



The ENDF Project

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16 Jan 2024





Outline

CSEWG

ENDF/B-VIII.1

Big Paper

ENDF format



Cross Section Evaluation Working Group

Before ENDF

By 1960, there were many data efforts worldwide

- different formats
- often hard-coded libraries
- proprietary data
- Notable efforts: UKNDL (AWE, UK), NDA library (US), ENDL (LRL, US)

~1962 H. Honeck (BNL), A. Henry (Westinghouse), G. Joanou (GA) met at Colony Restaurant in DC decided on action

- requested Reactor Mathematics and Computation Division of ANS to sponsor 2 meetings to link databases



<http://www.streetsofwashington.com/2013/10/fine-dining-in-washington-dc-in-1950s.html>



The Cross Section Evaluation Working Group produces ENDF/B library



- **Formed 1966 & Chaired by BNL**
- **Currently ~200 members of the collaboration from 25 institutions**
 - US programs, industry and international partners
 - If you see something in the library, at some point a sponsor somewhere wanted it
- **All steps of nuclear data pipeline coordinated through CSEWG**
- **Depending on what needs done, getting required data in library can be major effort**

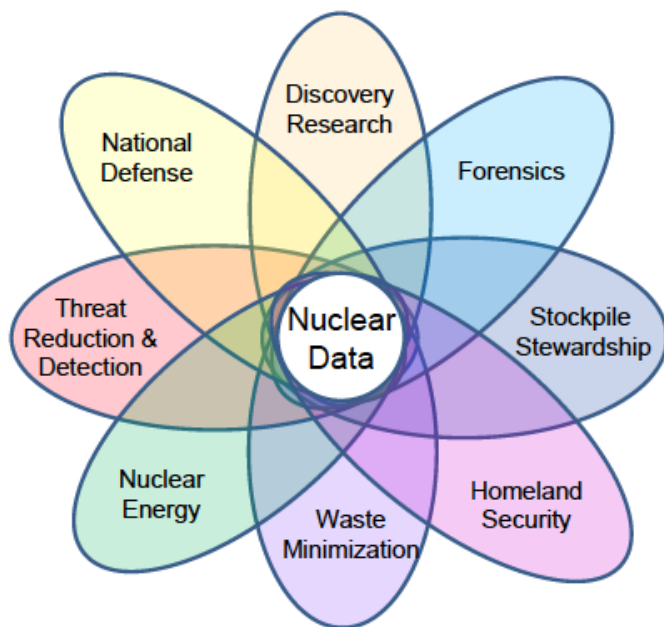


CSEWG collaboration meeting in November 2022: our first in-person meeting since the pandemic started!

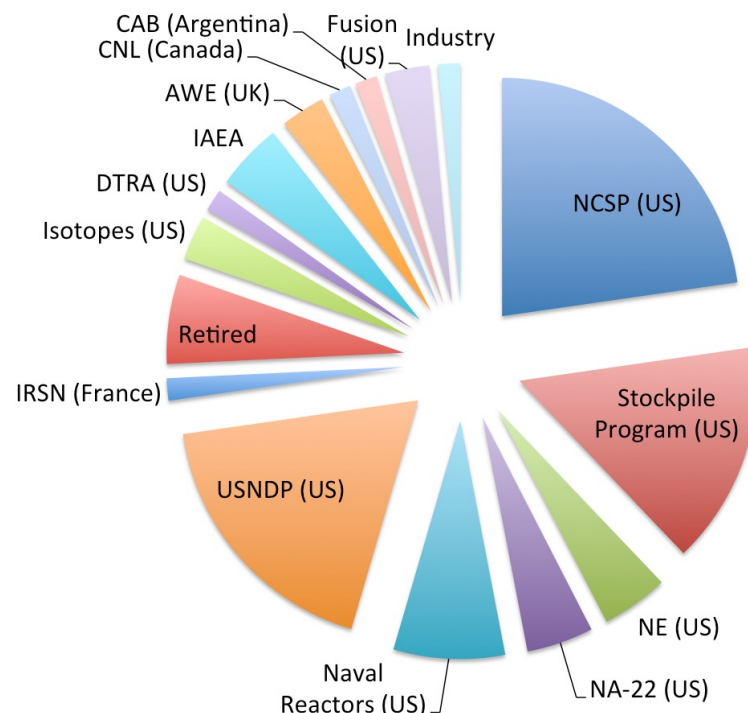
We are always open to new users and collaborators

CSEWG is a long standing collaboration between the data users who are also the biggest content providers

Intersections in Nuclear Data



Fraction of evaluations provided for ENDF/B-VIII.0



Dr. Timothy Hallman, Associate Director of the Office of Science for Nuclear Physics
April 5, 2015

By sharing through CSEWG many sponsors reap the benefit of collaboration

Program	Measurement	Theory	Compilation	Evaluation	QA (V&V, IE)	Infrastructure (gitlab, etc.)
DTRA	✓					
International (IAEA,NEA,...)	✓	✓	✓	✓	✓	✓
NA-22	✓	✓		✓		
Naval Reactors	✓			✓	✓	
NCSP	✓	✓		✓	✓	✓
Nuclear Energy					✓	
Other (IP, Fusion, ...)	✓	✓				
Defense Programs	✓	✓		✓	✓	✓
USNDP	✓	✓	✓	✓	✓	✓

TRIUMF

Argonne
NATIONAL LABORATORY

Canadian Nuclear
Laboratories
Laboratoires Nucléaires
Canadiens



Lawrence Livermore
National Laboratory

Sandia
National
Laboratories

Los Alamos
NATIONAL LABORATORY
EST. 1943

**OAK
RIDGE**
National Laboratory



Rensselaer

BROOKHAVEN
NATIONAL LABORATORY

NAVAL NUCLEAR
LABORATORY

NIST
NC STATE
UNIVERSITY



CSEWG is not limited to the Americas



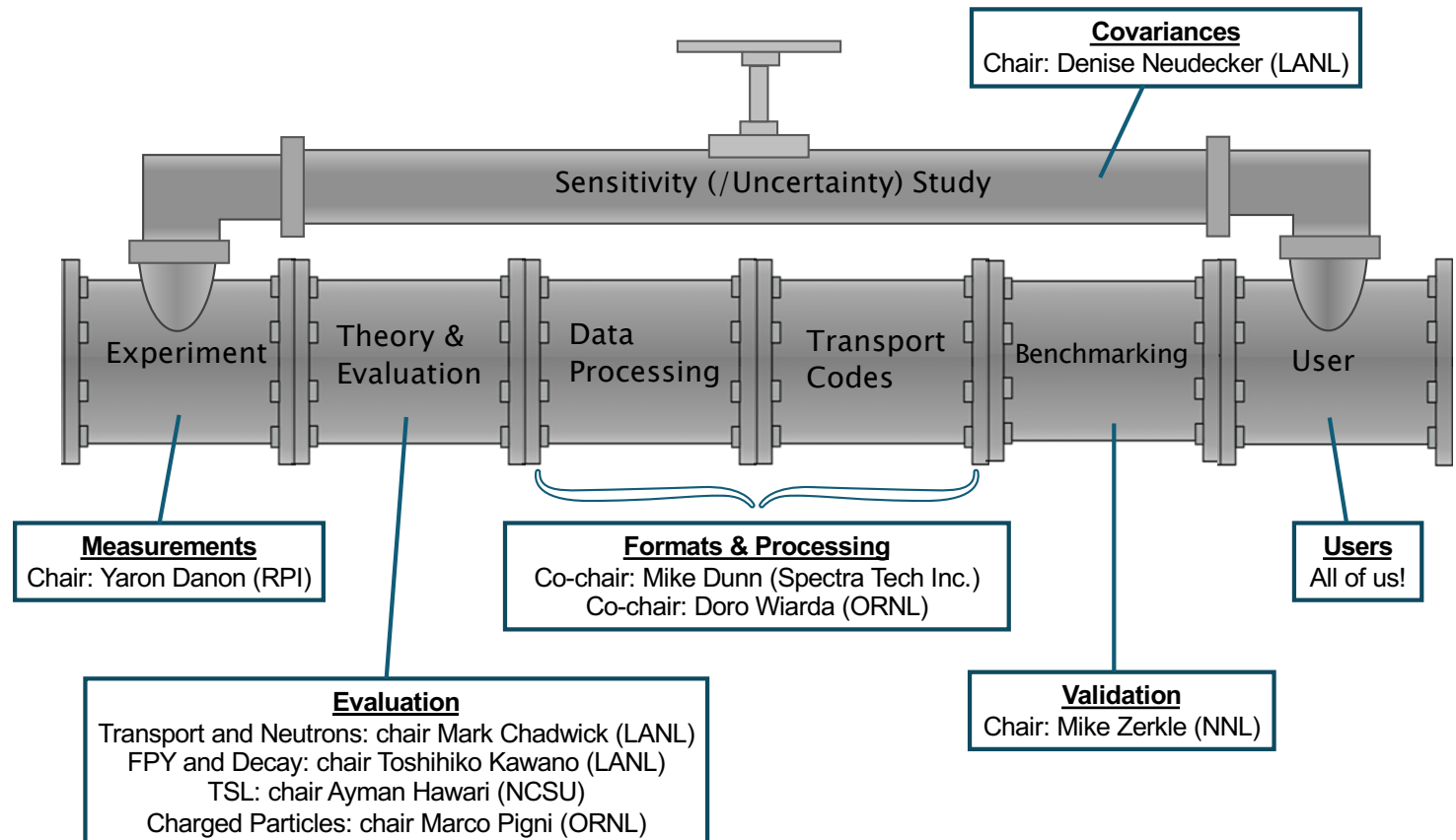
All steps of nuclear data pipeline are coordinated through CSEWG

Chair:

David Brown (BNL)
dbrown@bnl.gov

Library Manager:

Gustavo Nobre (BNL)
gnobre@bnl.gov



The next ENDF/B release

ENDF/B-VIII.1 release

The next release of the ENDF/B library is scheduled for **May 2024!**

Although technically “minor”, it will have **major** impact.

- Why **VIII.1** and not **IX**?
 - There are no planned updates of the standards library for this release
 - Standards are well-established cross sections, in specific energy ranges, used in ratios with other measurements
 - However, many, many important and impactful changes are on the way!!
- Next release will be in both legacy **ENDF-6** format and **GNDS-2.0**
- Will have an accompanying “**Big Paper**”
- Implemented review system: Multiple VIII.1 Beta versions have been released
- Preliminary validation indicate that this will be the best-performing library ever!



ENDF/B
VIII.1-β2

What to expect when expecting... ... the ENDF/B-VIII.1 release

Neutrons:

- Many INDEN contributions
- Actinides:
 - **²³⁹Pu**: multi-institution effort, with important updates to fission, nubar, PFNS, capture, URR, RRR, (n,2n)
 - **²³⁵U**: resonances, nubar, covariances,
 - **²³⁸U**: resonance update to improve performance on depletion benchmarks
 - **^{240,241}Pu**: work in concert with changes in ²³⁹Pu and ²³⁸U to recover burnup performance
- Stainless steel & other structure materials:
 - **^{54,56,57}Fe**: Corrects leakage deficiency from ENDF/B-VIII.0
 - **^{50,52,53,54}Cr**: Thorough re-evaluation, impact in criticality and leakage benchmarks
 - **^{206,207,208}Pb**: complete evaluations (RPI/LANL)
 - **^{63,65}Cu**: improved performance
 - **⁵⁵Mn**: Gamma spectra
 - **^{28,29,30}Si**: resonance evaluations
- Others:
 - **⁶Li, ⁹Be** (LANL)
 - **^{234,236}U** (LANL)
 - **^{140,142}Ce** (ORNL)
 - **¹⁰³Rh** (RPI/IRSN)
 - **⁸⁶Kr** (BNL)
 - **¹⁸¹Ta** (RPI/ORNL/LANL)
 - **⁹⁵Mo** (IRSN/LANL)
 - Many, many, many more...

What to expect when expecting... ... the ENDF/B-VIII.1 release

TSL:

- 70+ new updated/files
- **Polystyrene, zirconium hydride, UC, UN, UO₂, sapphire, lucite, FLiBe, etc...**
- Fuel materials with different enrichments
- So many new evaluations that we had to re-think how to identify them.
- Low-temperature extrapolations to light water

- Community-wide review and validation

Fission Yields:

- Many fixes
- ...but no changes to the actual yields

Photo-nuclear:



- **200+** updates coming from IAEA CRP

Charged particles:

- A few improvements and fixes

ENDF versioned repository: GitLab

USNDP Collaboration Platform

 **National Nuclear Data Center**


The U.S. nuclear data community working together to continuously advance the state of nuclear data for science and technology applications.

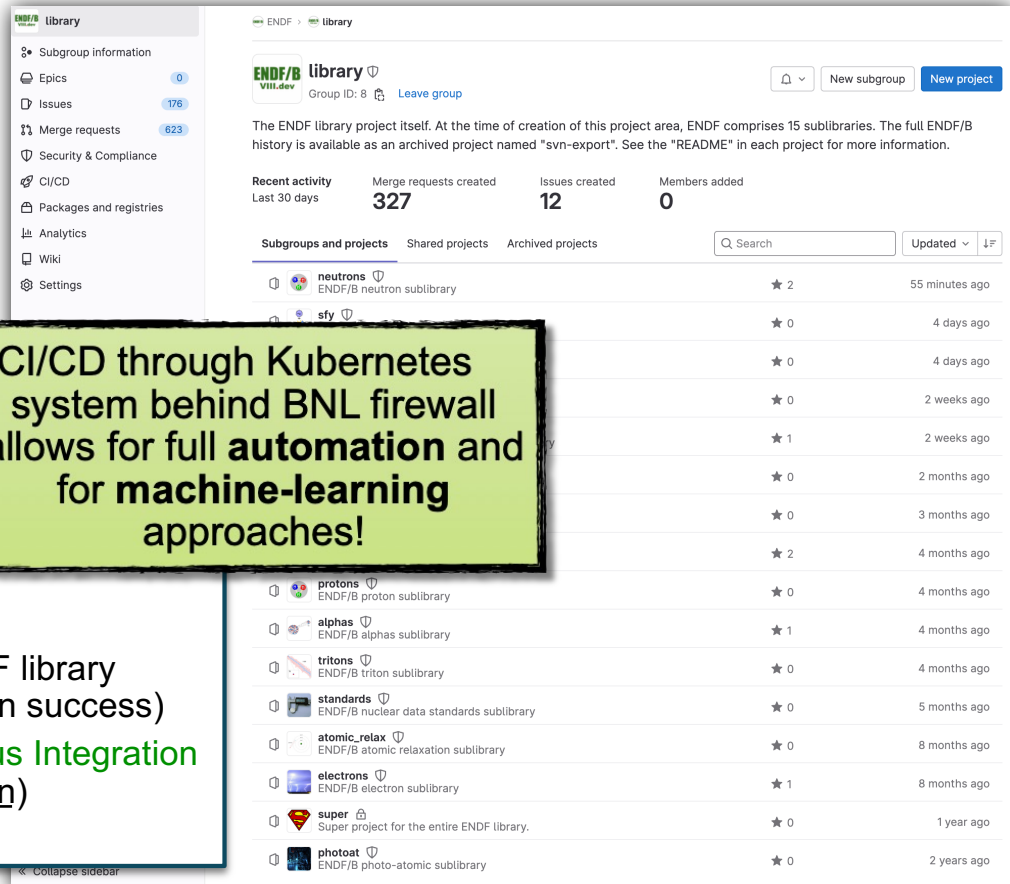
NOTICE TO USERS

This is a Federal computer system (and/or it is directly connected to a BNL local network system) and is the property of the United States Government. It

Username or email

Password

Remember me [Forgot your password?](#)



ENDF library

Subgroup information

- Epics 0
- Issues 176
- Merge requests 623
- Security & Compliance
- CI/CD
- Packages and registries
- Analytics
- Wiki
- Settings

ENDF library

Group ID: 8 [Leave group](#)

The ENDF library project itself. At the time of creation of this project area, ENDF comprises 15 sublibraries. The full ENDF/B history is available as an archived project named "svn-export". See the "README" in each project for more information.

Recent activity Last 30 days

Merge requests created	327
Issues created	12
Members added	0

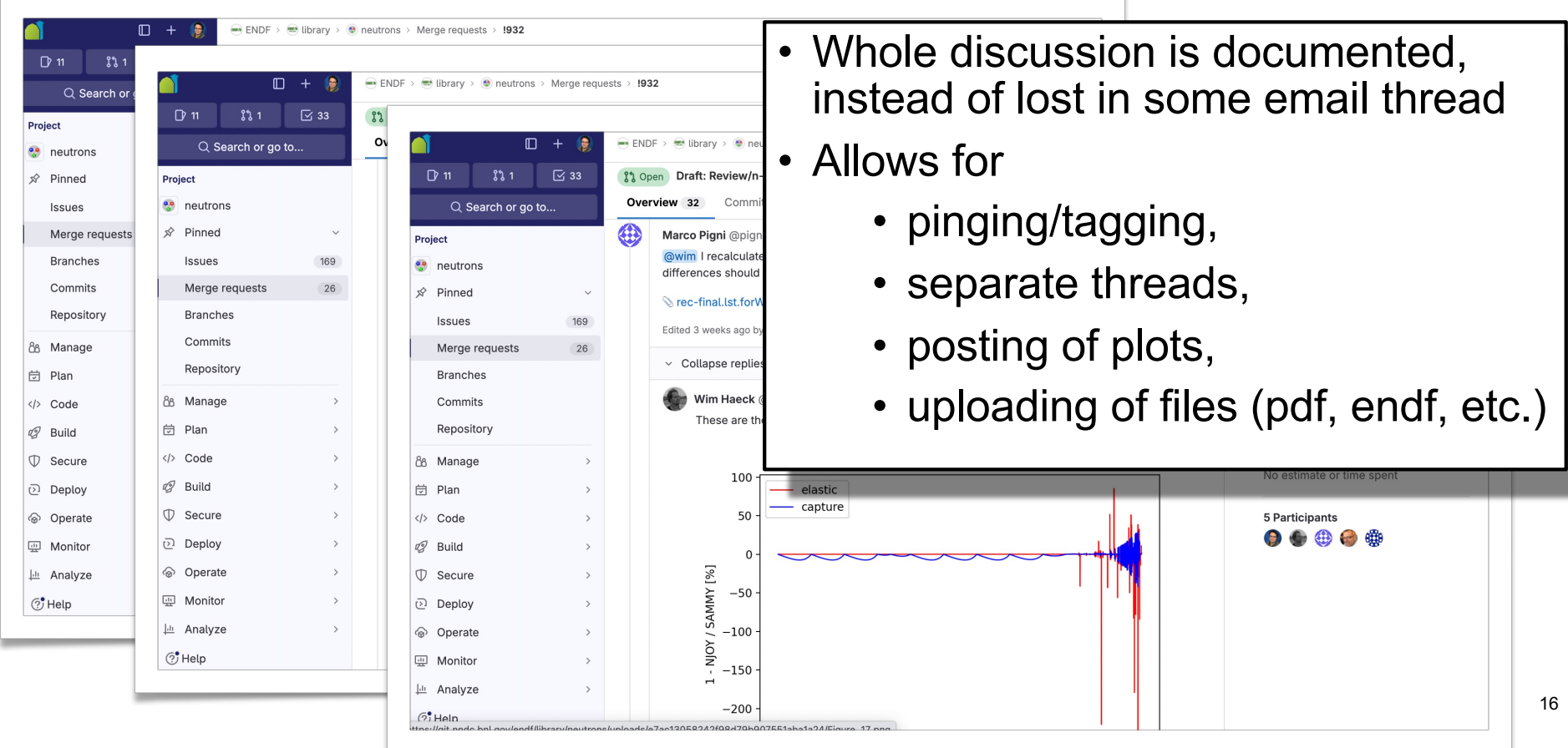
Subgroups and projects Shared projects Archived projects

Subgroup/Project	Stars	Last Updated
neutrons	2	55 minutes ago
sfy	0	4 days ago
protons	0	4 days ago
alphas	0	2 weeks ago
tritons	1	2 weeks ago
standards	0	2 months ago
atomic_relax	0	3 months ago
electrons	2	4 months ago
super	0	4 months ago
photoat	0	4 months ago

- Constantly updated and maintained
- Keeps track of
 - Any changes
 - Development, **review** and release branches
 - Issue trackers
 - etc...
- Usage is growing! Currently ~60 active members in ENDF library group (unfortunately there's a seat limit: victims of our own success)
- Integration of library repository in GitLab with a **Continuous Integration system: ADVANCE** (R. Arcilla, R. Coles, B. Shu, D. Brown)

CI/CD through Kubernetes system behind BNL firewall allows for full automation and for machine-learning approaches!

Showcase example: ^{88}Sr review



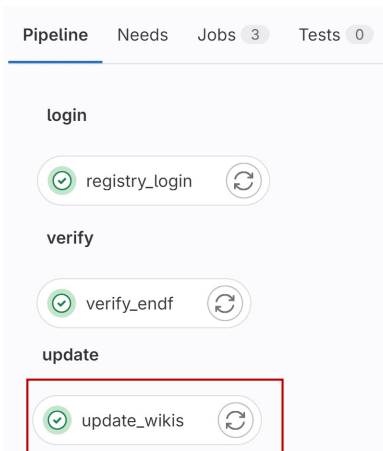
- Whole discussion is documented, instead of lost in some email thread
- Allows for
 - pinging/tagging,
 - separate threads,
 - posting of plots,
 - uploading of files (pdf, endf, etc.)

ENDF Repo Auto-updating Wikis

A new job is being added to GitLab ENDF repositories to auto-update the repo's wiki with useful information about job and artifact status.

Example: <https://git.nndc.bnl.gov/endl/library/neutrons/-/wikis/Neutron-Artifacts>

1) update_wikis job runs after verify_endf



2) Use Wiki table of contents to find your data

Neutron Artifacts

Atomic Number (Z)	Symbol	Atomic Mass (A)
000	n	001
001	H	001, 002, 003
002	He	003, 004
003	Li	006, 007
004	Be	007, 009
005	B	010, 011
006	C	012, 013
007	N	014, 015
008	O	016, 017, 018
009	F	019
010	Ne	020, 021, 022
011	Na	022, 023

3) Enjoy a record of job and artifact data all in one place

n 000_n_001

Last updated: 2023-11-09 19:06:57.358348

Legend: success = 🟢, failed = 🚫, warning = ⚠️, canceled = ✖, pending = ⏸, running = 🔄, emoji for unknown status

- 🟢 verify_endf 2023-11-09 19:06:57.358383
Job Status: Job Details
Artifacts created by this job: <https://git.nndc.bnl.gov/api/v4/projects/27/jobs/10305/artifacts>
- 🟢 verify_endf 2023-11-09 18:30:02.626032
Job Status: Job Details
Artifacts created by this job: <https://git.nndc.bnl.gov/api/v4/projects/27/jobs/10299/artifacts>
- 🟢 verify_endf 2023-11-06 14:41:16.202000
Job Status: Job Details
Artifacts created by this job: <https://git.nndc.bnl.gov/api/v4/projects/27/jobs/10122/artifacts>
- 🟢 verify_endf 2023-11-06 14:54:46.259000
Job Status: Job Details
Artifacts created by this job: None
- 🟢 verify_endf 2023-11-06 16:21:16.366000
Job Status: Job Details
Artifacts created by this job: None

Status of “Big Paper”

Past ENDF release published accompanying paper in Nuclear Data Sheets

Nuclear Data Sheets 112, 2887-2996 (2011)



Nuclear Data Sheets
Volume 112, Issue 12, December 2011, Pages 2887-2996



ENDF/B-VII.1 Nuclear Data for Science and Technology: Cross Sections, Covariances, Fission Product Yields and Decay Data

M.B. Chadwick^a, M. Herman^b, P. Obložinský^b, M.E. Dunn^c, Y. Danon^d, A.C. Kahler^e, D.L. Smith^f, B. Pritychenko^g, G. Arbanas^h, R. Arcillaⁱ, R. Brewer^j, D.A. Brown^k, R. Capote^l, A.D. Carlson^m, V.S. Choⁿ, H. Derrien^o, K. Guber^p, G.M. Hale^q, S. Hoblit^r, S. Holloway^s, T.D. Johnson^t, T. Kawano^u, B.C. Kiedrowski^v, H. Kim^w, S. Kurihara^x, N.M. Larson^y, L. Leal^z, J.P. Lestone^{aa}, R.C. Little^{ab}, E.A. McCutchan^{ac}, R.E. MacFarlane^{ad}, M. MacInnes^{ae}, C.M. Mattoon^{af}, R.D. McKnight^{ag}, S.F. Mughabghab^{ah}, G.P.A. Nobre^{ai}, G. Palmiotti^{aj}, A. Palumbo^{ak}, M.T. Pigni^{al}, V.G. Pronyaev^{am}, R.O. Sayer^{an}, A.A. Sonzogni^{ao}, N.C. Summers^{ap}, P. Talou^{aq}, I.J. Thompson^{ar}, A. Trkov^{as}, R.L. Vogt^{at}, S.C. van der Marck^{au}, A. Wallner^{av}, M.C. White^{aw}, D. Wiarda^{ax}, P.G. Young^{ay}

Cited 3,138 times!

Nuclear Data Sheets, 107 (2006), p. 2931



Nuclear Data Sheets
Volume 107, Issue 12, December 2006, Pages 2931-3060



ENDF/B-VII.0: Next Generation Evaluated Nuclear Data Library for Nuclear Science and Technology

ENDF
B-VII.0
Evaluated Nuclear Data File

Cited 2,753 times!

ENDF
B-VIII.0

Nuclear Data Sheets 148, 1-142 (2018)



Nuclear Data Sheets
Volume 148, February 2018, Pages 1-142



ENDF/B-VIII.0: The 8th Major Release of the Nuclear Reaction Data Library with CIELO-project Cross Sections, New Standards and Thermal Scattering Data

D.A. Brown^a, M.B. Chadwick^b, D. A. Brown^c, R. Capote^d, A.C. Kahler^e, A. Trkov^f, M.W. Herman^g, A.A. Sonzogni^h, Y. Danonⁱ, A.D. Carlson^j, M. Dunn^k, D.L. Smith^l, G.M. Hale^m, G. Arbanasⁿ, R. Arcilla^o, C.R. Bates^p, B. Beck^q, B. Becker^r, F. Brown^s, R.J. Casperson^t, J. Conlin^u, D.E. Cullen^v, M.-A. Descalle^w, R. Firestone^x, T. Gaines^y, K.H. Guber^z, A.I. Hawari^{aa}, J. Holmes^{ab}, T.D. Johnson^{ac}, T. Kawano^{ad}, B.C. Kiedrowski^{ae}, A.J. Koning^{af}, S. Kopecky^{ag}, L. Leal^{ah}, J.P. Lestone^{ai}, C. Lubitz^{aj}, J.I. Márquez Damián^{ak}, C.M. Mattoon^{al}, E.A. McCutchan^{am}, S. Mughabghab^{an}, P. Navrátil^{ao}, D. Neudecker^{ap}, G.P.A. Nobre^{aq}, G. Noguere^{ar}, M. Paris^{as}, M.T. Pigni^{at}, A.J. Plompen^{au}, B. Pritychenko^{av}, V.G. Pronyaev^{aw}, D. Roubtsov^{ax}, D. Rochman^{ay}, P. Romano^{az}, P. Schillebeeckx^{ba}, S. Simakov^{bb}, M. Sin^{bc}, I. Sirakov^{bd}, B. Sleaford^{be}, V. Sobes^{bf}, E.S. Soukhovitskii^{bg}, I. Stetcu^{bh}, P. Talou^{bi}, I. Thompson^{bj}, S. van der Marck^{bk}, L. Welser-Sherill^{bl}, D. Wiarda^{bm}, M. White^{bn}, J.L. Wormald^{bo}, R.Q. Wright^{bp}, M. Zerkle^{bq}, G. Zerovnik^{br}, Y. Zhu^{bs}

Cited 1,658 times!



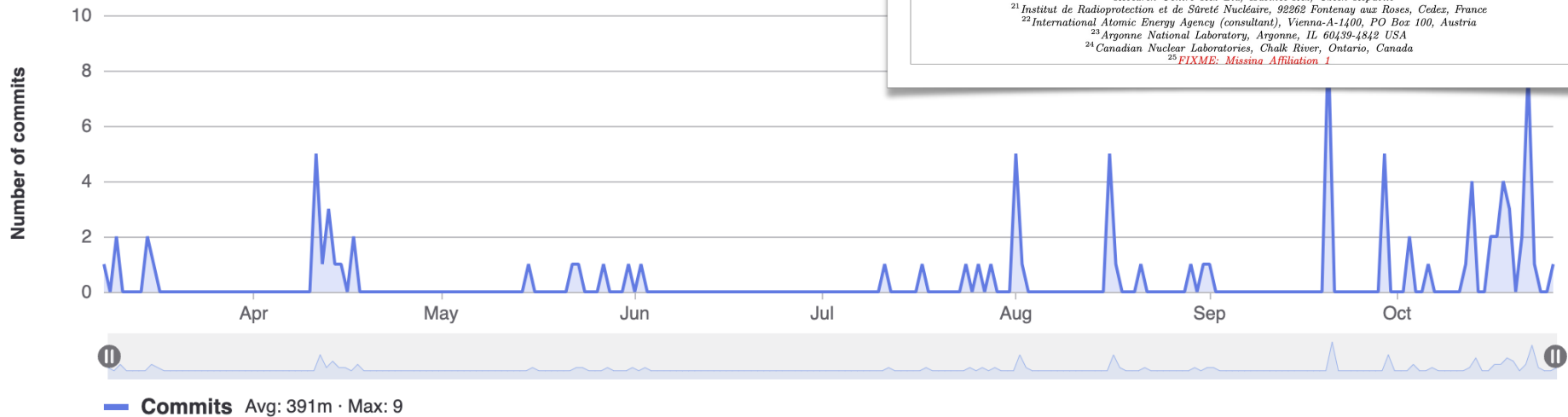
This is really, **really** impactful!

Big Paper updates

- Many contributions have been sent but there are still gaps that will be addressed after CSEWG Meeting
- Defined preliminary full author list and ordering
- Big Paper is shaping up: huge “stitching” effort
- Aiming to have a complete manuscript soon

Commits to development

Excluding merge commits. Limited to 6,000 commits.



FIXME: Full title of ENDF/B-VIII.1 paper

G.P.A. Nobre,^{1,*} R. Capote,² M.T. Pigni,³ A. Trkov,⁴ C.M. Mattoon,⁵ D. Neudecker,⁶ D.A. Brown,¹ M.B. Chadwick,⁶ A.C. Kahler,⁹ N.A. Kleedtke,⁶ M. Zerke,⁷ A.I. Hawari,⁸ C.W. Chapman,³ N.C. Fleming,⁸ J.L. Wormald,⁷ K. Ramić,³ Y. Danon,⁹ N.A. Gibson,⁶ P. Brain,⁹ M.W. Paris,⁶ G.M. Hale,⁶ I.J. Thompson,⁵ D.P. Barry,⁷ I. Stetcu,⁶ W. Haeck,⁶ A.E. Lovell,⁶ M.R. Mumpower,⁶ G. Potel Aguilar,⁵ K. Kravvaris,⁵ G. Noguere,¹⁰ A.D. Carlson,¹¹ M. Dunn,¹² T. Kawano,⁶ D. Wiarda,³ G. Arbanas,³ R. Arcilla,¹ B. Beck,⁵ D. Bernard,¹⁰ R. Beyer,¹³ J.M. Brown,³ O. Cabellos,¹⁴ R.J. Casperson,⁵ E.V. Chimanski,¹ R. Coles,¹ M. Cornock,¹⁵ J. Cotchen,⁷ J.P.W. Crozier,⁸ D.E. Cullen,² A. Daskalakis,⁷ M.-A. Descalle,⁵ D.D. DiJulio,¹⁶ P. Dimitriou,² A.C. Dreyfuss,⁵ **FIXME: Ignacio Duran**,¹⁷ R. Ferrer,¹⁸ T. Gaines,¹⁵ G. Gert,⁵ K.H. Guber,³ J.D. Haverkamp,⁷ M.W. Herman,⁹ J. Holmes,⁷ A.R. Junghans,¹⁹ K. Kelly,⁶ H.I. Kim,¹⁹ P.E. Koehler,⁹ M. Košťál,²⁰ B.K. Laramee,⁸ A. Lauer-Coles,¹ L. Leal,^{3,21} H.Y. Lee,⁶ A.M. Lewis,⁷ J. Malec,⁴ J.I. Márquez Damián,¹⁶ W.J. Marshall,³ A. Mattered,¹ J.D. McDonnell,³ G. Muhrer,¹⁶ A. Ney,⁷ W.E. Ormand,³ D.K. Parsons,⁶ C.M. Percher,⁵ B. Pritychenko,¹ V.G. Pronyaev,²² S. Quagliioni,⁵ M. Rapp,⁷ J.J. Ressler,⁵ P.K. Romano,²³ D. Roubtsov,²⁴ G. Schnabel,² M. Schulc,²⁰ A.A. Sonzogni,¹ P. Talou,⁵ J. Thompson,⁷ T.H. Trumbull,⁷ M. Vorabbi,¹ C. Wemple,¹⁸ K.A. Wendt,⁵ M. White,⁶ and R.Q. Wright²⁵

¹ Brookhaven National Laboratory, Upton, NY 11973-5000, USA

² International Atomic Energy Agency, Vienna-A-1400, PO Box 100, Austria

³ Oak Ridge National Laboratory, Oak Ridge, TN 37831-6171, USA

⁴ Jožef Stefan Institute, Jamova 39, SI-1000, Ljubljana, Slovenia

⁵ Lawrence Livermore National Laboratory, Livermore, CA 94551-0808, USA

⁶ Los Alamos National Laboratory, Los Alamos, NM 87545, USA

⁷ Naval Nuclear Laboratory, Schenectady, New York 12301-1072, USA

⁸ North Carolina State University, Department of Nuclear Engineering, Raleigh, North Carolina 27695

⁹ Rensselaer Polytechnic Institute, Troy, NY 12180, USA

¹⁰ CEA, DEN, DER, SPRC, Cadarache, 13108 Saint-Paul-lès-Durance, France

¹¹ National Institute of Standards and Technology, Gaithersburg, MD 20899-8463, USA

¹² Spectra Tech, Inc., Oak Ridge, TN 37830, USA

¹³ Physikalisches Technische Bundesanstalt, Department 6.4 - Ion and Neutron Radiation, Bundesallee 100, 38116 Braunschweig, Germany

¹⁴ Universidad Politécnica de Madrid, José Gutiérrez Abascal, 2 28006, Madrid, Spain

¹⁵ AWEP, Aldermaston, Reading, BERKSHIRE, RG7 4PR.

¹⁶ European Spallation Source ERIC, Lund, Sweden

¹⁷ **FIXME: Missing Affiliation 2**

¹⁸ **FIXME: Studsvik**

¹⁹ Korea Atomic Energy Research Institute, Daejeon, Republic of Korea

²⁰ Research Centre Řež Ltd, Husinec-Řež, Czech Republic

²¹ Institut de Radioprotection et de Sécurité Nucléaire, 92262 Fontenay aux Roses, Cedex, France

²² International Atomic Energy Agency (consultant), Vienna-A-1400, PO Box 100, Austria

²³ Argonne National Laboratory, Argonne, IL 60439-4842 USA

²⁴ Canadian Nuclear Laboratories, Chalk River, Ontario, Canada

²⁵ **FIXME: Missina Affiliation 1**

Final comments

- **CSEWG and ENDF:**
 - Welcomes collaboration in and out of US (but inside is easier)
 - Long tradition of quality/continuous improvement
 - Conservative, driven by experiment when possible
 - Golden Rule (whoever has the gold makes the rules)
- **USNDP and NNDC are a resource for many other data products**
- **When complete, data is OPEN**
 - (admittedly we are still learning what the evolving BNL process is for opening the data)



Old Sigma

(and what to replace it with)

David Brown
National Nuclear Data Center, Brookhaven National Laboratory
Upton, NY 11973, USA

16 Jan 2024



Sigma access points

National Nuclear Data Center

Databases

Structu

Oigma Evaluated Nuclear Data File (ENDF) Retrieval & Plotting

Periodic Table Browse Directory Tree Browse Basic Retrieval Advanced Retrieval Plot Cart Computations

Select first a library, then a sublibrary and finally click on a chemical element to obtain results.
Data are available for materials with a cyan background.

Library: ENDF/B-VII.1(USA, 2011) Sublibrary: Neutron reactions

BNL Homepage

Version History:

- New (December 2011)
 - ENDF/B-VII.1 evaluated neutron library.
- New in version 3.1 (October 2009)
- New in version 3.0 (February 2009)
- New in version 2.0 (April 2008)
- New in version 1.0 (April 2007)

National Nuclear Data Center

Databases

Structu

Oigma Evaluated Nuclear Data File (ENDF) Retrieval & Plotting

New: ENDF/B-VII.1 library.

Periodic Table Browse Directory Tree Browse Basic Retrieval Advanced Retrieval Plot Cart Computations

National Nuclear Data Center
Building 817

- ENDF Directory Tree
 - Neutron Reactions
 - Thermal Neutron Scattering
 - Neutron Cross Section Standards
 - Photonuclear Reactions
 - Charged Particle Reactions
 - Decay Data

Three options to navigate ENDF Directory Tree:

- Use single mouse click on the "plus" or "minus" signs
- Use double mouse click on the folders icons or folder content description

Use combination of the "Tab" + "Enter" keys on "plus" or "minus" signs

National Nuclear Data Center

Databases

Structu

Oigma Evaluated Nuclear Data File (ENDF) Retrieval & Plotting

Periodic Table Browse Directory Tree Browse Basic Retrieval Advanced Retrieval Plot Cart Computations

Target: _____ or Z: _____ and A: _____
Ex: 56fe or fe56 or 56fe,60ni

Select Reaction: And select quantity:

- (n,*)
 - (n,total)
 - (n,elastic)
 - (n,inelastic)
 - (n,f)
 - (n,y)
 - (n,2n)
 - (n,p)
 - (n,α)
- Cross sections
 - Resonance parameters
 - dσ/dω
 - dσ/dE
 - d²σ/dωdE
 - Covariances

Or select other ENDF topic:

- Neutron cross section standards
- Thermal neutron scattering
- Photonuclear reactions
- Proton reactions
- Decay data
- Neutron-induced fission yields
- Spontaneous fission yields
- Neutron-induced nu-bar's
- All nu-bar's

Library: All ENDF/B-VII.1 ENDF/B-VII.0 JEFF-3.1 JENDL-4.0 JENDL-3.3 ROSFOND CENDL-3.1 ENDF/B-VII.1 (USA, 2011) ENDF/B-VII.0 (USA, 2006) JEFF-3.1 (Europe, 2005) JENDL-4.0 (Japan, 2010) JENDL-3.3 (Japan, 2002) ROSFOND (Russia, 2008) CENDL-3.1 (China, 2009) ENDF/B-VII.1 (USA, 2001)

Submit Reset Help

Sigma access points

National Nuclear Data Center

Databases

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Oigma Evaluated Nuclear Data File (ENDF) Retrieval & Plotting

Periodic Table Browse Directory Tree Browse Basic Retrieval Advanced Retrieval Plot Cart Computations

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Library: ENDF/B-VII.1(USA, 2011) Sublibrary: Neutron reactions

National Nuclear Data Center

Databases

Structu

Oigma Evaluated Nuclear Data File (ENDF) Retrieval & Plotting

New: ENDF/B-VII.1 library.

Periodic Table Browse Directory Tree Browse Basic Retrieval Advanced Retrieval Plot Cart Computations

All of these entry points have been updated with new look-n-feel, but the reskinning has not been deployed

Not ready for containers

Version History:

- New: (December 2011)
- ENDF/B-VII.1 evaluated neutron library.
- New in version 3.1 (October 2009)
- New in version 3.0 (February 2009)
- New in version 2.0 (April 2008)
- New in version 1.0 (April 2007)

National Nuclear Data Center

Databases

Structu

Oigma Evaluated Nuclear Data File (ENDF) Retrieval & Plotting

Periodic Table Browse Directory Tree Browse Basic Retrieval Advanced Retrieval Plot Cart Computations

Target: _____ or Z: _____ and A: _____

Ex: 56fe or fe56 or 56fe,60ni

Select Reaction: (n,*) (n,total) (n,elastic) (n,inelastic) (n,f) (n,y) (n,Zn) (n,p) (n,α)

And select quantity: Cross sections Resonance parameters dσ/dω dσ/dE d²σ/dωdE Covariances

Or select other ENDF topic: Neutron cross section standards Thermal neutron scattering Photonuclear reactions Proton reactions Decay data Neutron-induced fission yields Spontaneous fission yields Neutron-induced nu-bar's All nu-bar's

Library: All ENDF/B-VII.1 ENDF/B-VII.0 JEFF-3.1 JENDL-4.0 JENDL-3.3 ROSFOND CENDL-3.1 ENDF/B-V

(USA, 2011) (USA, 2006) (Europe, 2005) (Japan, 2010) (Japan, 2002) (Russia, 2008) (China, 2009) (USA, 2007)

Submit Reset Help

Two other features: computations, shared plot cart with exfor-endf

National Nuclear Data Center Databases Structure

Evaluated Nuclear Data File (ENDF) Retrieval & Plotting

Periodic Table Browse Directory Tree Browse Basic Retrieval Advanced Retrieval Plot Cart **Computations**

Existing Datasets:

Dataset	Description	Number of total (x,y) points	Quantities evaluated between $x_{min}=0.0$ and $x_{max}=0.0$							
			Number of (x,y) points	y_{min}	y_{max}	$\int y dx$	Maxwellian weighted $kT=30$ keV	^{252}Cf weighted	$\int xy dx$	$\int \frac{xy}{y} dx$

The plot cart is empty, add plot(s) to it before using this page: go to Browse or Retrieval, select Plot, click Add to plot cart.

This part of Sigma allows the user to compute operations between different sets of (x,y) data points that are stored in the plot cart. A new set of (x,y) points, resulting in mathematical operations of other datasets, will be added to the plot cart using this feature.

Examples:

- To plot the first dataset's y column multiplied by 10, type $10*y0$.
- To plot the \log_{10} of the first dataset's y column, type $\log(y0)$. Use $\ln()$ for \log_e .
- To plot the first dataset's x column multiplied by its y column, type $x*y0$.
- To plot the integrand of a Maxwellian type averaged cross section, type $y0*x*\exp(-x/10000)$.
- To add the y columns for the first two datasets, type $y0+y1$.
- To plot the first dataset's y column plus 10 times the second dataset's y column type $y0+10*y1$.

The new dataset's x values are obtained as the union of all the datasets' x values involved in the operation. The expression is then evaluated in y values obtained from linear interpolations.

The ^{252}Cf weighted quantity corresponds to $\int C \exp(Ax) \sinh(\sqrt{Bx}) y dx$, with $A=-7.89E-1$ MeV $^{-1}$ and $B=4.47E-1$ MeV $^{-1}$

The abundances are obtained from NuDat2 while the fission yields are from ENDF/B-VII.0

Warnings:

- At the moment, computations are restricted to cross sections (MF=3).
- Operations among datasets containing more than 10,000 actual points will take a long time to execute and the connection will be timed out.
- The precision of the integrals in the table is good as long as the integrand is linear.

National Nuclear Data Center

Building 817
Brookhaven National Laboratory
Upton, NY 11973-5000

About
History

National Nuclear Data Center Databases Structure

Evaluated Nuclear Data File (ENDF) Retrieval & Plotting

Periodic Table Browse Directory Tree Browse Basic Retrieval Advanced Retrieval Plot Cart **Computations**

The plot cart is empty.

Using the plot cart it is possible to draw different evaluations in a single image. To add evaluations to the plot cart, click on the "Add to plot cart" button in the individual plots, or click on the "Plot selected files" "plot all" buttons in the results output. Up to 15 plots can be plotted simultaneously using this feature. Click here for a more detailed description.

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About
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Databases

Contact Us

(631) 344-2802

Outline & interpreted versions provided by Bob MacFarlane's endf2htm

 National Nuclear Data Center
 Databases
 Structure & Decay

 **Evaluated Nuclear Data File (ENDF) Retrieval & Plotting**


Periodic Table Browse
Directory Tree Browse
Basic Retrieval
Advanced Retrieval
Plot Cart
Computations

Select first a library, then a sublibrary and finally click on a chemical element to obtain results.
Data are available for materials with a cyan background.

Library:
Sublibrary:

1																	2				
H																	He				
3	4															5	6	7	8	9	10
Li	Be															B	C	N	O	F	Ne
11	12															13	14	15	16	17	18
Na	Mg															Al	Si	P	S	Cl	Ar
19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36				
K	Ca	Sc	Ti	V	Cr	Mn	Fe	Co	Ni	Cu	Zn	Ga	Ge	As	Se	Br	Kr				
37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54				
Rb	Sr	Y	Zr	Nb	Mo	Tc	Ru	Rh	Pd	Ag	Cd	In	Sn	Sb	Te	I	Xe				
55	56	57	72	73	74	75	76	77	78	79	80	81	82	83	84	85	86				
Cs	Ba	La	Hf	Ta	W	Re	Os	Ir	Pt	Au	Hg	Tl	Pb	Bi	Po	At	Rn				
87	88	89	104	105	106	107	108	109	110	111											
Fr	Ra	Ac	Rf	Db	Sg	Bh	Hs	Mt	Ds	Rg											
58	59	60	61	62	63	64	65	66	67	68	69	70	71								
Ce	Pr	Nd	Pm	Sm	Eu	Gd	Tb	Dy	Ho	Er	Tm	Yb	Lu								
90	91	92	93	94	95	96	97	98	99	100	101	102	103								
Th	Pa	U	Np	Pu	Am	Cm	Bk	Cf	Es	Fm	Md	No	Lr								

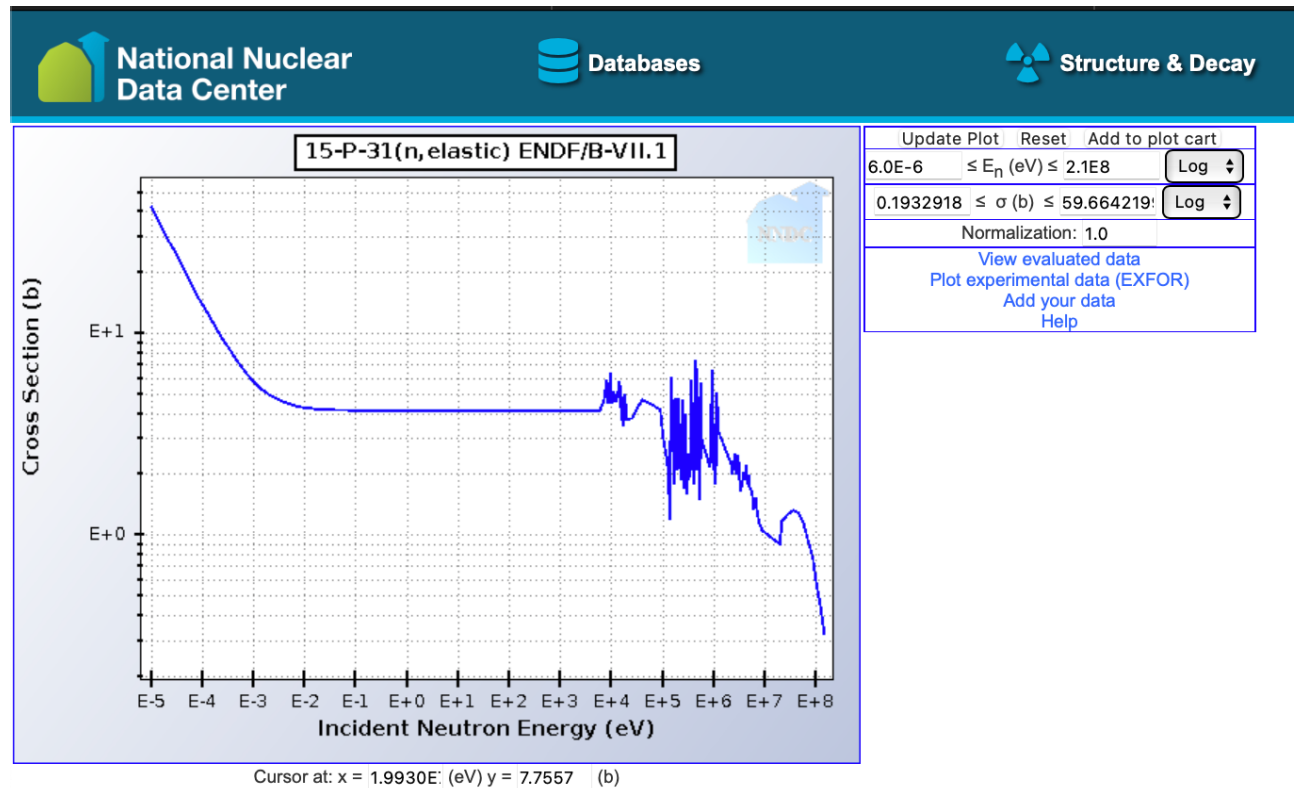
Results for **Z=15**

ENDF-6 format	Human-readable
Whole file - 	
Introduction	Interpreted
res. param.	Interpreted
Cross sections:	
(n,total)	Interpreted Plot
(n,elastic)	Interpreted Plot
(n,inelastic)	Interpreted Plot
(n,anything)	Interpreted Plot
(n,2n)	Interpreted Plot
(n,np)	Interpreted Plot
(n,n _γ)	Interpreted Plot
(n,ν)	Interpreted Plot
(n,p)	Interpreted Plot
(n,α)	Interpreted Plot
Angular distributions:	
(n,elastic)	Interpreted Plot
(n,2n)	Interpreted Plot
(n,np)	Interpreted Plot
(n,n _γ)	Interpreted Plot
Energy distributions:	
Full Spectra	Plot
(n,2n)	Interpreted Plot
(n,np)	Interpreted Plot
(n,n _γ)	Interpreted Plot
Energy-angle distributions:	

Version History:

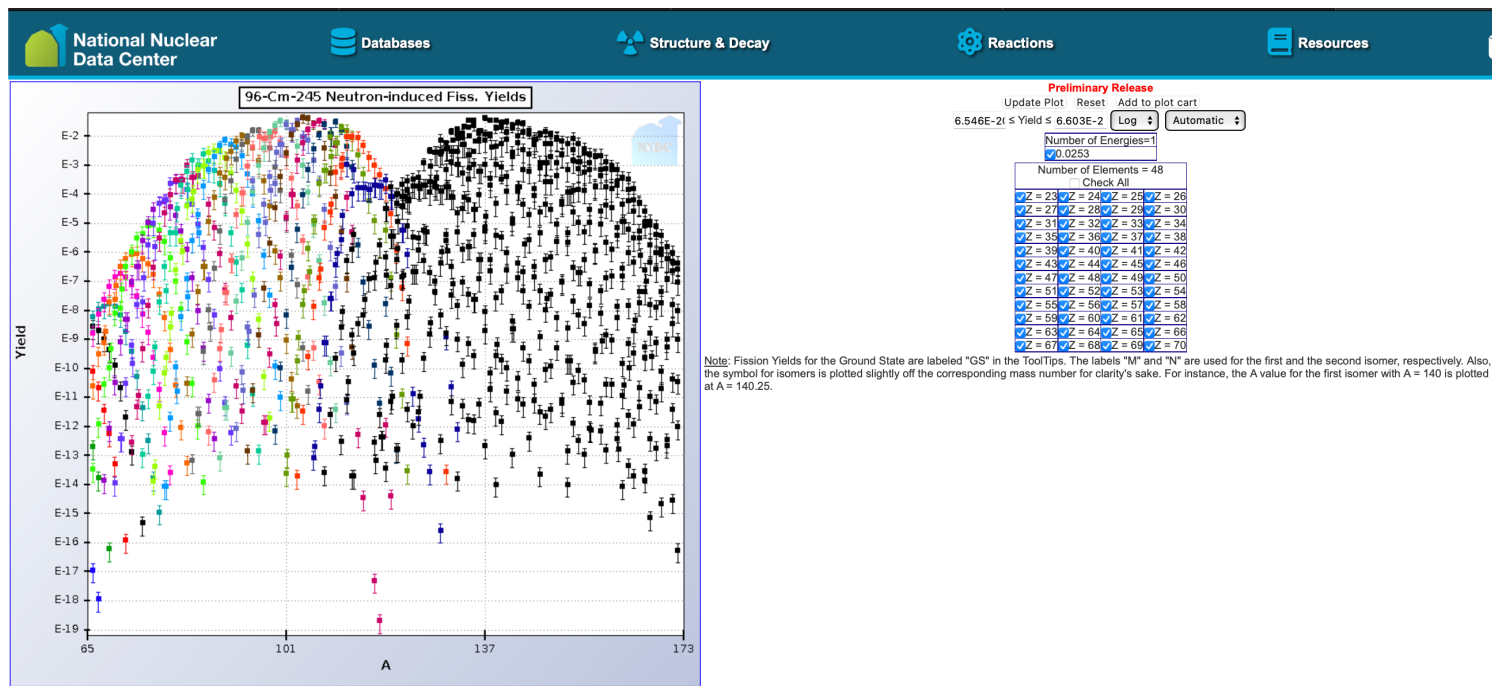
- **New: (December 2011)**
 - ENDF/B-VII.1 evaluated neutron library.
- + New in version 3.1 (October 2009)
- + New in version 3.0 (February 2009)
- + New in version 2.0 (April 2008)
- + New in version 1.0 (April 2007)

The plot link resolves to homebrew plotter with access to plot cart



To Zoom, left click on one of the limits and release the button on the other limit.
Zooming will work with the latest versions of FireFox and Internet Explorer.
Cross sections plots are reconstructed using the code PREPRO at T=300 K.

Most 2D data types can be plotted, as well as FPY



System not designed for heavy loads and is not flexible

Webserver:

- Provides the view
- Performs all needed computations <- don't do this!
- A lot of arcane glue <- very dark magic

MYSQL database:

- Contains precomputed items, including resonance reconstructed cross sections and Doppler broadened cross sections

All ENDF data must be preprocessed before loading to database

Codes in the backend & preprocessing steps (also significant amounts of java glue)

- endf2htm
- prepro (sigma1/recent)
- x4toc4
- lstab
- pltlst
- zvvddx <- only non-F77 code here!
- plotlst

It is getting harder to maintain these codes:

- New staff not trained on fortran
- gfortran *usually* works on them
- ENDF is cryptic & difficult
- EXFOR less cryptic & difficult
- Mapping between ENDF and EXFOR non-trivial

Proposal for a rewrite

- ENDF file preparation: use FUDGE and JSON!
 - ENDF -> GNDS (FUDGE does automatic resonance reconstruction)
 - Heating to RT
 - XML -> JSON is trivial
 - Need endf2htm analog
- EXFOR preparation
 - EXFOR -> JSON (four separate solutions!)
 - Need to replace EXFOR processing code (x4toc4)
- Replace MariaDB/MySQL with Couchdb or Mongodb
- Many off-the shelf client-side plotters available



GNDS

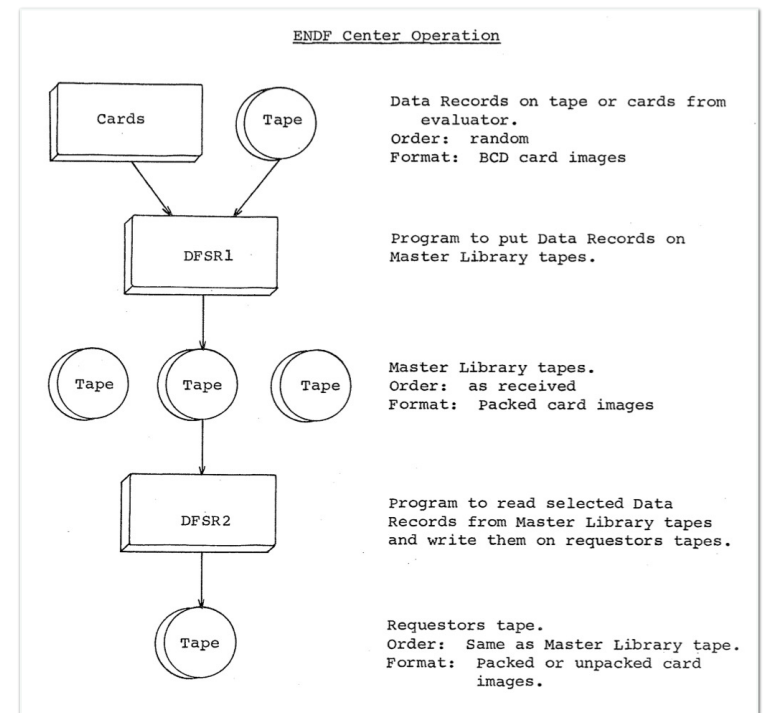
David Brown
National Nuclear Data Center, Brookhaven National Laboratory
Upton, NY 11973, USA

16 Jan 2024



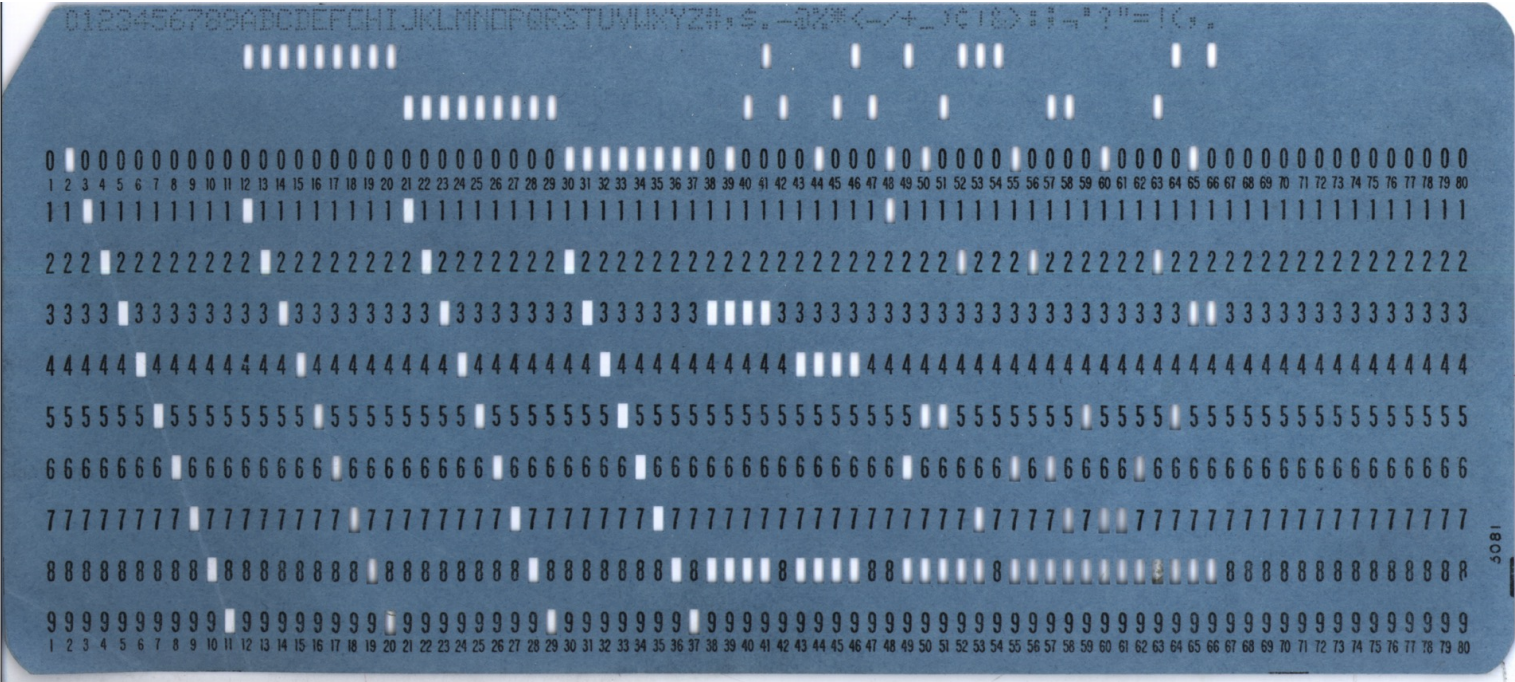
ENDF format & ontology was (and still is) tied to original infrastructure

- **Original format designed to fit on IBM 80 column punchcards**
 - Evaluations actually were occasionally submitted on punchcards
- **Original data stored on magnetic tapes**
- **It was possible to request ENDF data on tapes and/or punchcards**
 - Punchcard format was discouraged, BNL was trying to phase them out



From BNL-8381 (1966)

This is an IBM 80 column punchcard



https://en.wikipedia.org/wiki/Punched_card#/media/File:Blue-punch-card-front-horiz.png

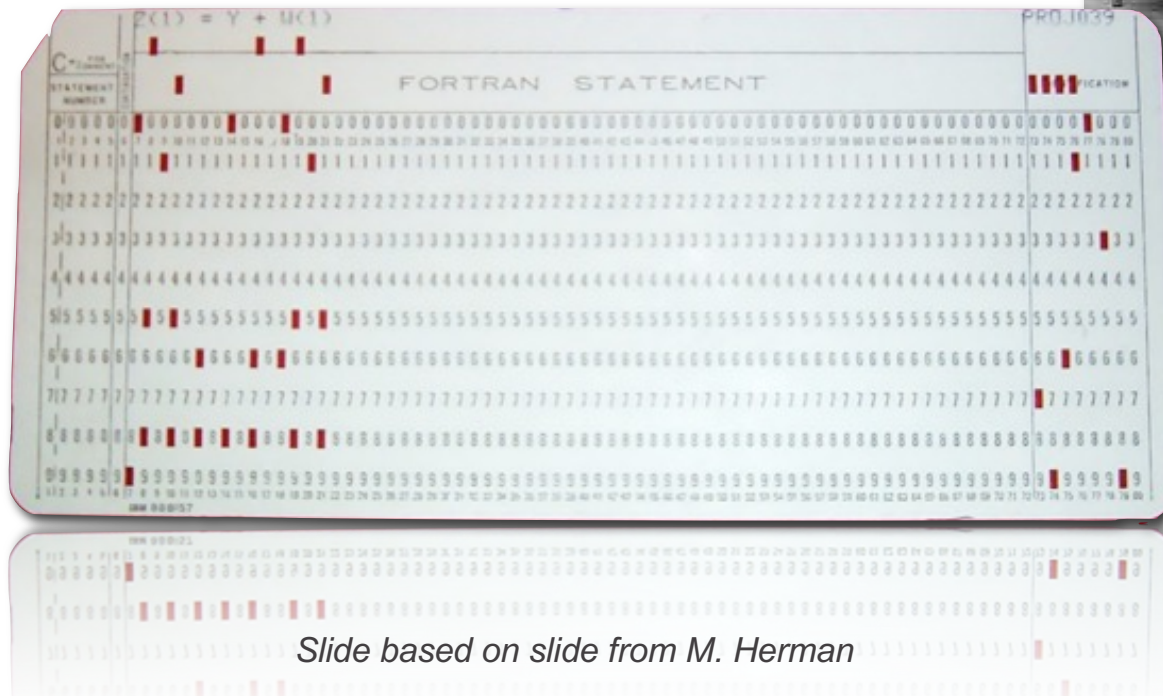
This is a chunk of the n+59Co evaluation: it's punchcard-ready

		14	83	1		02725	1451	286
		14	84	1		02725	1451	287
		14	85	1		02725	1451	288
		14	86	1		02725	1451	289
		14	87	1		02725	1451	290
		14	88	1		02725	1451	291
						2725	1	099999
						2725	0	0
2.705900+4	5.842690+1	0	0	1		02725	2151	1
2.705900+4	1.000000+0	0	0	1		02725	2151	
1.000000-5	1.000000+5	1	3	0		12725	2151	3
3.500000+0	6.672000-1	0	0	2		32725		
5.842690+1	6.672000-1	0	0	600		1002725		
-5.000000+3	3.000000+0	5.576800+2	9.215100+0	0.000000+0	0.000000+0	02725		
-5.000000+3	4.000000+0	1.898100+2	1.868200-1	0.000000+0	0.000000+0	02725		
-4.767000+2	4.000000+0	1.949000-2	2.148900+0	0.000000+0	0.000000+0	02725		
-2.258800+2	3.000000+0	9.164400+0	5.214100-2	0.000000+0	0.000000+0	02725		
1.320000+2	4.000000+0	5.270100+0	4.700000-1	0.000000+0	0.000000+0	02725		
4.323100+3	4.000000+0	1.041400+2	4.173700-1	0.000000+0	0.000000+0	02725		
5.016000+3	3.000000+0	6.789601+2	1.332200+0	0.000000+0	0.000000+0	02725		
6.389700+3	4.000000+0	1.681100+0	3.155600-1	0.000000+0	0.000000+0	02725	2151	13

Line number,
so you can
put your
punchcards
back in order
if you drop
them

Legacy formats

...but will we continue to be enslaved by this “modern technology”?



No, seriously

A good format can determine the data structures used to interact with it
These data structures are the components we use to create new things

**We are trying to create a development environment
(tools + components) that we enjoy working with**

We will be working with these tools for a long time

**Good tools == Happy
developers**

ENDF is resilient

Colony Restaurant closed 1963

AEC created CSEWG and ENDF; AEC ended in 1974, replaced with DOE in 1977

ENDF/B-V made “classified”, then unclassified

Management of CSEWG by DOE “faded away” in the 1990’s, but we kept going

Internet revolution(s)

11 US Gov’t administrations, so far

55th (-ish) anniversary this year



The most important part of ENDF is the ecosystem built on the format

PREPRO

NJOY

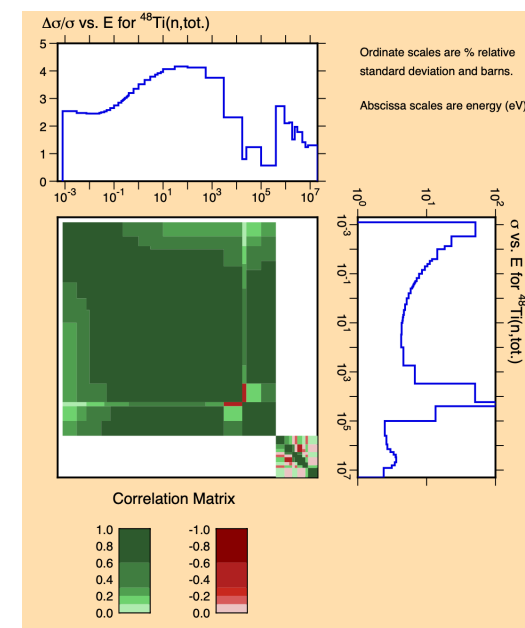
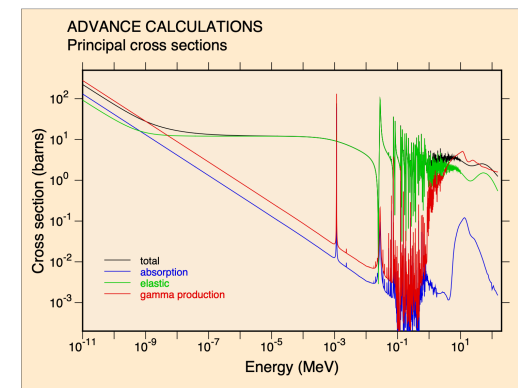
NNDC checking codes

AMPX

CALENDF

...

These are the tools that
get the data into user's
hands



ENDF is resilient, but...

Obsolete (and therefore confusing) constructs

- FEND, MEND, SEND and TEND “cards”
- line numbers (for the punchcards)

Limitations imposed by original physical format

- Fixed precision
- Limited MT's
- Limited MAT's

“Design by committee”

- MF6
- Fission data in MT1 not MT18
- Resonances
- ...

“Not fun to work with”, is often is barrier for newcomers



The biggest danger are the legacy tools becoming “black boxes”

Original developers are deceased, retired or soon to retire

- NJOY (LANL)
 - McFarlane retired
 - Kahler retired
- PREPRO (IAEA)
 - Cullen retired
- ndfgen/mcfgen (LLNL)
 - Perkins deceased
- AMPX (ORNL)
 - Greene retired
- CALENDF (CEA)
 - Ribon retired
- NNDC codes (BNL)
 - Dunford deceased

“if it ain’t broke, don’t fix it”, but...

A chance at a do-over

Want to preserve the evaluators' intent; but bad format meant evaluators put things in places they don't belong

- pseudo levels in 6,7Li (ab)used in MF4, before MF6 developed
- fission in W
- “battle over MT's” for high energy reactions
- gammas in MT3 or 4 rather than with the reaction that produced them
- ...

Bad design leads to mistakes, want to engineer them away

- Backgrounds in resonance region
- Multiple ways to store the same thing (gammas in MF12,13,14 vs. gammas in MF6), possible double counting
- Synchronization issues (masses, levels, ...)
- ...

ENDF & GNDS Formats

• New NCSP relevant formats

- Mixed elastic scattering TSL data for nuclides with both coherent and incoherent components (M. Zerkle NNL)
- Background R-matrix format (J. Brown ORNL)

• GNDS & ENDF have similar format approval process

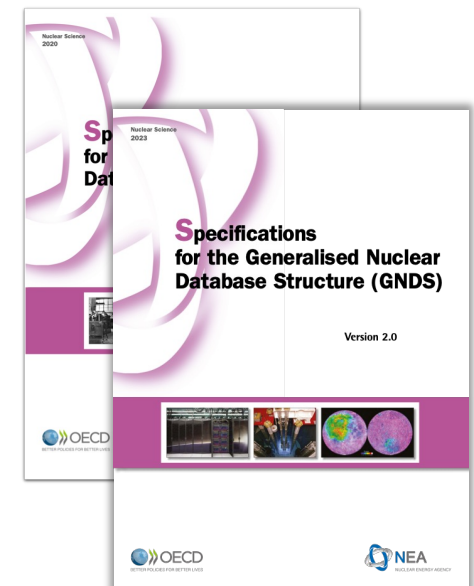
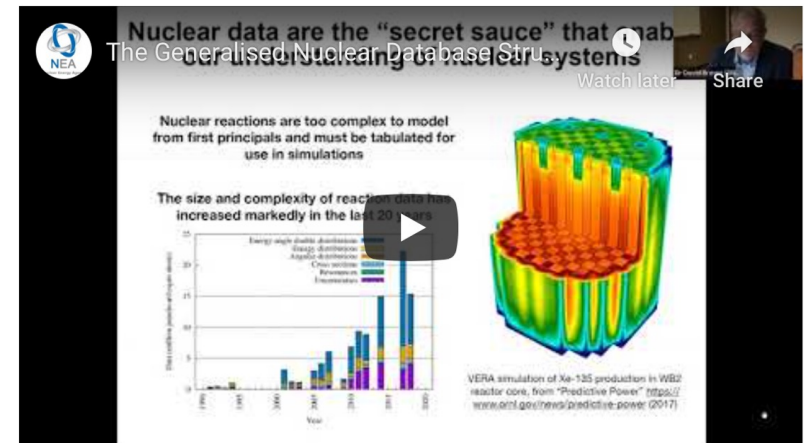
- Harness power of gitlab
- Contact D. Brown (EG-GNDS chair) or D. Wiarda (ENDF Formats chair) for information

• GNDS-1.9 published

- Specifications: <https://oecd-nea.org/download/wpec/documents/7519-GNDS.pdf>
- XML Schema for GNDS-1.9: <https://www.oecd-nea.org/download/wpec/gnds/gnds.xsd>
- GNDS Webinar: <https://www.youtube.com/watch?v=h9Byrkxr8LE&feature=youtu.be>

• GNDS-2.0 published

- Specifications: https://oecd-nea.org/upload/docs/application/pdf/2023-09/7647_gnds-2.0.pdf





GNDS Data Model

GNDS types of files (“top level nodes”)

reactionSuite – reaction data for transport/inventory calculations

covarianceSuite – uncertainty data for the reactionSuite

PoPs – properties of particles, subsuming RIPL-like functions and the decay sublibrary. Planned for major revisions in next GNDS, now that there is an ENSDF-JSON.

fissionFragmentData – fission product yield data is separate since it is bigg-ish and not needed for every application. Planned for major revisions in next GNDS.

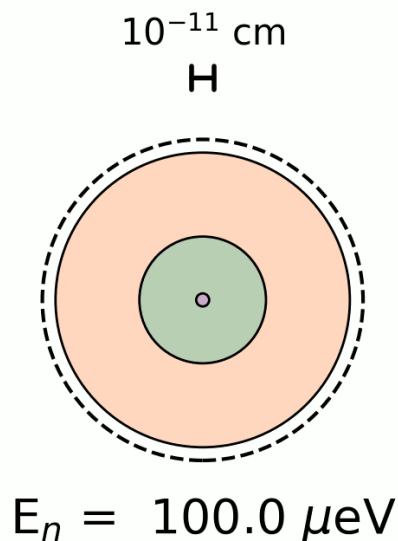
map – like NJOY’s xmdir file, but with some added functionality. Great for organizing a library.

The reactionSuite data model is modeled after particle transport

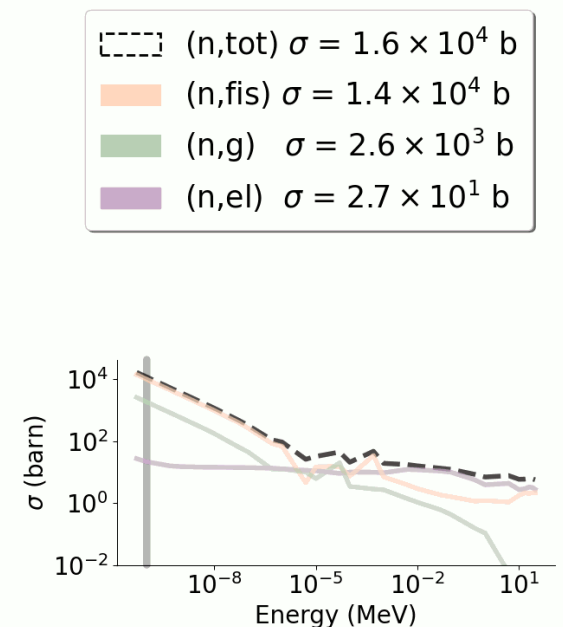
Transport algorithm in a nutshell:

- In a zone, use atom density and total cross sections to decide what to hit
- In a nucleus, use partial cross sections to determine what reaction to sample
- In a reaction, sample outgoing distributions for each emitted particle

Cross Section for neutrons on ^{235}U



From Matteo Vorabbi



All ENDF data has a home in GNDS, but the arrangement is much more rational

reactionSuite

- **styles** – different styles of data live in same structure (evaluated, various kinds of processed data, etc.)
- PoPs – the internal particle database
- resonances – resonance data (gets processed into pointwise cross sections)
- **reactions** – main collection of data
- sums – ENDF's many sumrules, rationalized
- productions – production cross sections
- applicationData – optional institution specific information

Dive into the reactions

```
<reactions>
  <reaction label="n + Al27" ENDF_MT="2">
    <crossSection>
      <resonancesWithBackground label="eval"> ... </resonancesWithBackground>
      <XYS1d label="recon"> ... </XYS1d></crossSection>
    <outputChannel genre="twoBody">
      <Q> ... </Q>
      <products>
        <product pid="n" label="n">
          <multiplicity> ... </multiplicity>
          <distribution>
            <angularTwoBody label="eval" productFrame="centerOfMass"> ... </angularTwoBody></distribution></product>
          <product pid="Al27" label="Al27">
            <multiplicity> ... </multiplicity>
            <distribution>
              <angularTwoBody label="eval" productFrame="centerOfMass"> ... </angularTwoBody></distribution></product></
          </products></outputChannel></reaction>
```


Styles contain the documentation for each style of data in evaluation

```
<styles>
  <evaluated label="eval" date="2001-02-01" library="ENDF/B" version="8.0.1">
    <documentation>
      <authors>
        <author name="M.B.Chadwick+,Derrien+"></author></authors>
      <dates>
        <date value="2001-02-01" dateType="created"/></dates>
      <title><![CDATA[ENDF-6 file form ENDF/B version 8.0.1 translated to GNDS by FUDGE.]]></title>
      <body><![CDATA[See the endfCompatible section.]]></body>
      <endfCompatible><![CDATA[ ]]></endfCompatible></documentation>
      <temperature value="0" unit="K"/>
      <projectileEnergyDomain min="1e-5" max="1.5e8" unit="eV"/></evaluated>
    <crossSectionReconstructed label="recon" derivedFrom="eval" date="2016-11-06"></crossSectionReconstructed></styles>
  <ReDs name="prototype-internal" version="1.0" format="2.0"></ReDs>
```

The evaluated style is the one created by the evaluator, so it has the ENDF documentation. We have a much more expressive documentation markup, based on DataCite schema, but can't really use it for legacy data.

Example files