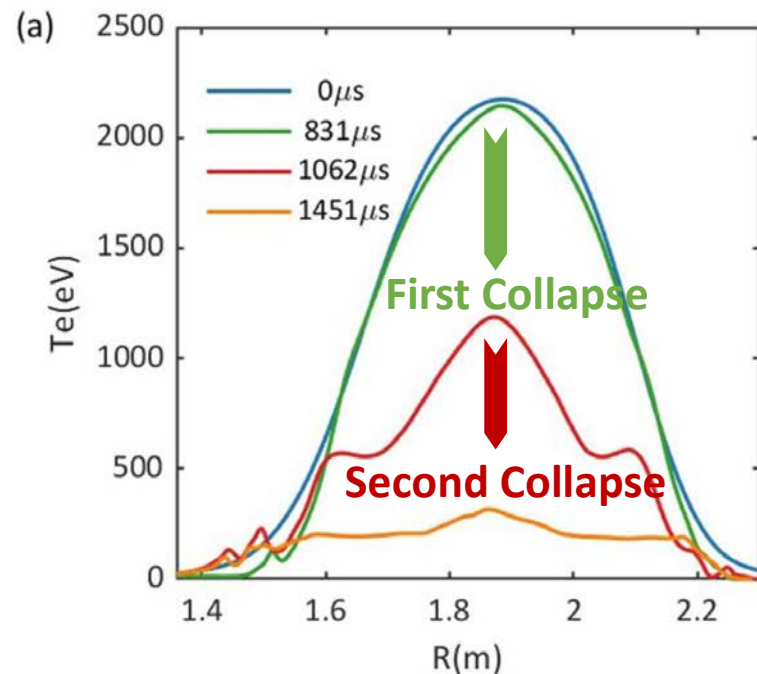
- Two distinct temperature collapses occur at $t_1 = 850 \mu\text{s}$ and $t_2 = 1400 \mu\text{s}$.
 - First collapse:** The edge temperature collapses firstly, accompanied with core thermal energy loss and the formation of a temperature plateau.



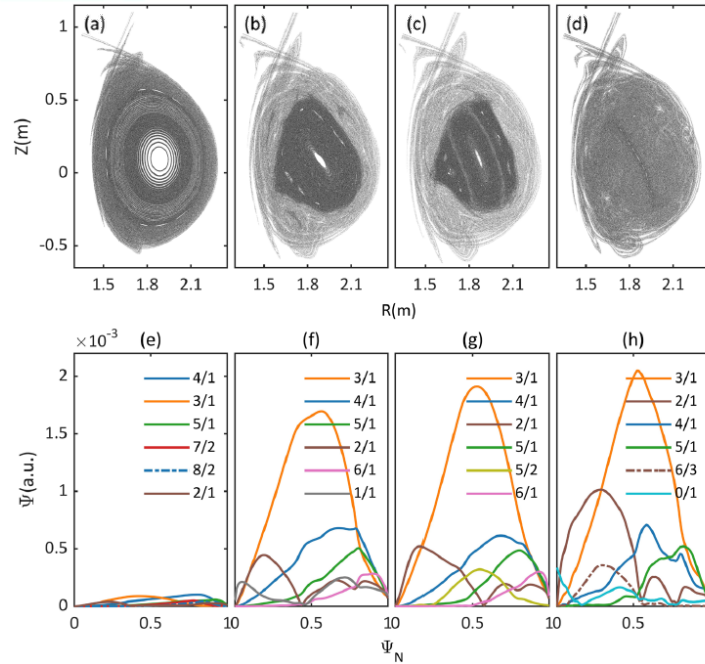
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 - consistent with the experimental observation.



MHD activities during typical double-stage TQ



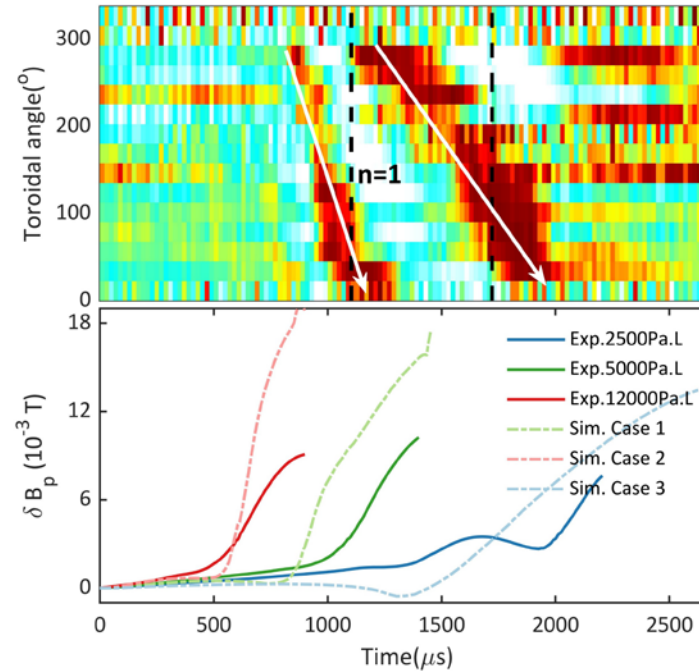
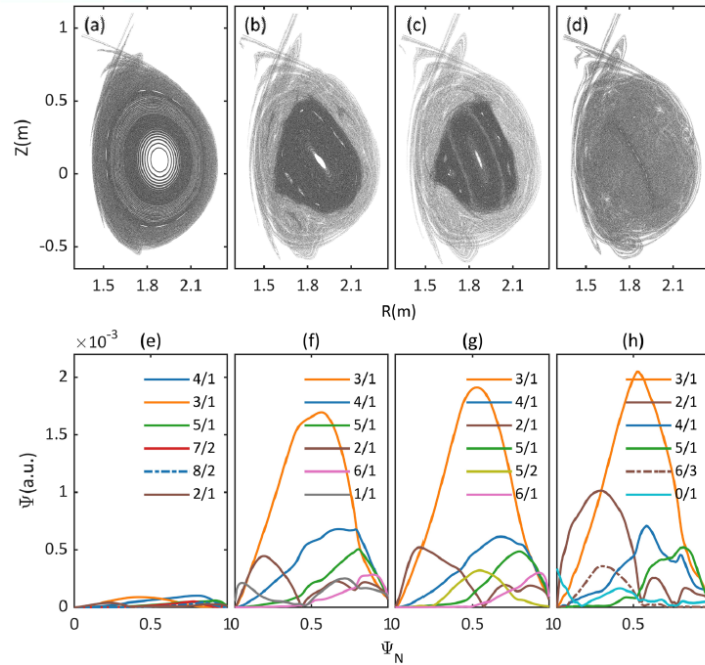
- **1-2 Delay stage**

- The growth of the 2/1 island results in a decrease in core temperature. **The 3/1 mode couples with the 4/1 and 5/1 modes, contributing to edge stochasticity.**

- **Second collapse**

- Non-linear interaction between **the 3/1 and 2/1 modes** primarily leads to **global stochasticity.**

MHD activities during typical double-stage TQ



The **n=1 mode** grows significantly before the TQ, consistent with the simulation results.

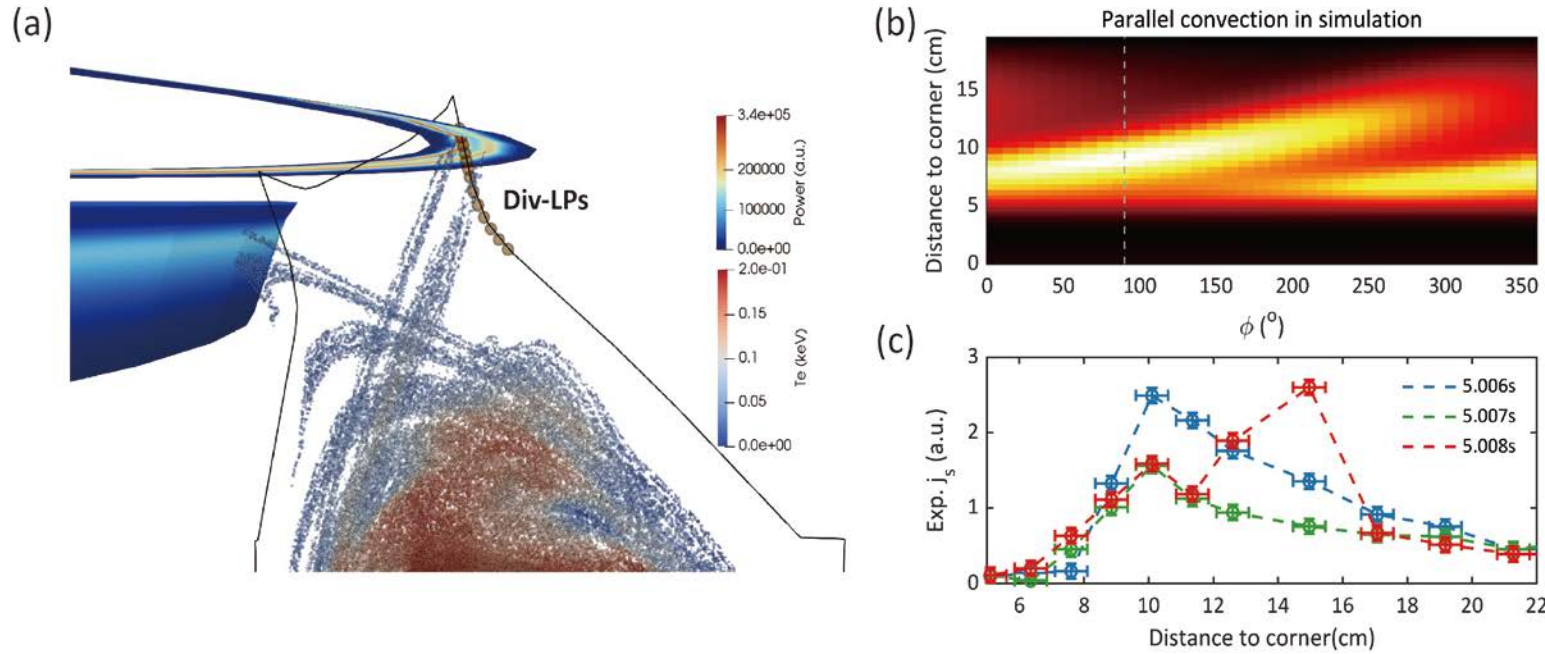
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Strike point splitting on the divertor during the TQ



Saturated ion flux measurement obtained by Langmuir probes consistent with simulation results

- The simulated parallel convective particle flux at the upper-outer target, combined with **the strike-point splitting, exhibits an $n = 1$ structure.**
 - This result provides *an explanation for the broadening of energy deposition width during the TQ in ASDEX Upgrade and JET.*

- Thermal quench dynamics in massive-impurity-injection triggered EAST disruptions have been demonstrated in experiments and simulations.
 - ✓ **Sudden change in mode frequency:** *A sudden change of the radial electric field (E_r), caused by collisionality transition from the banana to the P-S regime, can drive the poloidal rotation and enhance impurity influx.*
 - ✓ **Double-stage TQ:** *The 3/1 mode couples with the 4/1 and 5/1 modes, contributing to edge stochasticity. The non-linear interaction between the 3/1 and 2/1 modes primarily leads to global stochastic.*
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Thanks!

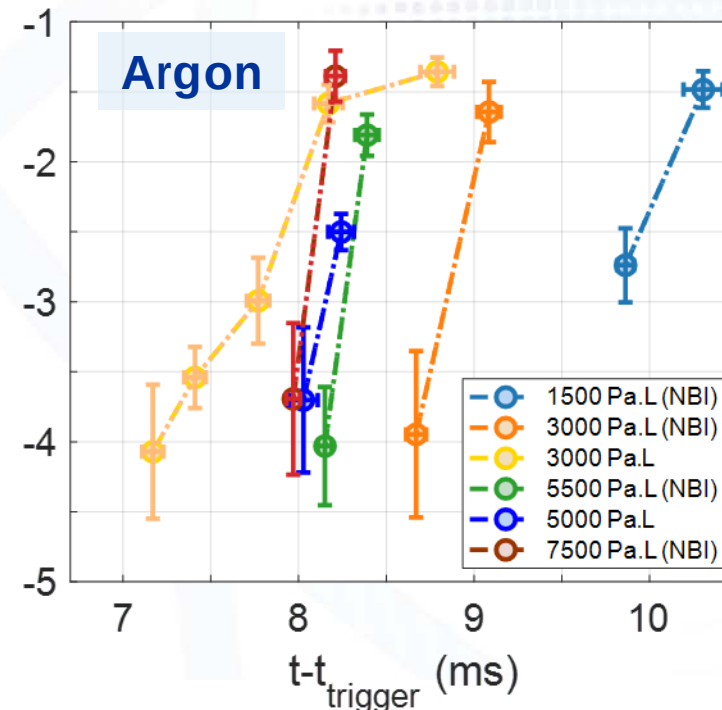
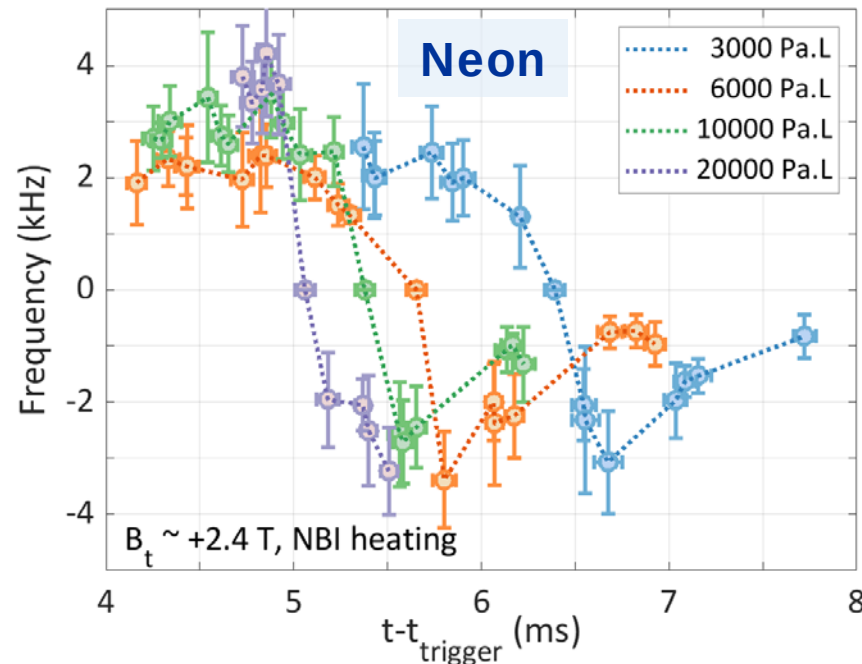


Backup

Parameter dependence of mode frequency



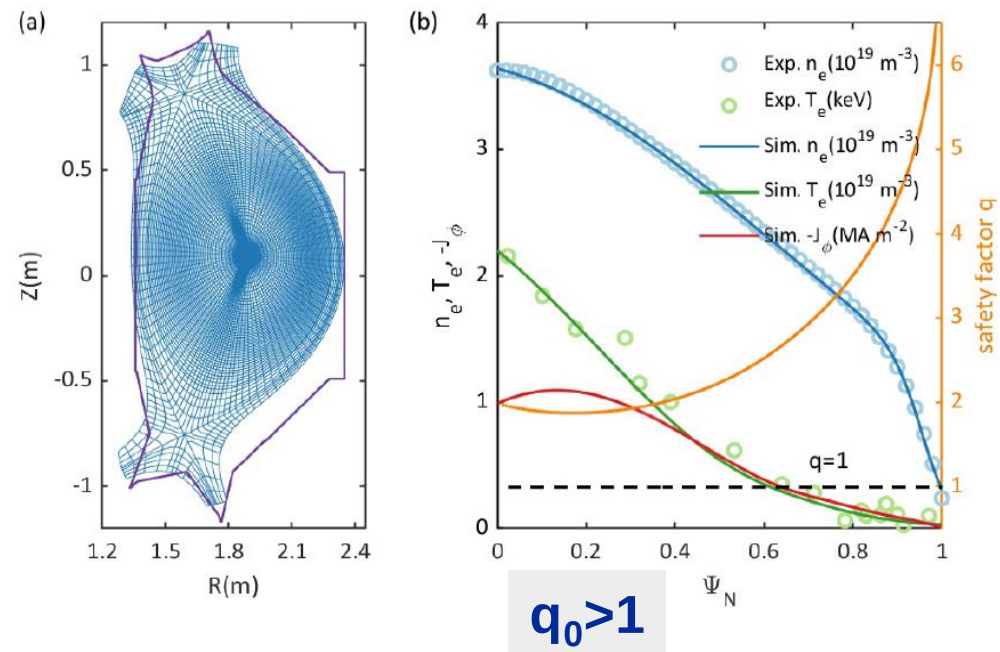
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Models and simulation settings



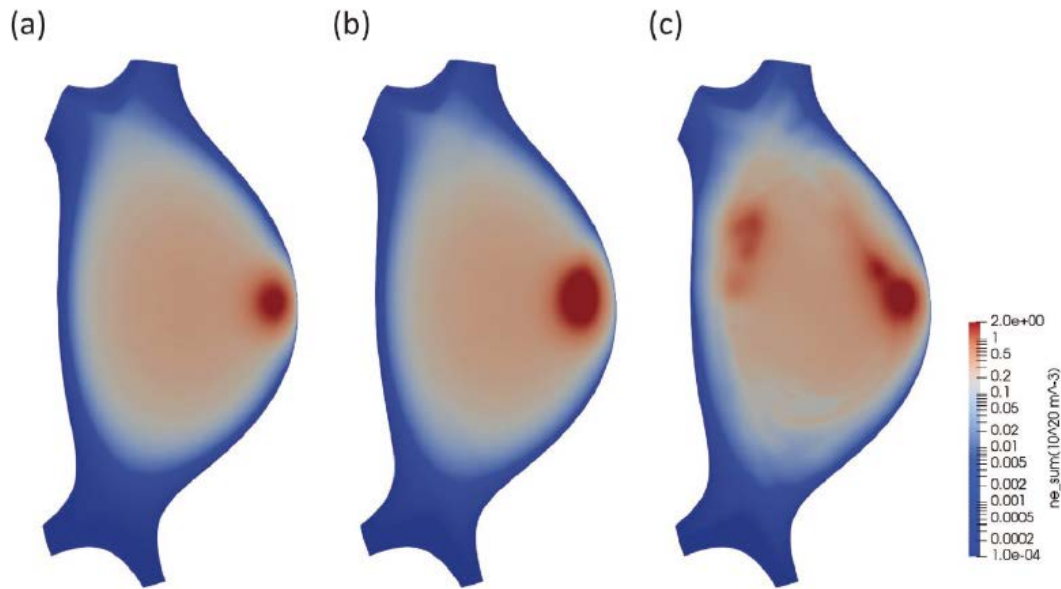
Models: single-fluid reduced MHD model with **non-equilibrium impurity treatment**



initial central resistivity	η_0	$8 \times 10^{-9} \Omega \cdot \text{m}$
dynamic viscosity	μ_0	$3 \times 10^{-6} \text{ kg} \cdot \text{m}^{-1} \text{s}^{-1}$
	$\mu_{ ,0}$	$1.5 \times 10^{-5} \text{ kg} \cdot \text{m}^{-1} \text{s}^{-1}$
particle diffusivities	$D_{\perp,0}$	$1 \text{ m}^2 \text{s}^{-1}$
	$D_{ ,0}$	$1.3 \times 10^5 \text{ m}^2 \text{s}^{-1}$
heat diffusivity	$\chi_{\perp,0}$	$15 \text{ m}^2 \text{s}^{-1}$
	$\chi_{ ,0}$	$7.7 \times 10^{10} \text{ m}^2 \text{s}^{-1}$

The perpendicular diffusivity are chosen to **keep the profile approximately constant** in the simulation.

Neon particles



Move inward along the major radius

Deposit at $\Psi_N \sim 0.7$

Extend along magnetic field lines

*Create **new deposition spots** at different poloidal positions*