

# High Density as an Avenue towards High Confinement Quality and Core-Edge Integration in Advanced Tokamaks

S. Ding<sup>1</sup>, A. M. Garofalo<sup>1</sup>, X. Z. Gong<sup>2</sup>, H. Q. Wang<sup>1</sup>, L. Wang<sup>2</sup>, W. Choi<sup>1</sup>, J. P. Qian<sup>2</sup>, J. Huang<sup>2</sup>, M. Kotschenreuther<sup>3</sup>, D. Hatch<sup>3</sup>, S. Mahajan<sup>3</sup>, D. B. Weisberg<sup>1</sup>, Z. Y. Li<sup>1</sup>, Z. Yan<sup>4</sup>, X. Jian<sup>2</sup>, S.-G. Baek<sup>5</sup>, P. Bonoli<sup>5</sup>, G. Wallace<sup>5</sup>, D. Eldon<sup>1</sup>, B. S. Victor<sup>6</sup>, A. Marinoni<sup>7</sup>, Q. M. Hu<sup>8</sup>, I. S. Carvalho<sup>1</sup>, T. Odstrčil<sup>1</sup>, K. D. Li<sup>2</sup>, A. W. Hyatt<sup>1</sup>, T. H. Osborne<sup>1</sup>, J. McClenaghan<sup>1</sup>, C. T. Holcomb<sup>6</sup>, J. M. Hanson<sup>9</sup>, Y. X. Sun<sup>2,10</sup> and Z. H. Wang<sup>2,10</sup>

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2<sup>nd</sup> IAEA Technical Meeting on Long-Pulse Operation of Fusion Devices  
Oct 14-18, 2024  
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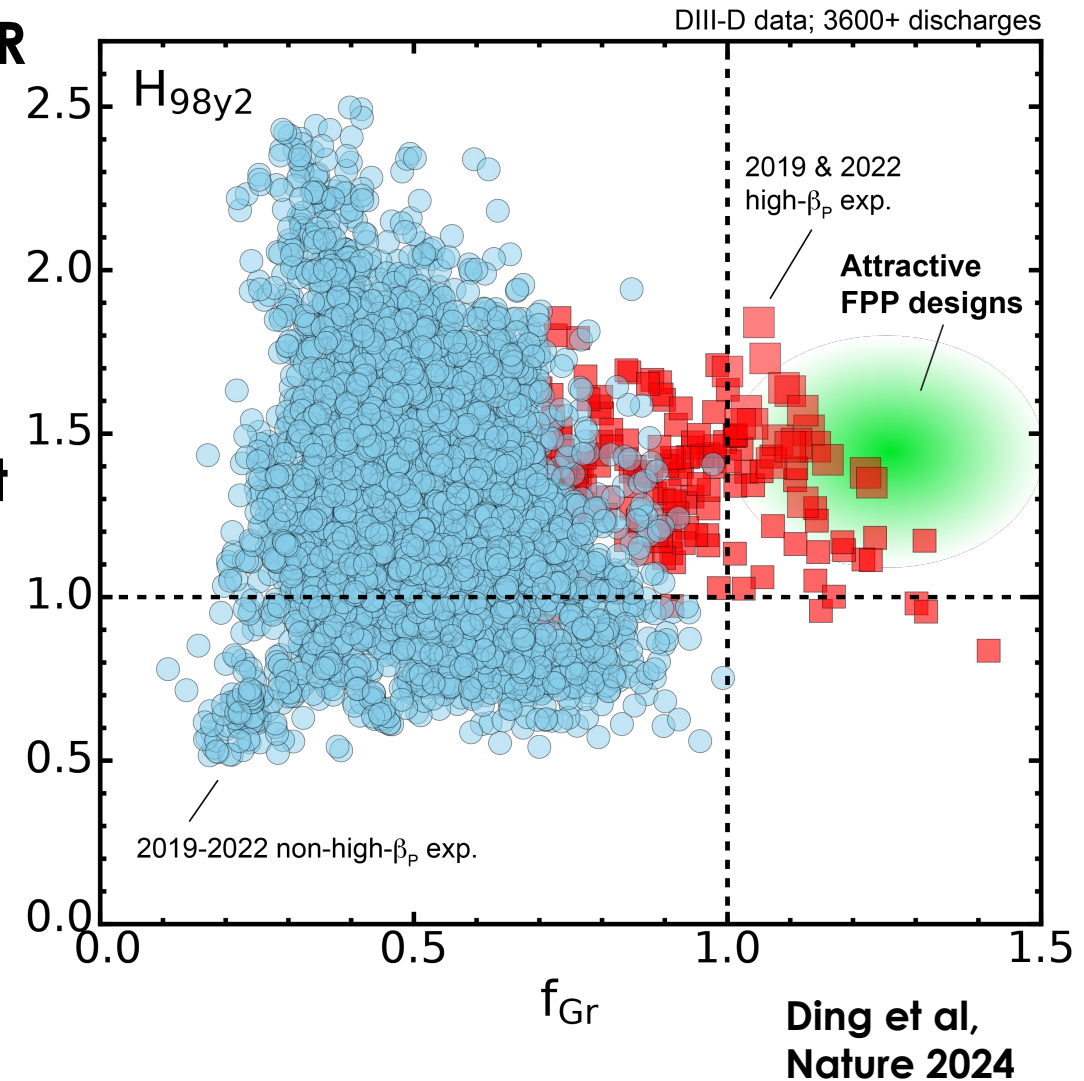
2<sup>nd</sup> IAEA Technical Meeting on Long-Pulse Operation of Fusion Devices, Oct 14-18, 2024, Vienna, Austria

Oct 14-18, 2024, Vienna, Austria  
View

**First demonstration of high  $\beta_p$  scenario with large radius ITB obtained on DIII-D with KSTAR operation constraints on June 14! [YoungMu Jeon, et al, APS 2024]**

# Takeaway: Breakthroughs Achieved for Improved Energy Confinement at Plasma Density near the Greenwald Value

- Both steady-state fusion pilot plants (FPPs) and ITER  $Q=10$  at significantly lower  $I_p$  require density Greenwald fraction ( $f_{Gr}$ )  $> 1$  with  $H_{98y2} > 1$
- High- $\beta_p$  experiments on DIII-D achieve first such demonstration in a tokamak
- Key physics: enhanced  $\alpha$ -stabilization of turbulent transport at high density gradient
- Same physics applied to EAST leads to nearly doubled  $T_i$
- Excellent core-edge integration: small ELMs and divertor detachment also achieved at high  $H_{98y2}$  and high  $f_{Gr}$



# Outline

- On the performance requirements for steady-state fusion pilot plants (FPPs) and for ITER  $Q=10$   $P_{fus}=500$  MW operation at reduced  $I_p$
- DIII-D high- $\beta_p$  experiments and transport analysis
- Simulations of EAST high- $\beta_p$  plasmas and theory-guided experiments
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# High Energy Confinement Quality and High Fuel Density are Two Key Elements for Economically Attractive Fusion

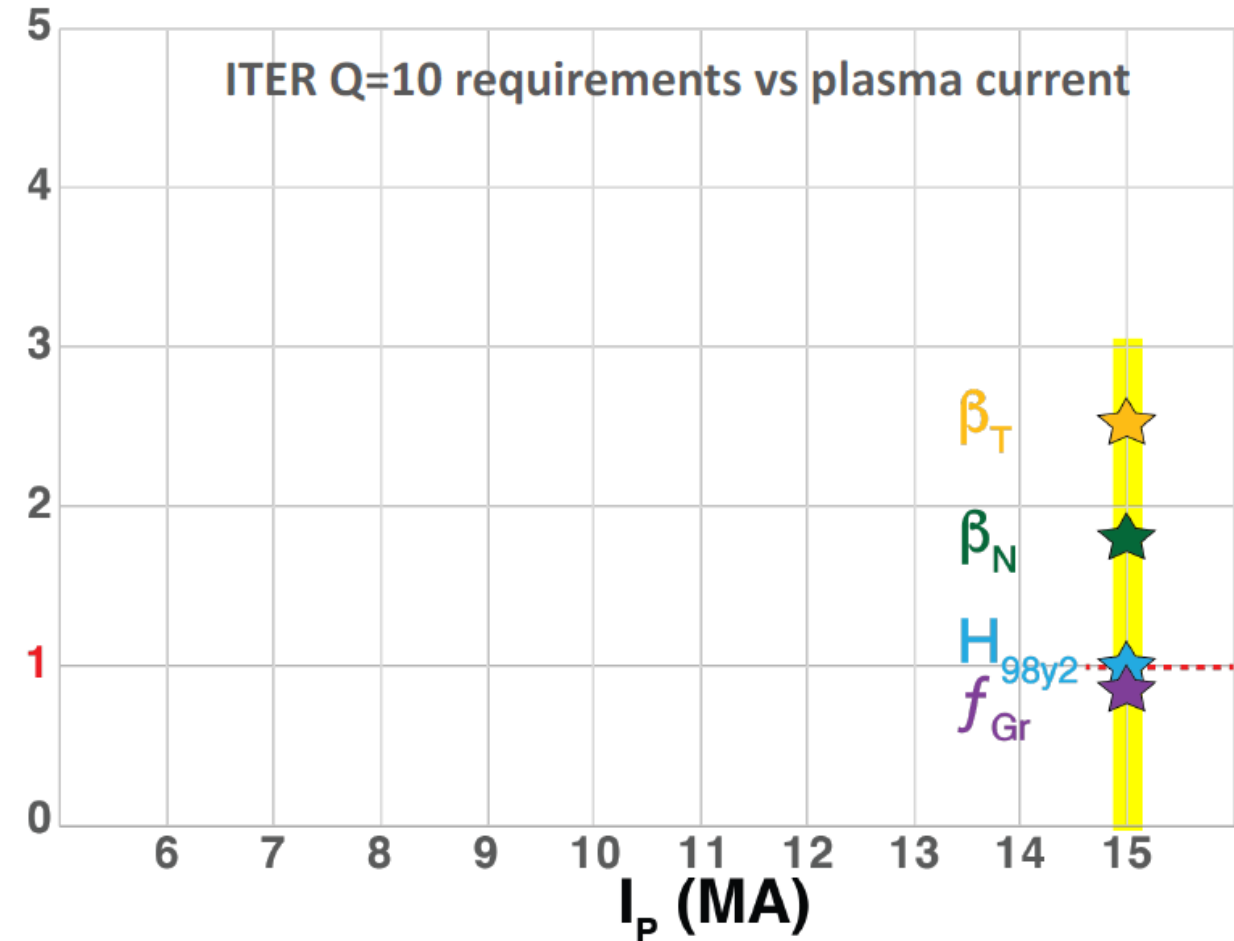
- **Energy confinement quality ( $H_{98y2}$ ) is the highest leverage parameter for fusion capital cost** Wade, Fusion Sci. Technol. 2021      Hammett, Maxwell Prize Address, APS 2024
- **Thermonuclear power density  $p \sim n_i^2 \langle \sigma v \rangle$**  Wesson, Tokamaks 2004
- **Both  $f_{Gr}$  and  $H_{98y2}$  above unity proposed in steady-state FPP Designs**
  - $f_{Gr} = \text{line-avg } n_e / n_{Gr}$
  - $n_{Gr} = I_p / \pi a^2$ , an empirical density limit on H-mode pedestal

FPP Device	$f_{Gr}$	$H_{98y2}$
CFETR	0.96	1.42
K-DEMO	1.0, 1.13	$\geq 1.3$
ARIES-ACT	1.0, 1.3	1.22 – 1.65
CAT-DEMO	$> 1.0$	1.2 – 1.51
GA-FPP	1.2	$\geq 1.5$

Zhuang, Nucl. Fusion 2019  
 Yeom, Fusion Eng. Des. 2013  
 Kessel, Fusion Sci. Technol. 2015  
 Wade, Fusion Sci. Technol. 2021  
 Buttery, Nucl. Fusion 2021  
 Shi, APS 2022  
 Shi, APS 2023

# ITER Operation at $I_p$ Significantly Lower than 15 MA could Provide A Safer Path to $Q=10$ $P_{fus}=500$ MW, but Requires $H_{98y2}>1.2$ at $f_{Gr} \geq 1$

- **15 MA approach:  $q_{95} \sim 3$ ,  $H_{98y2} = 1.0$ ,  $f_{Gr} = 0.85$** 
  - Risks: Disruption, MHD, divertor heat flux...



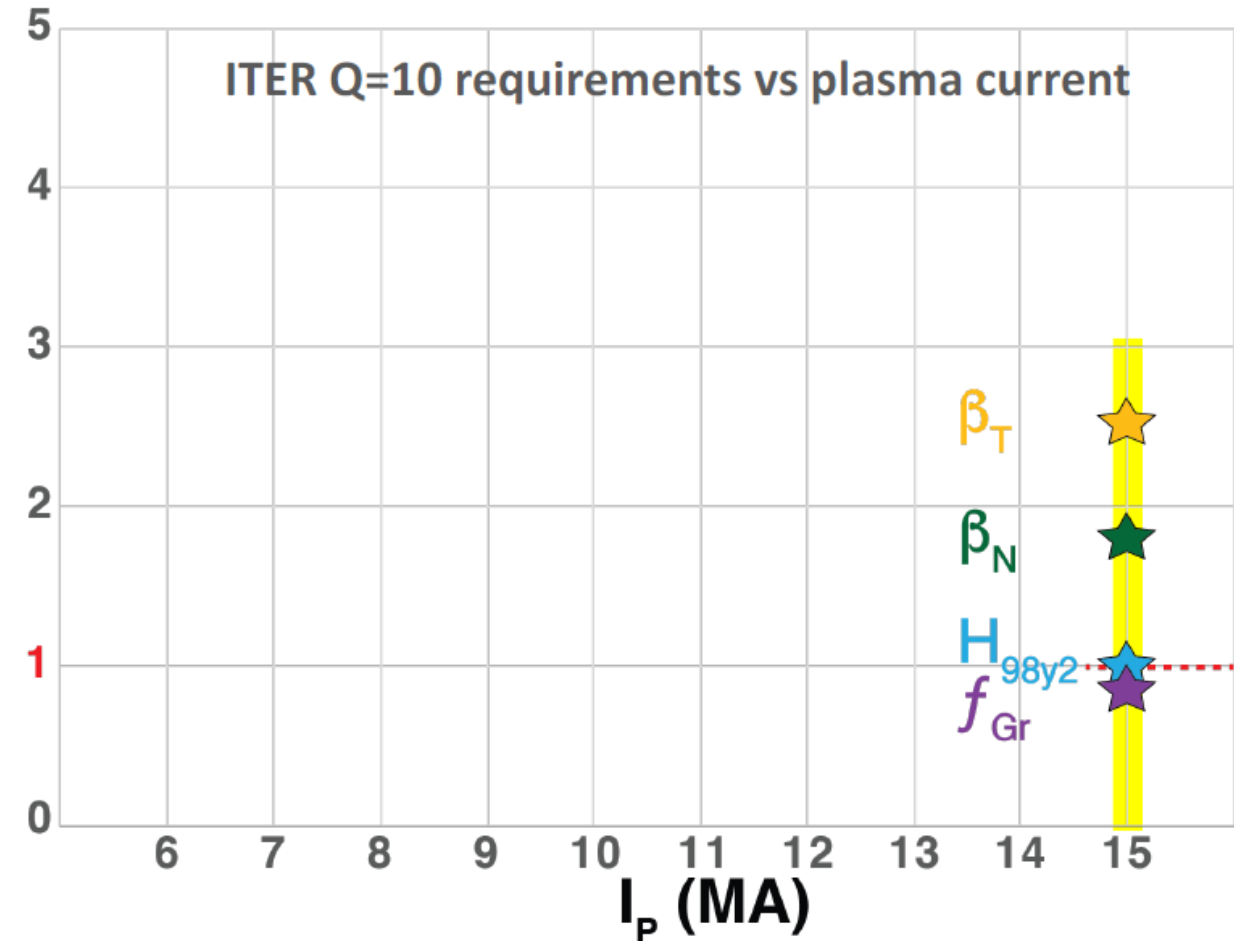
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**ITER is seeking an alternative higher  $q_{95}$  ( $\geq 6$ ) plan for its  $Q=10$  mission**

- Target  $I_p \geq 7.5$  MA

Loarte, ITR-24-004 (2024)

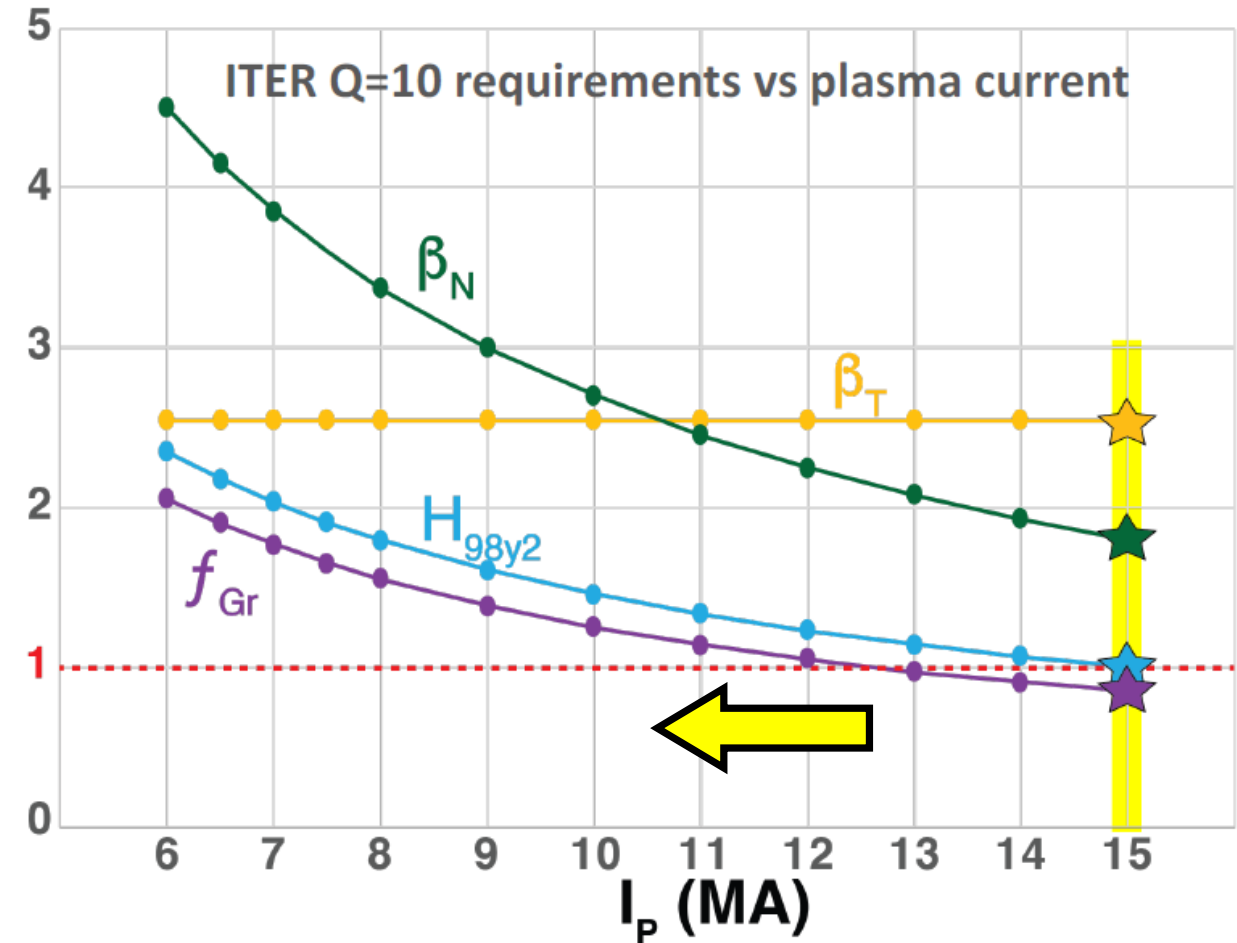




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  - Risks: Disruptions, MHD, divertor heat flux...
- **Reduce  $I_p$ , while keeping constant  $B_T$ ,  $P_{aux}$ ,  $n$  and  $T$  in 0D modeling**
  - Constant  $\tau_E$
  - Constant  $Q$  and  $P_{fus}$
- $\beta_T = nk_B T / (B^2 / 2\mu_0) = \text{constant}$
- $\beta_N = \beta_T / (I_p / aB) \sim 1 / I_p$
- $f_{Gr} = \langle n_e \rangle / (I_p / \pi a^2) \sim 1 / I_p$
- $H_{98y2} = \tau_E / \tau_{98y2} \sim 1 / I_p^{0.93}$

$$\tau_{98y2} = 0.0562 I_p^{0.93} B_T^{0.15} n_e^{0.41} P_H^{-0.69} M^{0.19} R^{1.97} \epsilon^{0.58} \kappa^{0.78}$$



Garofalo et al, APS 2024

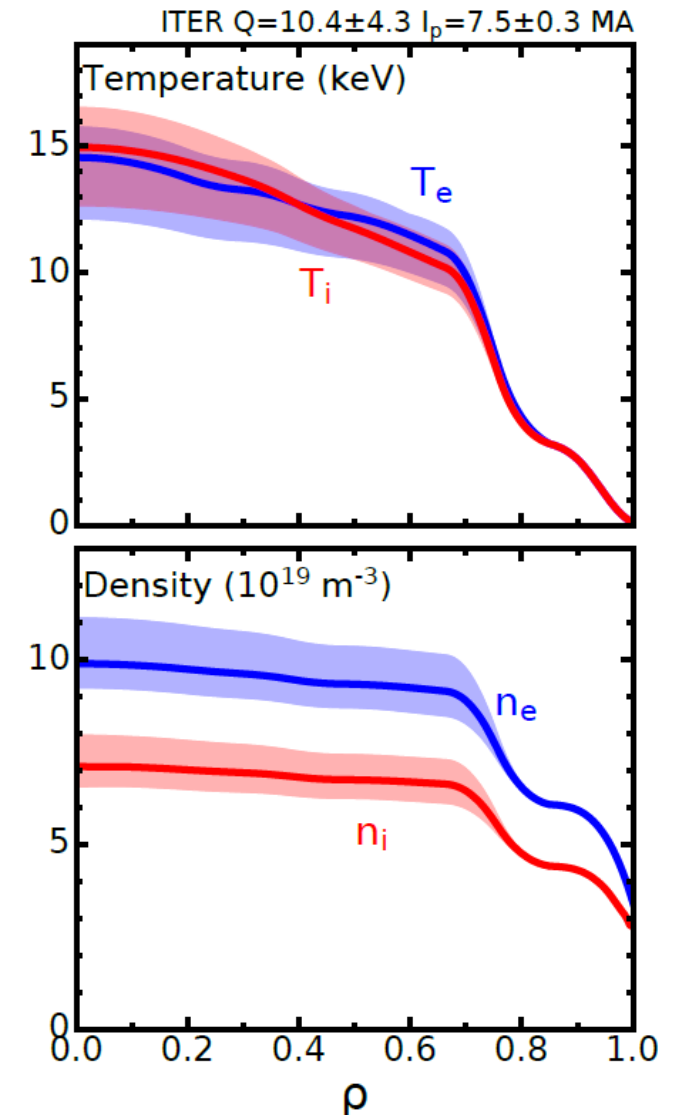
# Theory-based High Fidelity Modeling Shows High $\beta_p$ Scenario Can Enable ITER Q Goals

- Self-consistent ITER modeling using high  $\beta_p$  scenario by STEP in OMFIT
- Q=5 steady-state predicted with  $f_{Gr} \sim 1.3$  and  $H_{98y2} > 1$  at  $I_p \sim 8.3$  MA ( $q_{95} \sim 7$ ,  $\beta_N \sim 3.0$ )
  - Day-1 H&CD powers
  - $P_{fus} \sim 400$  MW
- Q=10 predicted with  $f_{Gr} \sim 1.4$  and  $H_{98y2} \sim 1.7$  at  $I_p \sim 7.5$  MA ( $q_{95} \sim 8$ ,  $\beta_N \sim 2.8$ )
  - Day-1 H&CD powers
  - $P_{fus} \sim 350$  MW

McClenaghan et al,  
Nucl. Fus. 2020

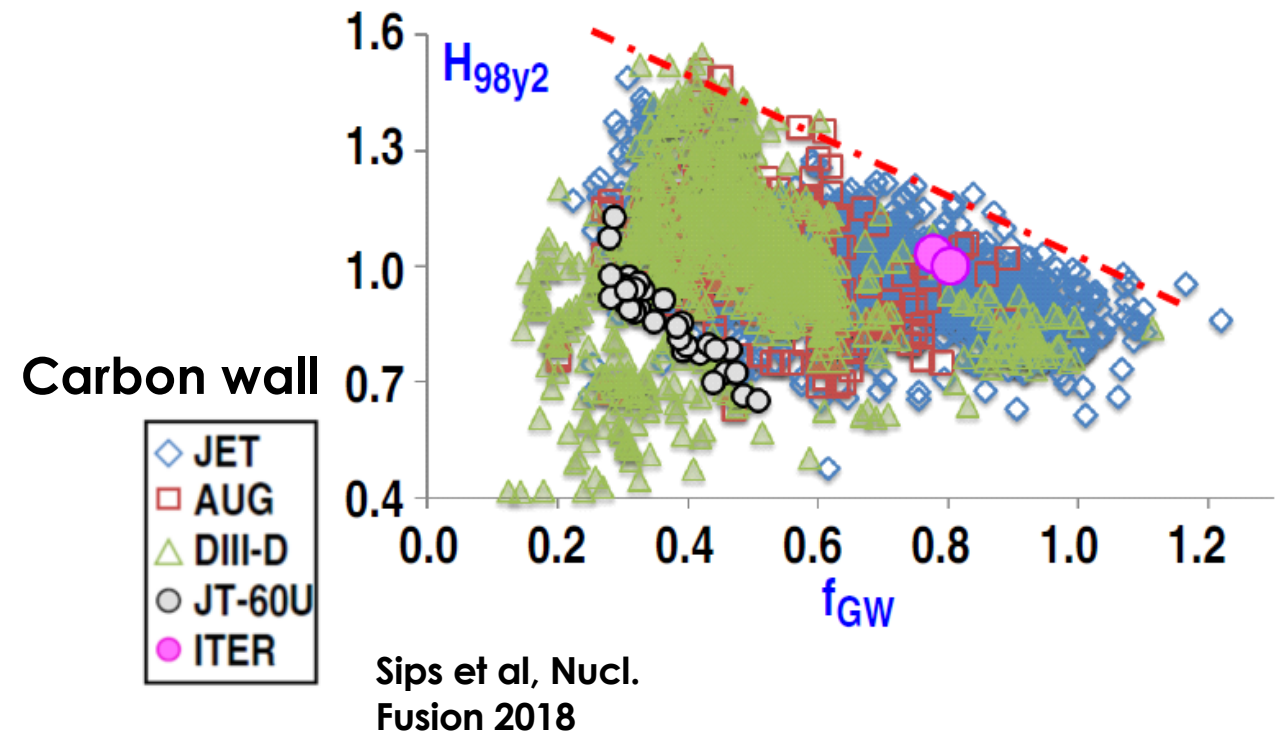
Ding et al, IAEA FEC  
2021

Operating regimes with  $f_{Gr} > 1$  and  $H_{98y2} > 1$  need to be demonstrated in experiments



# Simultaneous $f_{Gr} > 1$ and $H_{98y2} > 1$ is Difficult for Experiments in Present Tokamaks $\rightarrow$ a “Performance” Limit

- ITPA database for ITER Q=10 H-mode  $q_{95}=2.7-3.3$ 
  - International tokamaks
  - No constraints on toroidal rotation/injected torque

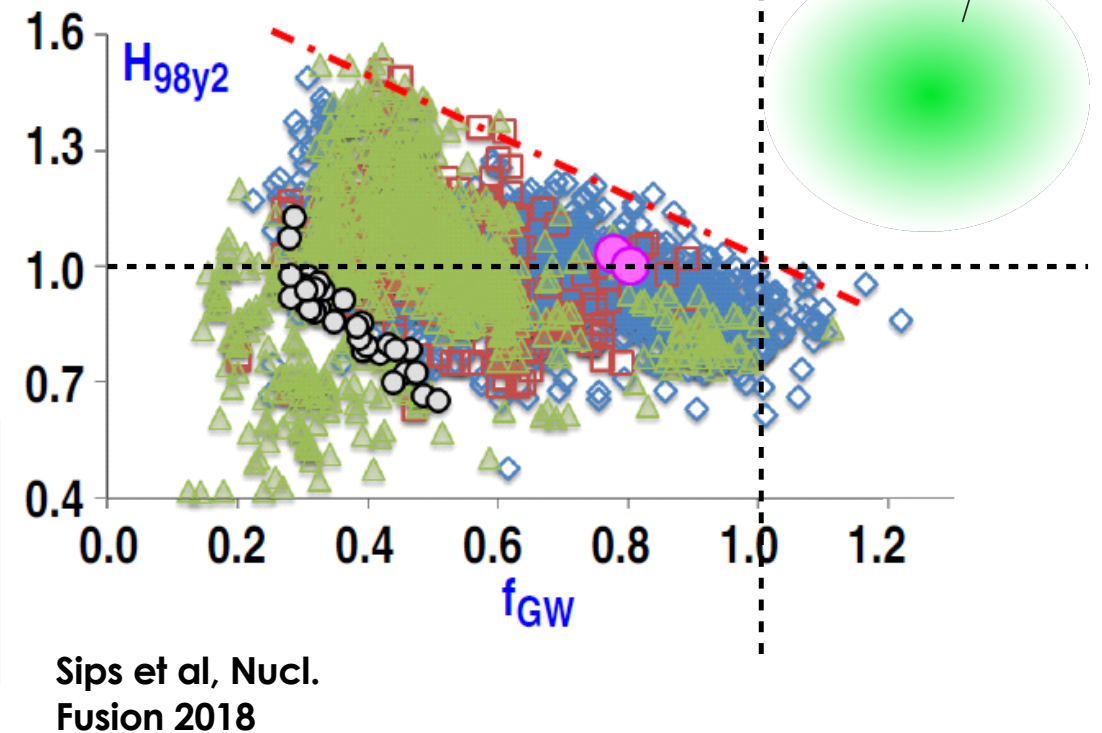
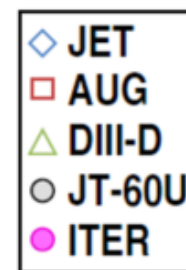


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No experiment shows significant  $H_{98y2} > 1$  at  $f_{Gr} > 1$

Carbon wall

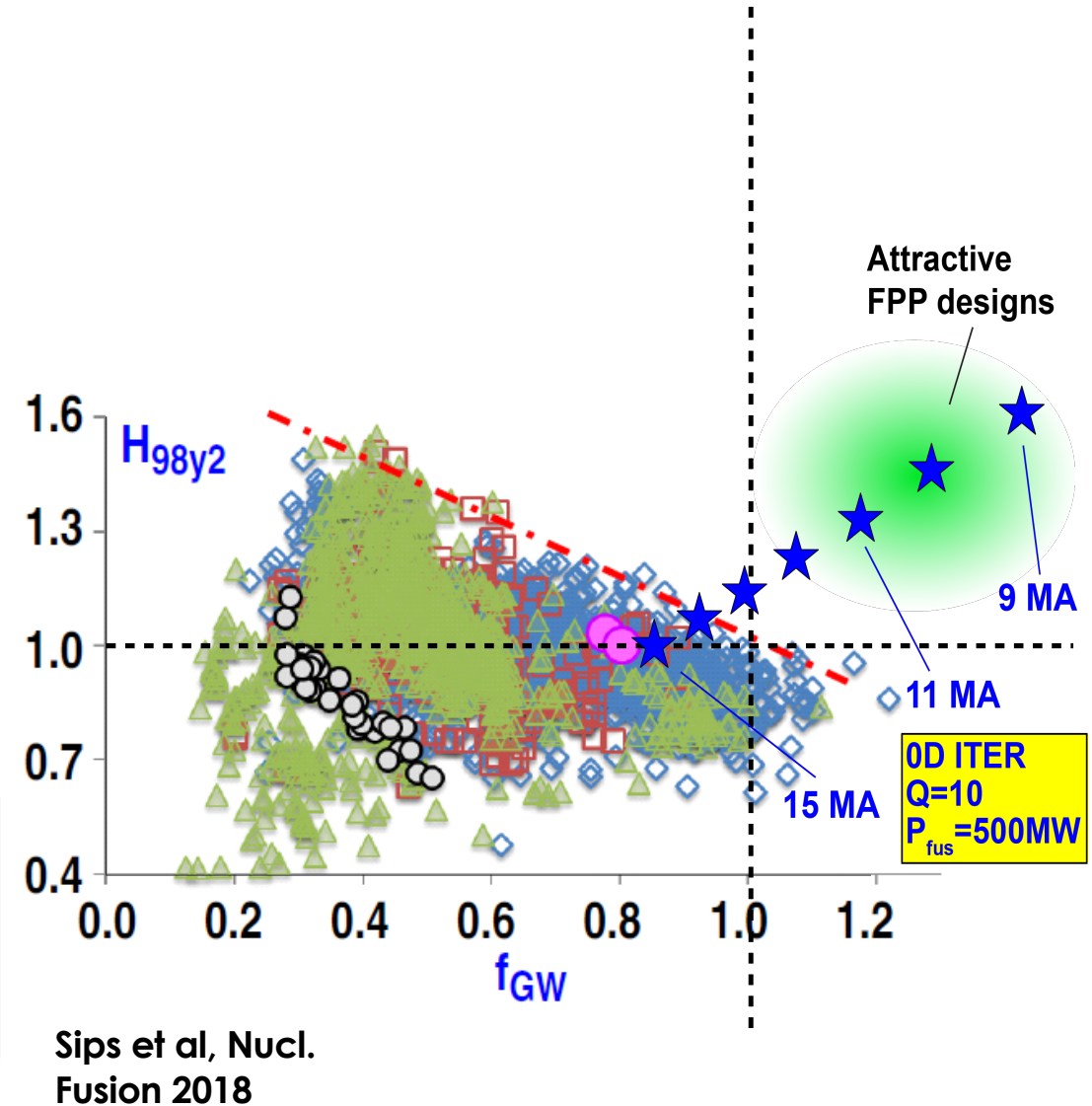
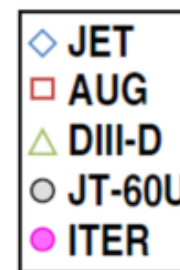


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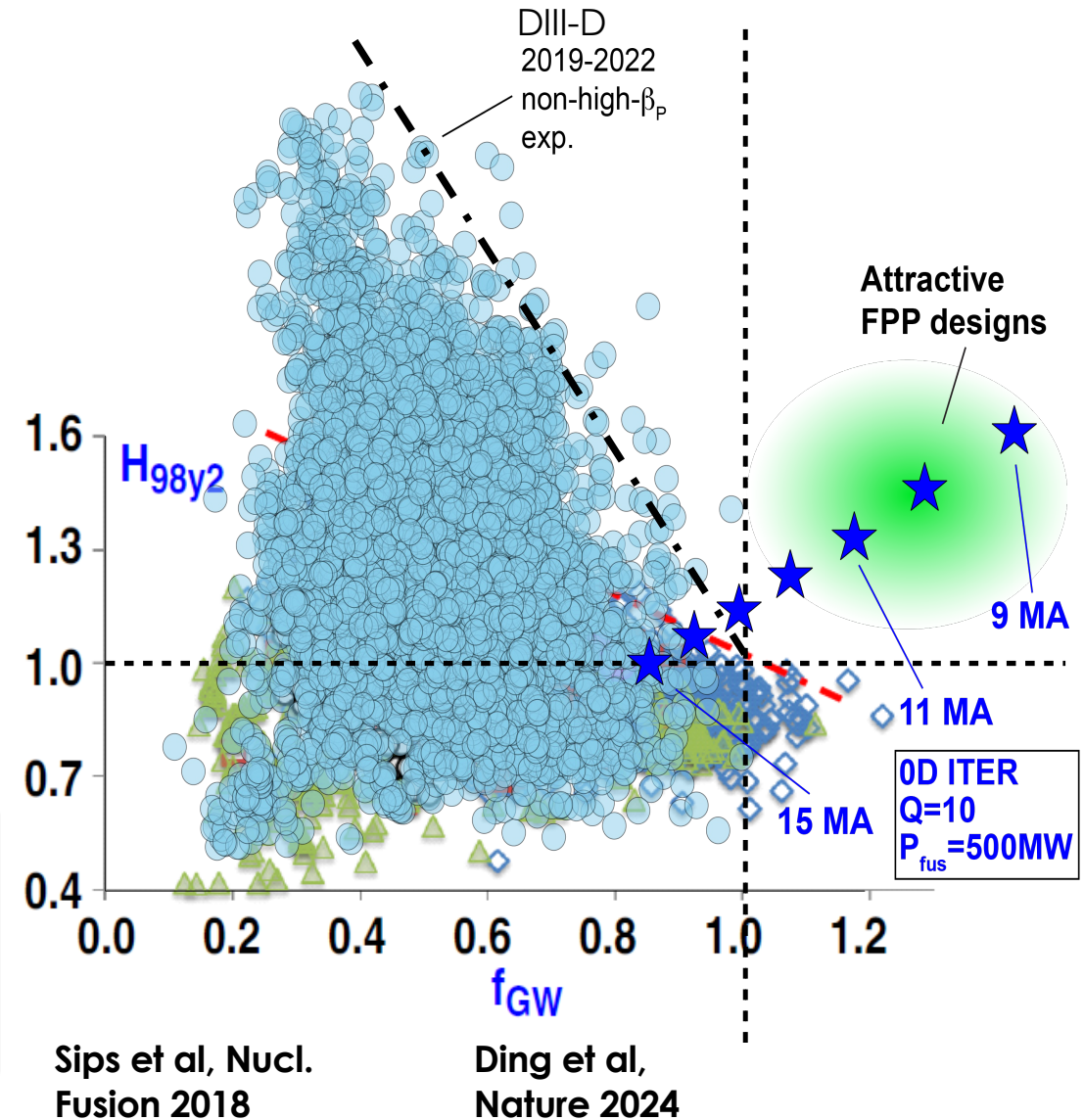
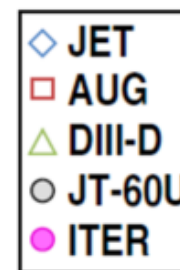


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- Remove  $q_{95}$  constraint, single tokamak (DIII-D)

**Still** No experiment shows significant  $H_{98y2} > 1$  at  $f_{Gr} > 1$

Carbon wall

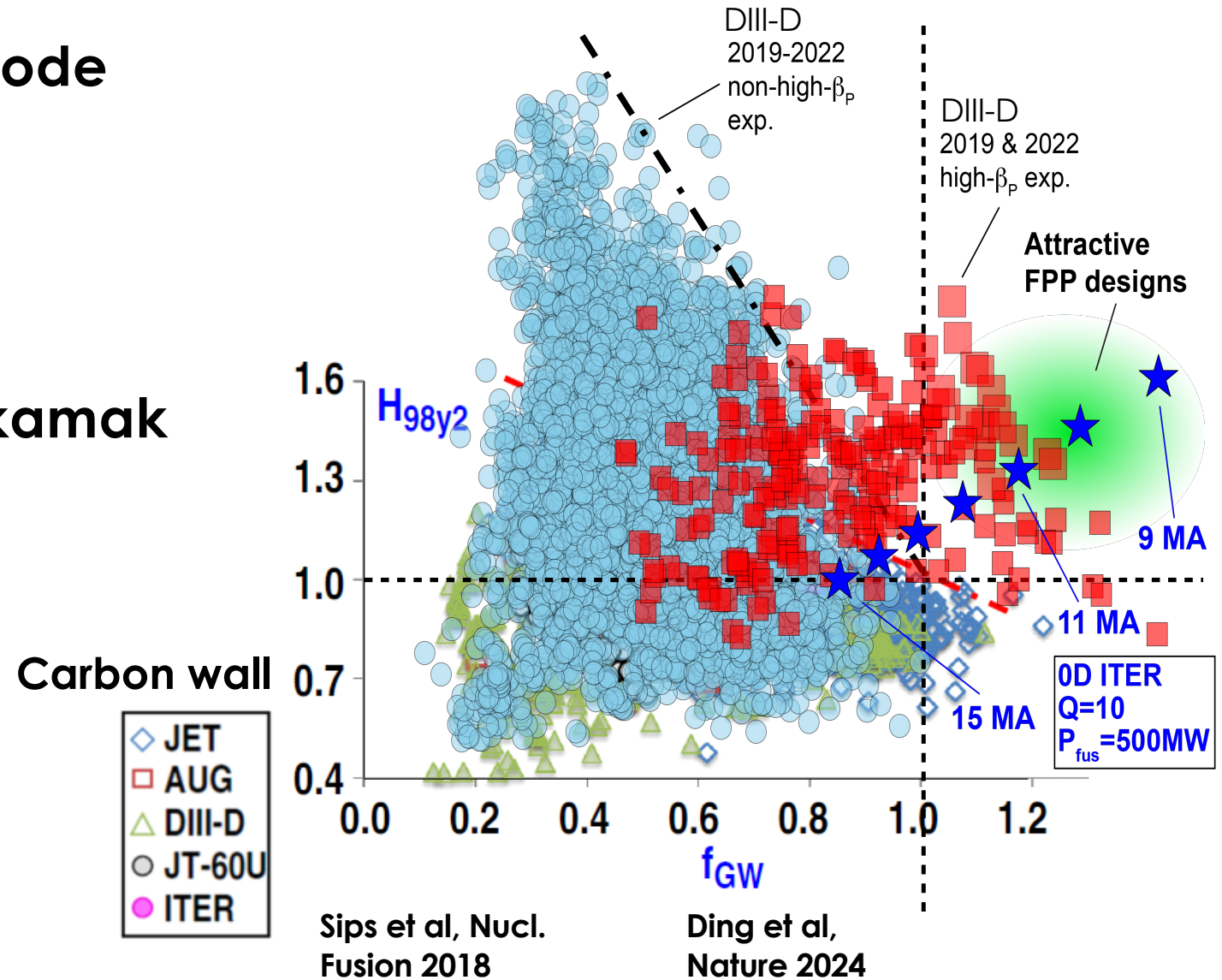


# Only High- $\beta_p$ Scenario Has Overcome the Performance Limit

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Until breakthrough from high- $\beta_p$  experiments

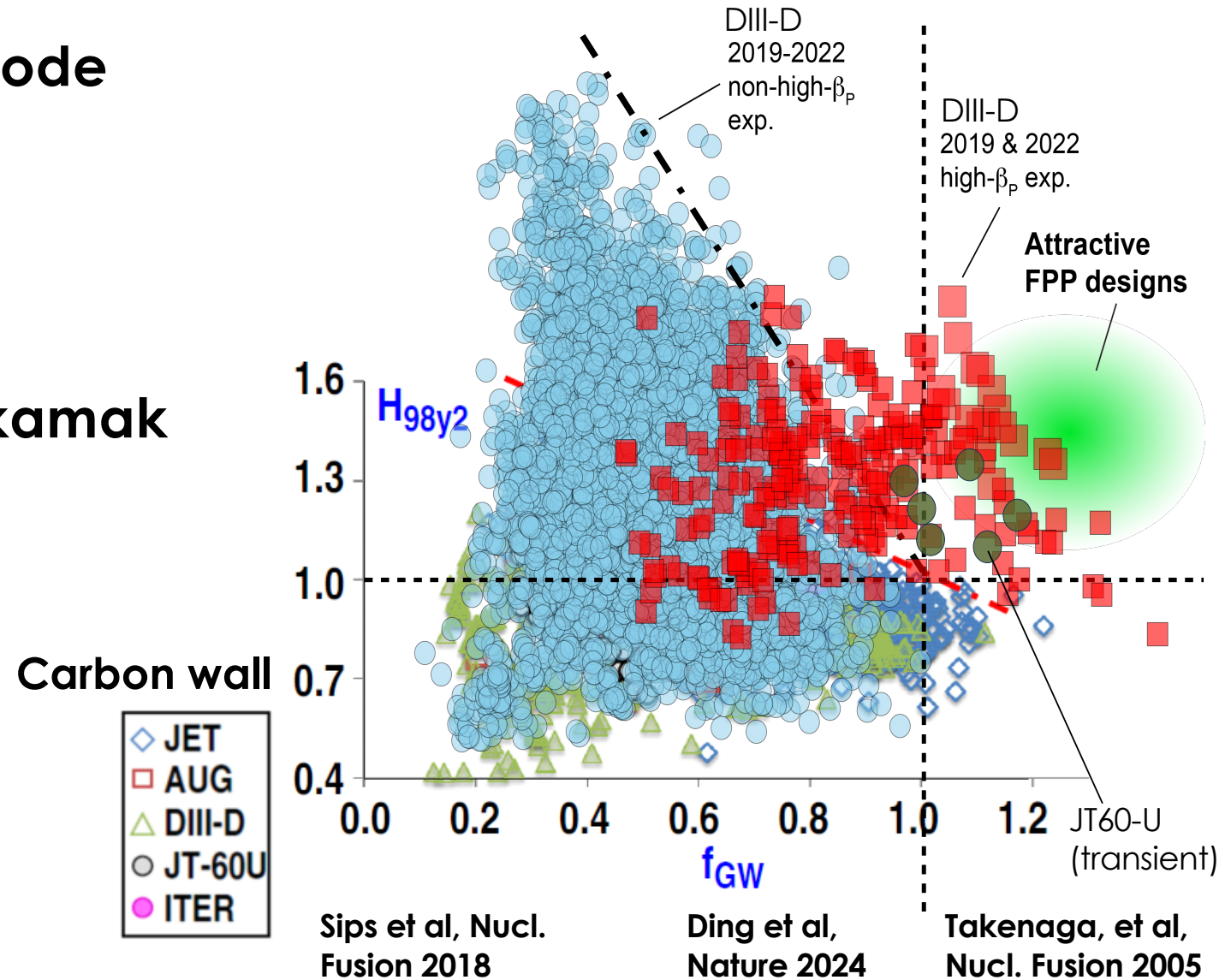


# Only High- $\beta_p$ Scenario Has Overcome the Performance Limit, both on JT60-U and DIII-D

- ITPA database for ITER Q=10 H-mode  $q_{95}=2.7-3.3$ 
  - International tokamaks
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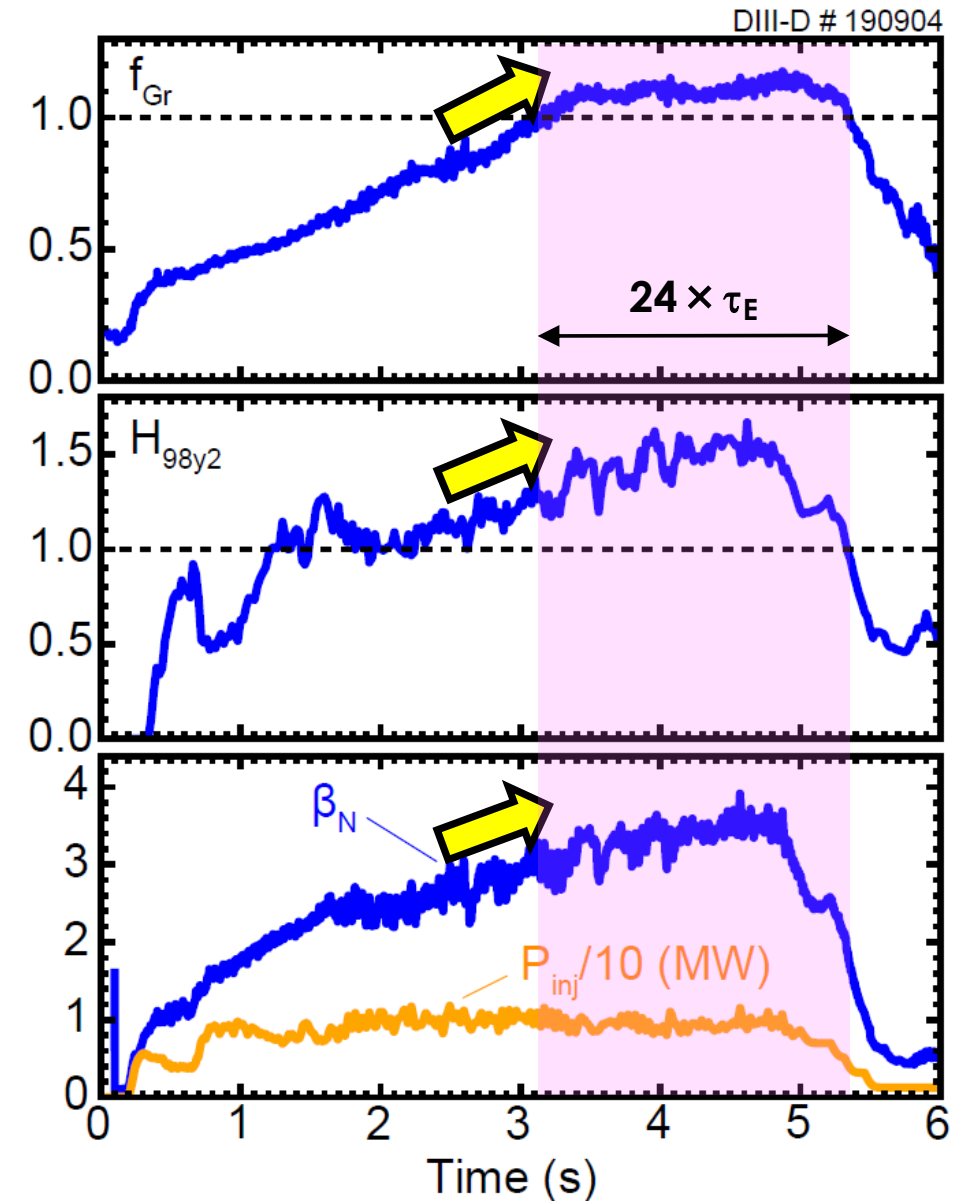


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# Breakthrough: Stored Energy and Confinement Quality Increase as Density Exceeds the Greenwald Value

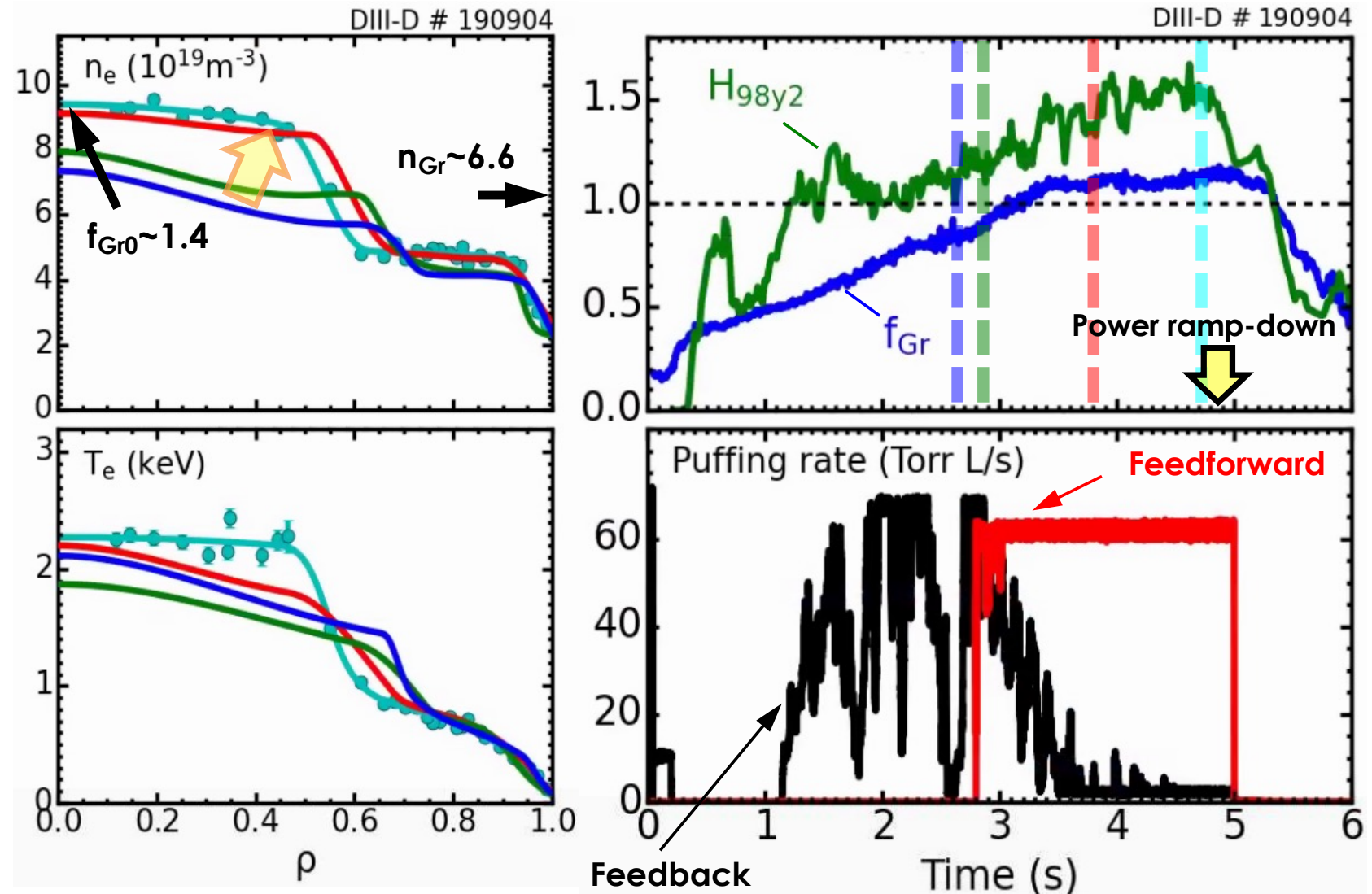
- First time achieve sustained  $H_{98y2} \sim 1.5$  at  $f_{Gr} > 1.1$
- $\beta_N > 3$
- Mixed co-/counter- $I_p$  NBI injection
- $T_{i0}/T_{e0} \sim 1.25$
- $D_2$  gas puffing



Ding et al,  
Nature 2024

# Strong Density Internal Transport Barrier (ITB) Develops during Strong D<sub>2</sub> Gas Puffing

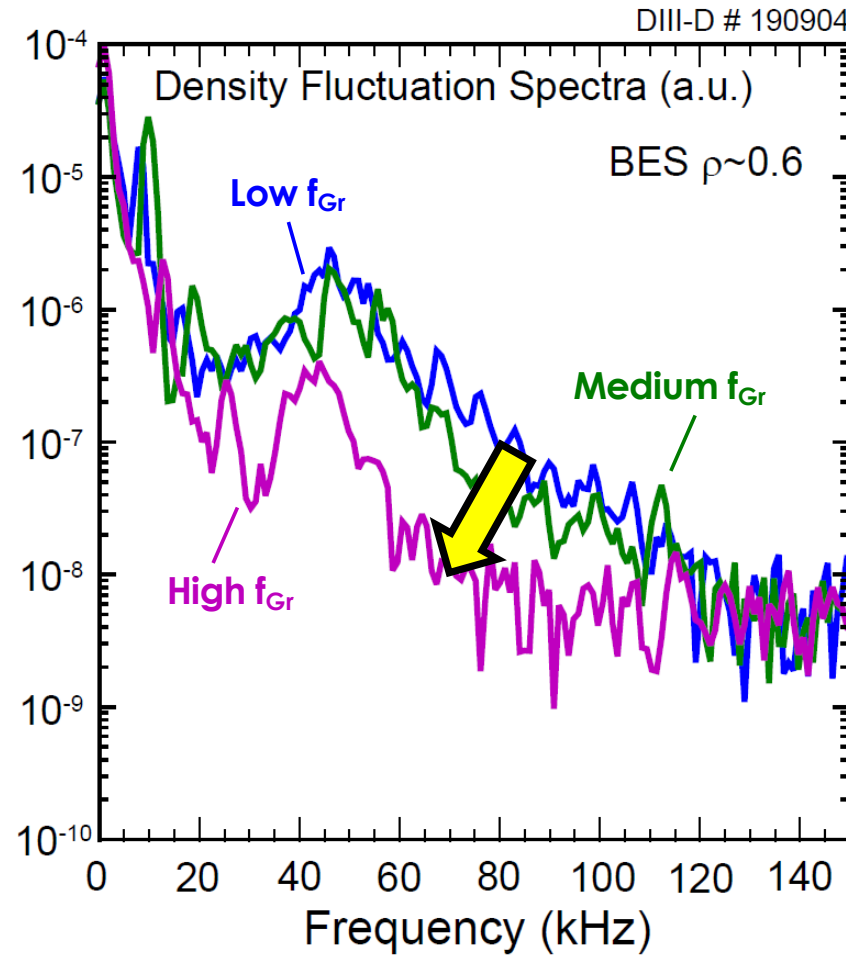
- Strong density ITB elevates core density, while keeping pedestal density below  $n_{Gr}$ 
  - $f_{Gr,ped} \sim 0.7$ ,  $f_{Gr,0} \sim 1.4$
- Strong temperature ITB develops as well



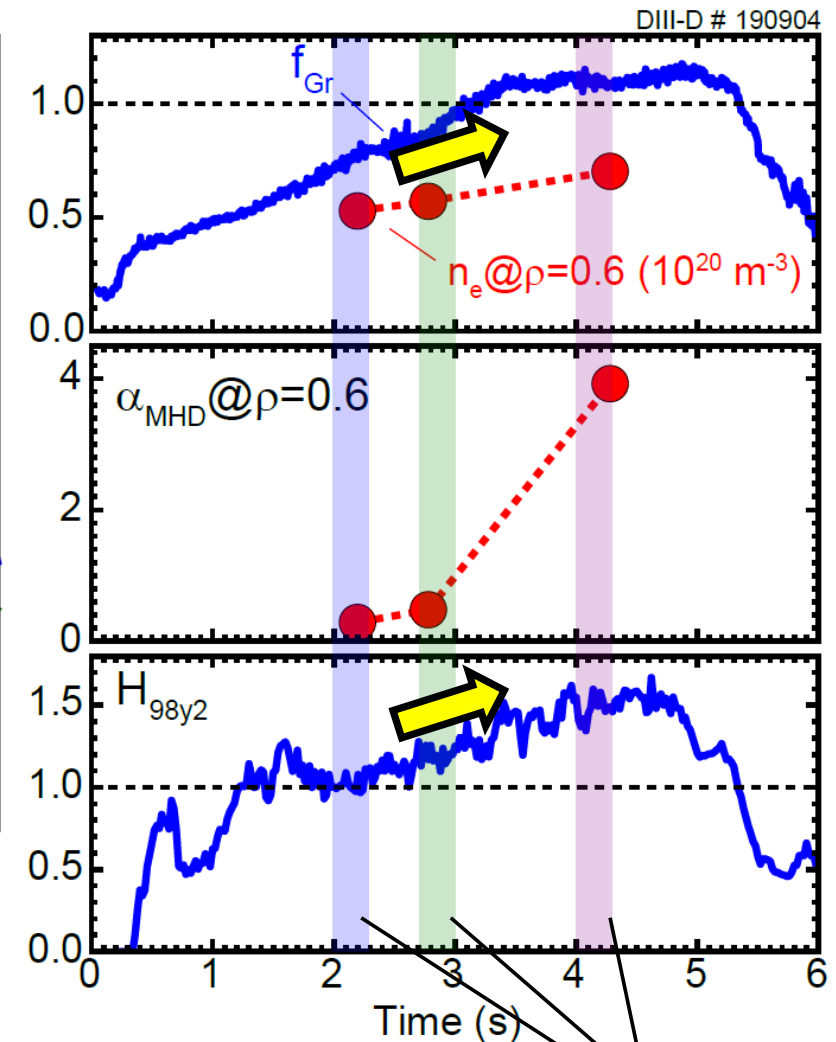
Ding et al, Nature 2024

# Weak Turbulence Measured at High Density and High $\alpha_{\text{MHD}}$

- $\alpha_{\text{MHD}} \sim -\frac{q^2}{B^2} R \frac{dp}{dr} \sim \frac{d\beta_P}{dr}$ ,  
a normalized pressure gradient
- Low-k fluctuations measured at mid-radius  
– 0.1-0.2 cm<sup>-1</sup>



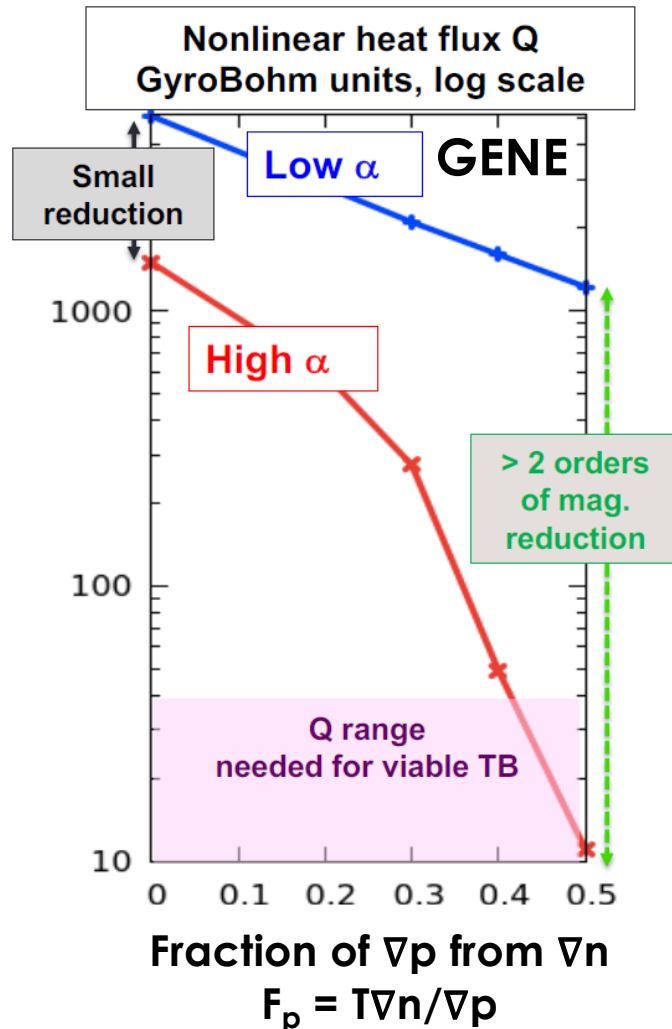
Ding, Invited Talk,  
APS 2023



BES  
measurement

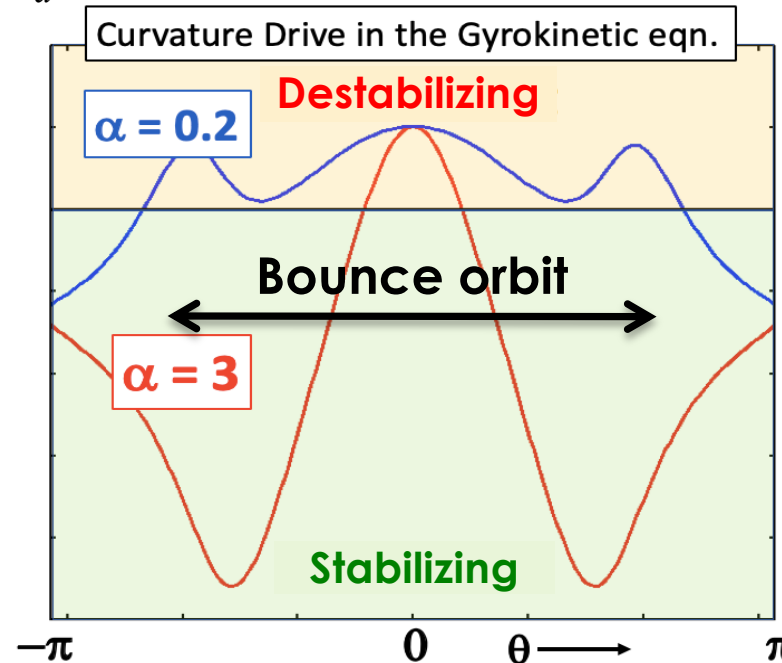
# Theory Predicts Enhanced Density Gradient Amplifies Turbulence Suppression by High $\alpha_{MHD}$

- GENE simulation based on theoretical parameters
- As  $\nabla n$  increases, turbulence transitions from  $\nabla T$  driven (ITG) to  $\nabla n$  driven (TEM)
- At high  $\alpha_{MHD}$ , unstable eigenfunction becomes narrower in poloidal angle ( $\theta$ )



Kotschenreuther et al, NF 2024

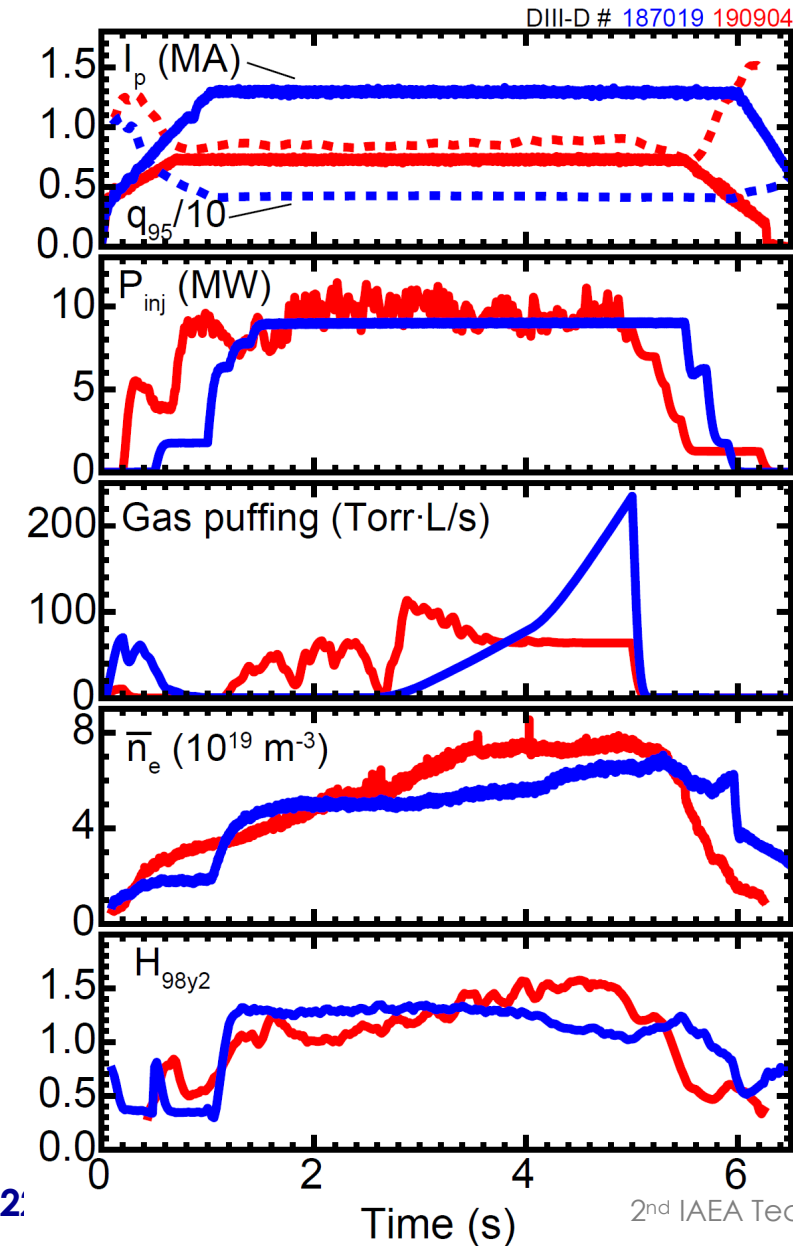
$$\omega_d \sim \cos \theta + (s\theta - \alpha \sin \theta) \sin \theta$$



- **Narrow eigenfunction couples poorly to  $\nabla n$  driven modes**
  - Electrons trapped in large banana orbits
  - Electrons react adiabatically
- **Eventually the turbulence loses free energy drive**

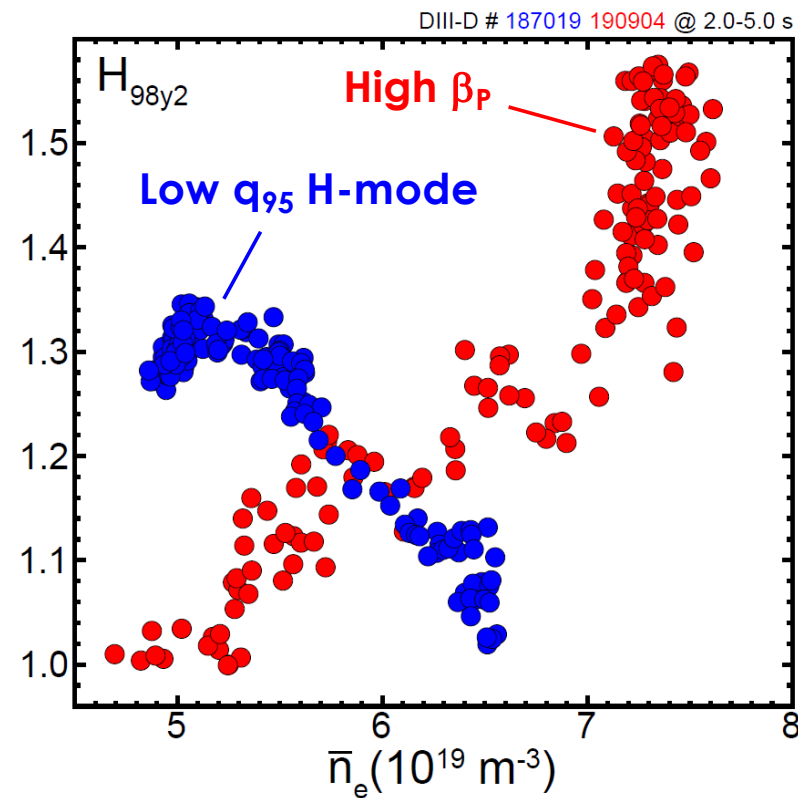
# Experiments Confirm High $\beta_p$ Favorable for Low Turbulence at High Density

Ding, Invited Talk,  
APS 2023



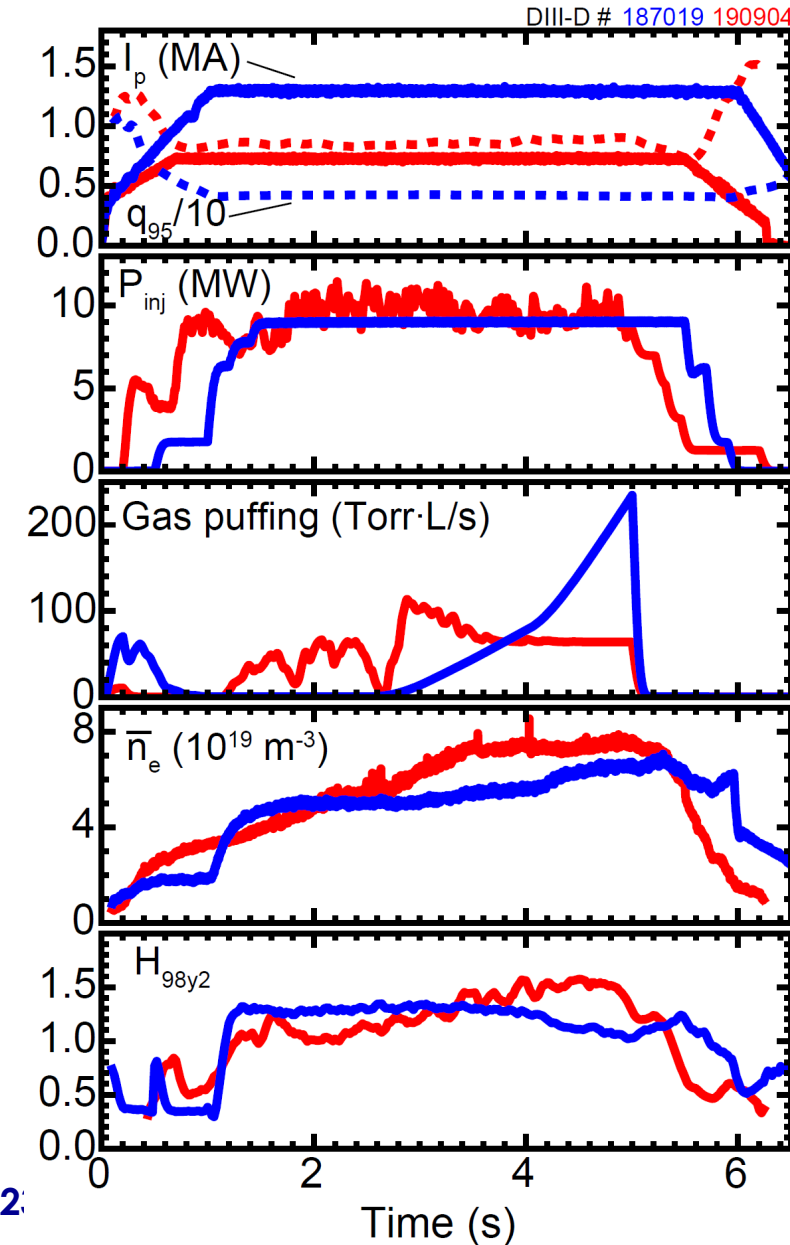
- **High  $\beta_p$**  VS **Low  $q_{95}$  H-mode**

–  $q_{95}$ : 8.5 vs 4



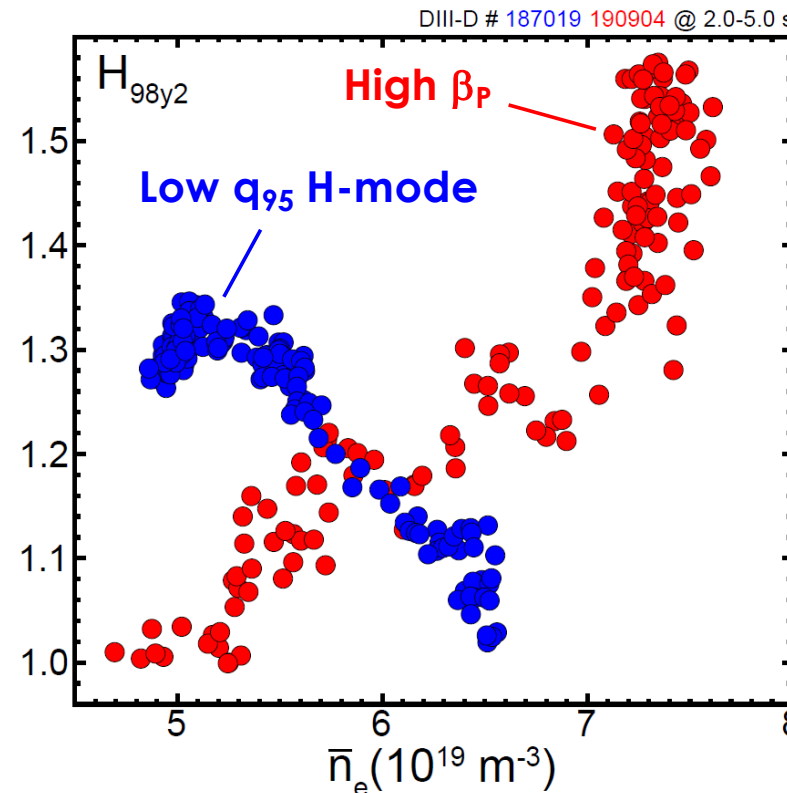
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APS 2023

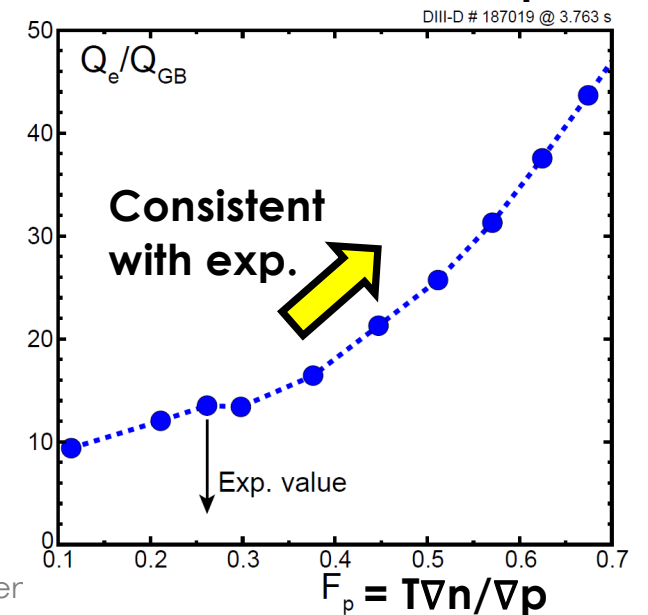
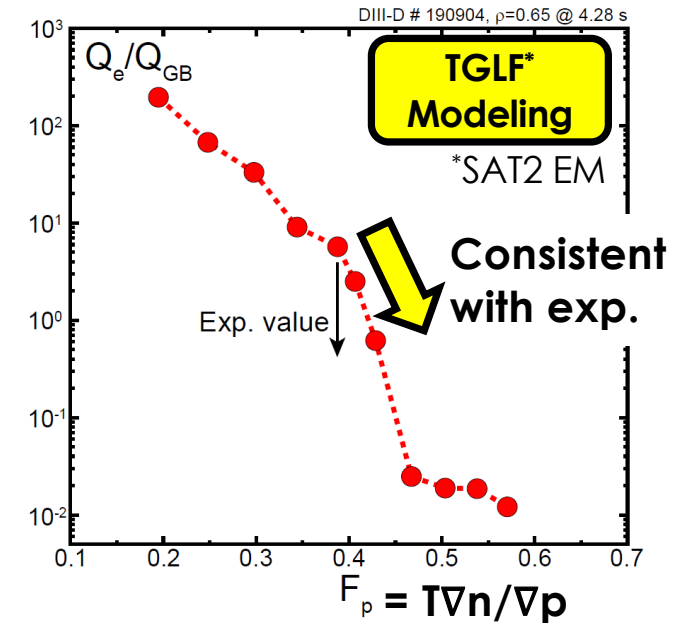


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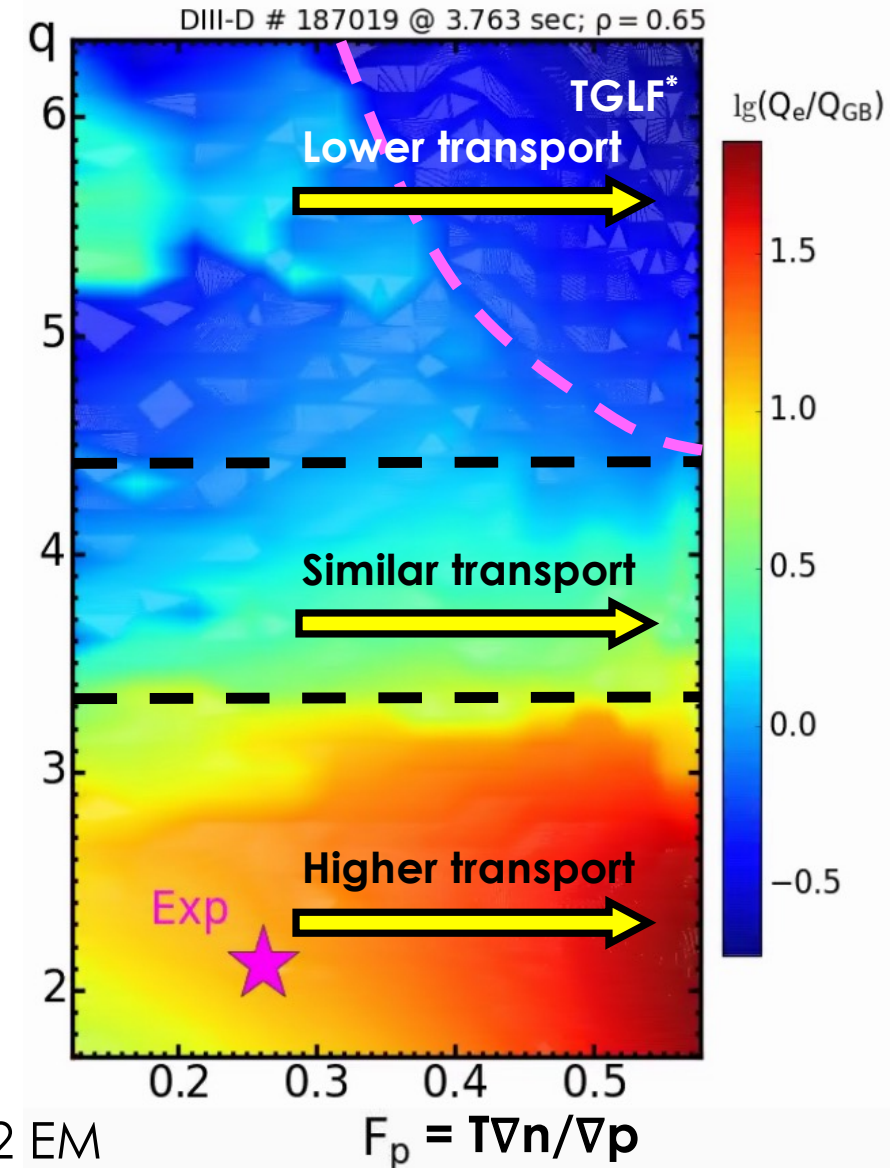
- TGLF core transport analysis using experimental data
- Scan  $\nabla n$  at constant  $\nabla p$  to vary  $F_p$



# Dependence of Turbulent Transport on $F_p$ Reverses from Low to High Local $q$

- Use low- $q$  experimental data at  $\rho=0.65$
- Roughly three regimes
  - High  $q$ : low transport at high  $F_p$
  - Medium  $q$ : similar transport at high  $F_p$
  - Low  $q$ : high transport at high  $F_p$
- Stronger  $\alpha$ -stabilization effect at high  $q$ 
  - $\alpha_{\text{MHD}} \sim q^2$

Safety factor



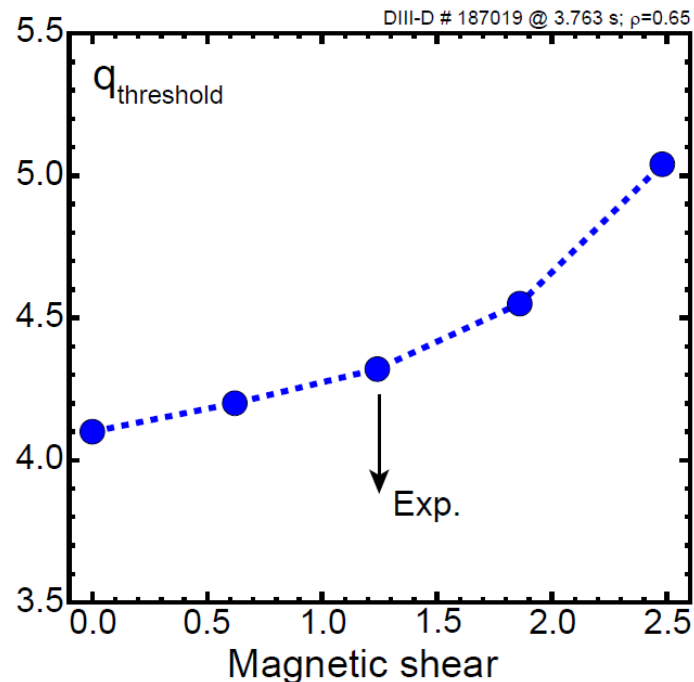
Constant  $\nabla T$  for  $F_p$  scan  
same magnetic shear



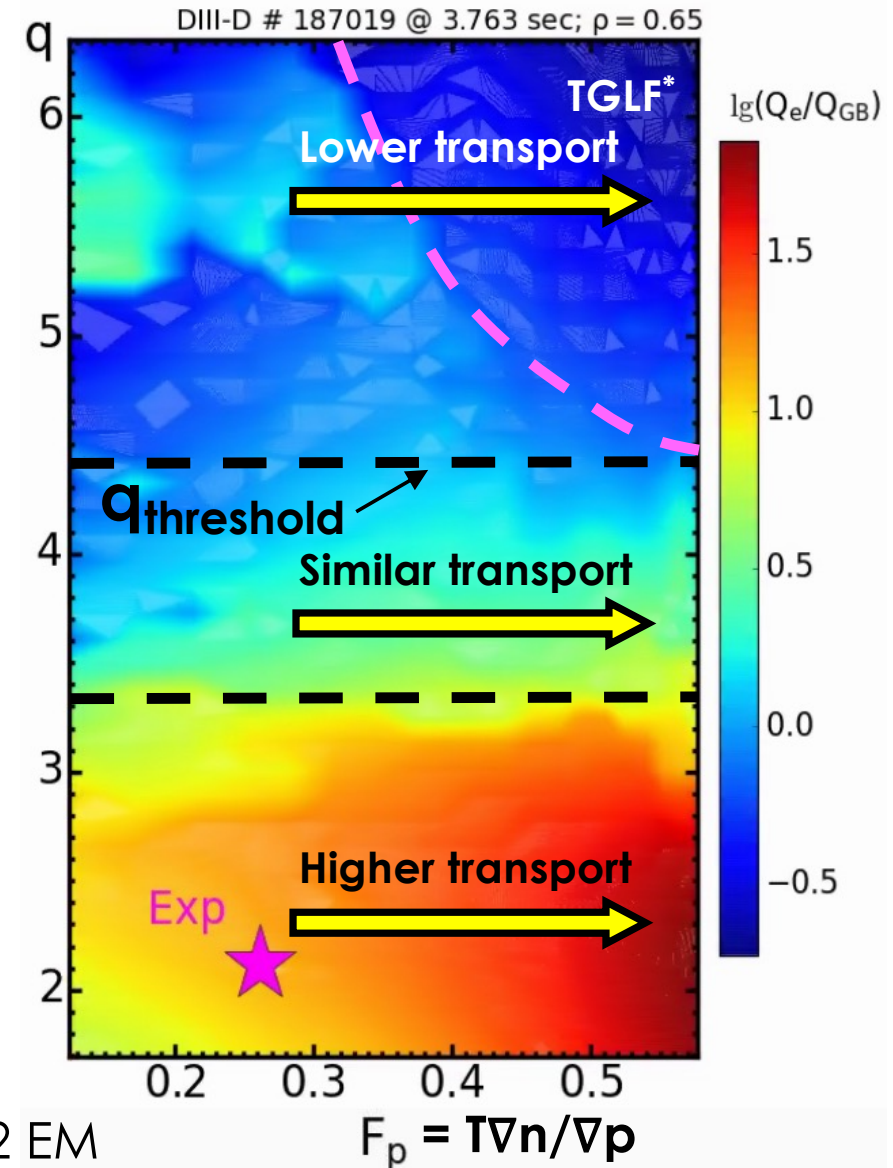
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- Stronger  $\alpha$ -stabilization effect at high  $q$ 
  - $\alpha_{\text{MHD}} \sim q^2$
- Low magnetic shear lowers  $q$  threshold

Safety factor



Constant  $\nabla T$  for  $F_p$  scan  
same magnetic shear



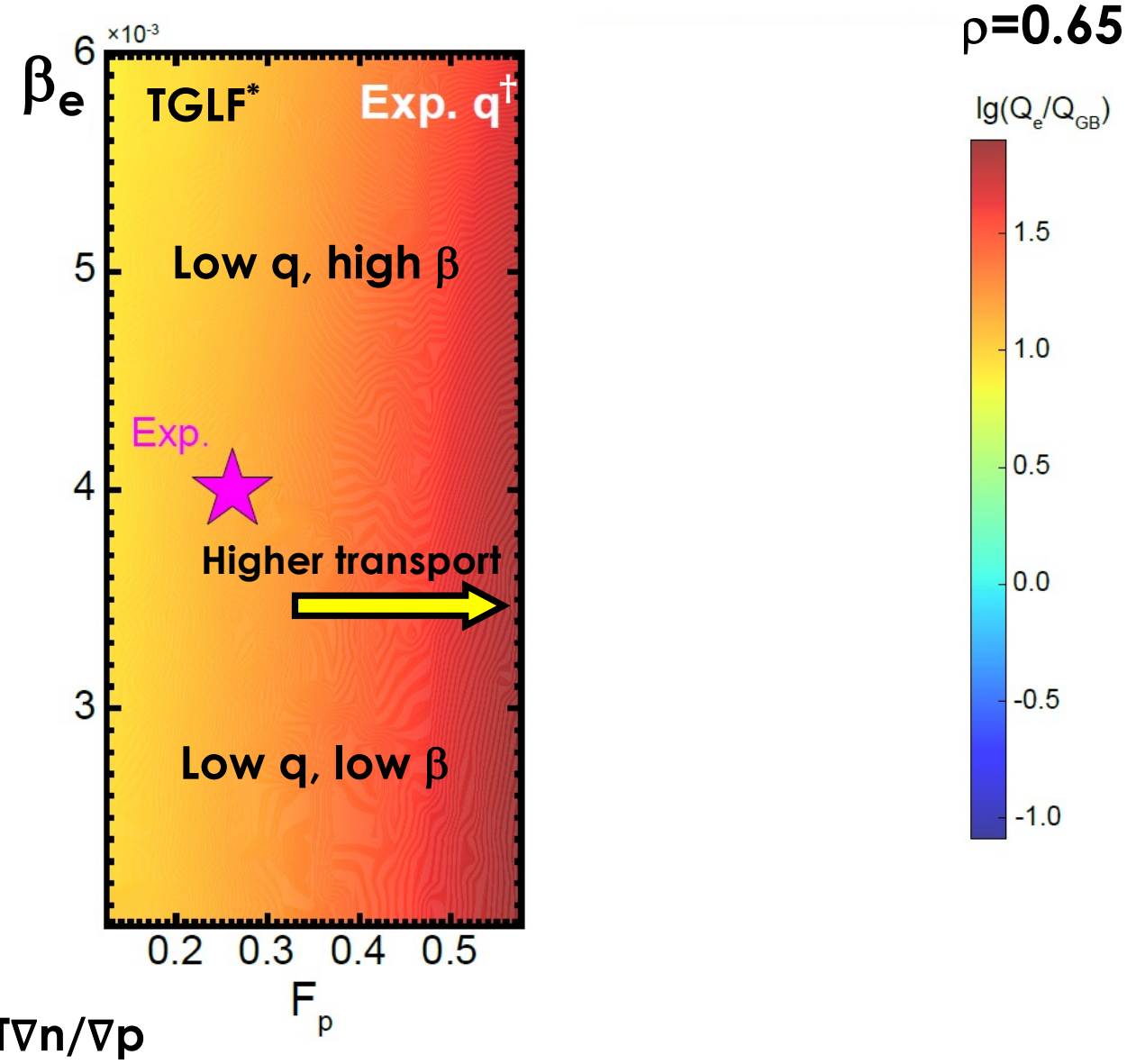
# Reduced Turbulence Transport at High Density Gradient Only Predicted at High $q$ and High $\beta$

- At low- $q$ , transport increases with  $F_p$  for all tested  $\beta_e$

Ding, *Invited Talk*,  
APS 2023

\*SAT2 EM

†Local  $q_{exp}=2.12$



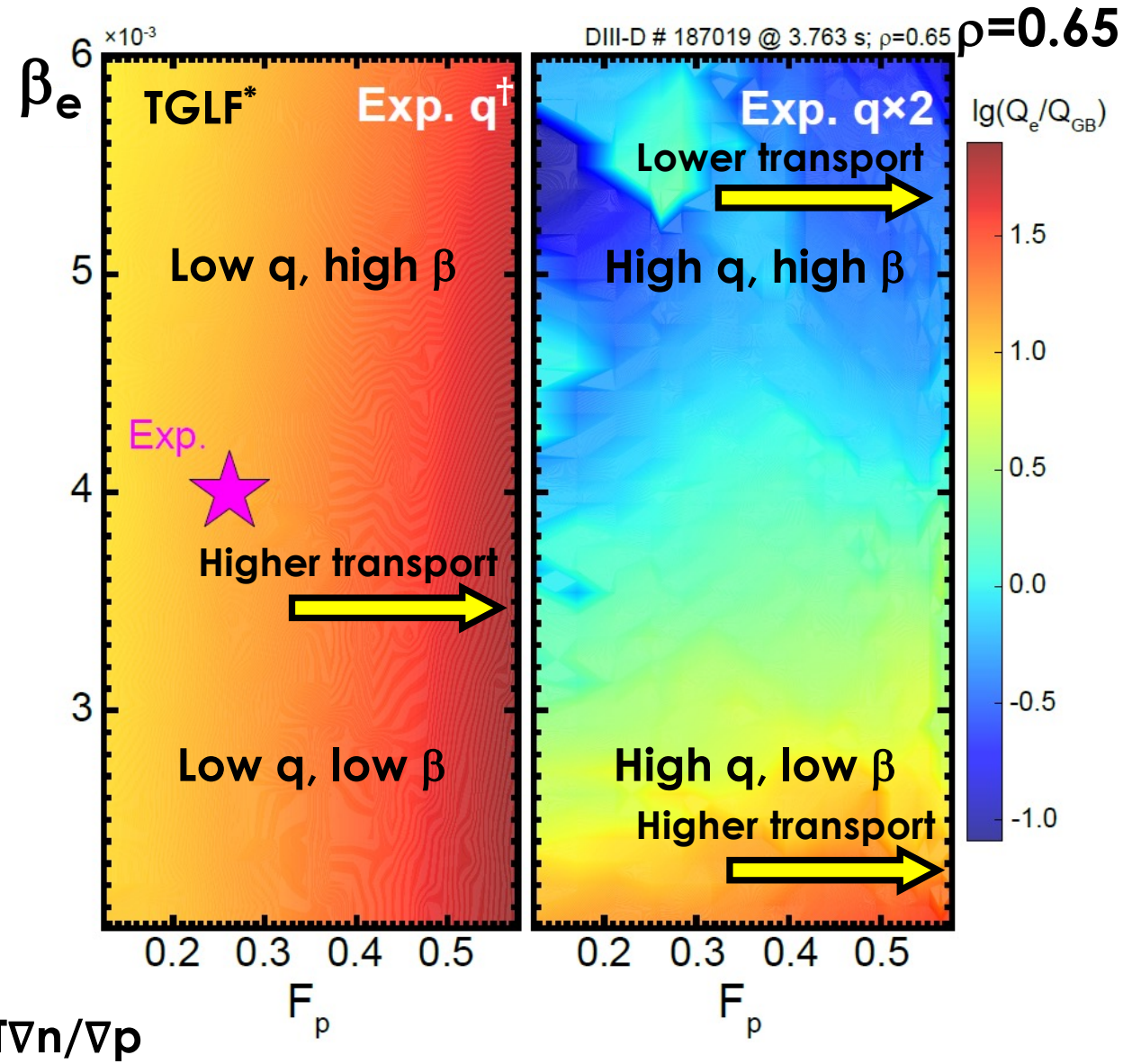
# Reduced Turbulence Transport at High Density Gradient Only Predicted at High q and High $\beta$

- At low-q, transport increases with  $F_p$  for all tested  $\beta_e$
- At high-q, higher turbulence transport at higher  $F_p$ , if  $\beta_e$  is low

Ding, *Invited Talk*,  
APS 2023

\*SAT2 EM

†Local  $q_{exp}=2.12$



**Favorable conditions for accessing low transport at high density  
(simultaneous  $f_{Gr} > 1$  and  $H_{98y2} > 1$ ):**

**High Local  $q$ , high  $\beta$  ( $\rightarrow$  high  $\alpha_{MHD}$ ) and low magnetic shear**

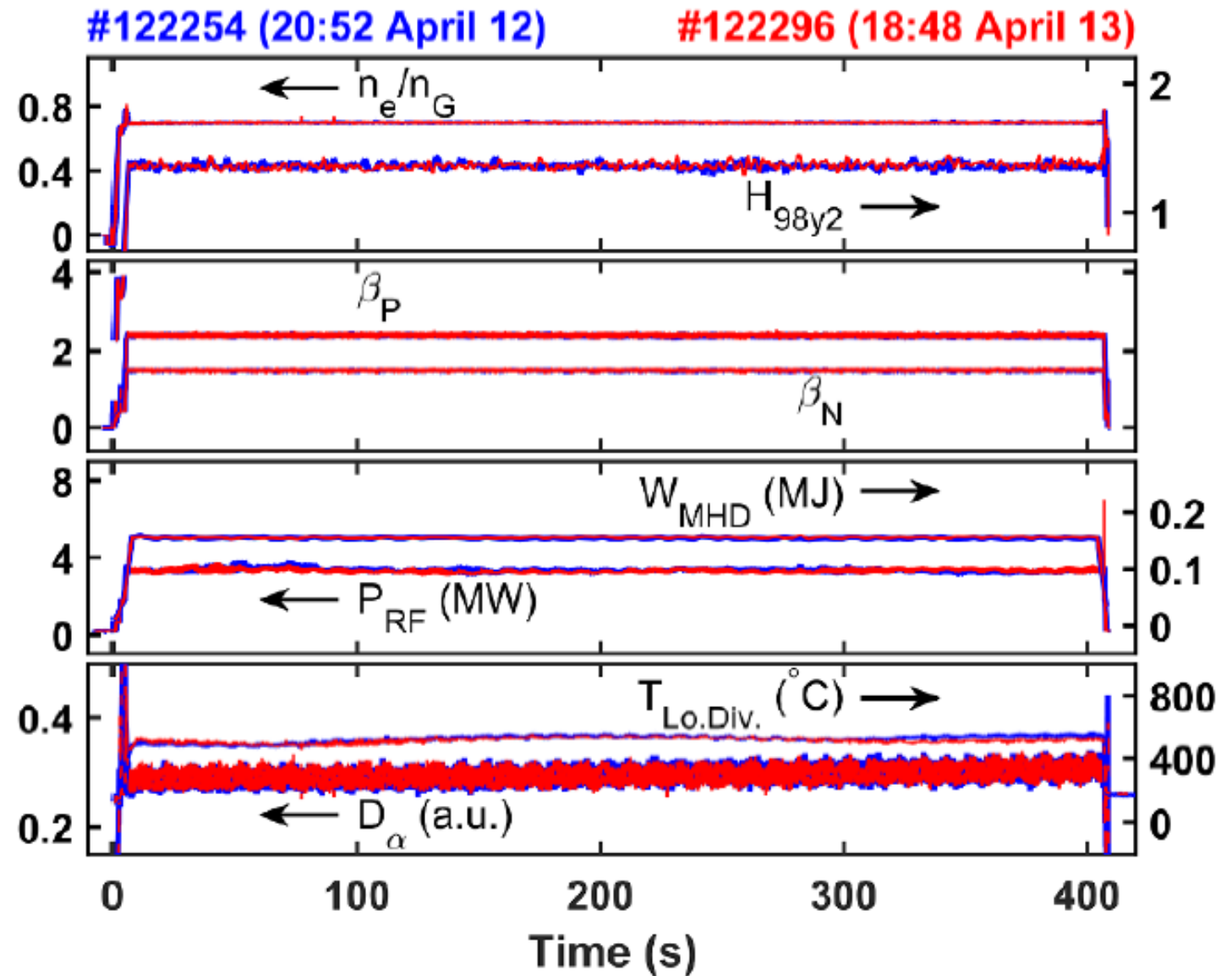
# Outline

- On the performance requirements for steady-state fusion pilot plants (FPPs) and for ITER  $Q=10$   $P_{fus}=500$  MW operation at reduced  $I_p$
- DIII-D high- $\beta_p$  experiments and transport analysis
- **Simulations of EAST high- $\beta_p$  plasmas and theory-guided experiments**
- Core-edge integration in DIII-D high- $\beta_p$  plasmas

# EAST Long-pulse Plasmas Are a Version of High- $\beta_p$ Scenario

- EAST achieved for 403 s H-mode with  $\beta_p \sim 2.5$  and  $f_{Gr} \sim 0.7$  using only RF power

Gong et al, Nucl. Fusion  
2024



# EAST High $\beta_p$ Plasma Has Comparable $\beta_p$ to DIII-D Case, but No Large-Radius ITB

- **Scientific challenges:**

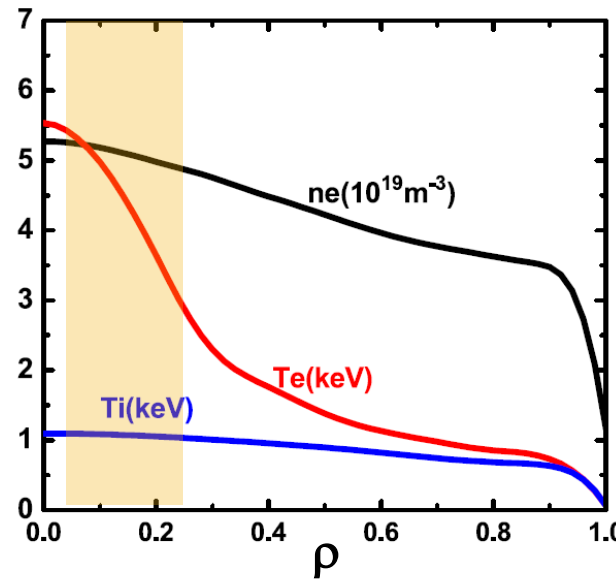
- $T_e$  ITB at small radius
- $T_i \ll T_e$  (Long-standing limitation)

- **Important to understand how to increase pressure gradient at mid-radius**

- Turbulence transport?
- Not enough power?
  - 3.25 MW in the discharge in this slide

- **Use four different codes to understand/predict how to improve this**

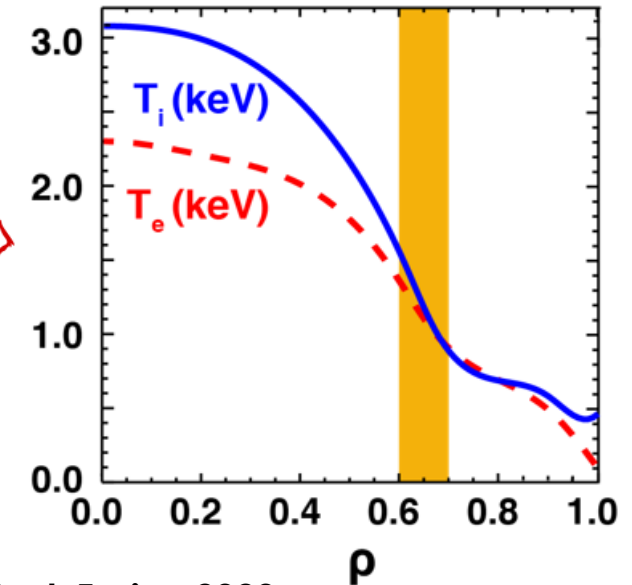
EAST # 81481 @ 5.7 s



Wu, Nucl. Fusion 2019



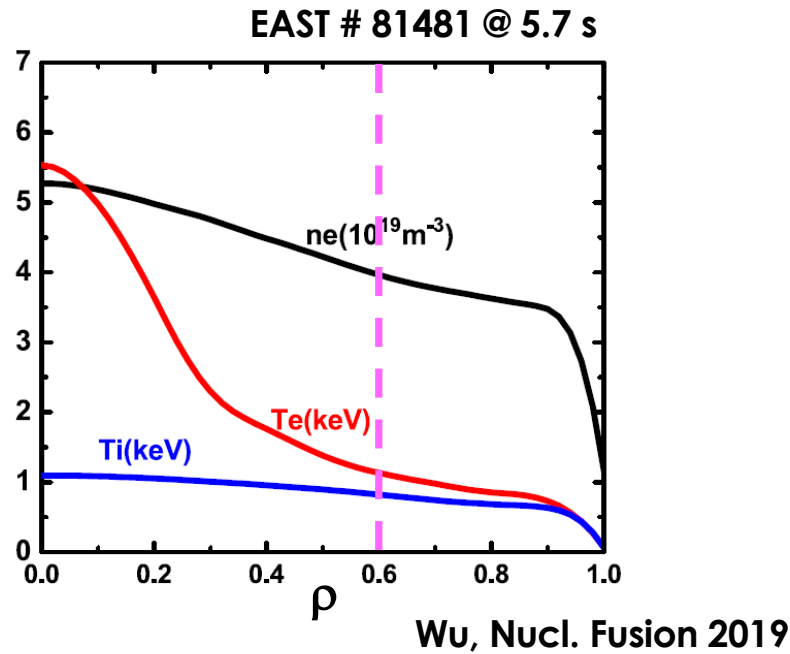
176090@4150ms DIII-D



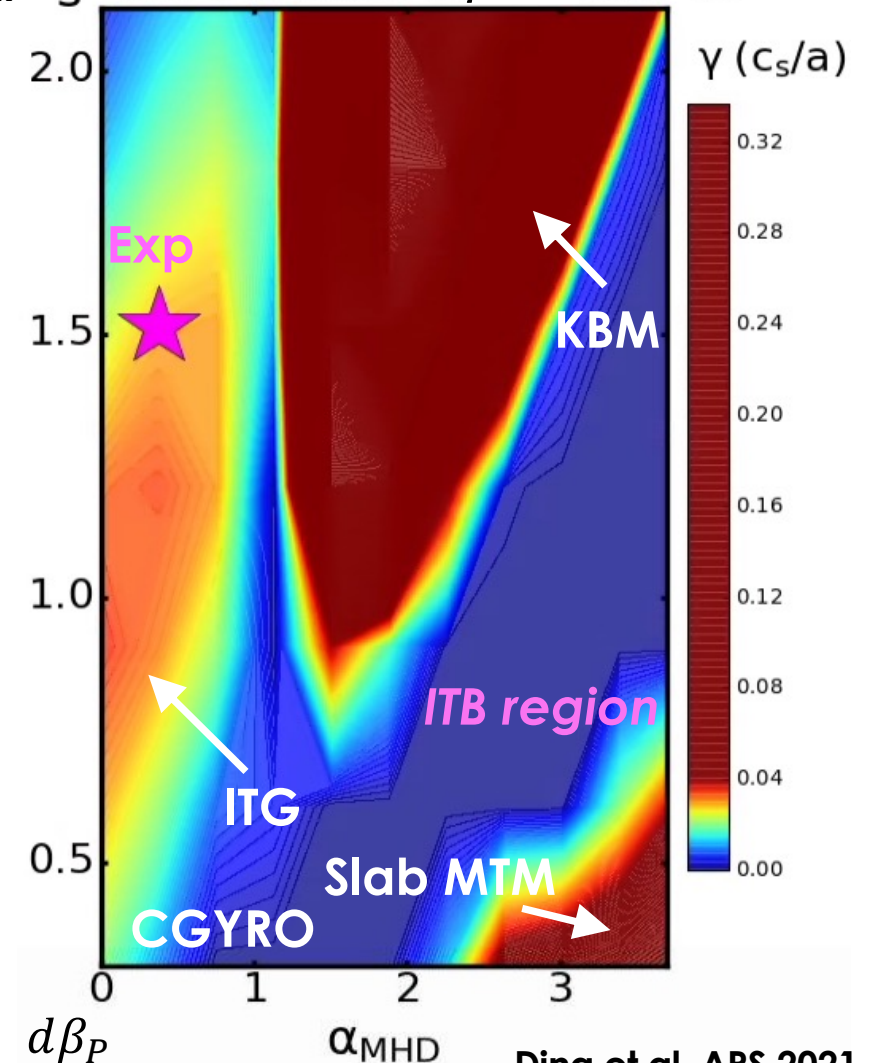
Huang, Nucl. Fusion 2020

	EAST	DIII-D
$q_{95}$	6.5-11	6-12
$\beta_p$	1.95-4.2	1.7-3.5
$\rho_{ITB}$	0.3	0.6-0.8

# Ion Temperature Gradient (ITG) Instability is Dominant at Mid-Radius in EAST Plasma



Magnetic Shear  $\hat{s}$  EAST # 81481, 5.7 s,  $\rho=0.6$ ,  $k_{\theta}\rho_s=0.3$



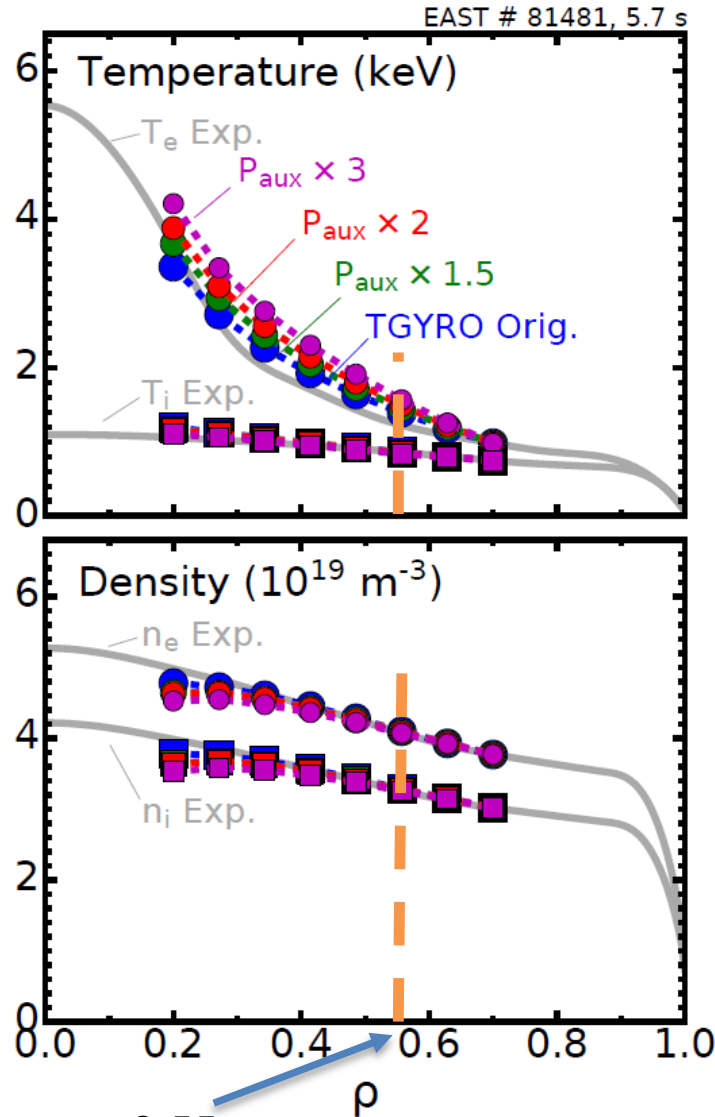
- Gyrokinetic simulations using CGYRO
- Exp. equilibrium 'Trapped' in an ITG mountain
  - Reduced magnetic shear could avoid ITGs

$$\alpha_{\text{MHD}} \sim -\frac{q^2}{B^2} R \frac{dp}{dr} \sim \frac{d\beta_P}{dr}$$

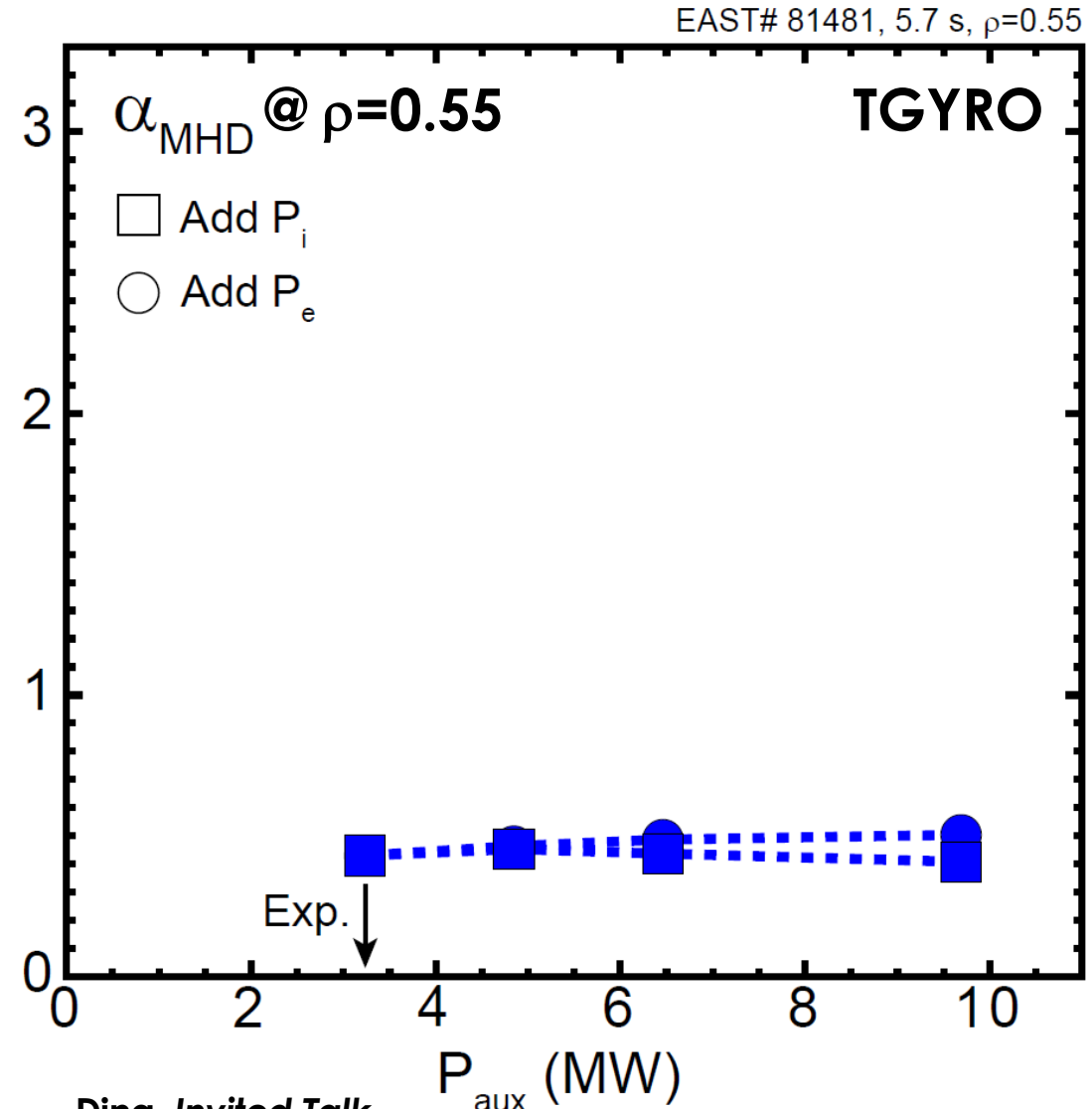


# Transport Modeling Suggests Adding Power Alone Cannot Increase Core Pressure Gradient

- **TGYRO+TGLF\*** reproduces experimental profiles
  - Predict  $T_e$ ,  $T_i$  and  $n_e$
- **Power scan up to 3x**
  - More  $P_e$  or  $P_i$



Focus on  $\rho=0.55$

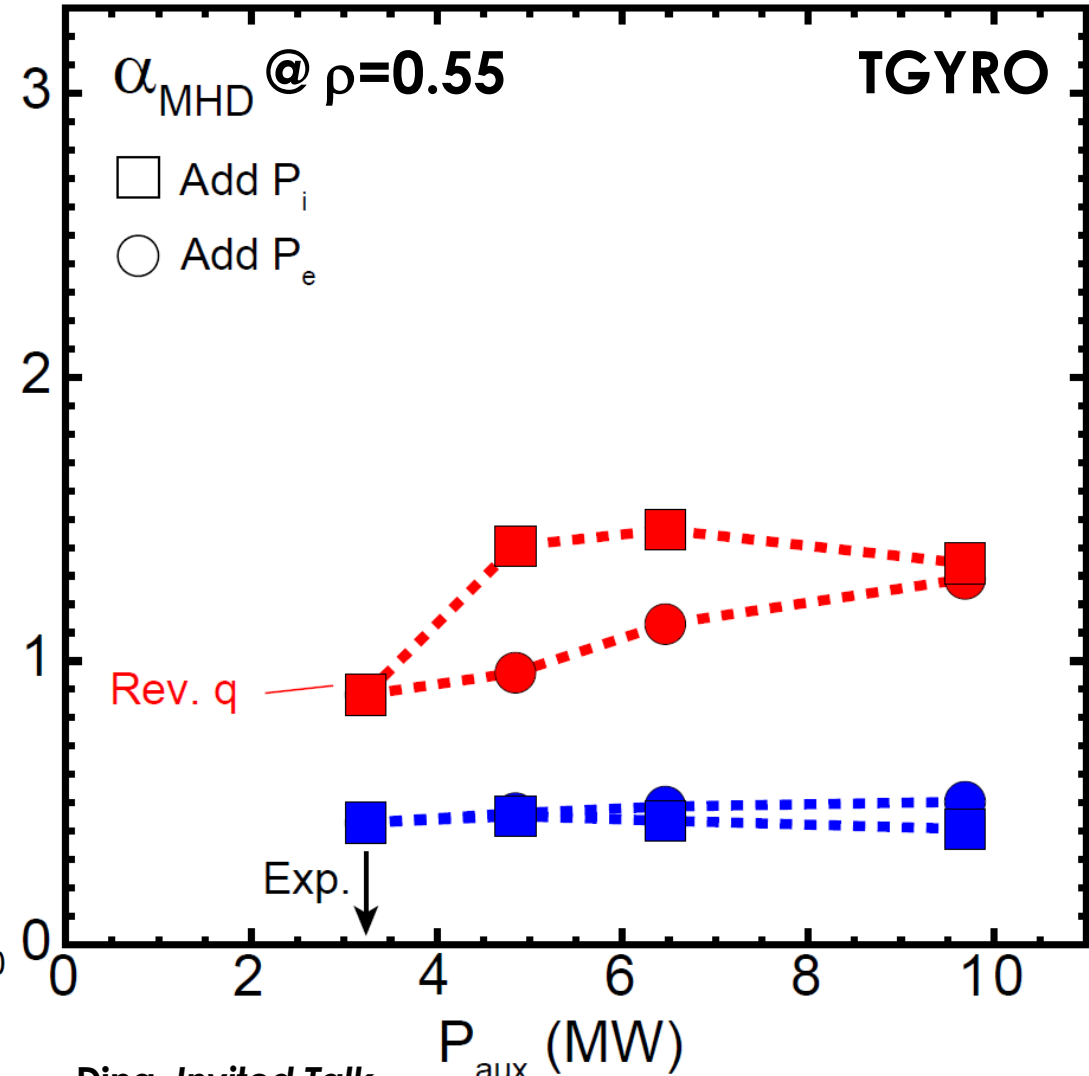
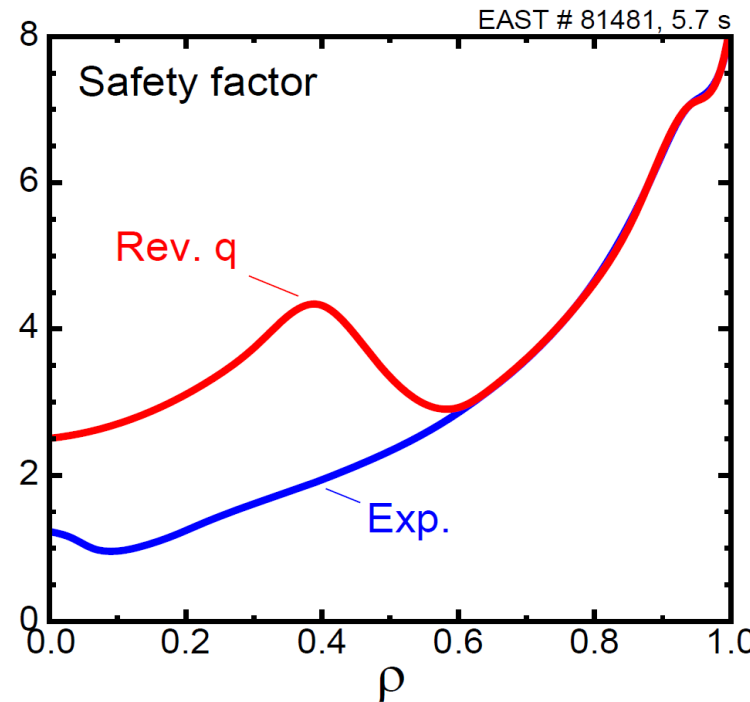


Ding, Invited Talk, APS 2023

# Transport Modeling Suggests Improved Normalized Pressure Gradient at Mid-Radius with Higher $q_{\min}$

EAST# 81481, 5.7 s,  $\rho=0.55$

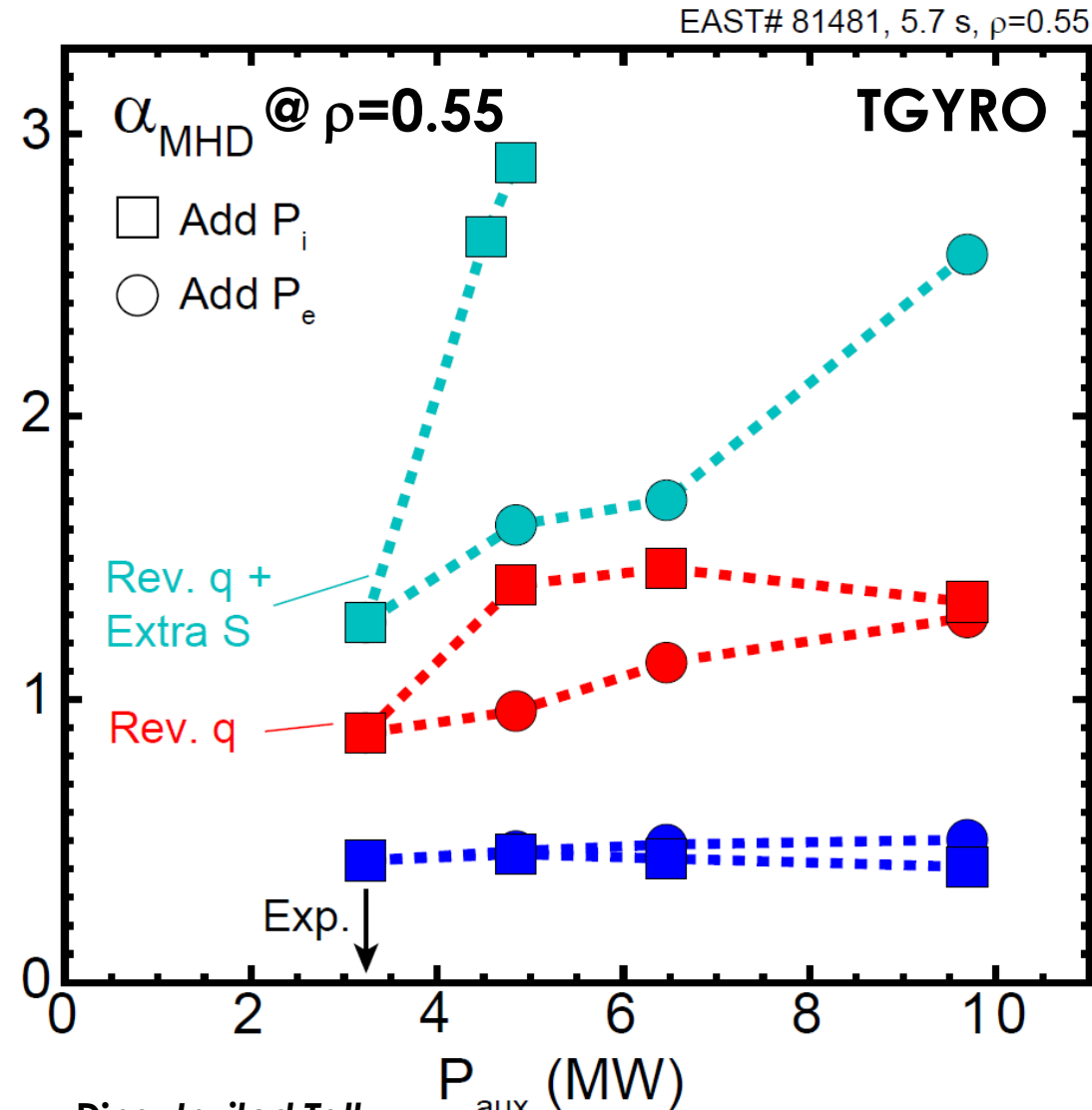
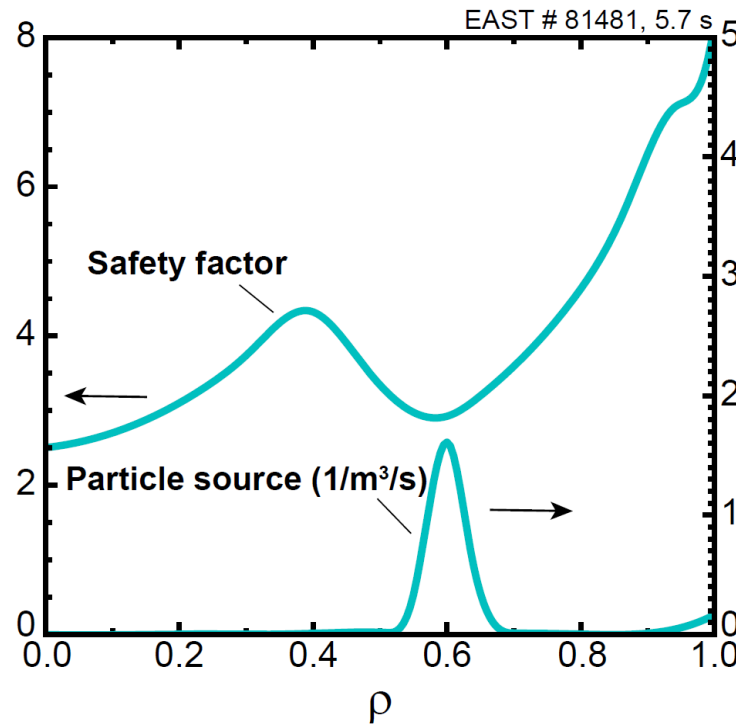
- **TGYRO+TGLF\*** reproduces experimental profiles
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- **Power scan up to 3 ×**
  - More  $P_e$  or  $P_i$
- **Reduced magnetic shear at large radius**



Ding, Invited Talk, APS 2023

# Transport Modeling Suggests Strong $\alpha_{\text{MHD}}$ Improvement at Mid-Radius with Combined Actuators

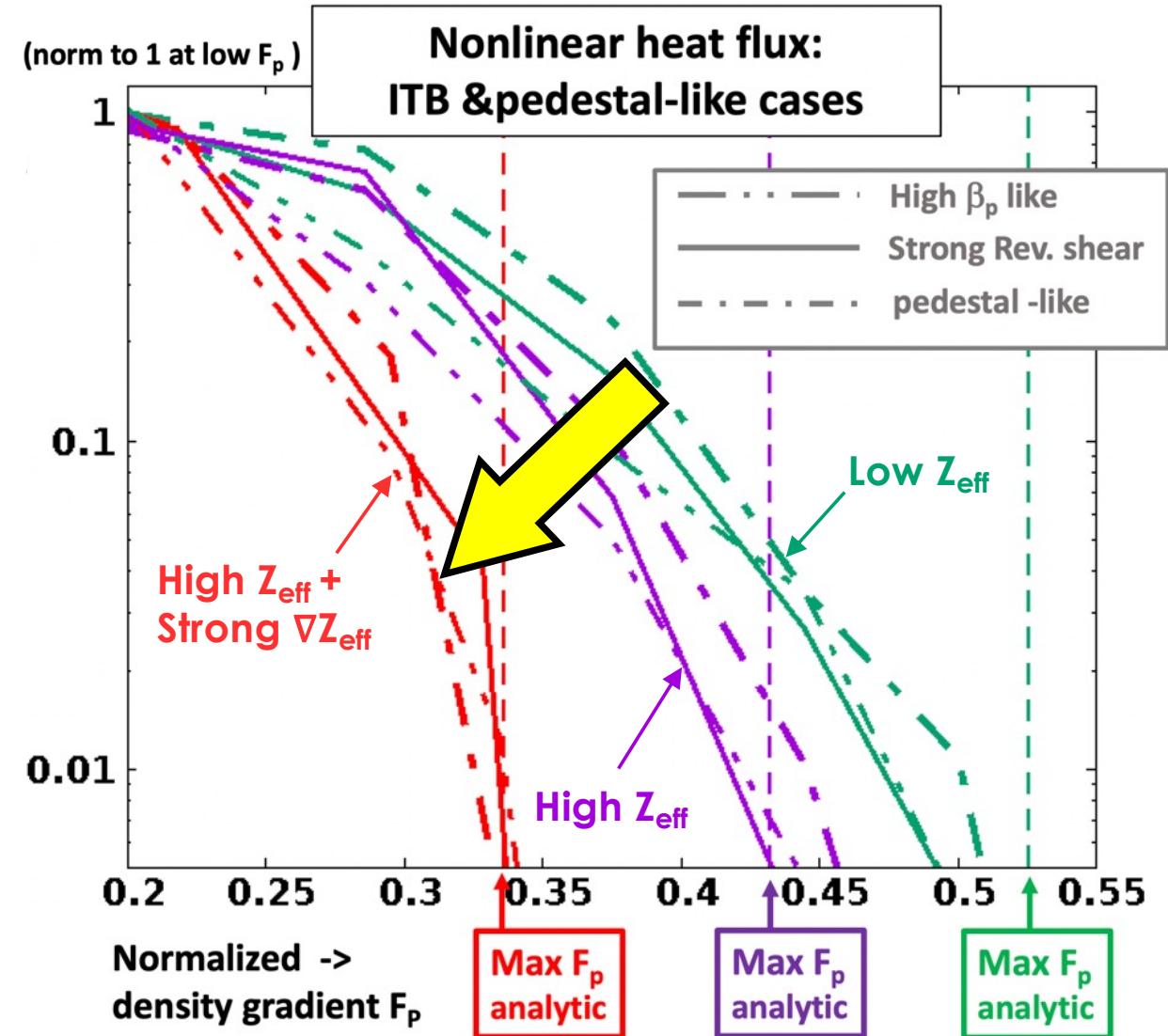
- **TGYRO+TGLF\*** reproduces experimental profiles
  - Predict  $T_e$ ,  $T_i$  and  $n_e$
- **Power scan up to 3x**
  - More  $P_e$  or  $P_i$
- **Reduced magnetic shear at large radius**
- **Add particle source at large radius**



Ding, Invited Talk, APS 2023

# High Density Gradient, High $Z_{\text{eff}}$ and High $Z_{\text{eff}}$ Gradient Expected to Suppress Turbulent Transport

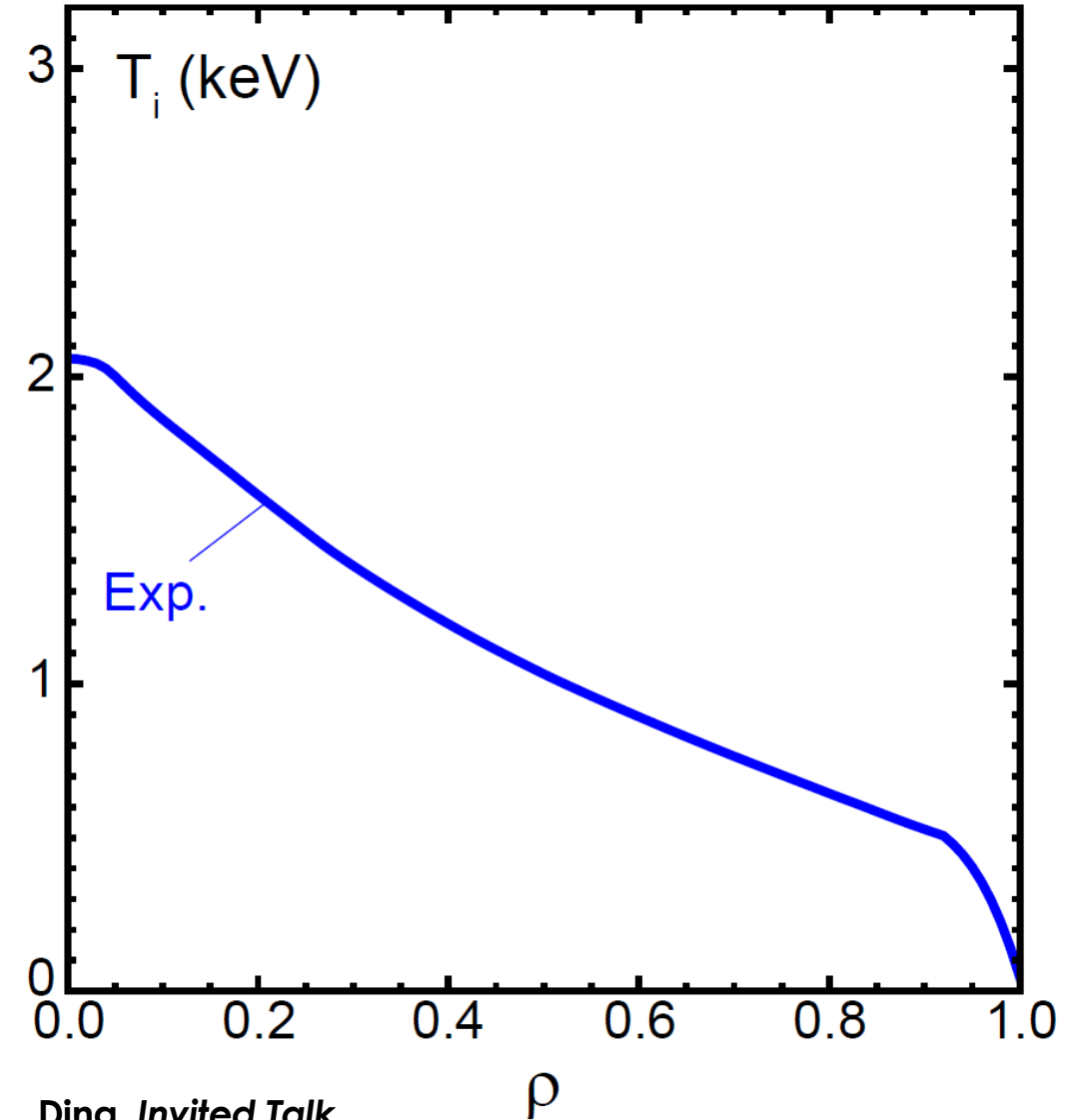
- GENE nonlinear modeling
- High  $F_p$  can suppress turbulent heat flux
- Increase  $Z_{\text{eff}}$  enhances the suppression effect
- Suggests experimental approach: impurity injection at large radius



Kotschenreuther et al, Nucl. Fusion 2024

# Time-Dependent Modeling Suggests Highest $T_i$ with Combined Actuators

- **FASTRAN+TGLF\***, **EAST experimental data†** as starting point
  - $T_i$ ,  $n$  predictions



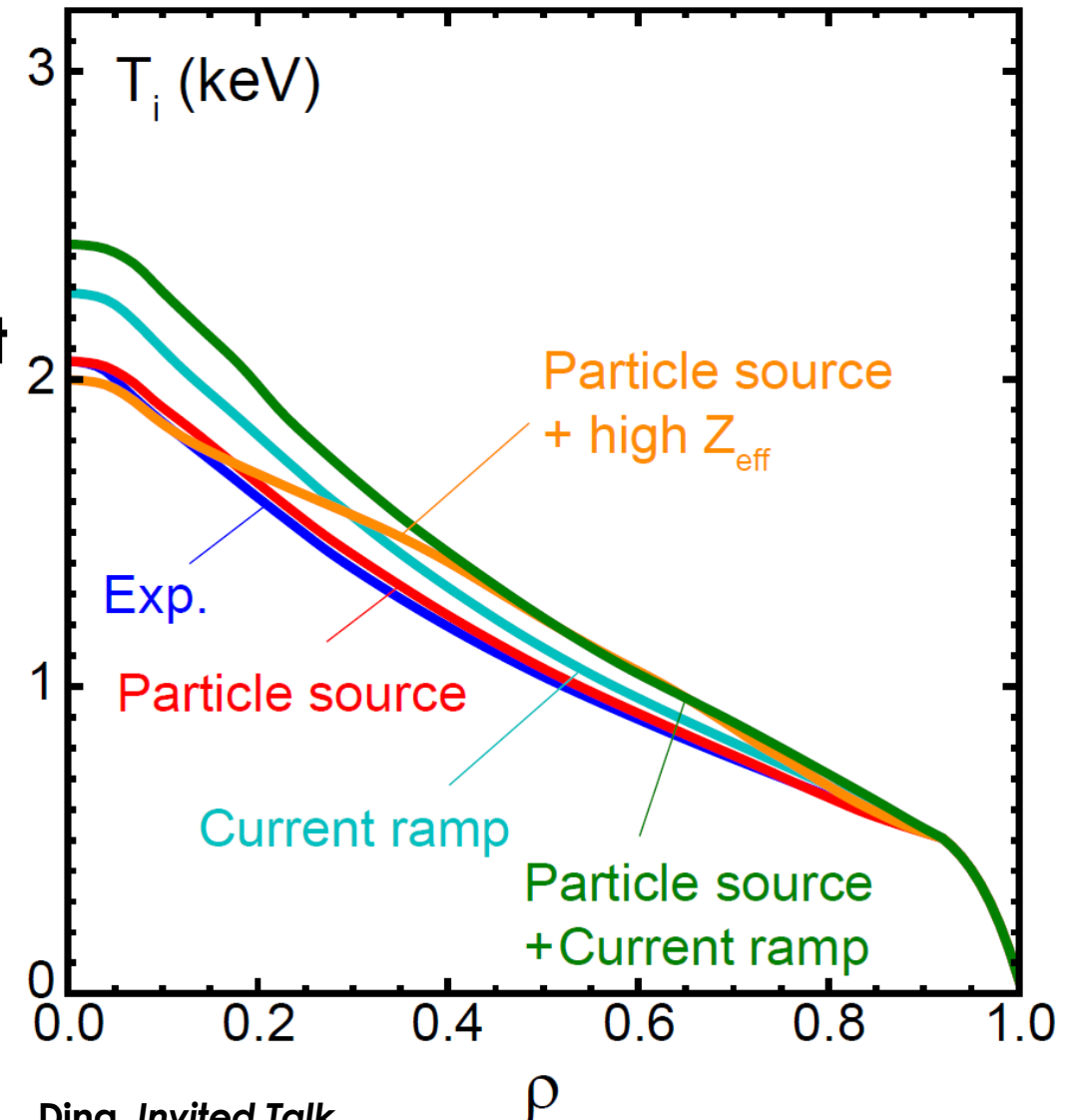
Ding, *Invited Talk*,  
APS 2023

\*SAT2 EM

† $B = 2.5$  T,  $I_p = 0.37$  MA,  $P_{\text{NBI}} = 2.5$  MW,  $P_{\text{RF}} = 6$  MW

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- **Modeling approaches for better confinement**
  - Particle source: increase density gradient
  - Current ramp: reduce magnetic shear
  - Higher  $Z_{\text{eff}}$ : mimic impurity injection



Ding, *Invited Talk*,  
APS 2023

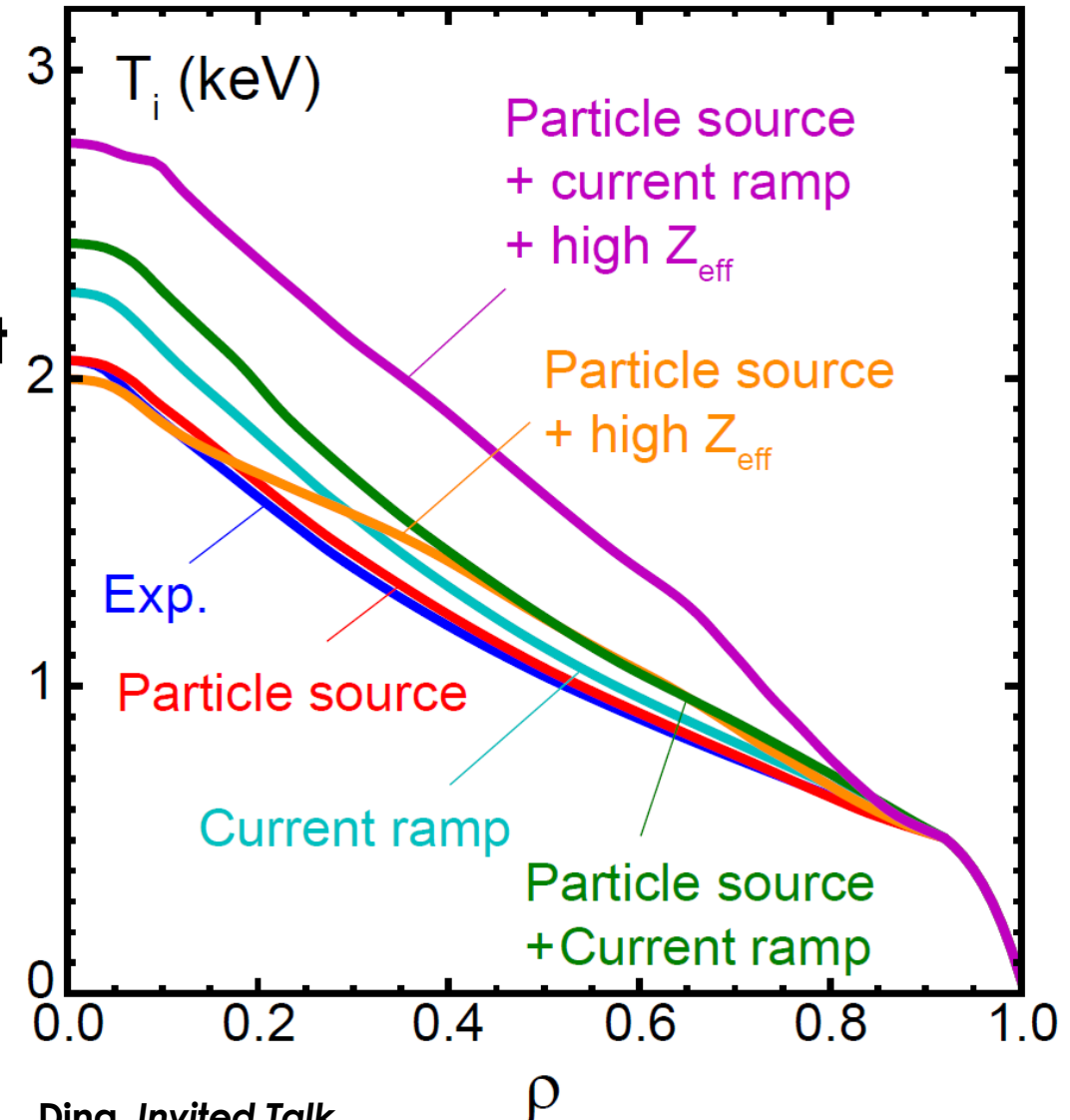
\*SAT2 EM

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- **Modeling approaches for better confinement**
  - Particle source: increase density gradient
  - Current ramp: reduce magnetic shear
  - Higher  $Z_{\text{eff}}$ : mimic impurity injection
  - **Combinations of the above**

**Experiment proposal developed based on the guidance of all modeling results**



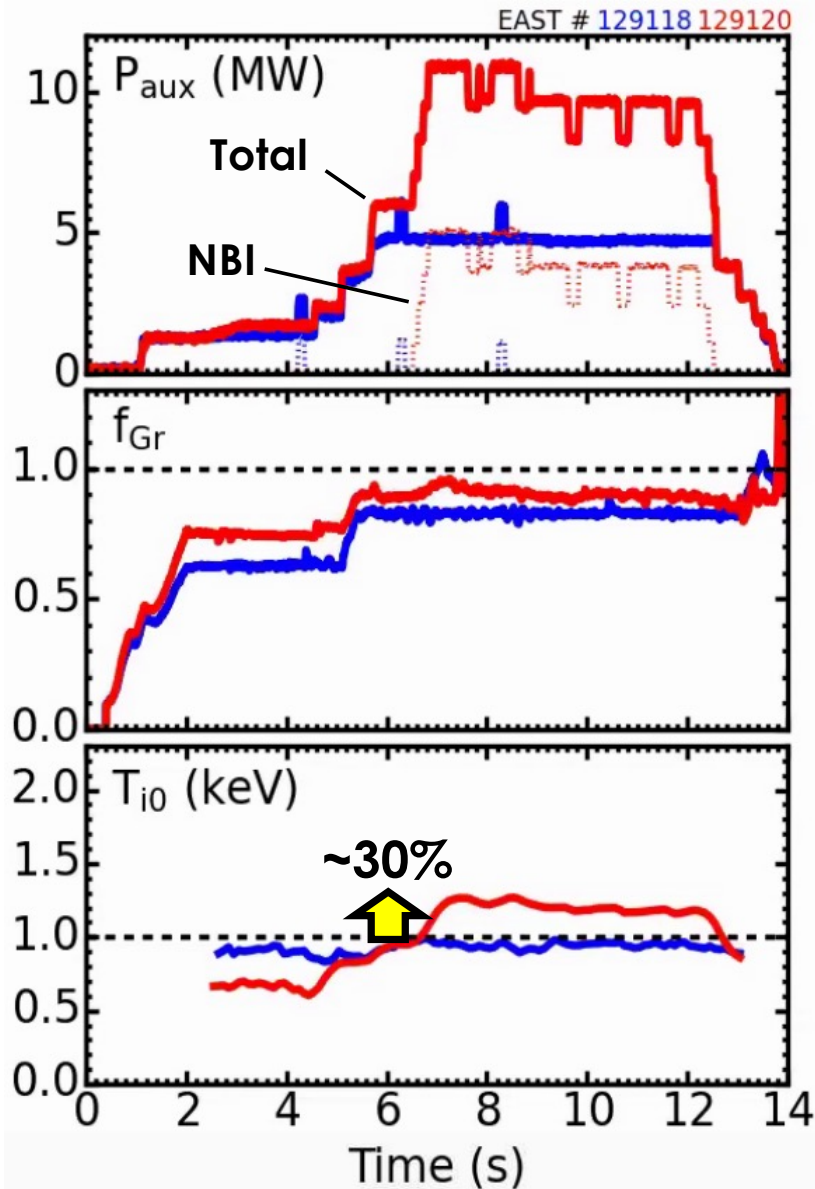
Ding, *Invited Talk*,  
APS 2023

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2<sup>nd</sup> IAEA Technical Meeting on Long-Pulse Operation of Fusion Devices, Oct 14-18, 2024, Vienna, Austria

# Doubled Input Power Shows Small $T_{i0}$ Increase

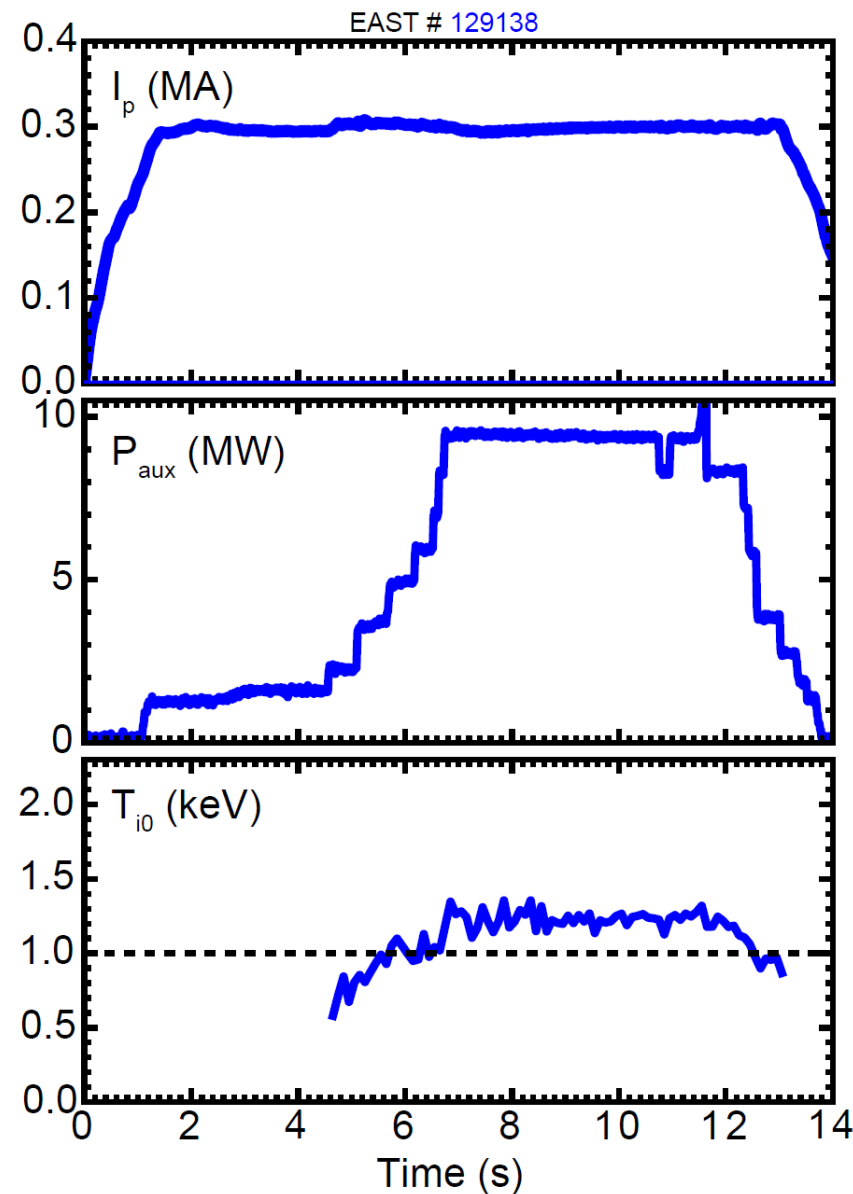


- Power level from 4.8 MW to 10.9 MW
- 30% increase in  $T_{i0}$
- Confirms the previous transport modeling result about limited effect of increasing power alone
- Need to incorporate other physics for turbulence suppression
  - Follows the guidance of transport modeling

Ding, *Invited Talk*,  
APS 2023

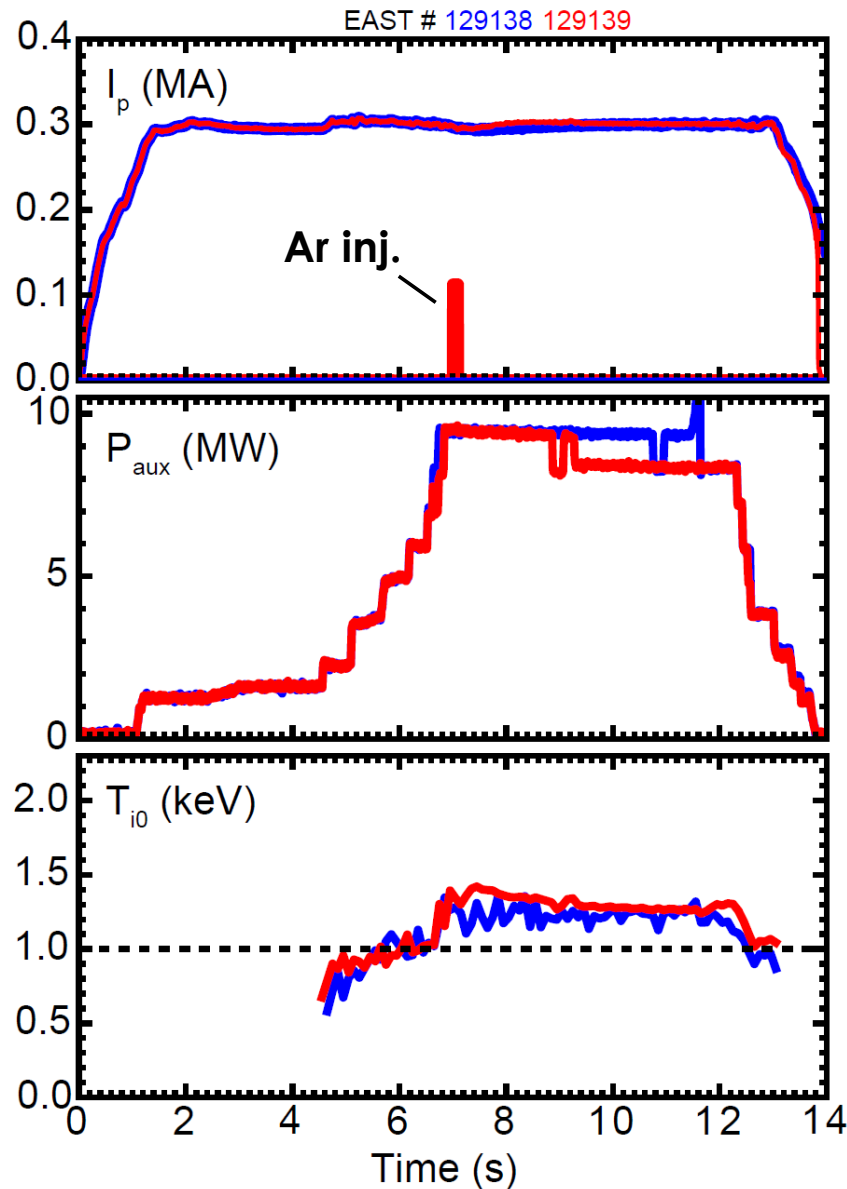


# Nearly Doubled $T_i$ Achieved by Combination of 2<sup>nd</sup> $I_p$ Ramp-Up and Ar Injection at $f_{Gr} \sim 0.9$



Ding, *Invited Talk*,  
APS 2023

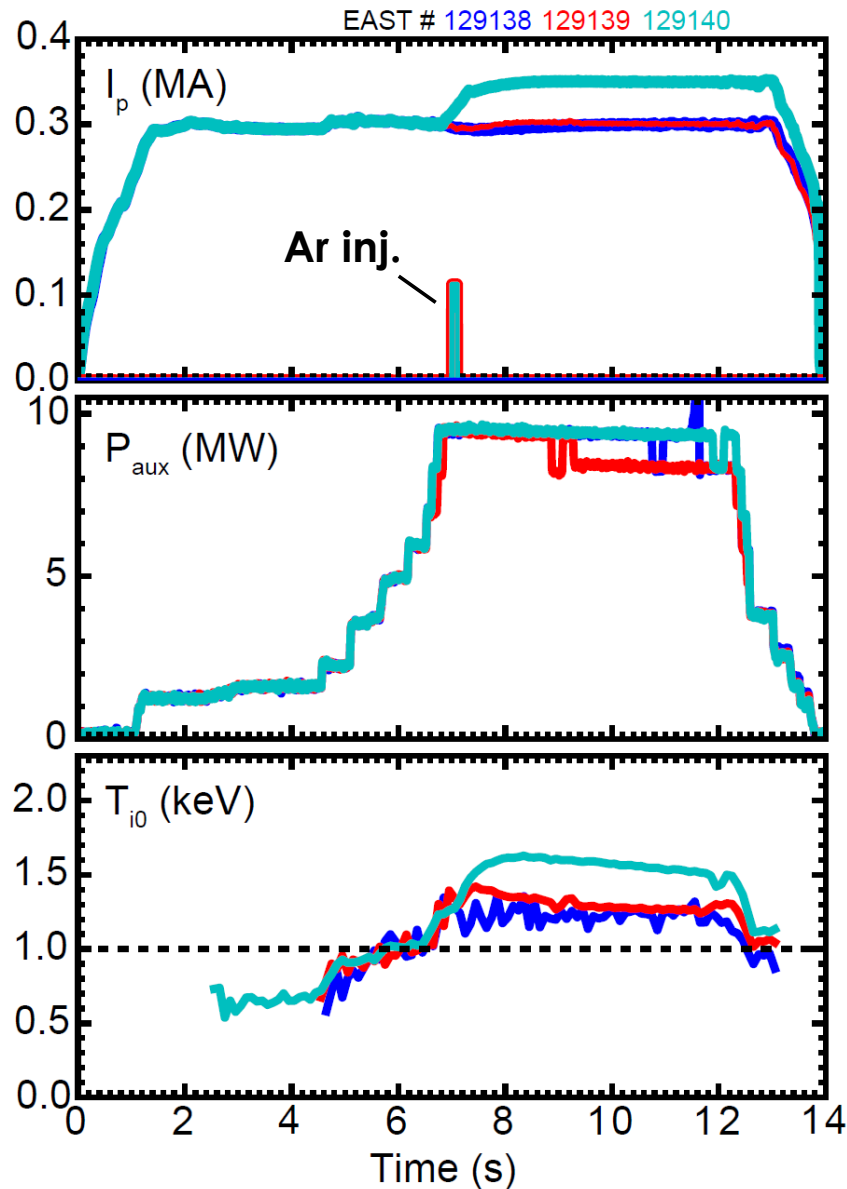
# Nearly Doubled $T_i$ Achieved by Combination of 2<sup>nd</sup> $I_p$ Ramp-Up and Ar Injection at $f_{Gr} \sim 0.9$



- Ar injection: small increase of  $T_i$

Ding, *Invited Talk*,  
APS 2023

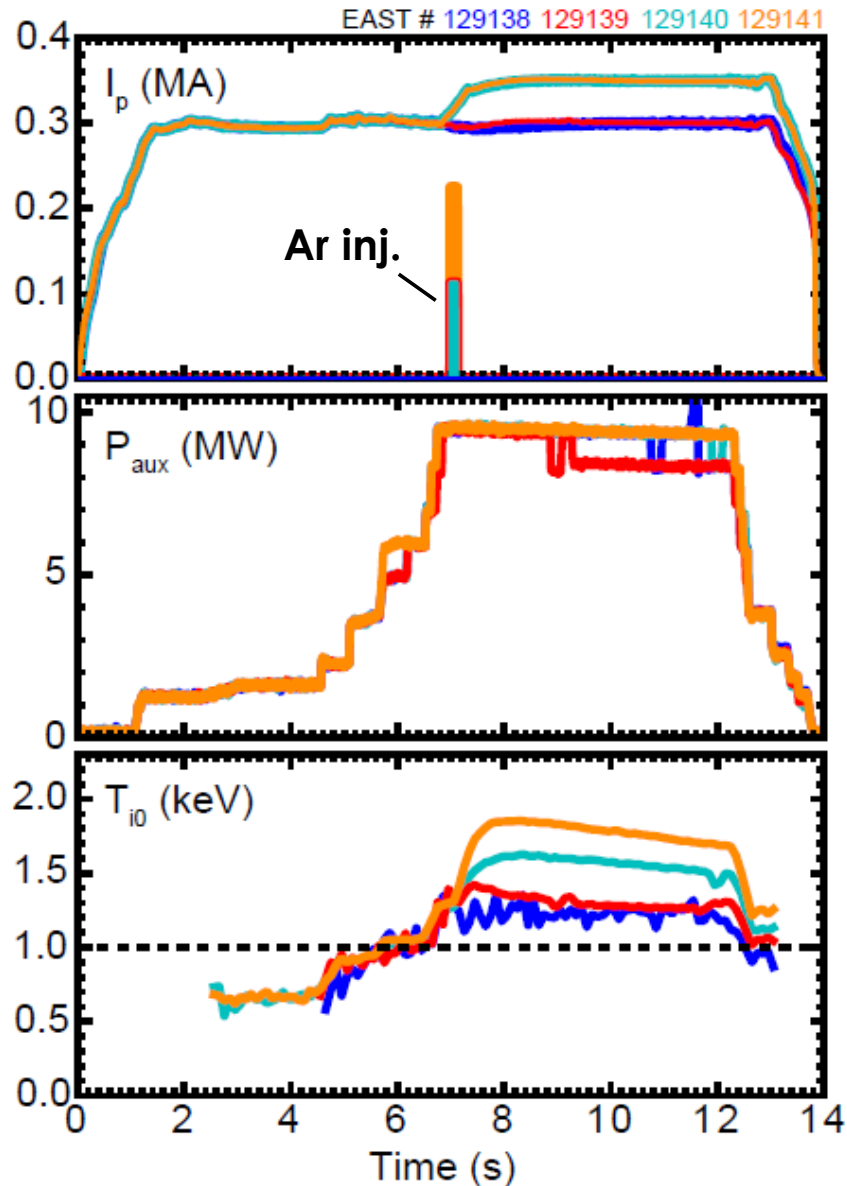
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- Ar injection: small increase of  $T_i$
- Add 2<sup>nd</sup>  $I_p$  ramp-up: further improved  $T_i$

Ding, *Invited Talk*,  
APS 2023

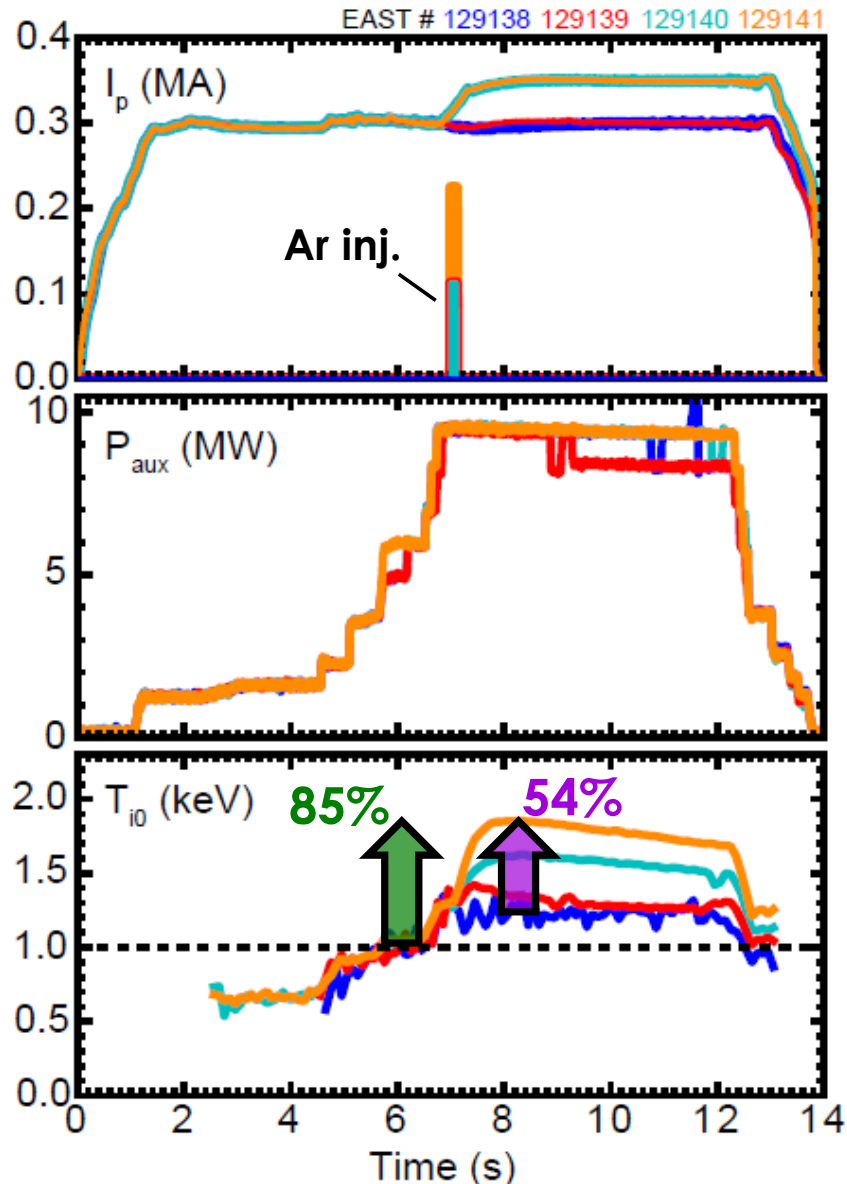
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- Ar injection: small increase of  $T_i$
- Add 2<sup>nd</sup>  $I_p$  ramp-up: further improved  $T_i$
- Increase Ar amount: highest  $T_i$ 
  - Add 2<sup>nd</sup> Ar injector, total amount doubled

Ding, *Invited Talk*,  
APS 2023

# Nearly Doubled $T_i$ Achieved by Combination of 2<sup>nd</sup> $I_p$ Ramp-Up and Ar Injection at $f_{Gr} \sim 0.9$



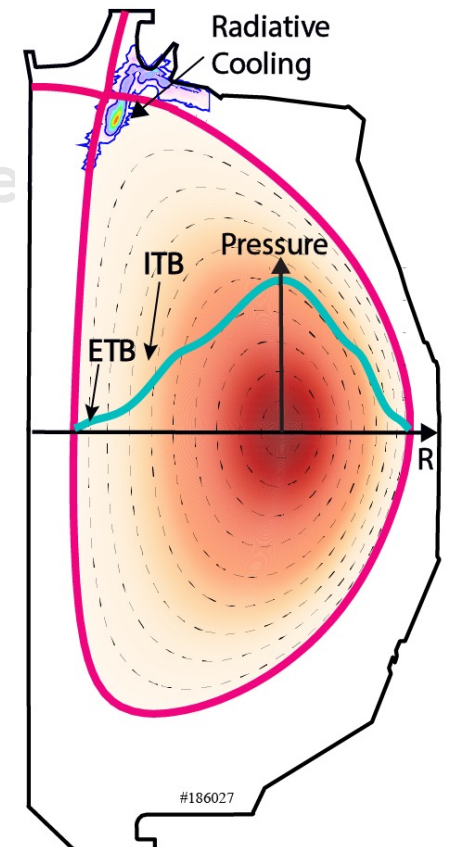
- Ar injection: small increase of  $T_i$
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- Increase Ar amount: highest  $T_i$ 
  - Add 2<sup>nd</sup> Ar injector, total amount doubled
- Confirms transport modeling results on the best turbulence suppression by the combined actuators

Ding, Invited Talk,  
APS 2023

# Outline

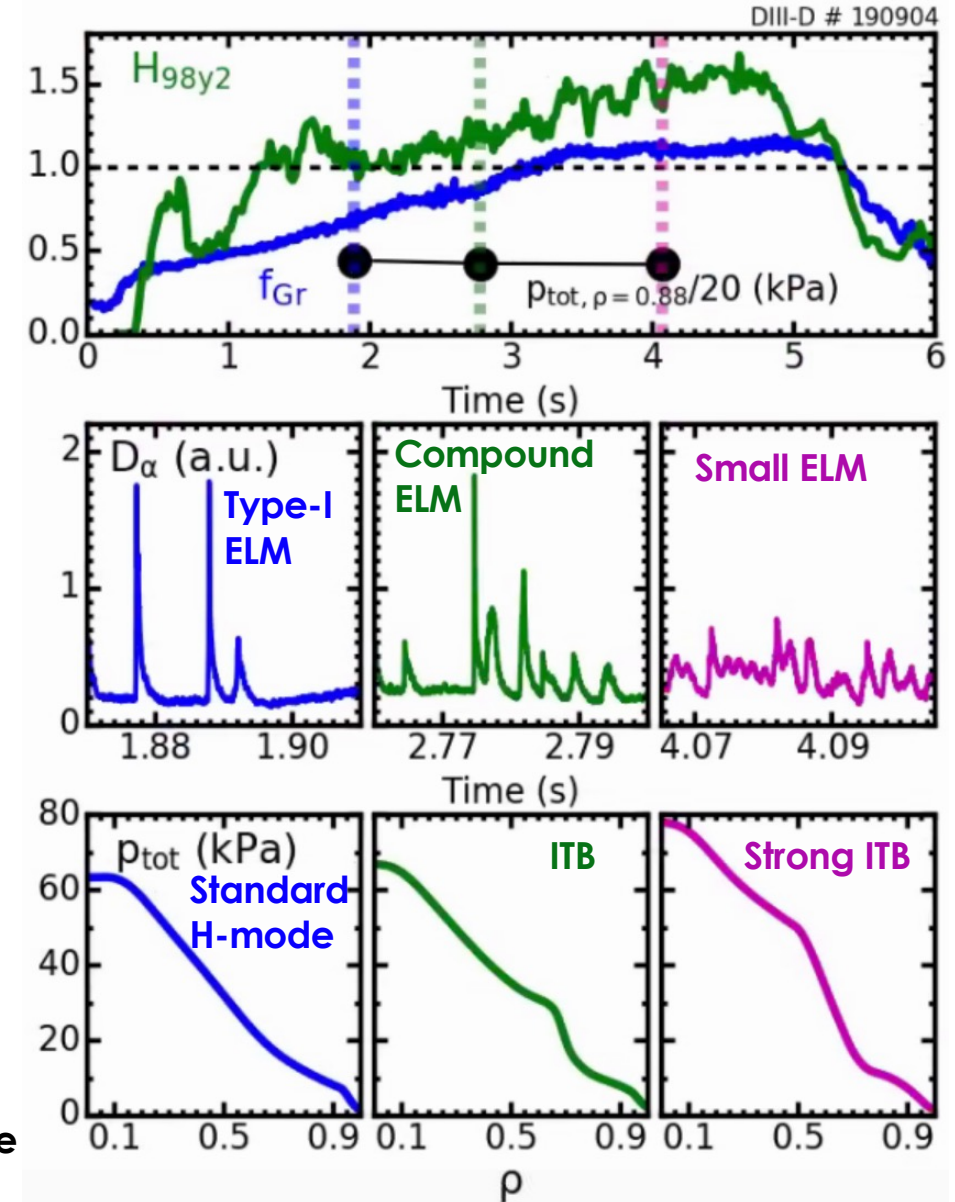
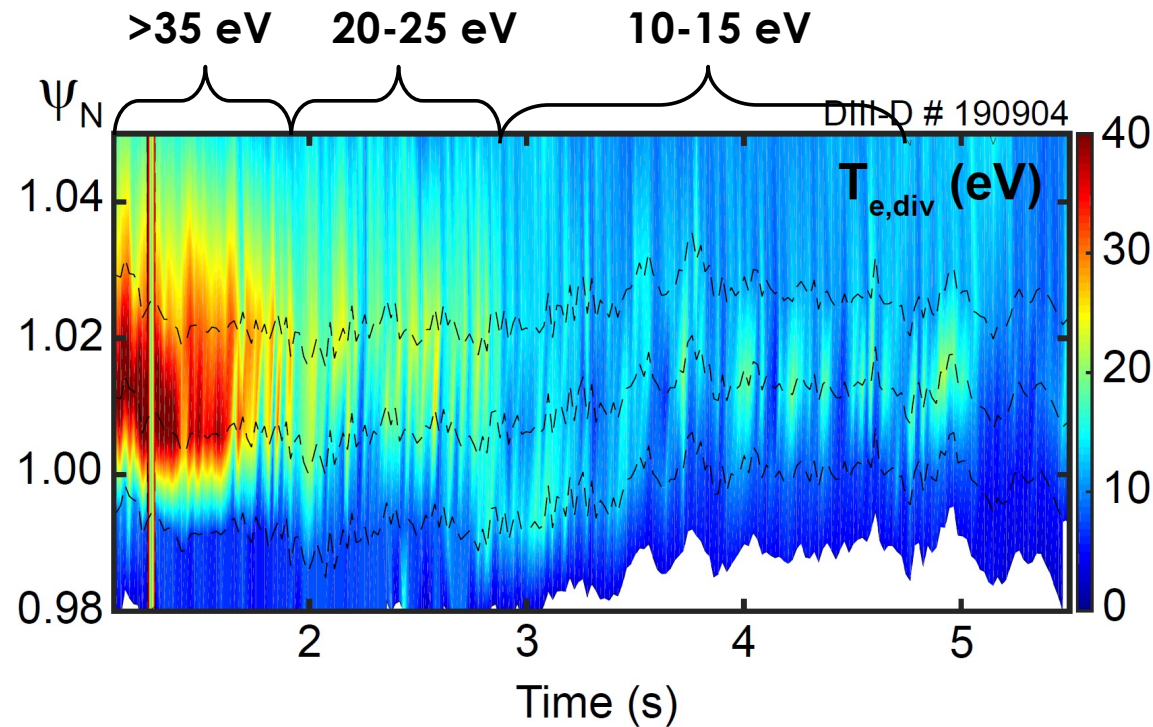
- On the performance requirements for steady-state fusion pilot plants (FPPs) and for ITER  $Q=10$   $P_{fus}=500$  MW operation at reduced  $I_p$
- DIII-D high- $\beta_p$  experiments and transport analysis
- Simulations of EAST high- $\beta_p$  plasmas and theory-guided experiments
- **Core-edge integration in DIII-D high- $\beta_p$  plasmas**
  - No large ELMs
  - Divertor detachment } + high confinement quality

**Hot & dense core surrounded by radiative mantle**



# Small ELMs Observed at High Normalized Density and Confinement

- Total pedestal pressure remains ~constant
- Reduced divertor heat load at higher density with small ELMs
  - Divertor close to detachment



Ding et al, Nature  
2024

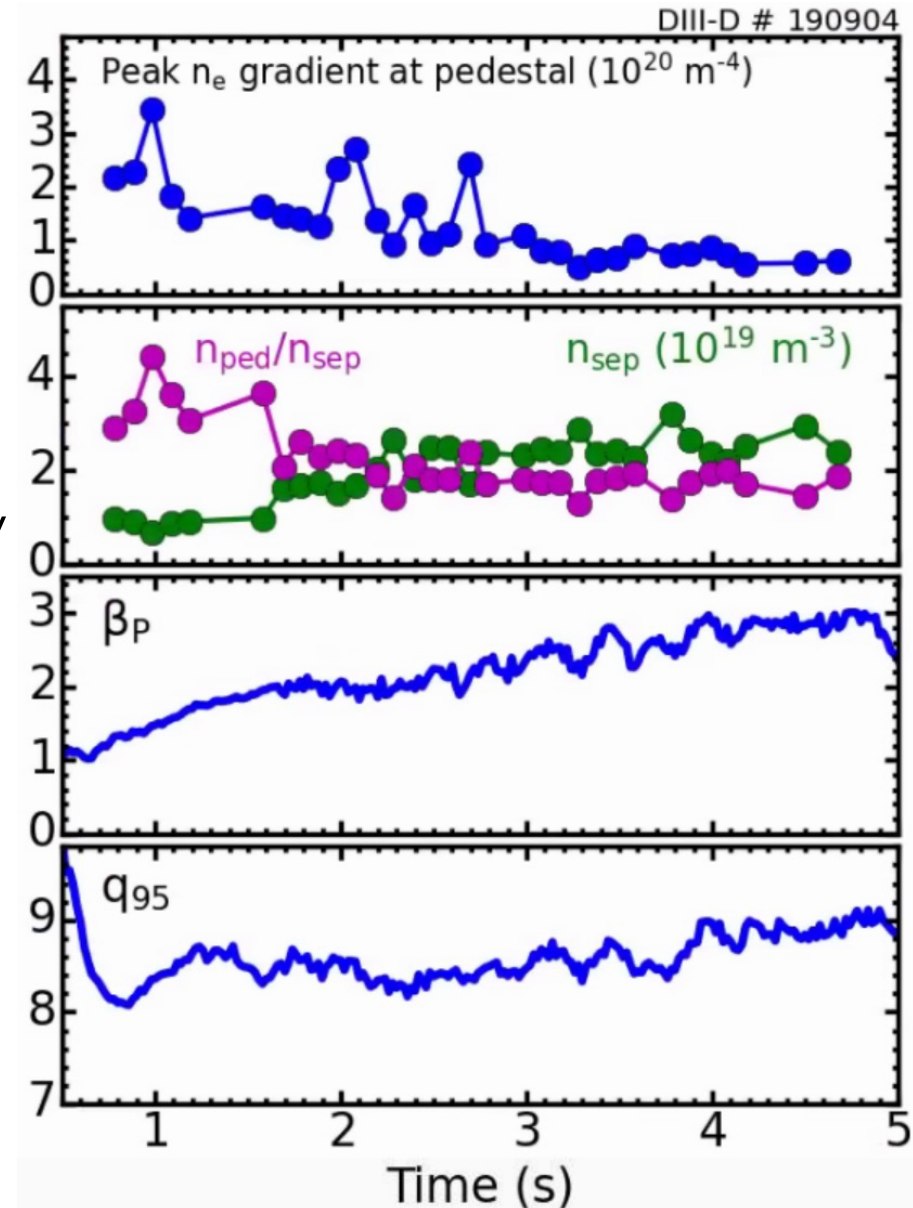
2<sup>nd</sup> IAEA Technical Meeting on Long-Pulse Operation of Fusion Devices, Oct 14-18, 2024, Vienna, Austria

# Discharge Naturally Evolves towards Expected Parameter Domain of Small ELM Regime

## Literature points out key parameters related to small ELM regime

Xu, Rev. Mod. Plasma Phys. 2023

- **Flat pedestal density profile**
  - Reduced peak pedestal  $n_e$  gradient at high density
- **High separatrix density**
  - Increased  $n_{sep}$  and reduced  $n_{ped}/n_{sep}$  at high density
  - May be related with strong gas puffing
- **High  $\beta_P$** 
  - $\beta_P$  increasing over time
- **High  $q_{95}$** 
  - Slightly increasing with  $\beta_P$



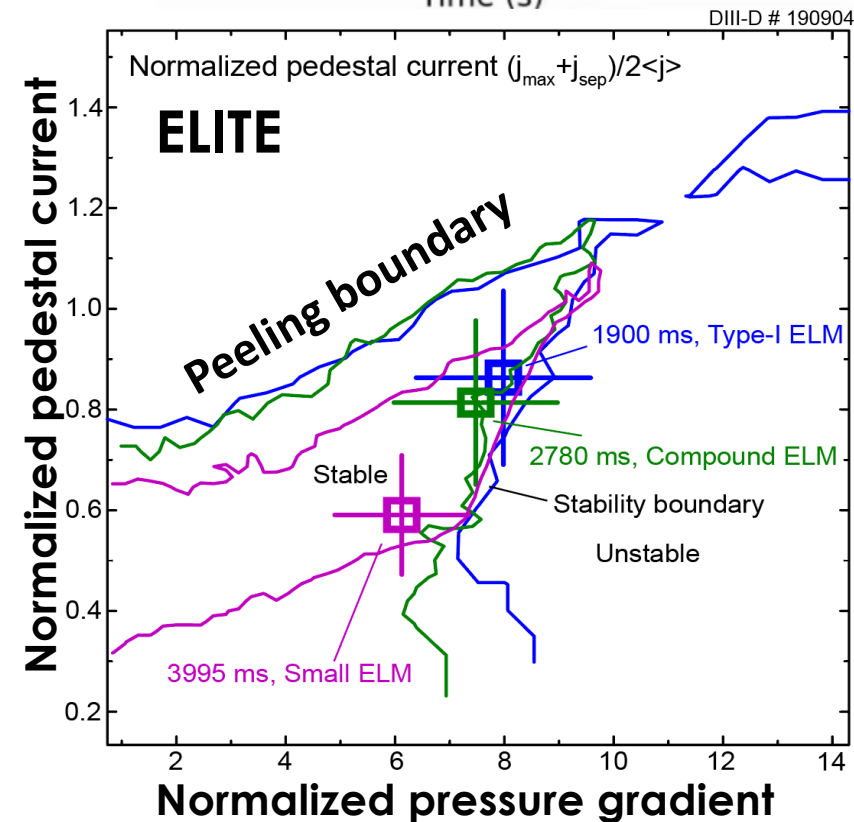
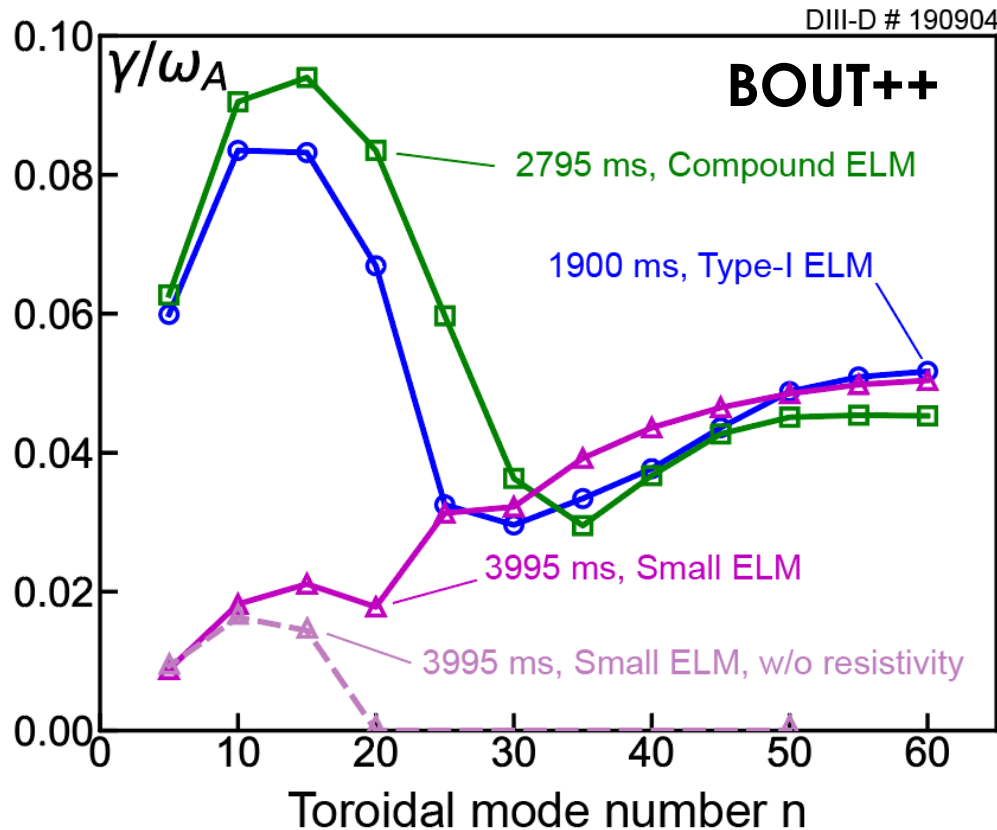
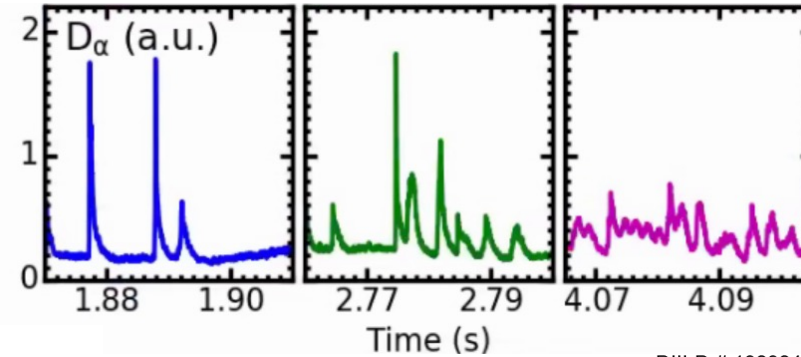
Ding et al, Nature  
2024

2<sup>nd</sup> IAEA Technical Meeting on Long-Pulse Operation of Fusion Devices, Oct 14-18, 2024, Vienna, Austria



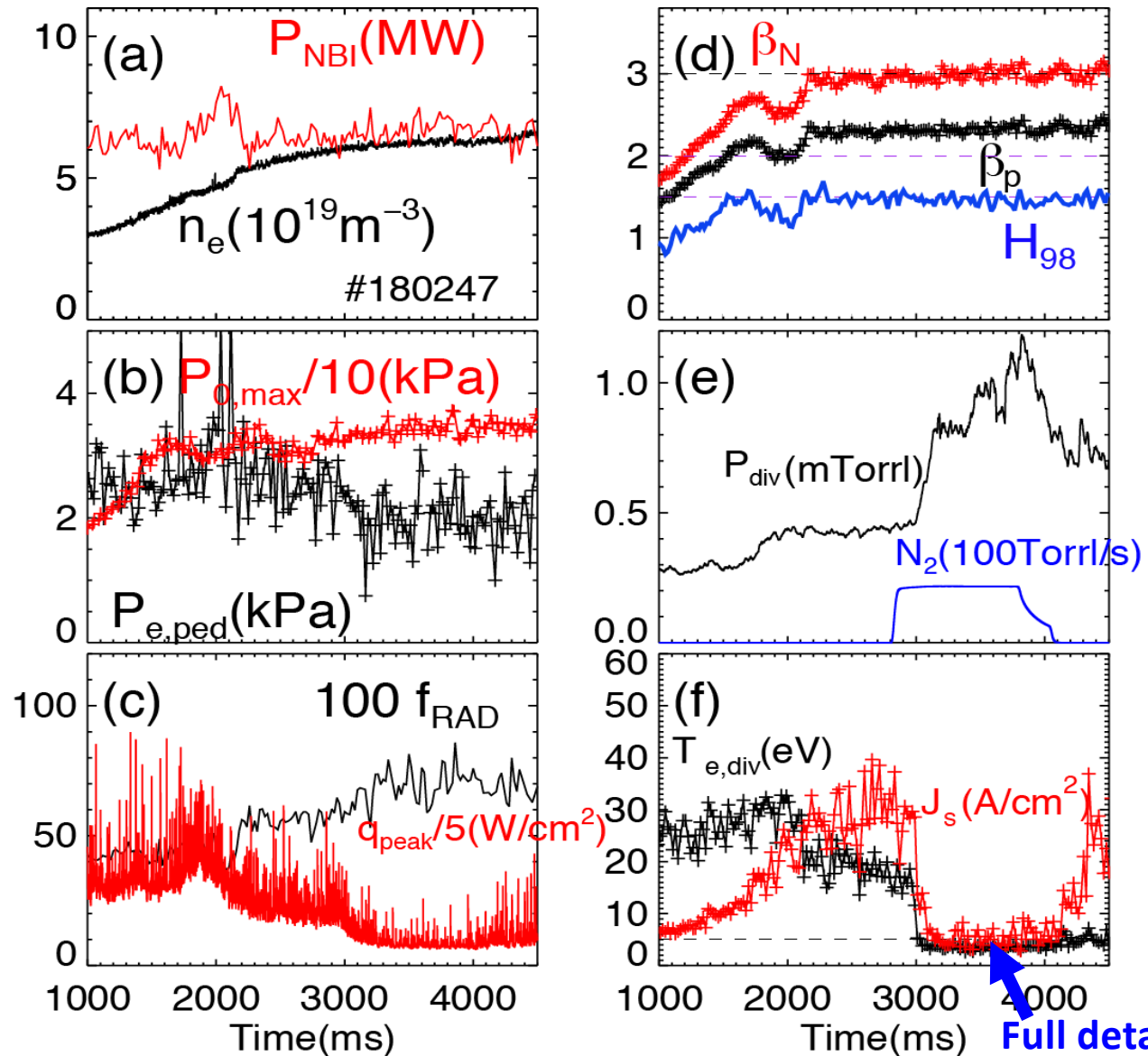
# Low Growth Rate of Low-n Modes and Predominance of High-n Resistive Ballooning Mode near the Separatrix Lead to Small ELMs

- **Low-n (~10) PBM at pedestal peak gradient**
  - BOUT++ agrees with ELITE
  - Small ELM case has lowest growth rate
- **High-n RBM at SOL and near separatrix**



Ding et al, Nature 2024

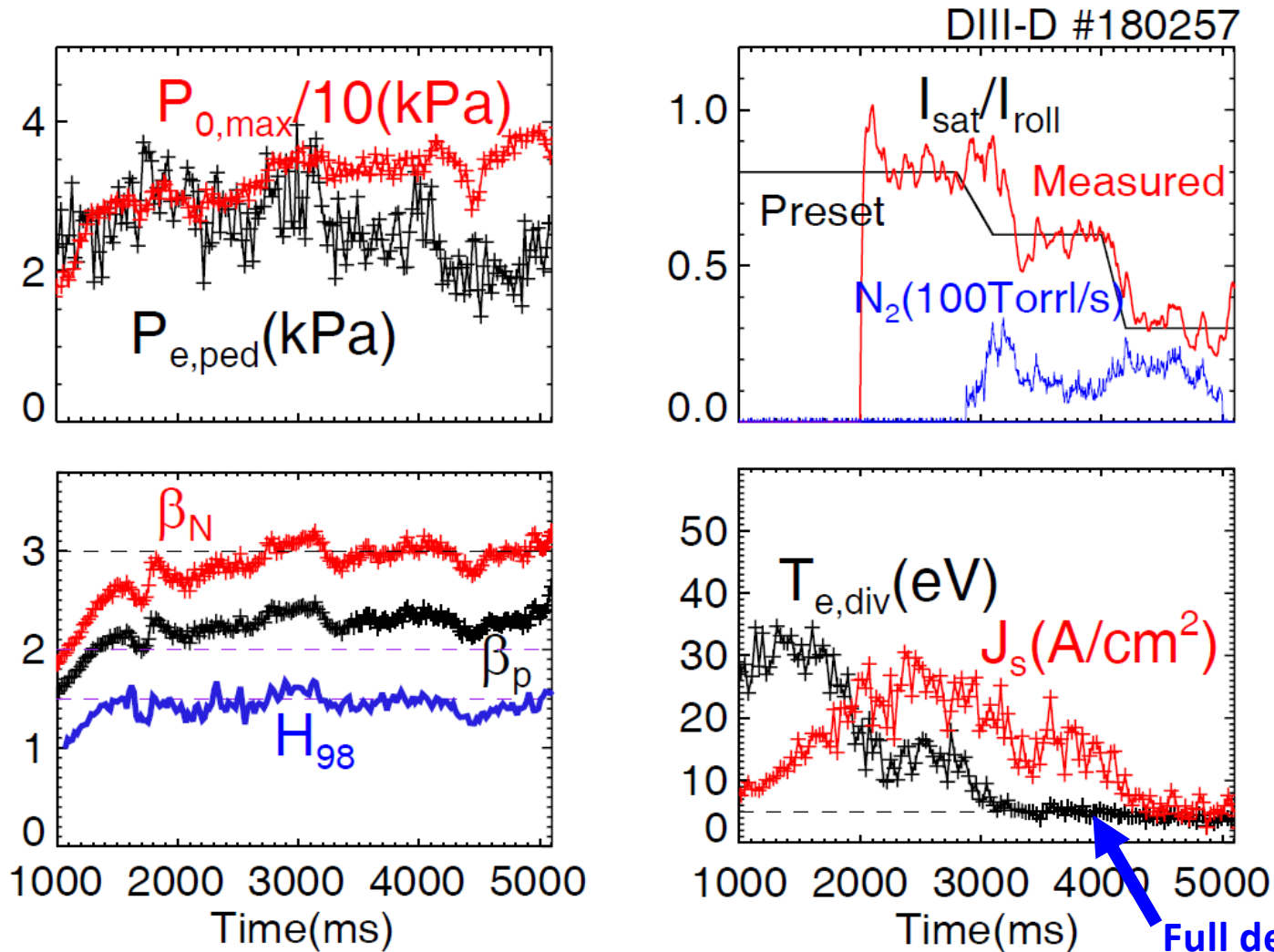
# High $\beta_p$ Plasmas Provided First Demonstrations of Excellent Energy Confinement Quality with Full Divertor Detachment



- $\text{N}_2$  injection in feedforward
- Pedestal height degrades, while core pressure is maintained

Wang et al, Nat. Commun. 2021  
Wang et al, Phys. Plasmas 2021

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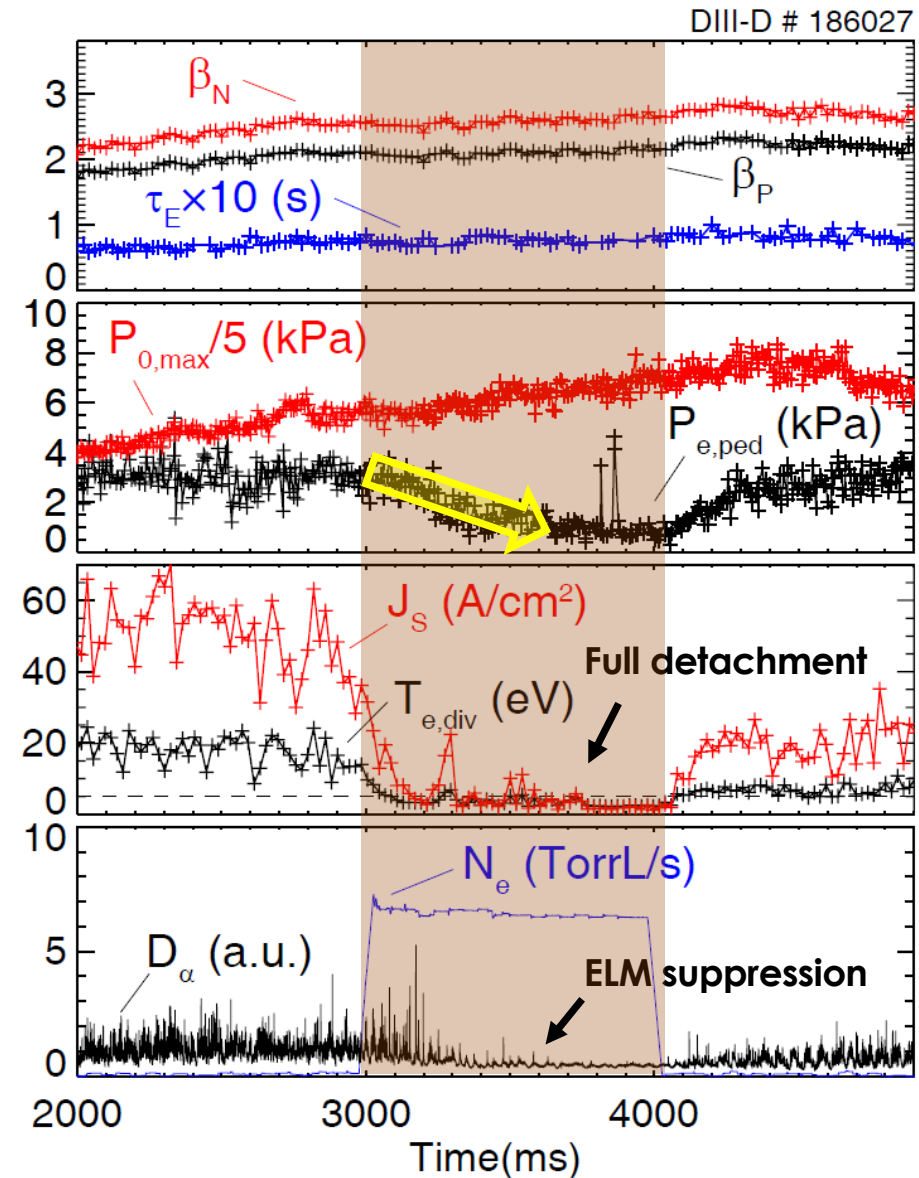
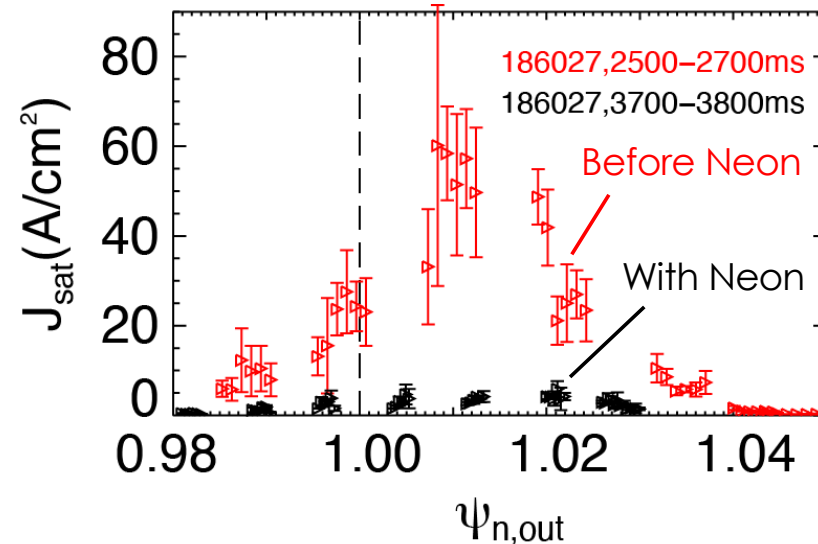


- $N_2$  injection under feedback control
- Pedestal height degrades, while core pressure is maintained

Wang et al, Nat. Commun. 2021  
 Wang et al, Phys. Plasmas 2021

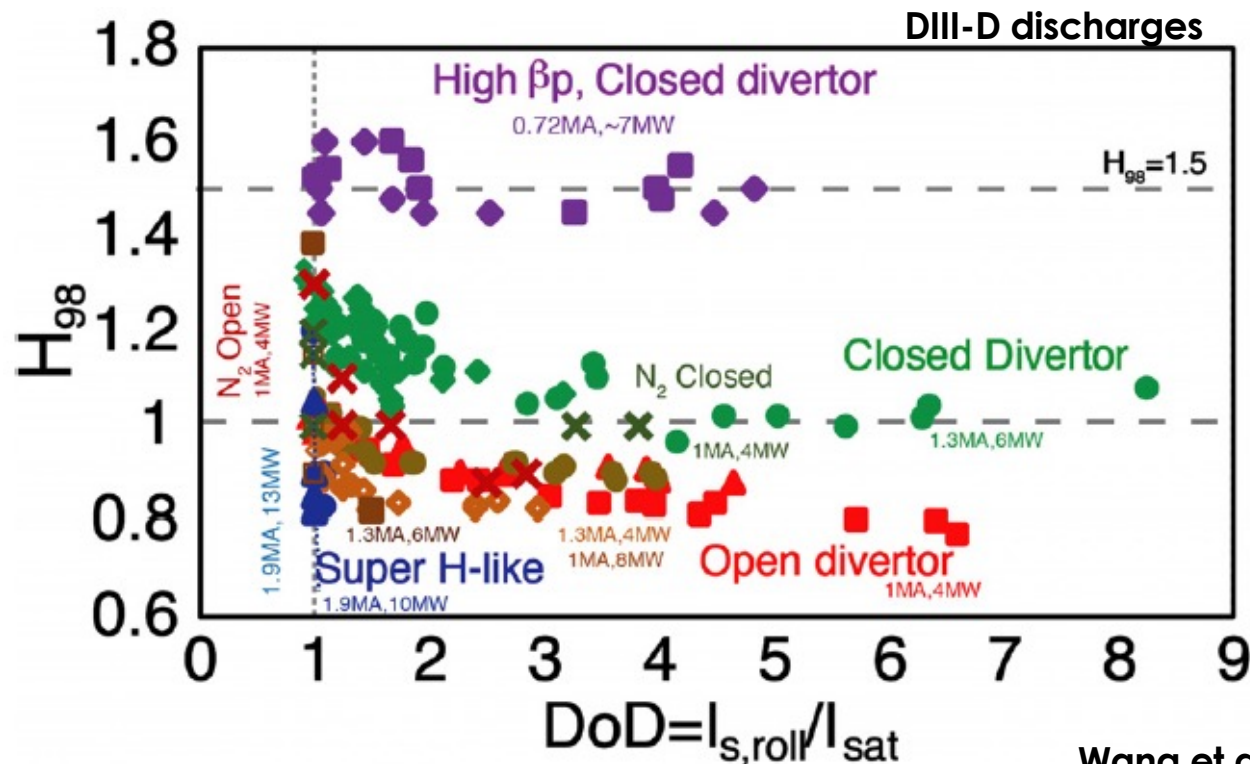
# High $\beta$ ITB Core + Full Divertor Detachment + ELM Suppression in ITER-Similar Shape Achieved on DIII-D

- Neon injection for divertor detachment
- Steady ELM suppression
- Core performance maintained

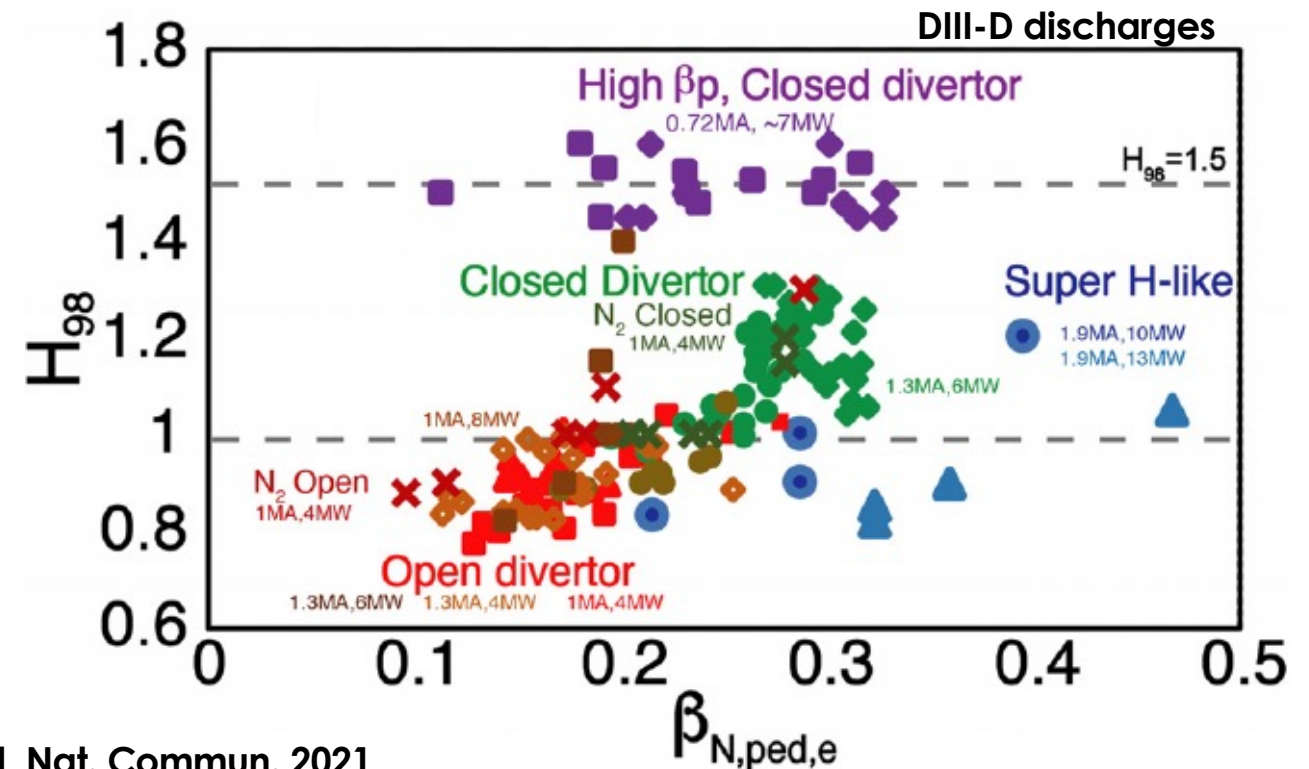


# High $\beta_p$ Plasmas Exhibit Core-Edge Integration Advantages Compared to Other H-Mode Scenarios

- High  $H_{98y2}$  with high Degree of Detachment (DoD) and low pedestal height



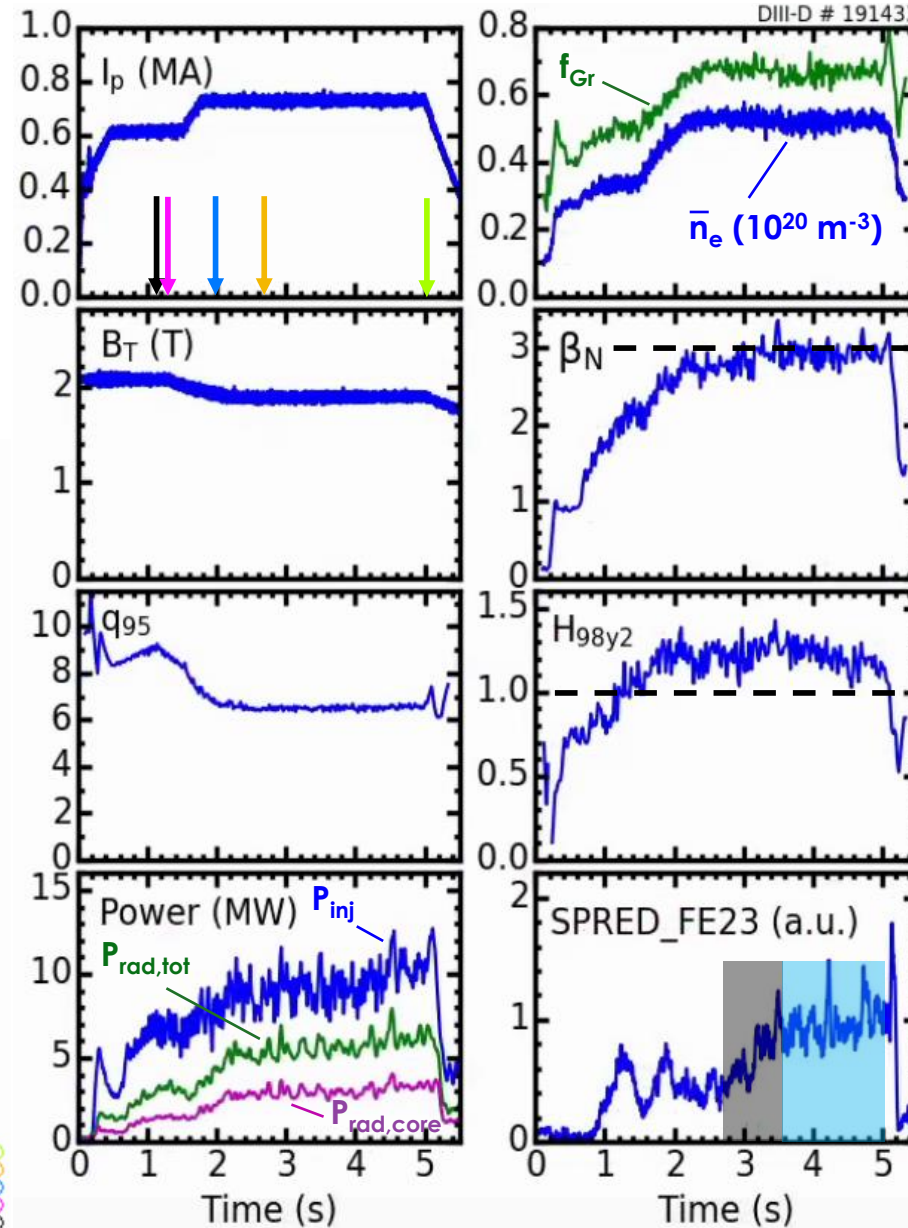
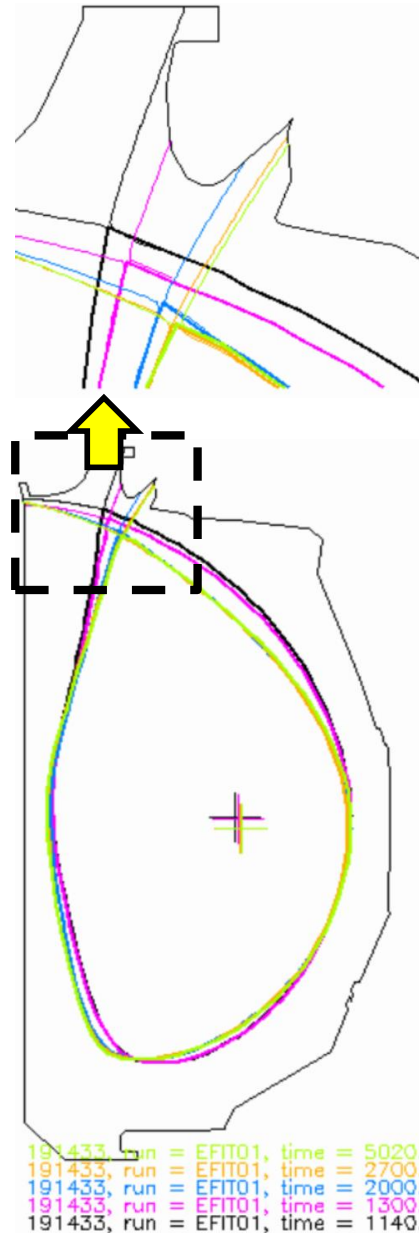
Wang et al, Nat. Commun. 2021



Make up for pedestal degradation and maintain global performance by breaking core stiffness and developing large-radius ITB

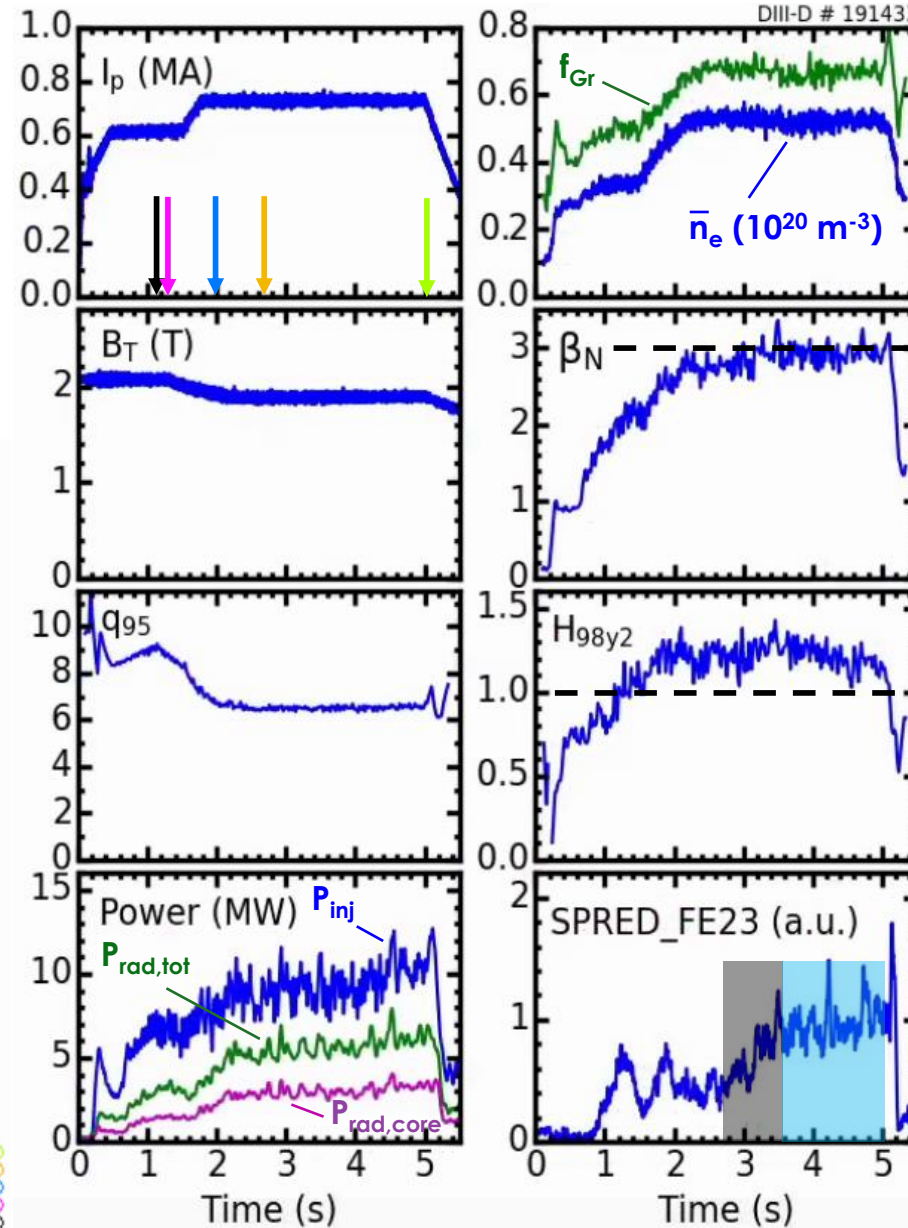
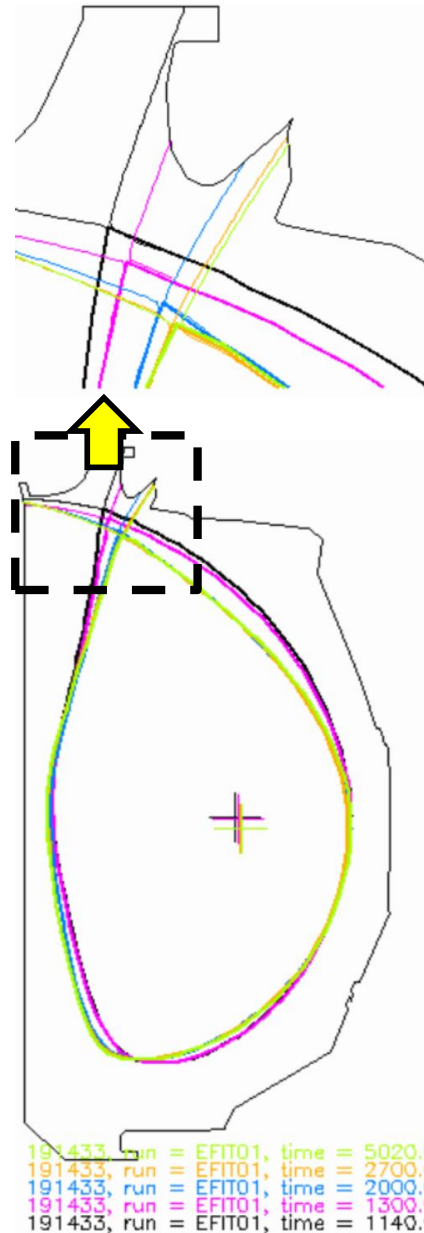
# On DIII-D, High $\beta_p$ Is Only Scenario Not Affected by Operation with Strike Point on Tungsten Divertor Ring

- Well-controlled strike point at tungsten ring from 2.7 s to 5 s
- High-Z emission first increases, then stays constant
- **No ECH**



# On DIII-D, High $\beta_p$ Is Only Scenario Not Affected by Operation with Strike Point on Tungsten Divertor Ring

- Well-controlled strike point at tungsten ring from 2.7 s to 5 s
- High-Z emission first increases, then stays constant
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- Strike point moves onto W surface when core state well developed
- Can we access the same core state if the strike point is on W from the onset?

# Density $\leftrightarrow$ Confinement Synergy Is Extremely Favorable towards a Core-edge Integration Solution

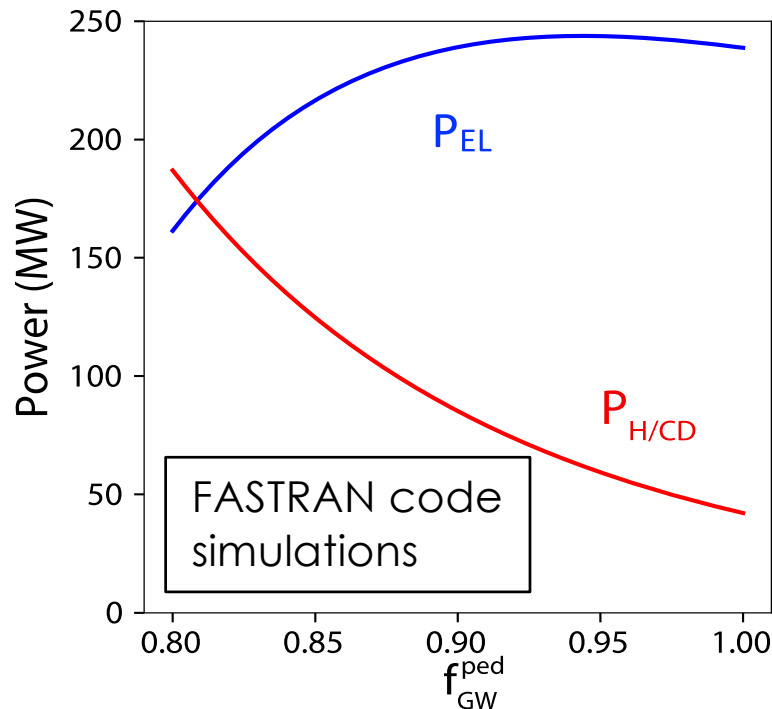
- **High density reduces the temperature at divertor and wall**
  - Experiment close to full detachment even without impurity seeding
- **High density strengthens ITB & reduces edge pedestal  $\rightarrow$  smaller ELM risk**
  - ELM damage  $\sim$  Peak ELM fluence  $\sim$  Pedestal pressure [Eich et al, NME 2017]



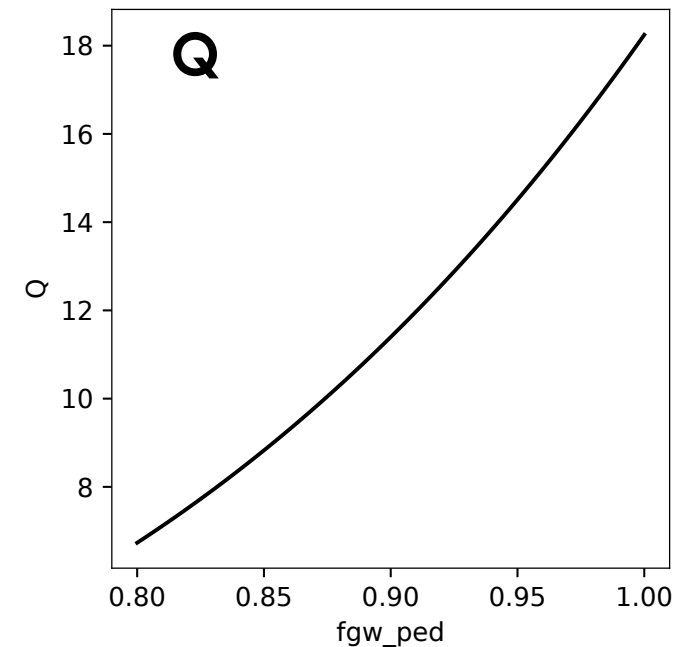
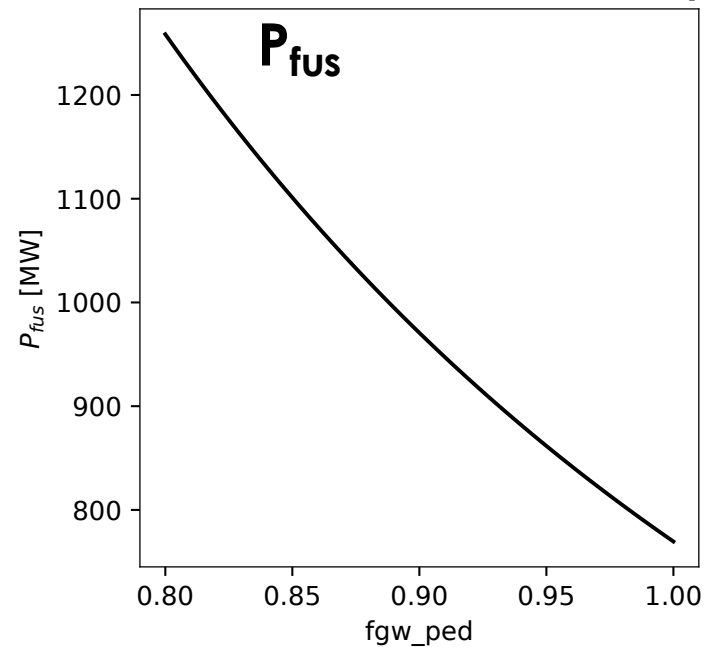
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  - ELM damage  $\sim$  Peak ELM fluence  $\sim$  Pedestal pressure [Eich et al, NME 2017]
- **Higher density Greenwald fraction can REDUCE external CD power, for fixed  $\beta_N$  target**
  - Higher  $f_{Gr}$   $\rightarrow$  lower  $I_p$ , lower  $P_{fus}$ , and much lower auxiliary power are required to maintain  $\sim$ constant electric power output

Buttery et al, NF 2021, "The advanced tokamak path to a compact net electric fusion pilot plant"

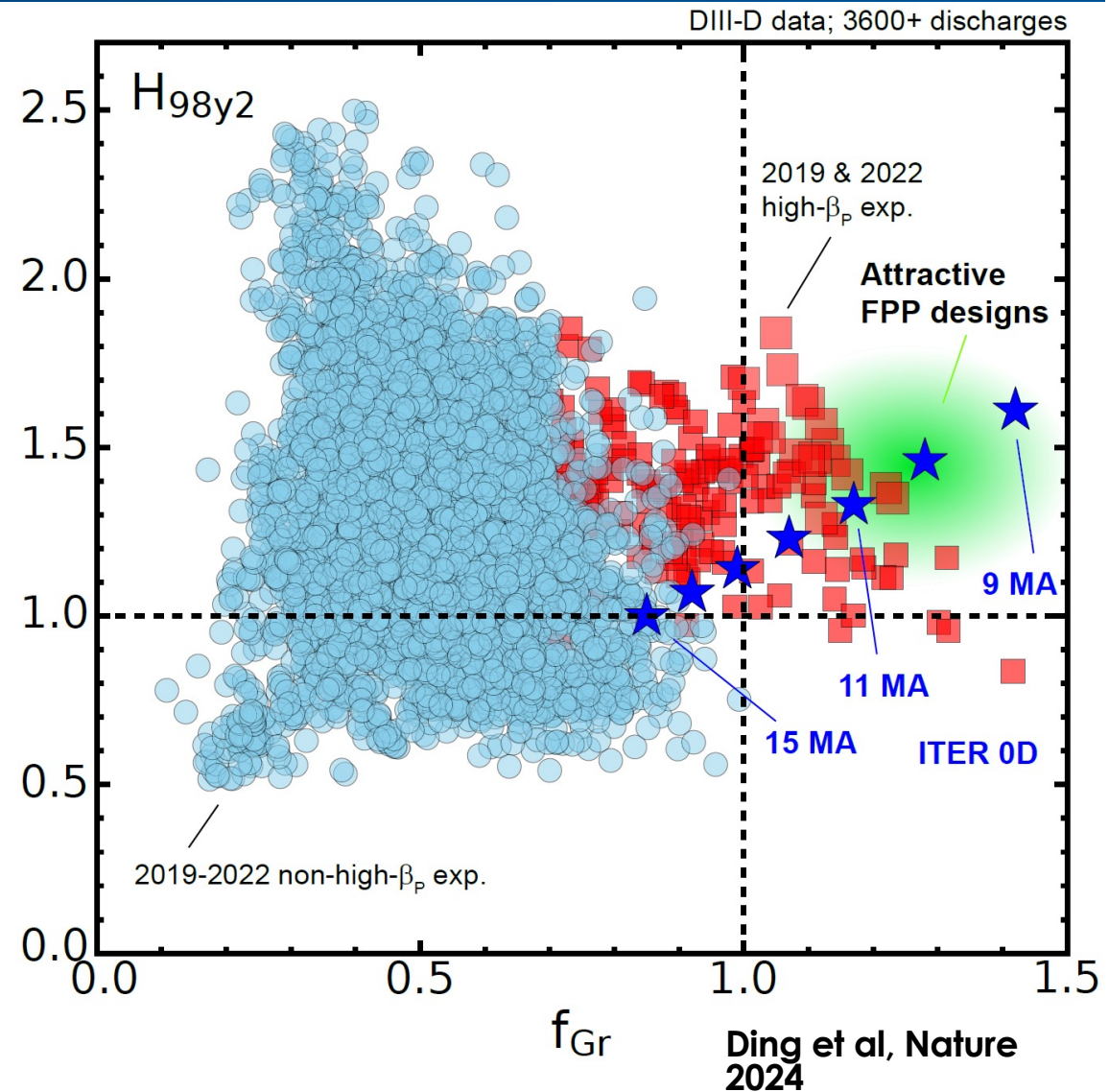


Courtesy of J.M. Park



# Coordinated Breakthroughs on DIII-D and EAST Overcome Long-standing Performance Limit, Show Path to High Confinement at High Density

Key physics:  $\alpha$ -stabilization of turbulence, amplified by high density gradients

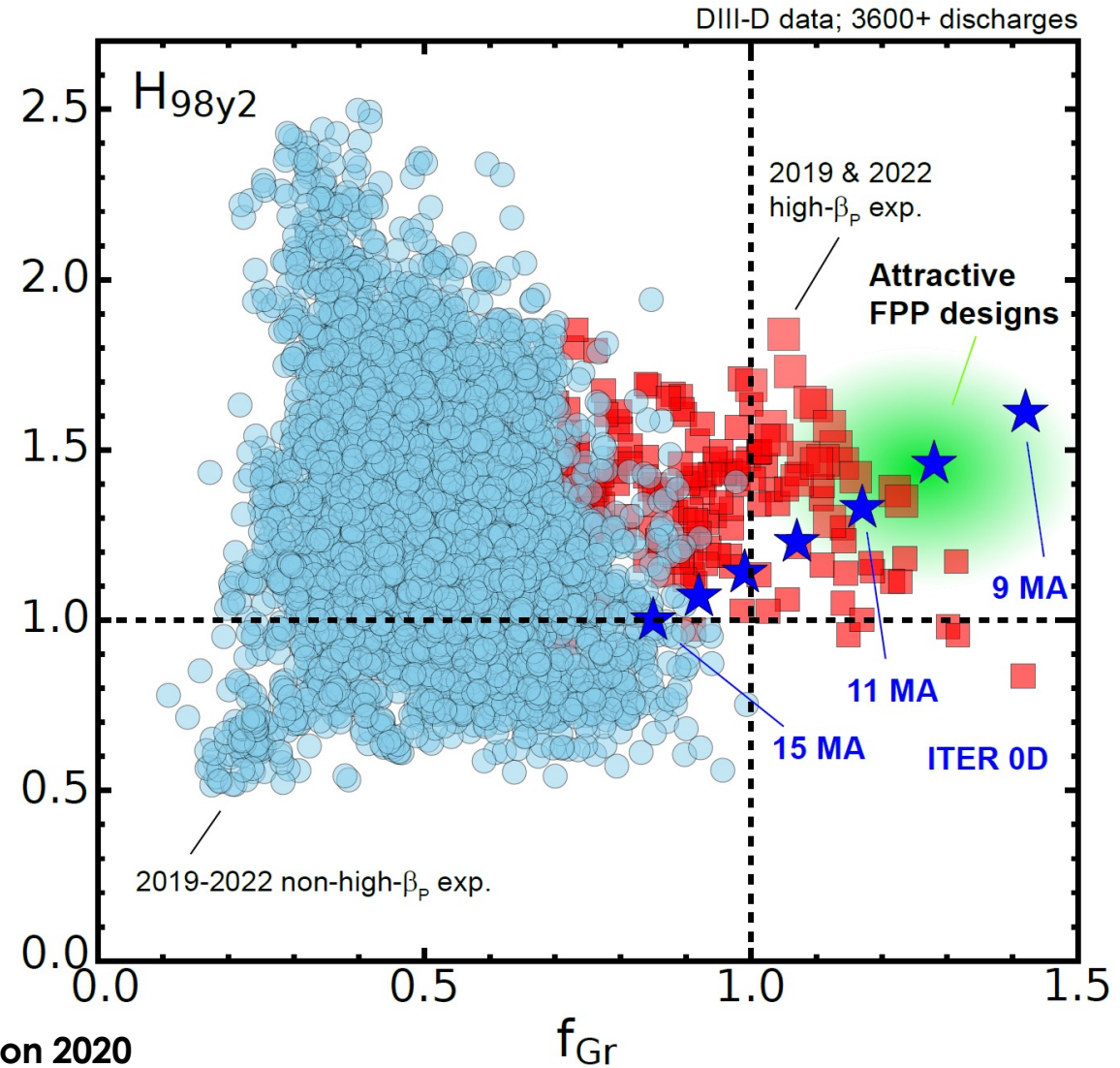


# Coordinated Breakthroughs on DIII-D and EAST Overcome Long-standing Performance Limit, Show Path to High Confinement at High Density

## Key physics: $\alpha$ -stabilization of turbulence, amplified by high density gradients

- Only path to sustained operation with  $f_{Gr} > 1$  and  $H_{98y2} > 1$  Ding et al, Nature 2024
- Only path to sustained operation with detached divertor and  $H_{98y2} > 1$  Wang et al, Nat. Comm. 2021
- Only path for long-pulse H-mode on tungsten divertor (EAST, KSTAR) and high performance with strike point on tungsten ring (DIII-D) Gong et al, Nucl. Fusion 2024 Kim H-S et al, this meeting
- Only path achieving SS  $Q=5$  & low- $I_p$   $Q=10$  using day-1 H&CD in theory-based predictions for ITER McClenaghan et al, Nucl. Fusion 2020

Ding et al, IAEA FEC 2021



# Outstanding Challenges

- **What determines the radius of ITB?**
- **Impurity transport (W inward, helium outward)**
- **RWM stabilization at low rotation**
- **Impact of collisionality**
- **Scenario access compatible with reactor constraints**