
**Measurement of the cross section of $^{235}\text{U}(n,f)$
induced by high-energy neutrons relative to
n-p elastic scattering performed
at the n_TOF facility at CERN**

A. Manna for the n_TOF Collaboration

IAEA Technical Meeting on Neutron Standards - 18-21 October 2022



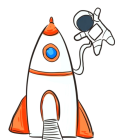
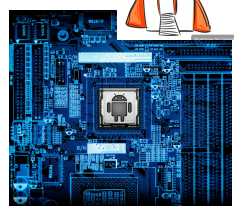
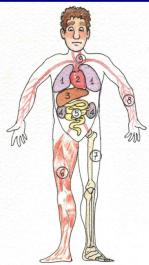
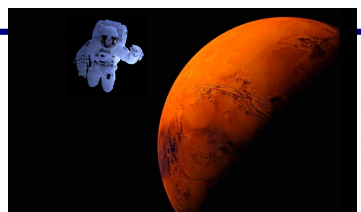
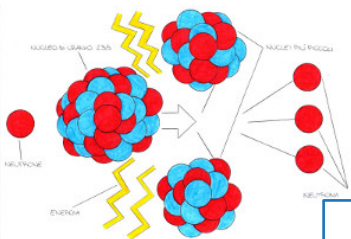
Motivations



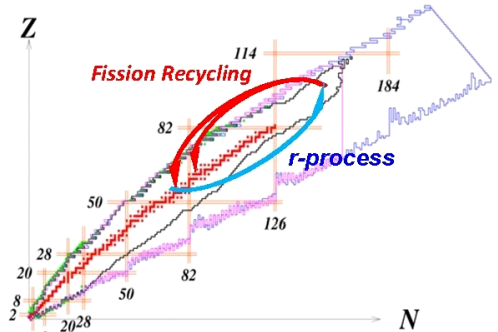
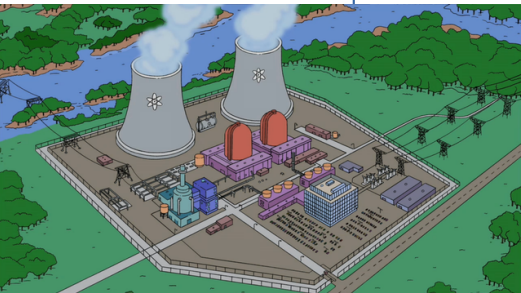
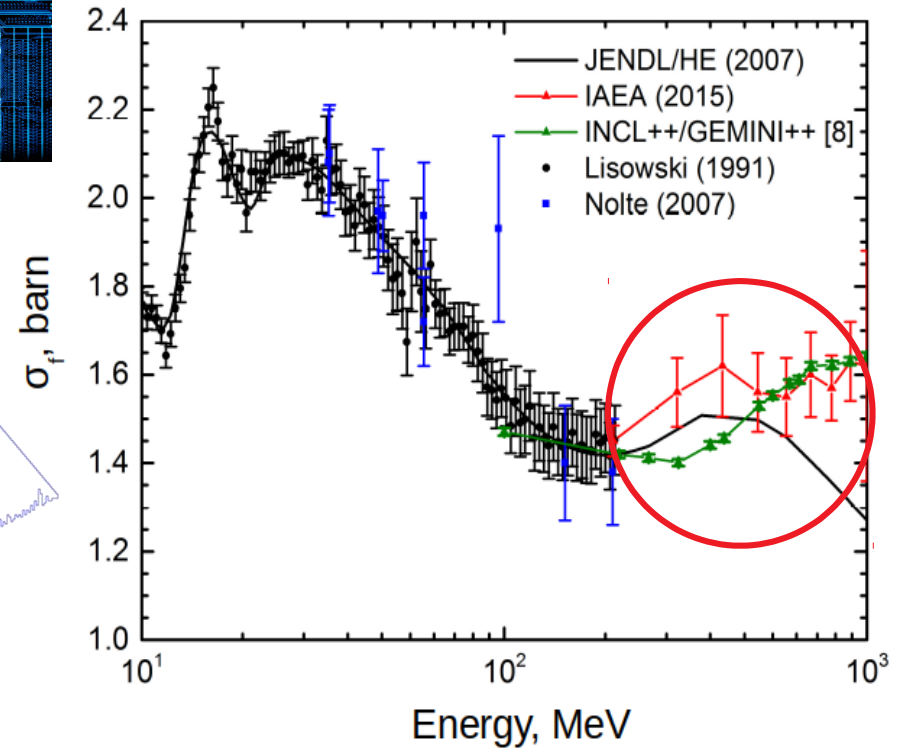
INDC International Nuclear Data Committee

“...Our analysis indicates that the new absolute measurements of the neutron induced fission cross section (e.g. relative to n-p scattering) on Uranium, Bismuth, Lead and Plutonium have the highest priority in establishing neutron induced fission reaction standard above 200 MeV...”

(INDC(NDS)-0681 Distr. ST/J/G/NM, IAEA 2015)

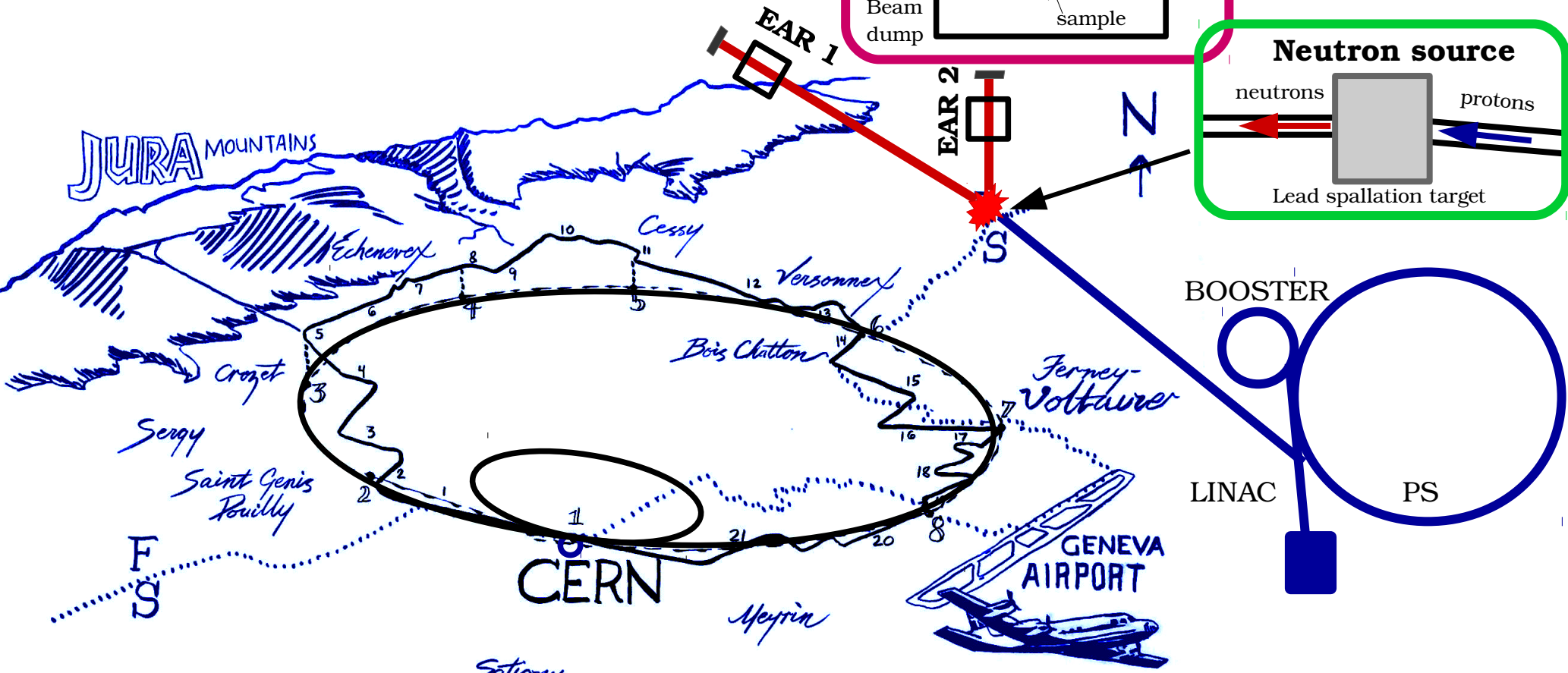
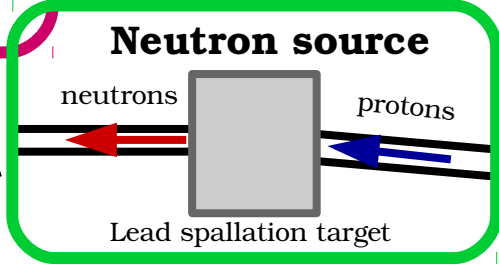
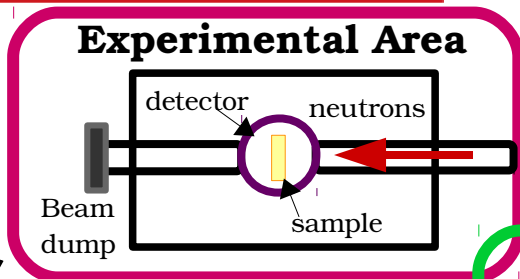


$^{235}\text{U}(n,f)$ is one of the most significant cross-section standards at 0.025 eV and [0.15-200] MeV **BUT** there are **no experimental data above 200 MeV**



The n_TOF facility

neutron Time Of Flight



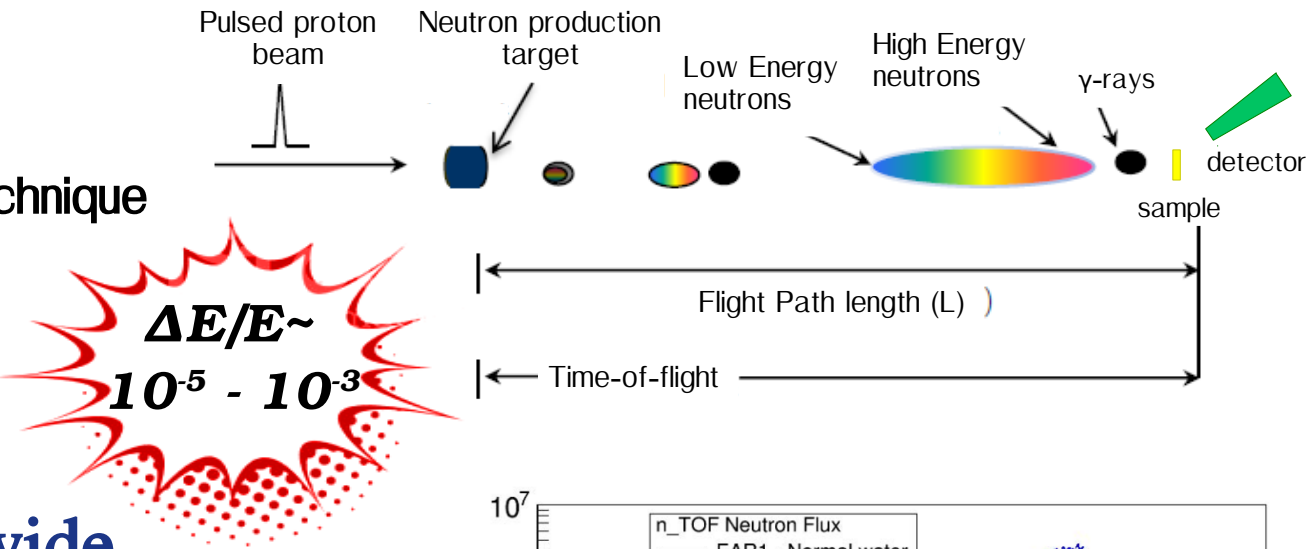
The n_TOF facility

neutron Time Of Flight

✦ **High energy resolution**

Time of Flight (ToF) technique

with a long flight path:
 185 m @ EAR 1
 20 m @ EAR 2

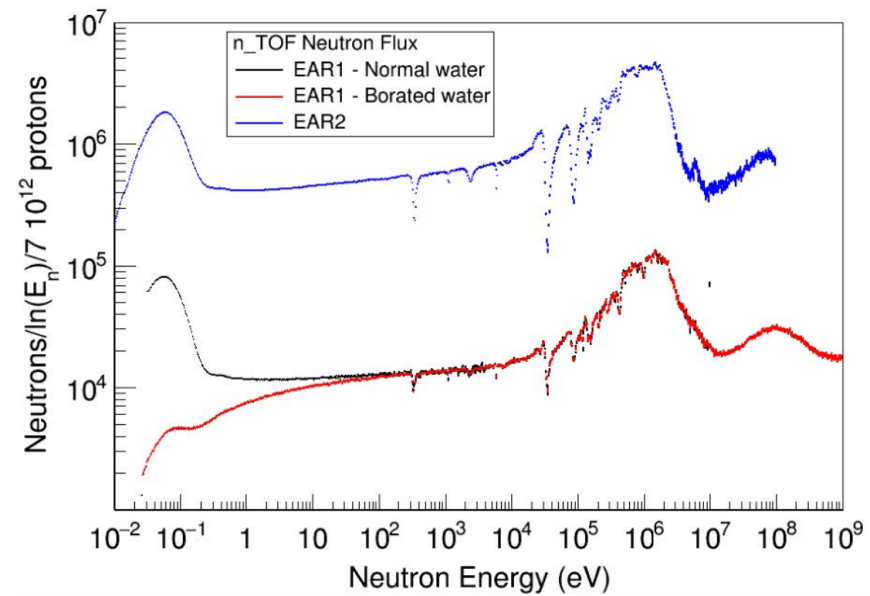


✦ **High neutron flux & wide energy range**

Spallation reaction

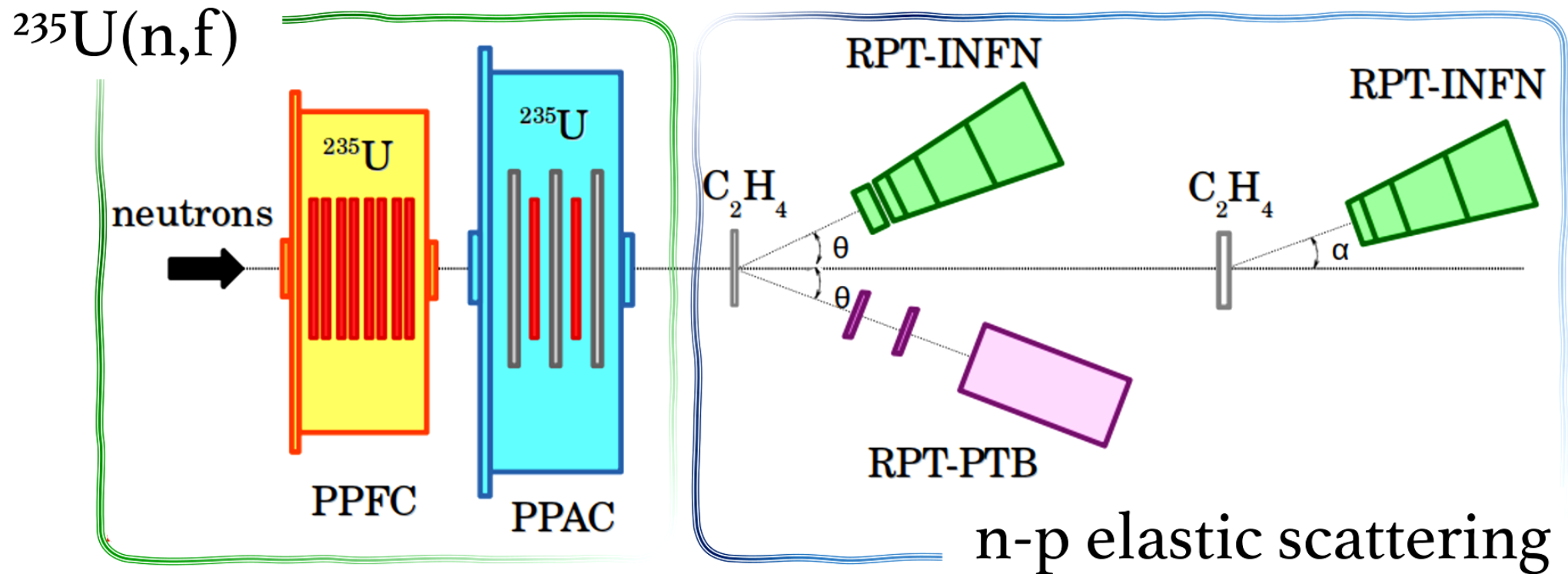
$7 \cdot 10^{12}$ protons,
 20 GeV/c momentum
 +
 1.3 ton Pb Target

350
 neutrons per
 incident proton

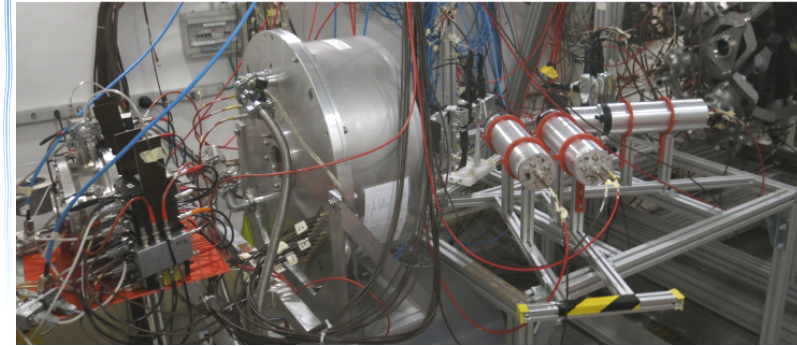
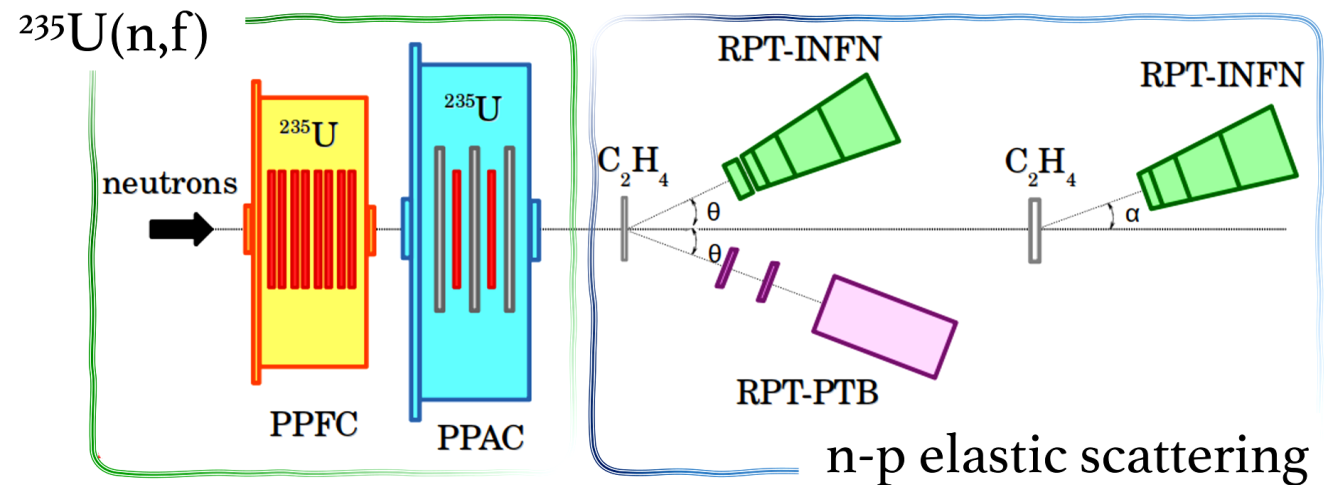


Experimental setup

$$\sigma_f(E_n) = \frac{N_f(E_n)}{n_U \varepsilon_f} \cdot \frac{1}{\Phi(E_n)} = \frac{N_f(E_n)}{n_U \varepsilon_f} \frac{nH \varepsilon_p \Omega}{N_p(E_n)} d\sigma_{(n,p)}/d\Omega$$



Experimental setup



^{235}U fission reaction

Fission fragment

Parallel Plate Fission Chamber

Neutron energy: 0,025 eV - 200 MeV

4 double sided U (99.93% ^{235}U) = 32,660 mg

Parallel Plate Avalanche Counter

Neutron energy: 0,025 eV - 1 GeV

2 samples U (92.7% ^{235}U) = 28 mg

Neutron flux

Elastic scattering

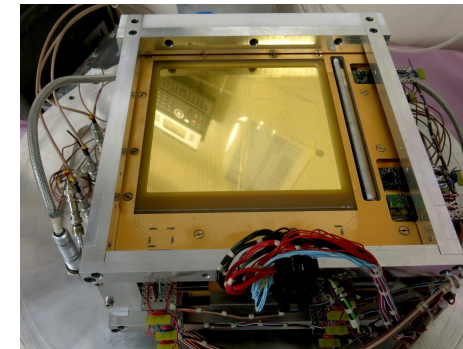
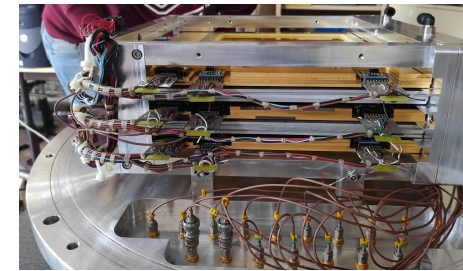
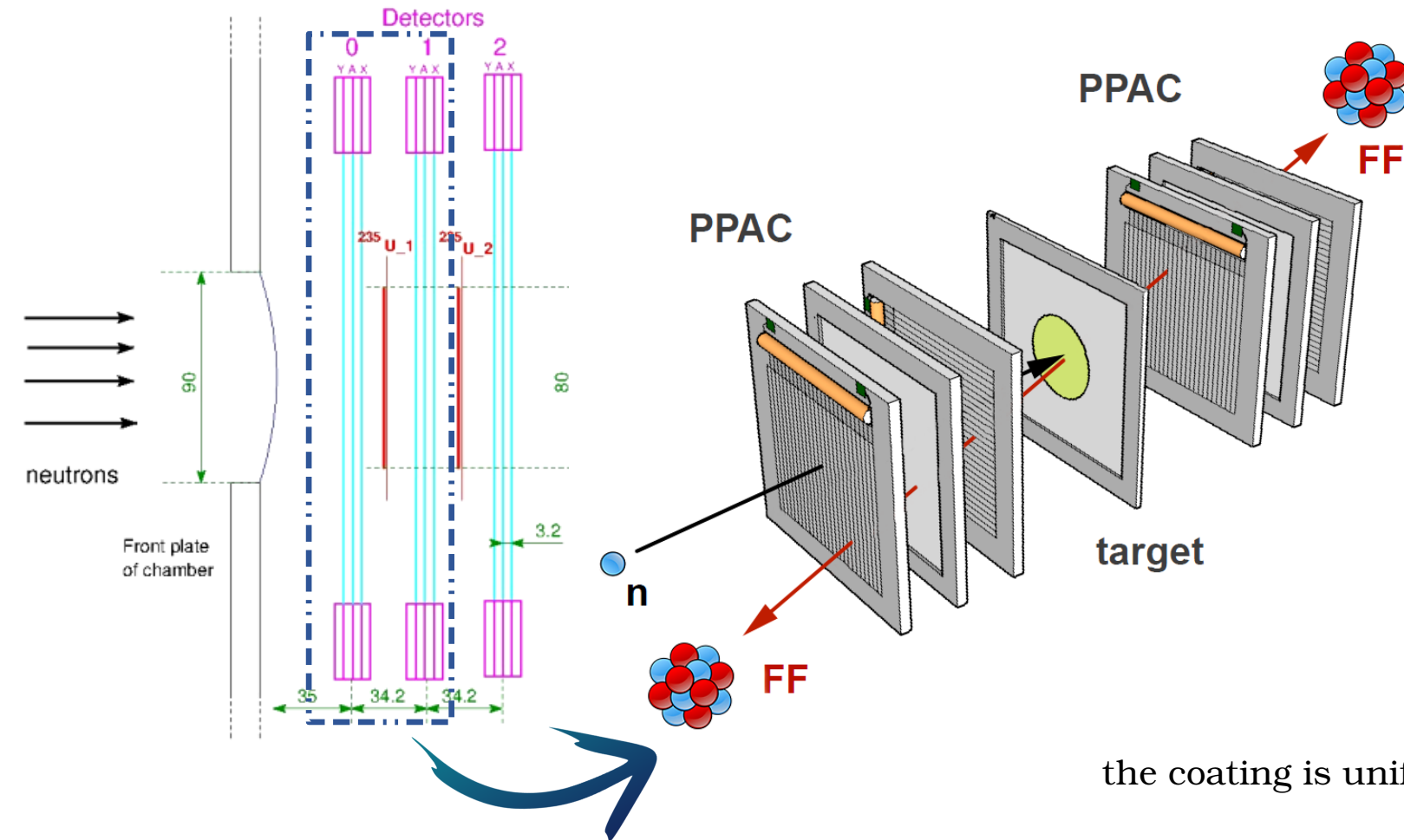
3 Proton Recoil Telescopes

located out of the neutron beam

- 2 @ 25° pointing at
a Polyethylene: 1/2/5 mm thick

- 1 @ 20° pointing
a Polyethylene: 5 mm thick

PPACs



The electrodes:

area of $20 \times 20 \text{ cm}^2$
 $1.7 \text{ }\mu\text{m}$ mylar foils,
 th gold, 700 nm thick

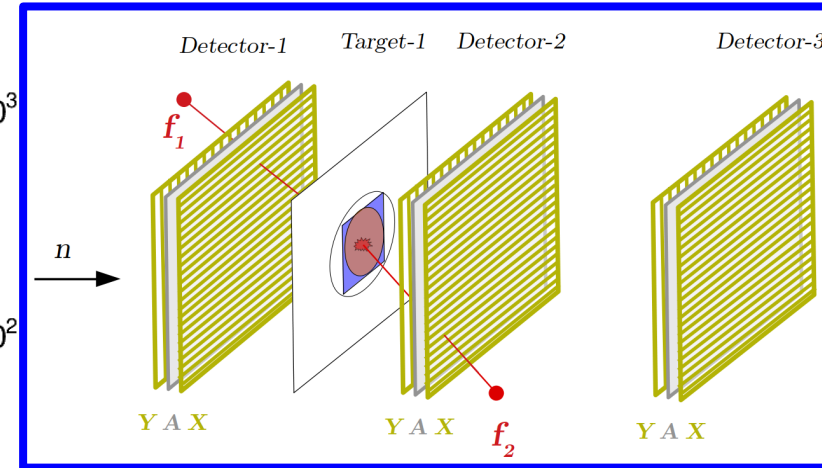
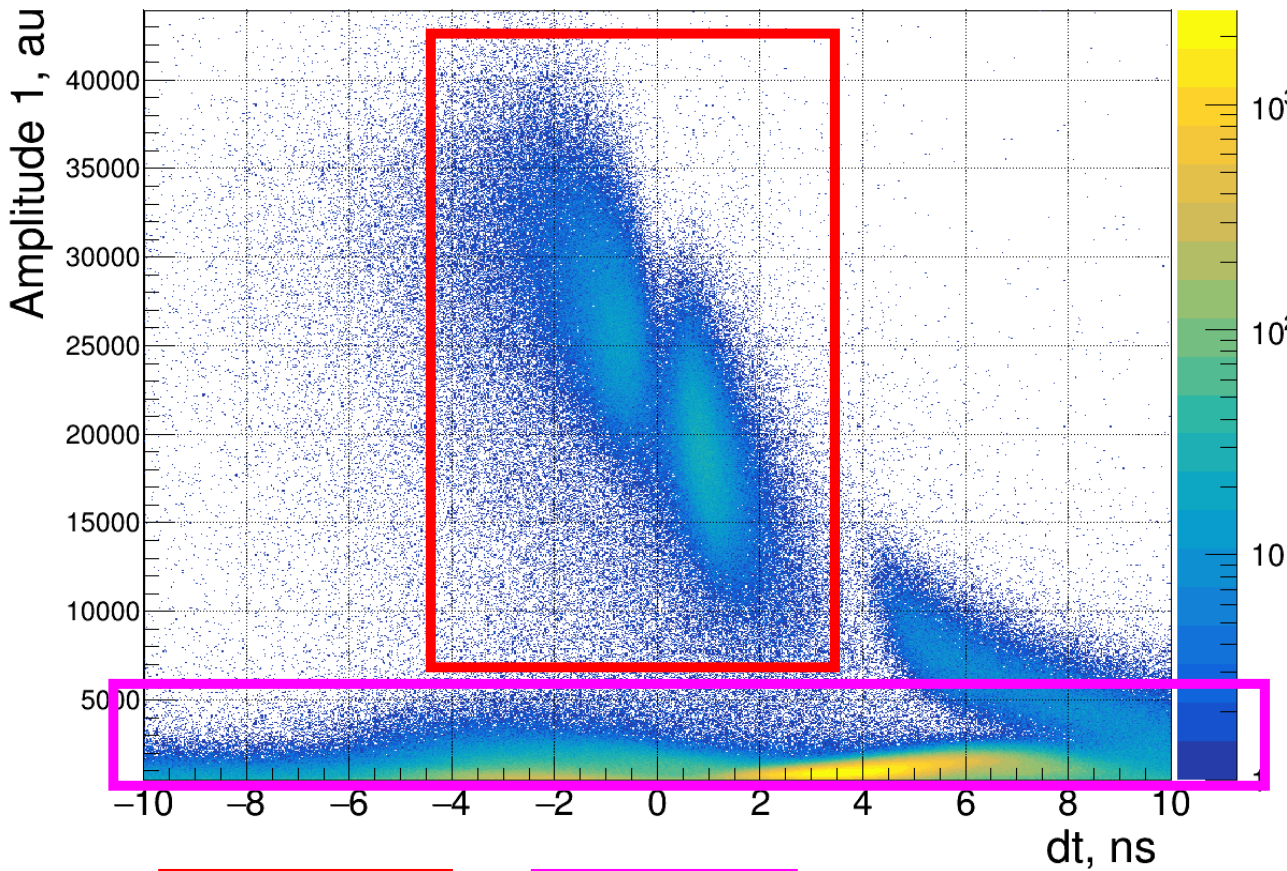
Anode:

the coating is uniform and double sided

Cathodes:

the coating is divided into 2 mm wide strips

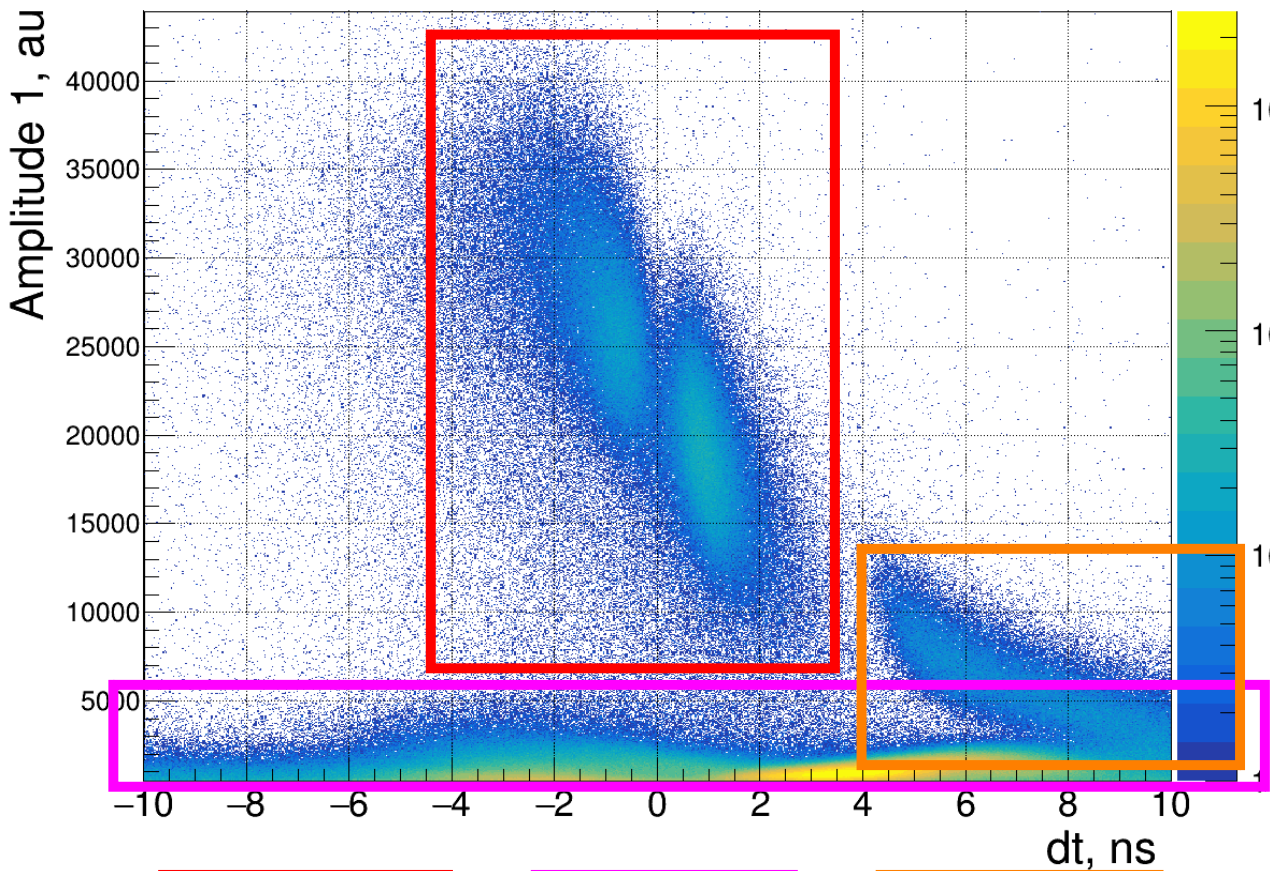
Fission events signature



FFs from
1st ^{235}U

α from
2nd ^{235}U

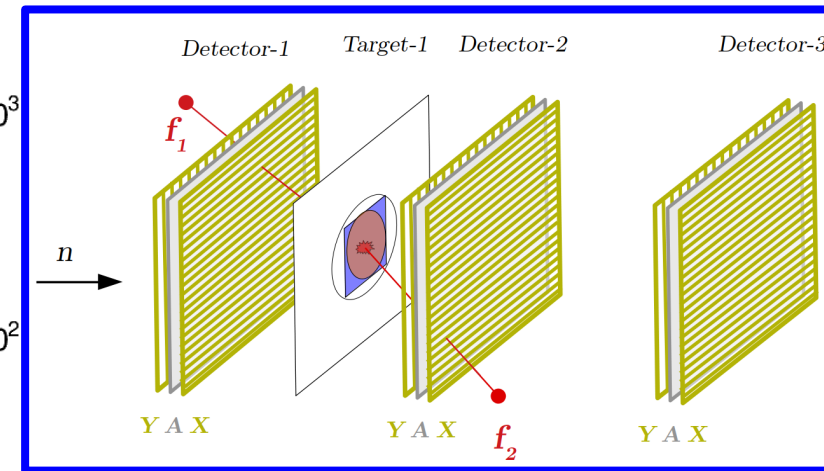
Fission events signature



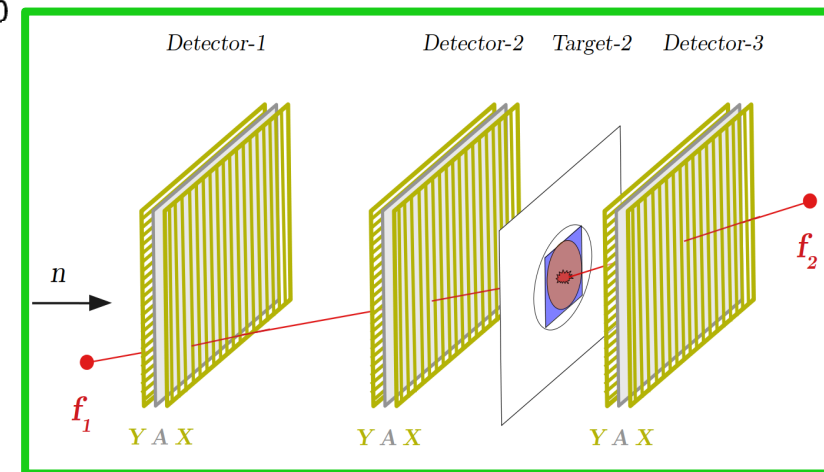
FFs from
1st ^{235}U

α from
2nd ^{235}U

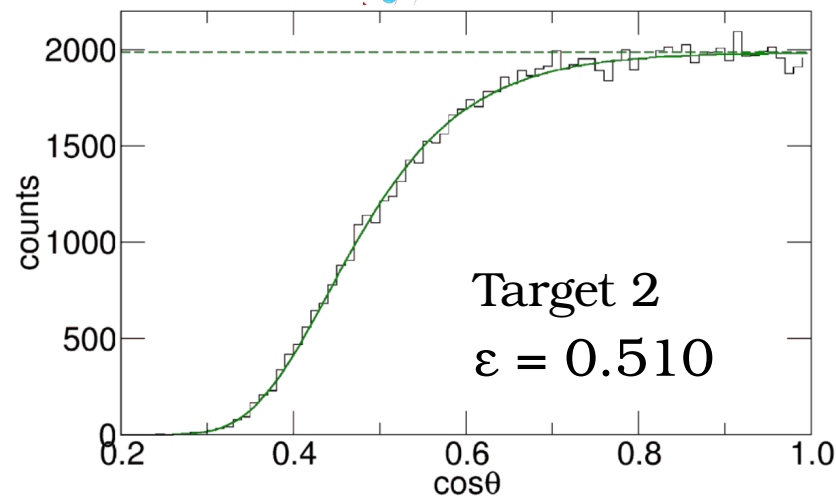
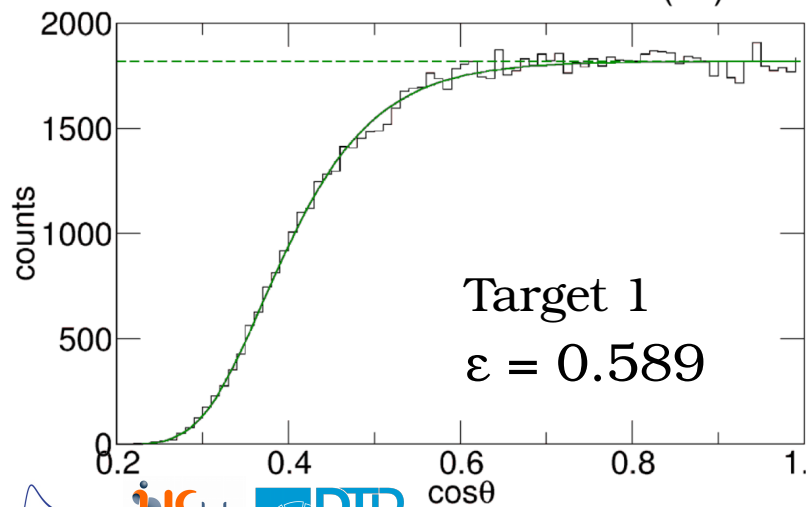
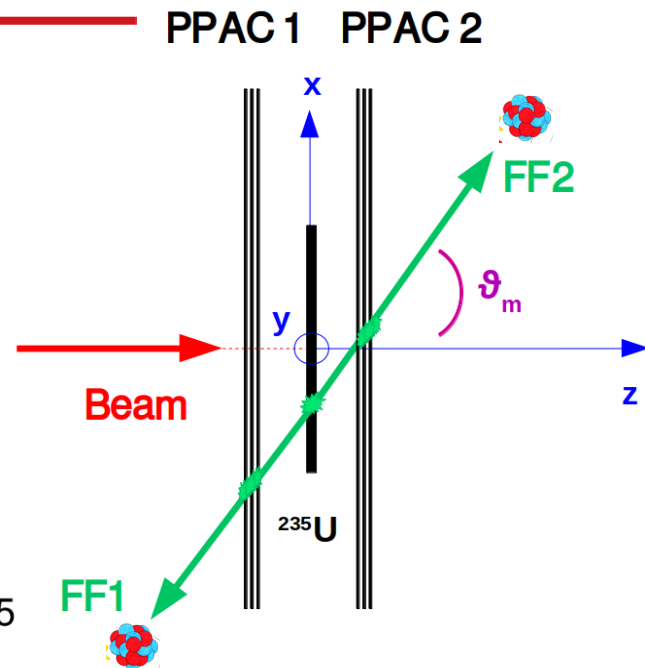
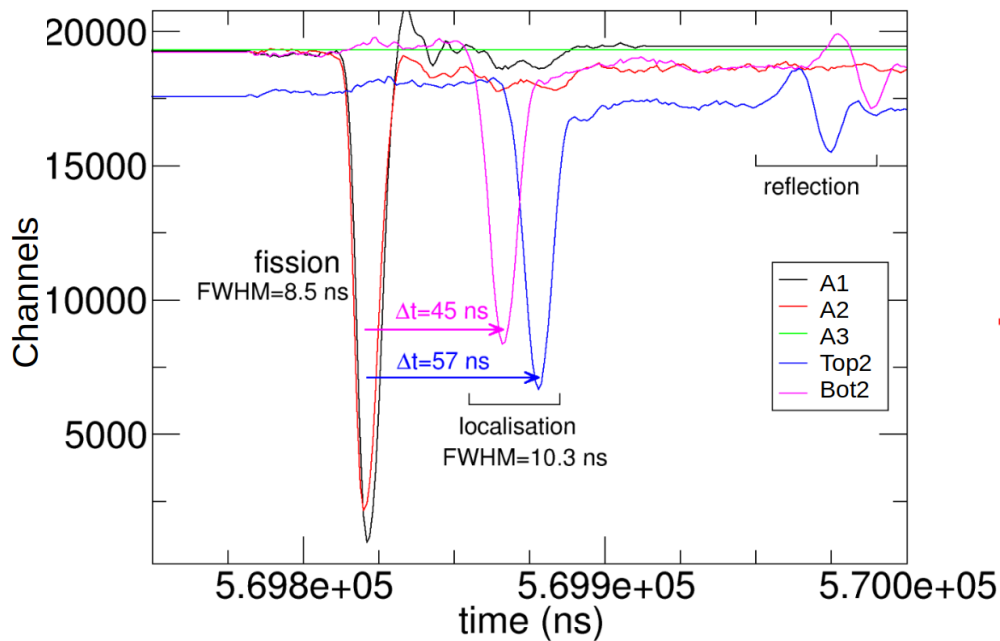
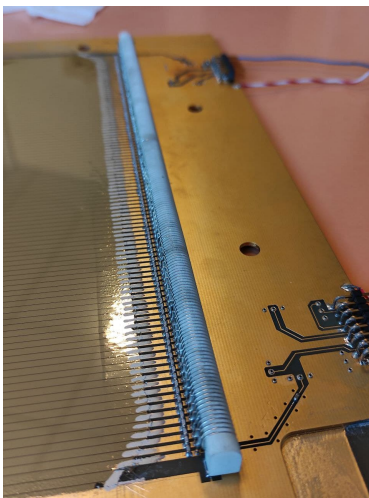
FFs from
2nd ^{235}U



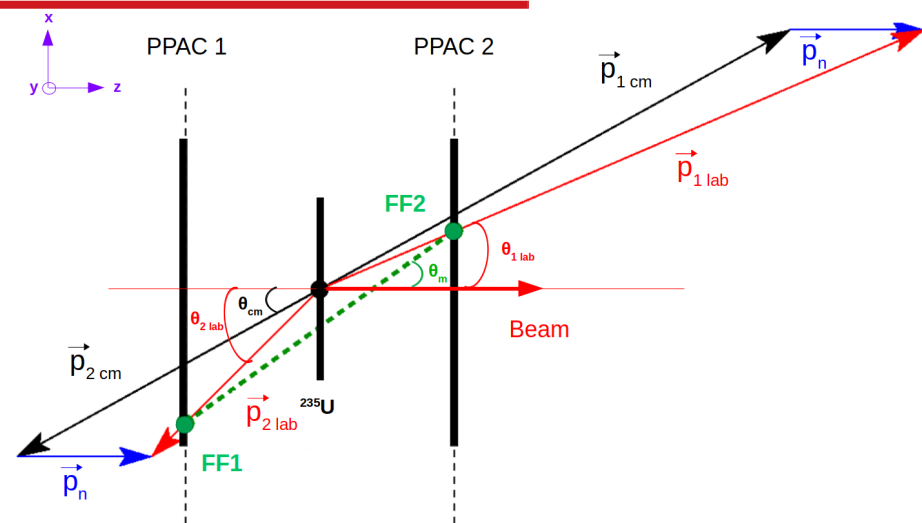
+



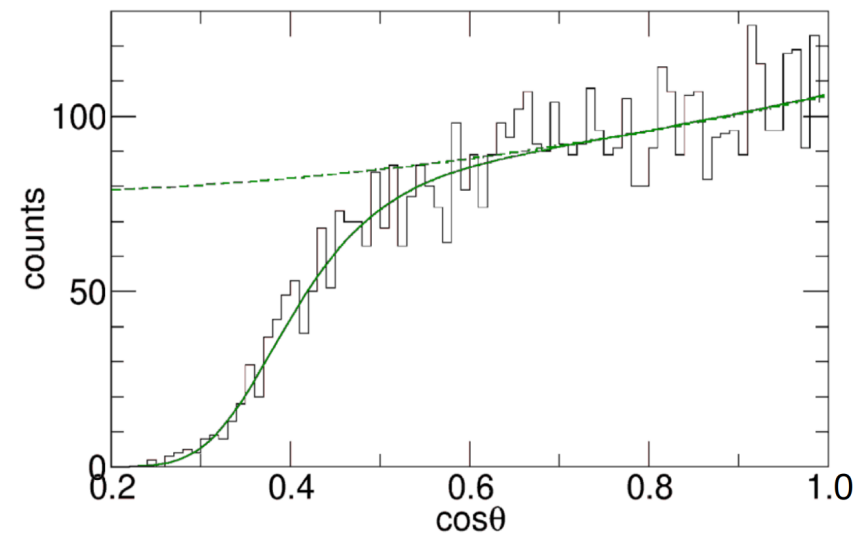
PPAC Efficiency



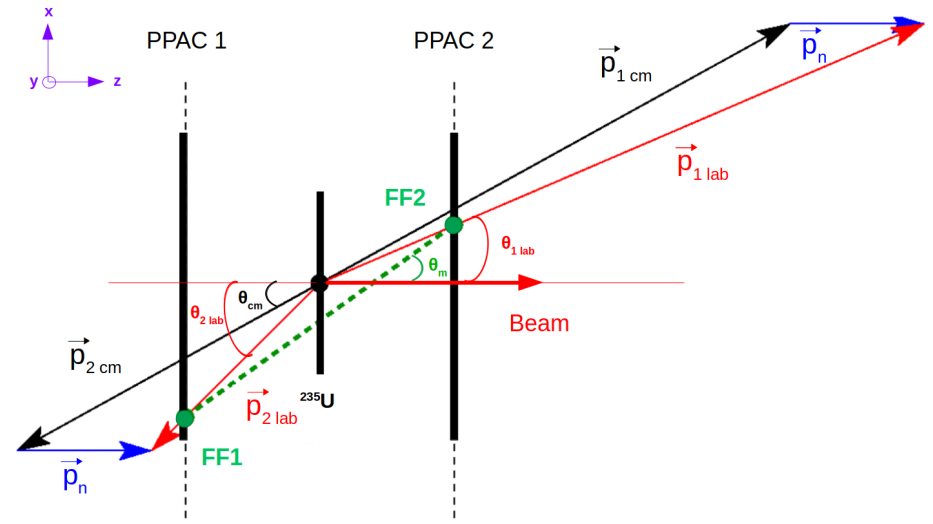
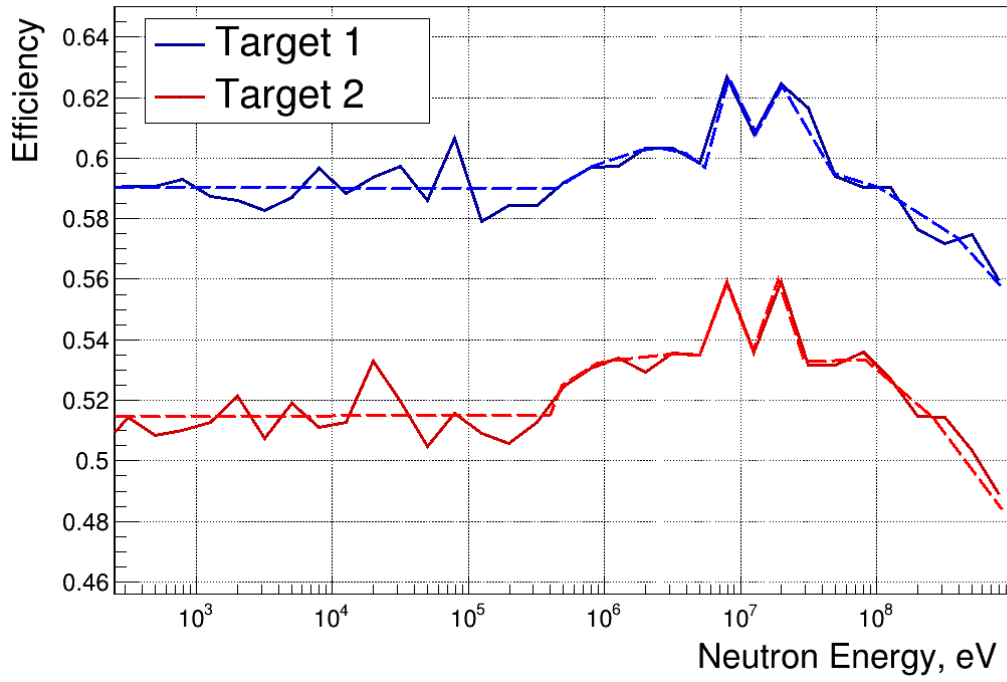
PPAC Efficiency



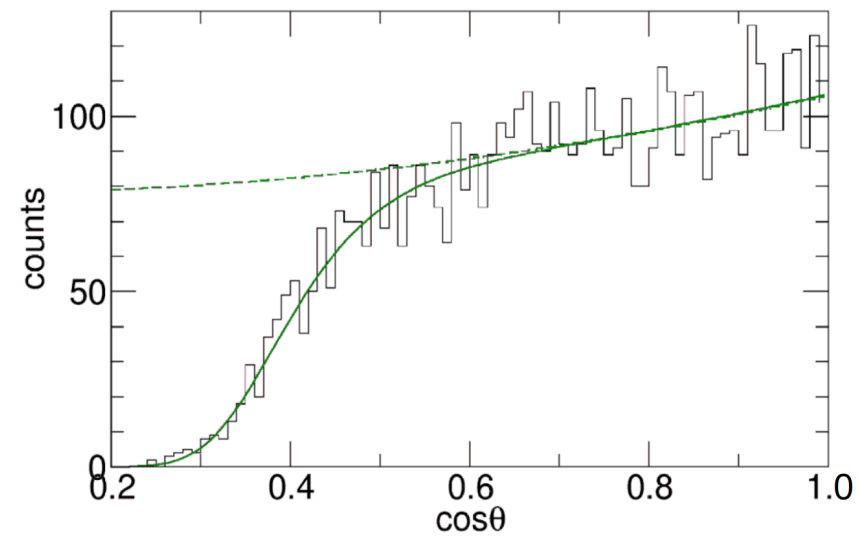
$6.3 \text{ MeV} < E_n < 10 \text{ MeV}$



PPAC Efficiency



6.3 MeV < E_n < 10 MeV



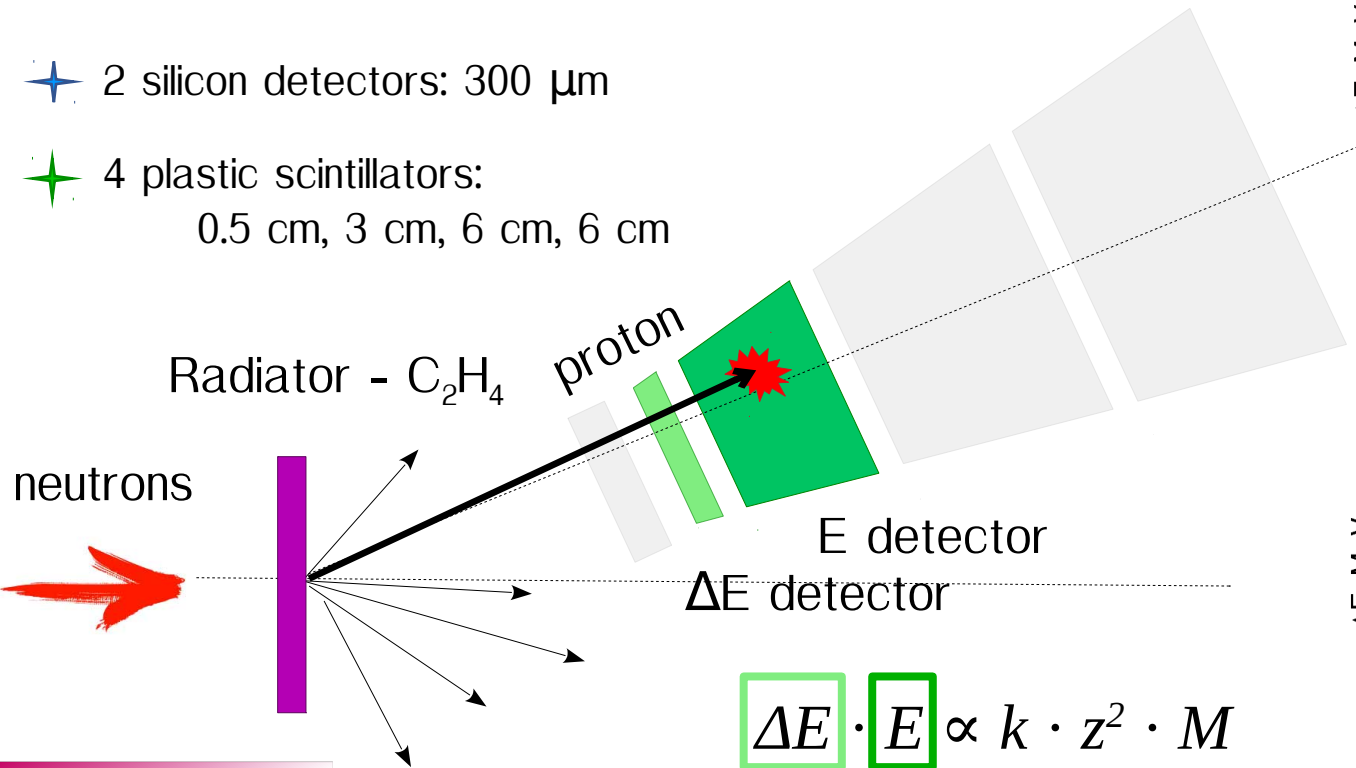
geometrical efficiency geometrical efficiency
 +
 angular distribution
 +
 linear momentum transfer

Recoil Proton Telescope

n-p elastic scattering reaction

★ 2 silicon detectors: 300 μm

✦ 4 plastic scintillators:
0.5 cm, 3 cm, 6 cm, 6 cm

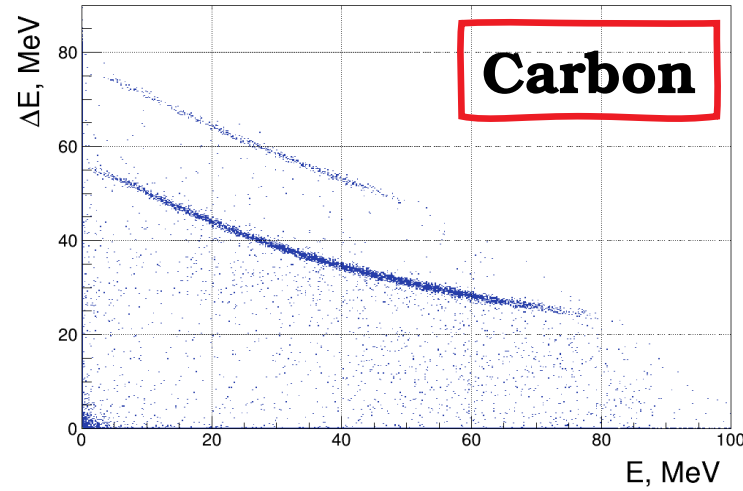
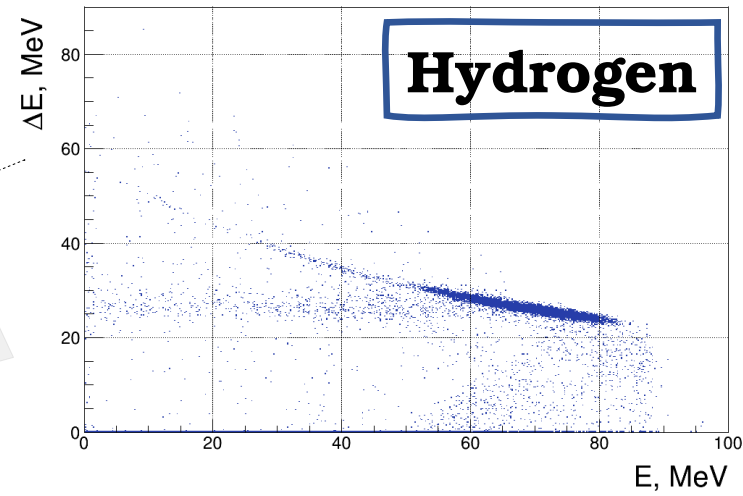


Pyramidal shape

$$\Delta E \cdot E \propto k \cdot z^2 \cdot M$$

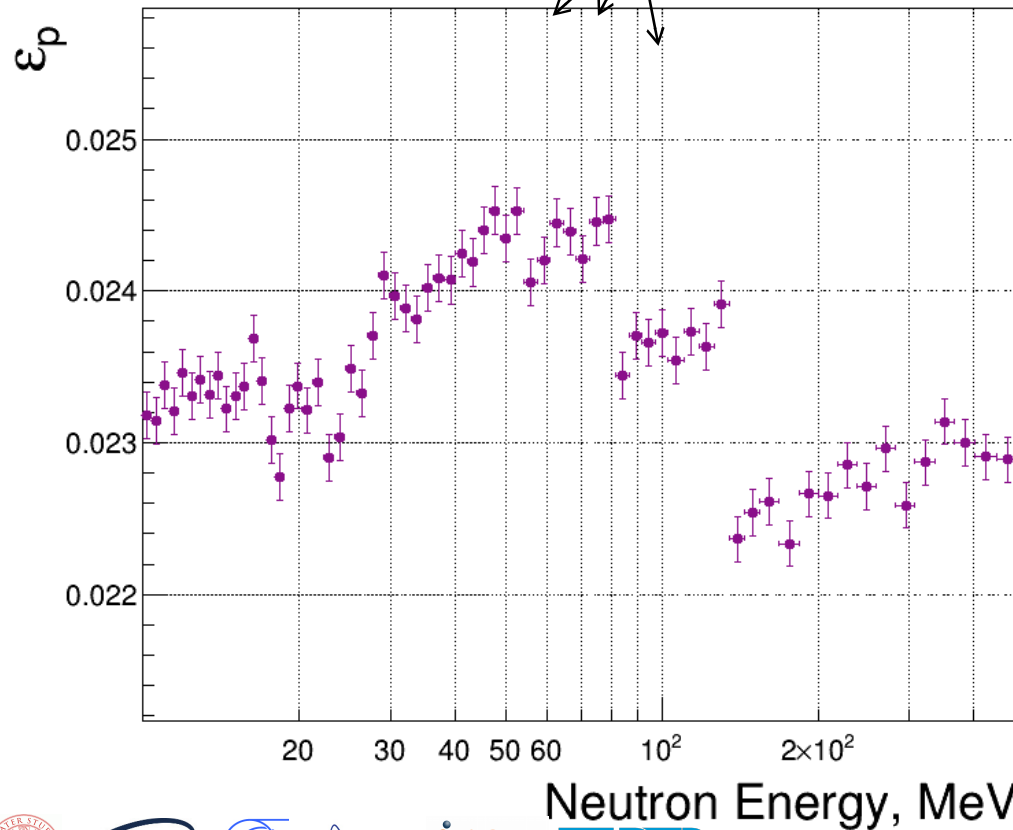
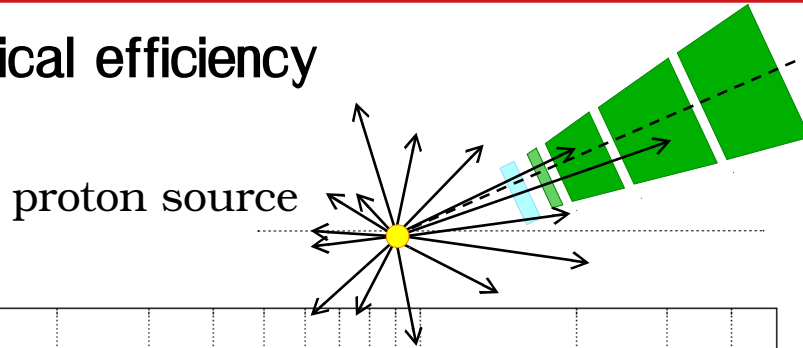
k: material absorption properties
M, *E*, *z*: interacting particle properties

*non-relativistic approximation



Monte Carlo simulation - RPT

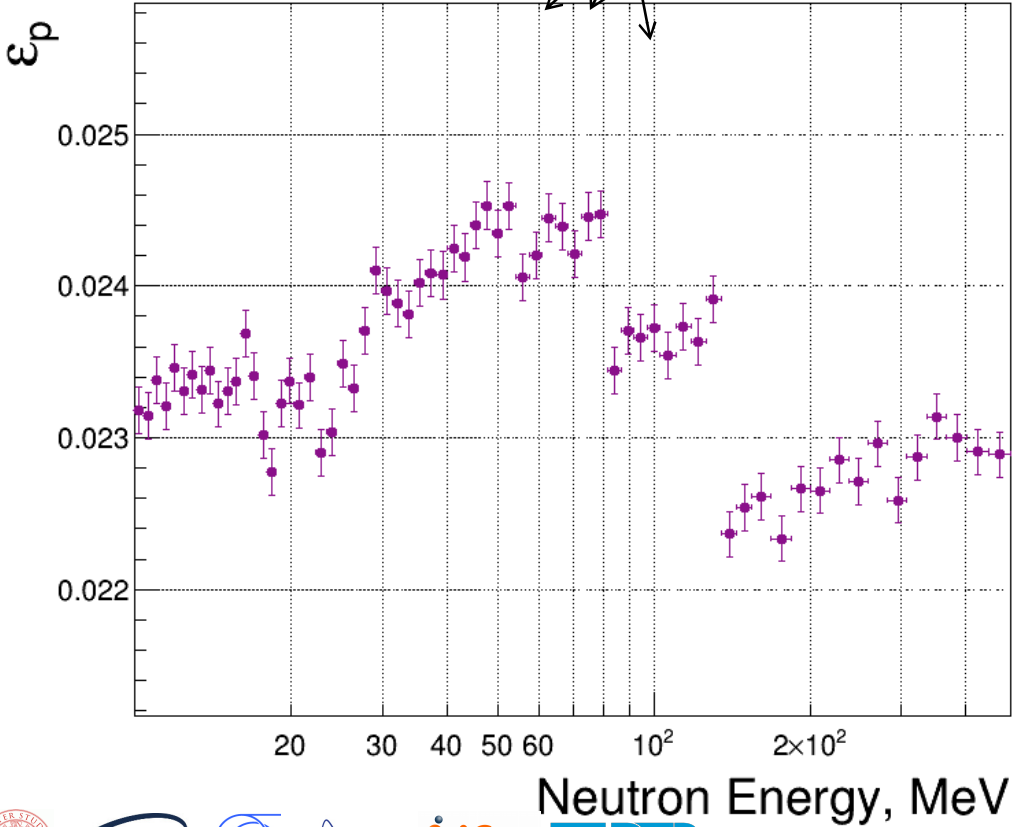
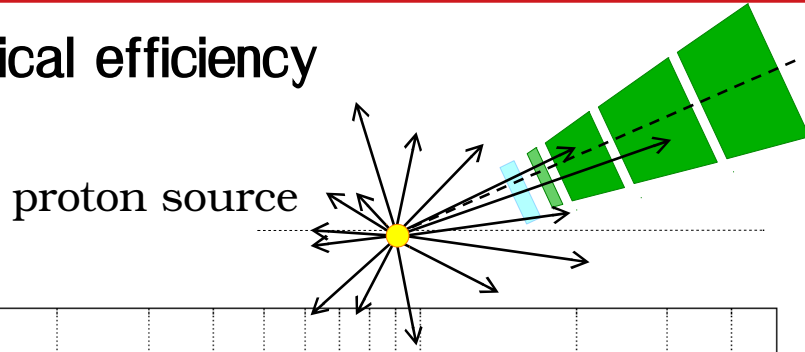
Geometrical efficiency



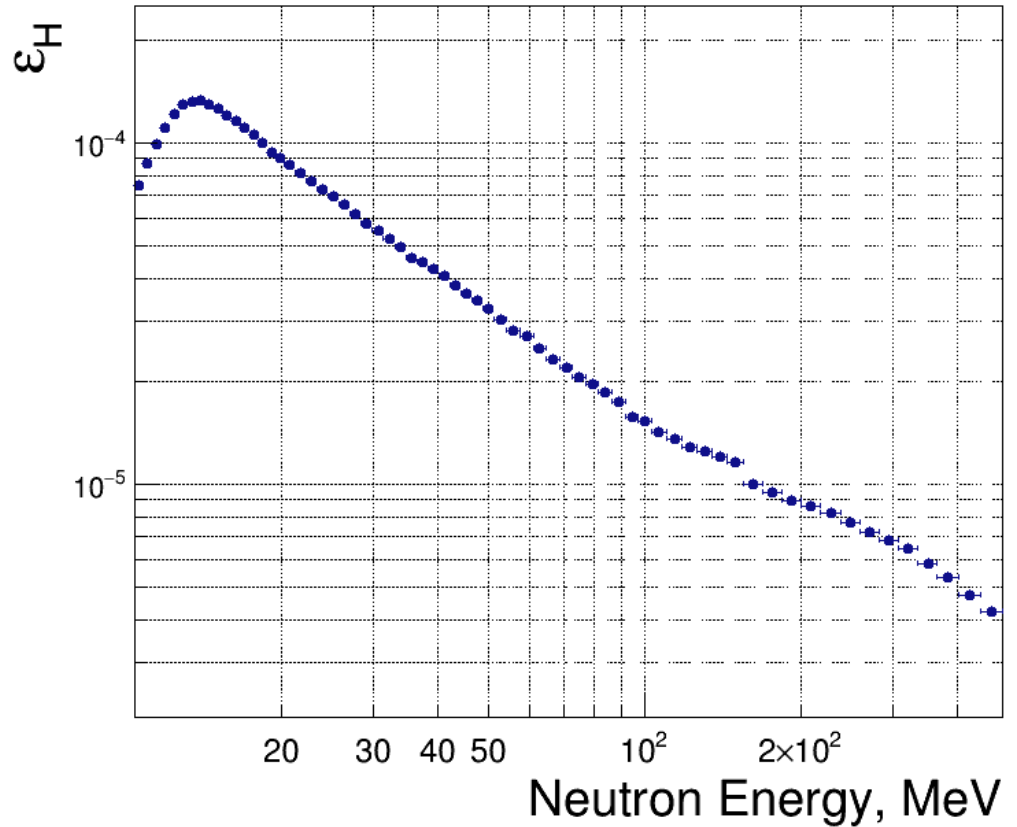
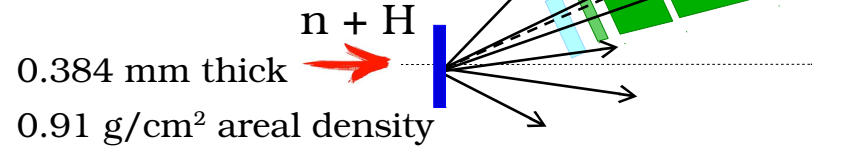
Monte Carlo simulation - RPT

Geant4 and MCNP

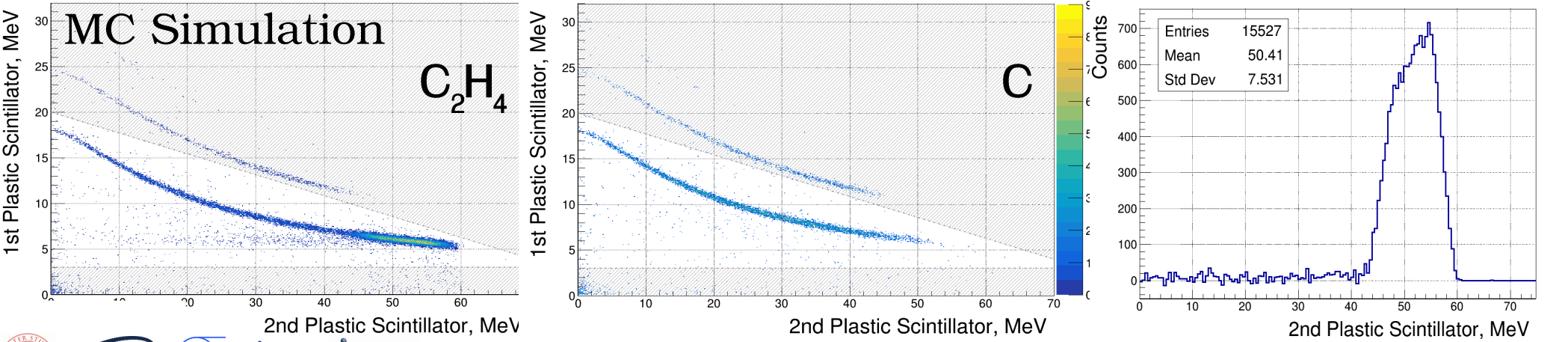
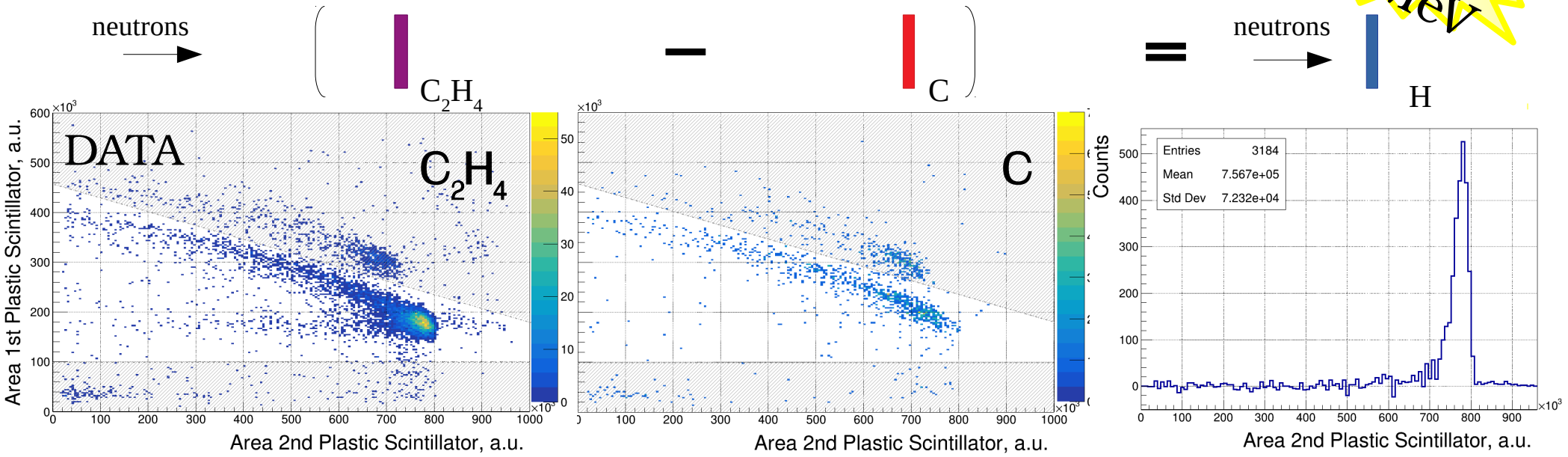
Geometrical efficiency



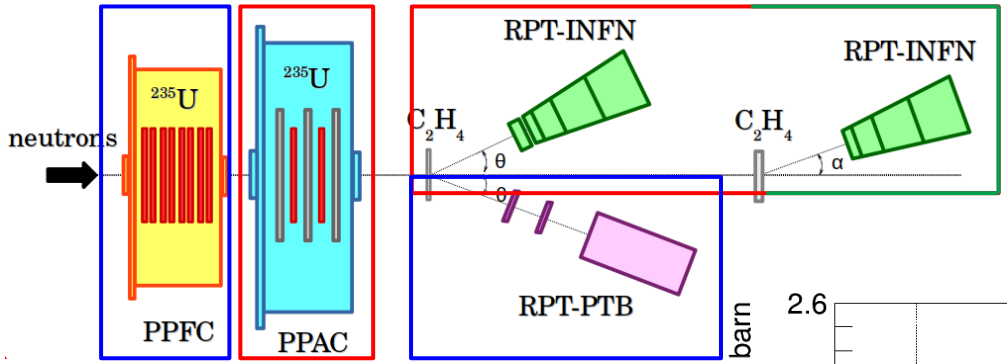
... x neutron-proton differential cross section



Neutron flux extraction



$^{235}\text{U}(n,f)$ cross section

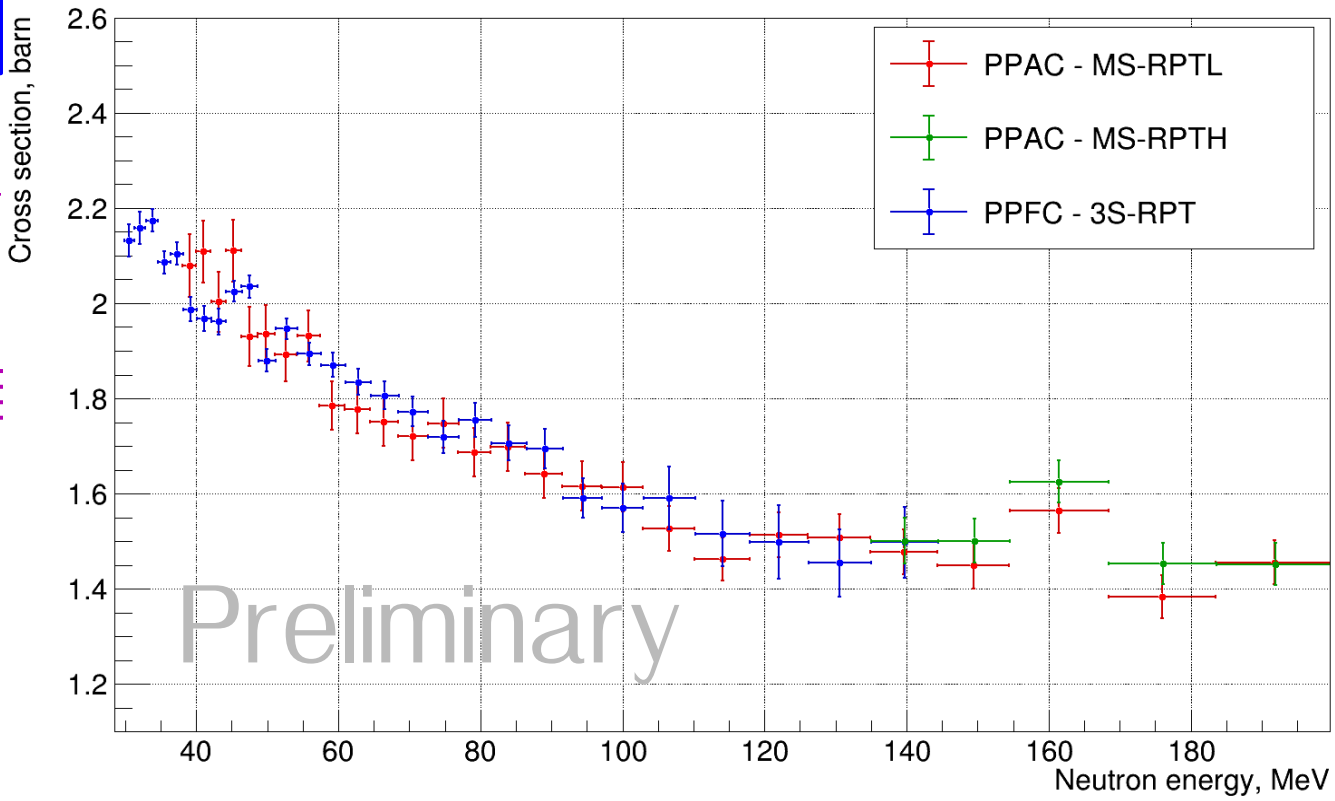


Arndt, VL40 ←

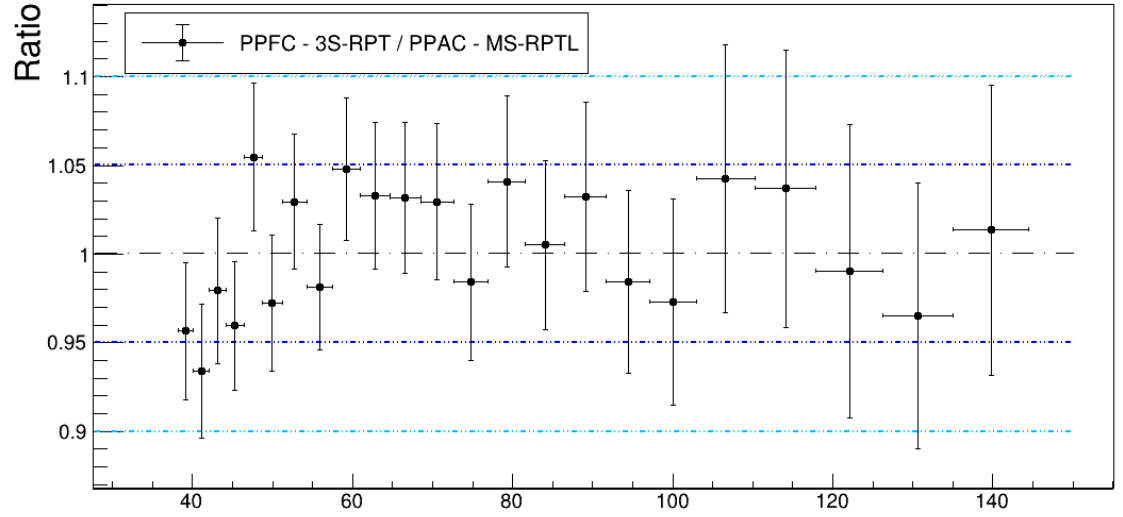
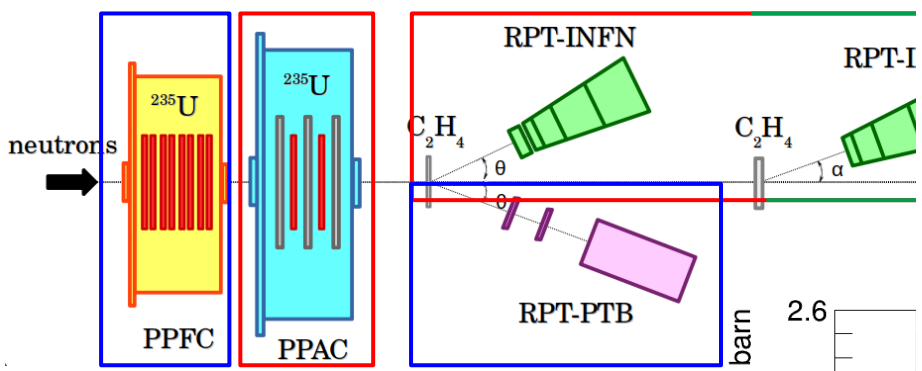
$$\Phi(E_n) = \frac{C_{C_2H_4}(E_n) - r_C C_C(E_n)}{n_H \varepsilon(E_n) \cdot d\sigma_{n,p}(E_n)/d\Omega}$$

→

$$\sigma_f(E_n) = \frac{C(E_n)}{N \Phi(E_n)}$$



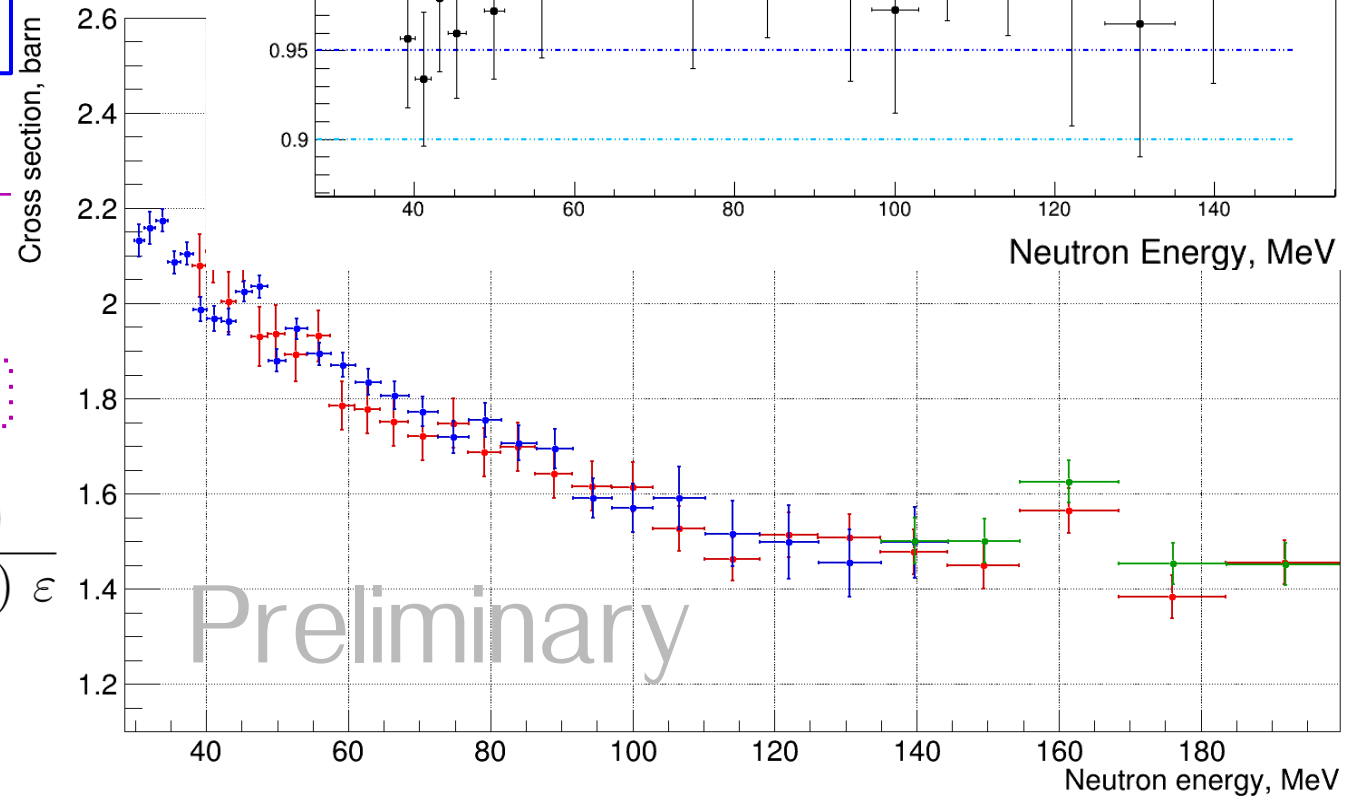
$^{235}\text{U}(n,f)$ cross section



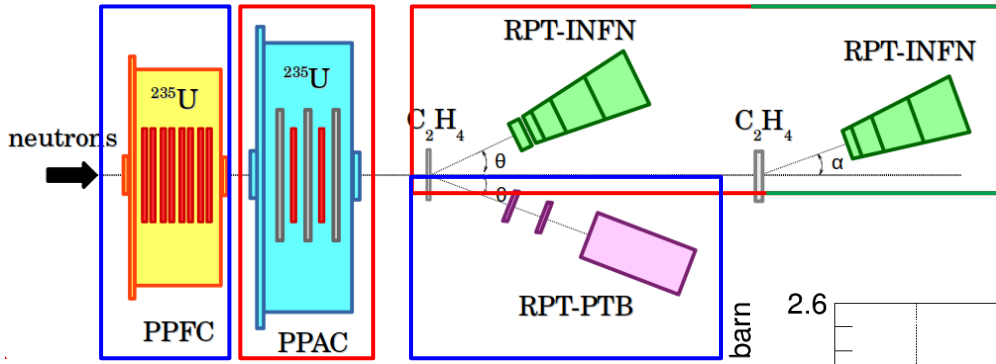
Arndt, VL40 ←

$$\Phi(E_n) = \frac{C_{\text{C}_2\text{H}_4}(E_n) - r_C C_C(E_n)}{n_H \varepsilon(E_n) \cdot d\sigma_{n,p}(E_n)/d\Omega}$$

$$\sigma_f(E_n) = \frac{C(E_n)}{N \Phi(E_n) \varepsilon}$$



$^{235}\text{U}(n,f)$ cross section

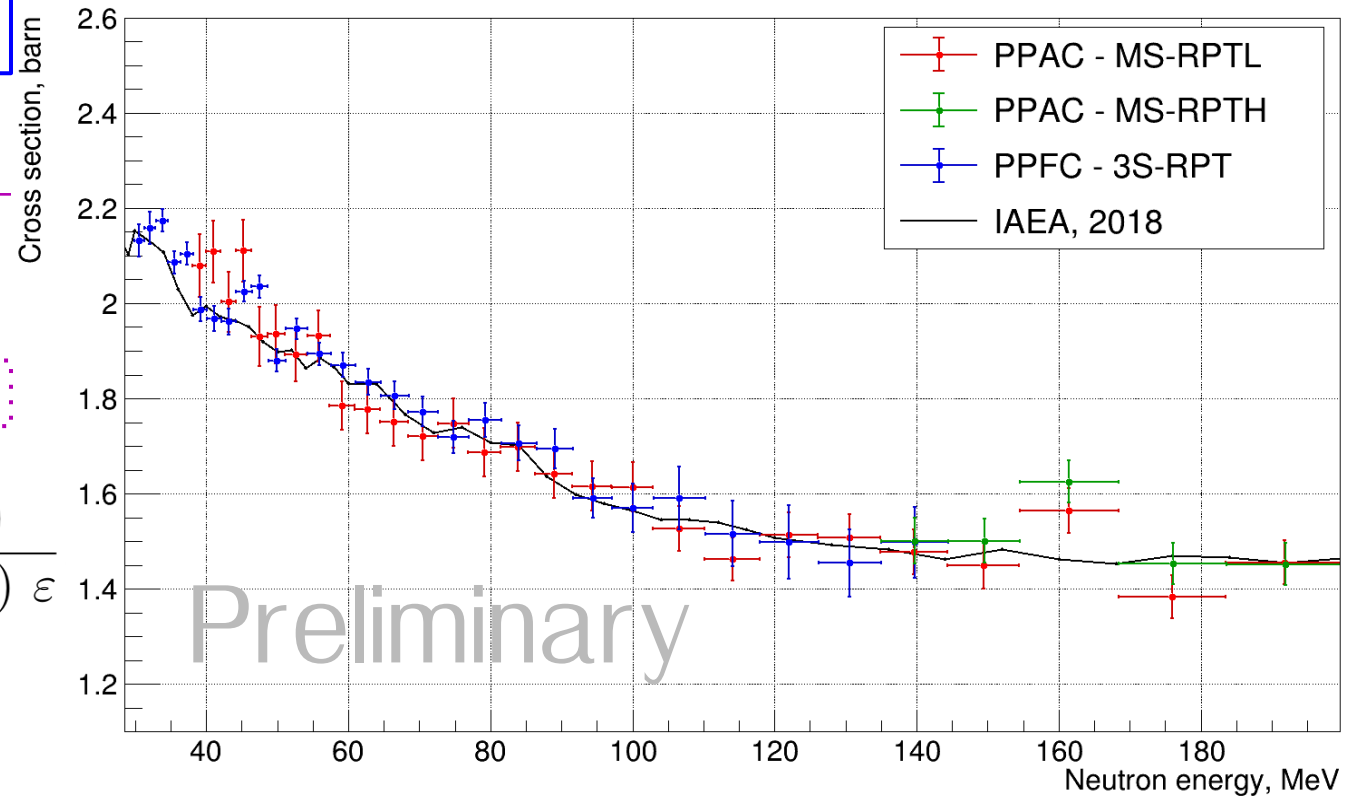


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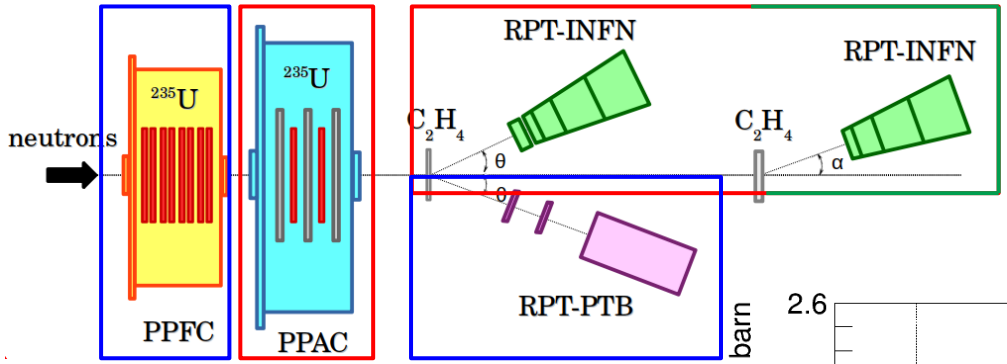


$$\sigma_f(E_n) = \frac{C(E_n)}{N \Phi(E_n) \varepsilon}$$



Preliminary

$^{235}\text{U}(n,f)$ cross section

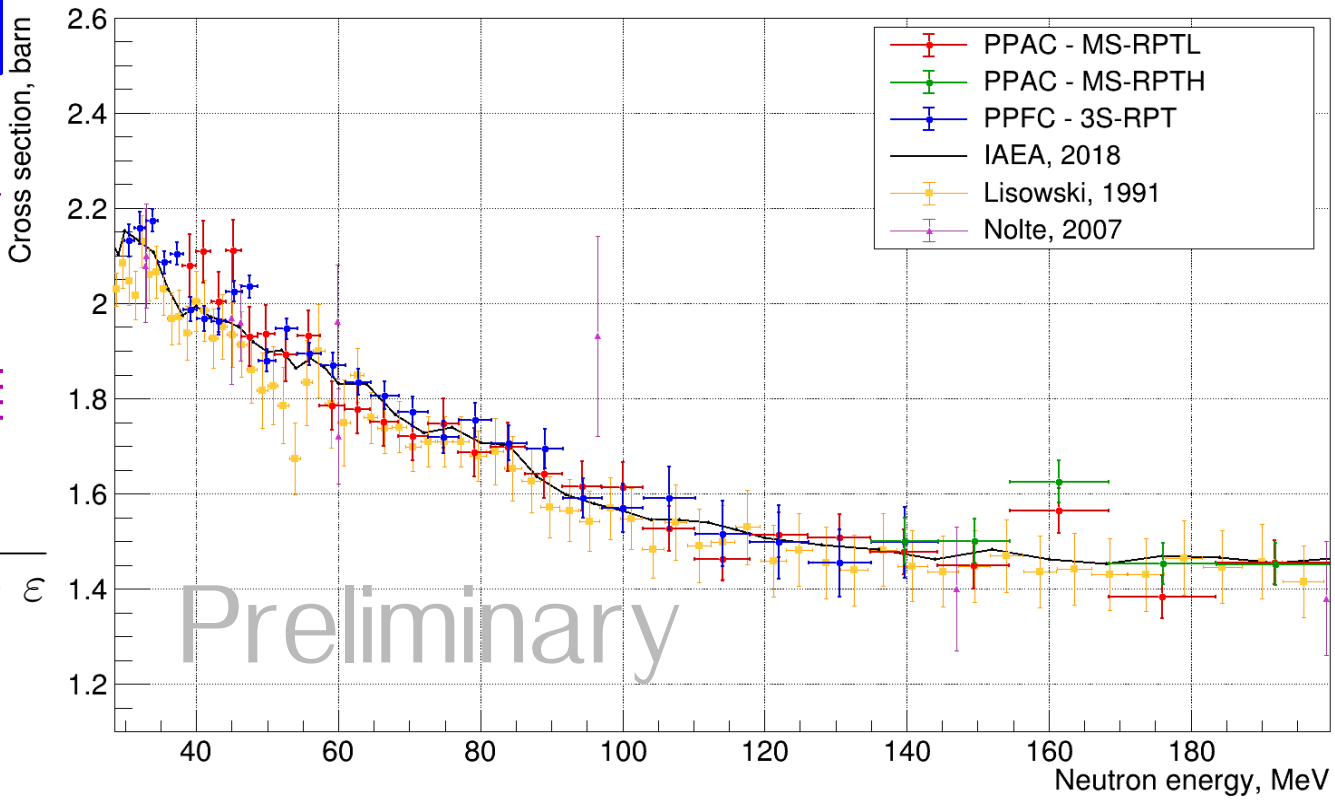


Arndt, VL40 ←

$$\Phi(E_n) = \frac{C_{\text{C}_2\text{H}_4}(E_n) - r_C C_C(E_n)}{n_H \varepsilon(E_n) \cdot d\sigma_{n,p}(E_n)/d\Omega}$$



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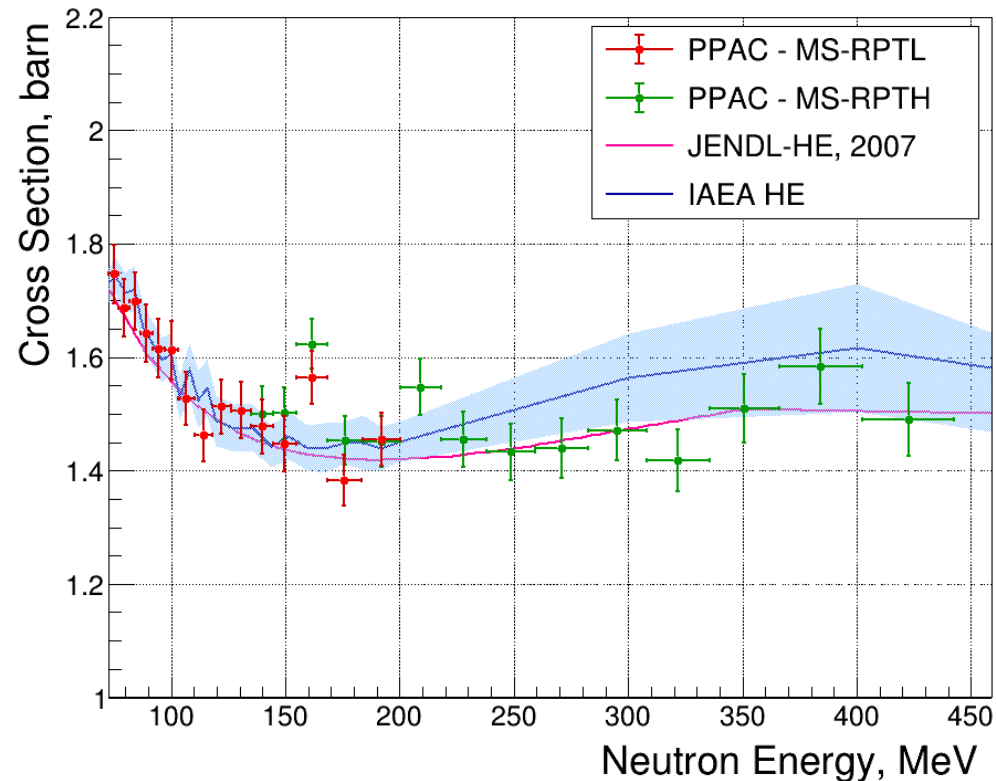


Preliminary

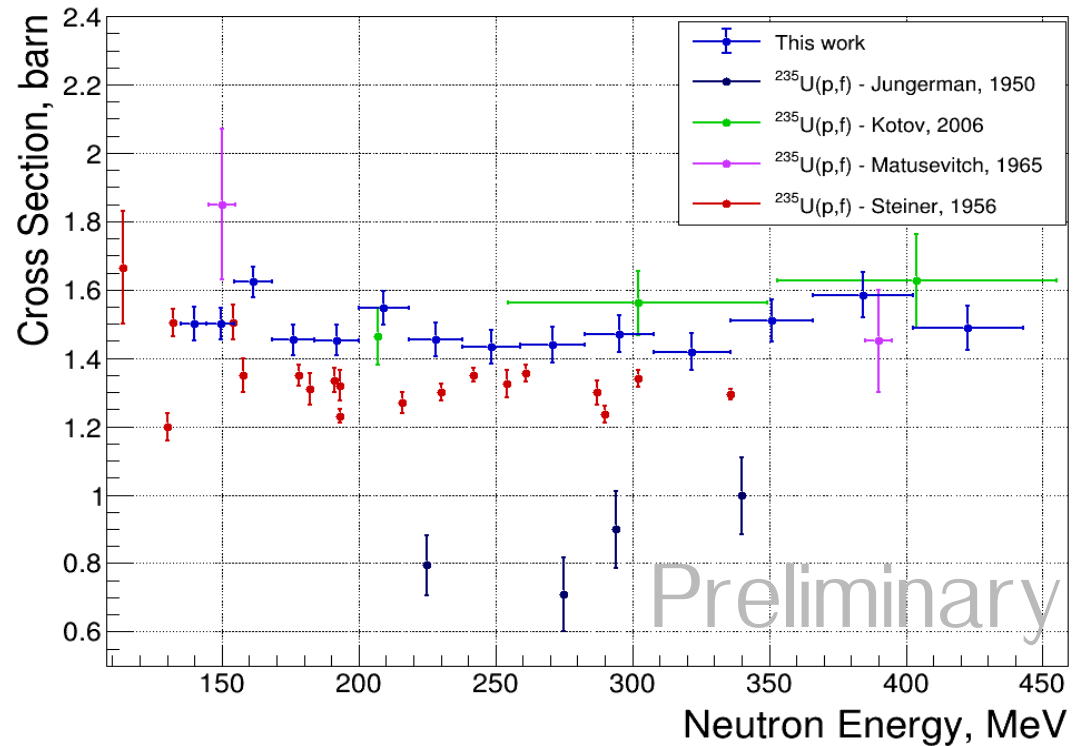
$^{235}\text{U}(n,f)$ cross section

First $^{235}\text{U}(n,f)$ cross section above 200 MeV

Comparison with models

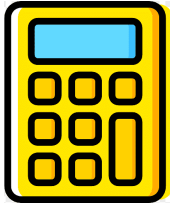


Comparison with $^{235}\text{U}(p,f)$ data



Preliminary

Uncertainties



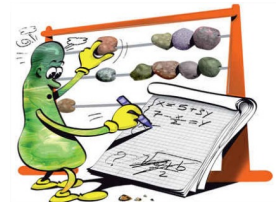
The energy range studied by the RTPL-INFN into three regions
 ↳ different detectors used or different working conditions

	Uncertainty En = [10-30 MeV]	Uncertainty En = [38-200 MeV]	Uncertainty En > 200 MeV
Systematics	6.1%	2.8%	2.8%
Statistics	1.7-3.8%	1.0-2.4%	1.8-4.5%
Systematics	2.5%	2.5%	2.2%
Statistics	2.5-4.2%	2.5-4.2%	2.0-3.1%
Systematics	6.6%	3.8%	3.7%
Statistics	3.0-5.7%	2.7-4.8%	2.7-5.5%

...for neutron flux measurement
RPTs-INFN related

...for FF events
PPAC related

Total



Uncertainties

Source of uncertainty	Uncertainty $E_n = [10-30]$ MeV	Uncertainty $E_n = [38-200]$ MeV	Uncertainty $E_n > 200$ MeV
C ₂ H ₄ mass	0.4%	0.2-0.5%	0.2-0.5%
C mass	0.9%	0.5-0.6%	0.5-0.6%
Signal Reconstruction	1.8%	0.5%	0.7%
Dead time correction	2.0%	1.0%	1.0%
Cuts in the ΔE -E matrix	5.0%	2.0%	2.0%
Telescope angle	0.6%	0.9%	1.0%
Telescope position	0.7%	0.7%	0.7%
Beam transmission	0.8%	0.8%	0.8%
Beam profile	0.5%	0.5%	0.5%
Counting statistics	1.7-3.8%	1.0-2.4%	2.8-4.5%
Total	6.2 - 7.1%	2.9 - 3.7%	4.4 - 5.4%

...for neutron flux measurement
RPTs-INFN related

...for FF events
PPAC related

Source of uncertainty	Uncertainty $E_n < 200$ MeV	Uncertainty $E_n > 200$ MeV
Sample mass	1.0%	1.0%
Trajectories reconstruction	0.4%	0.4%
Efficiency calculation fit	2.0%	2.0%
Anisotropy correction	1.2%	-
Counting statistics	2.5 - 4.2%	2.0 - 3.1%
Total	3.6 - 4.9%	3.0 - 3.8%



Conclusion

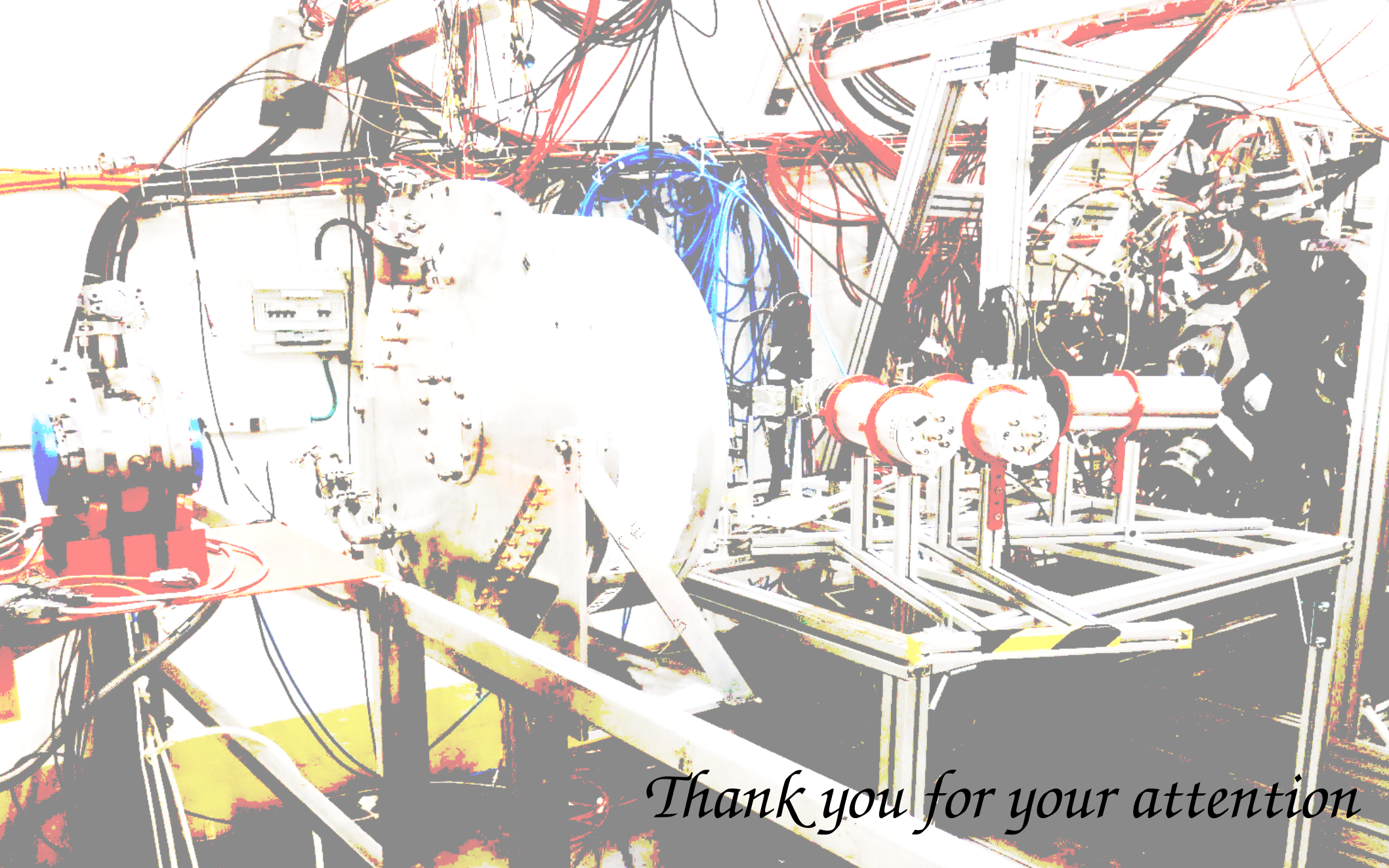
First measurement of neutron-induced fission cross section on ^{235}U in the energy range between 200 and 450 MeV

First measurement of the n_TOF flux in the energy range between 20 and 450 MeV

reliability of results by agreement with previous measurements
at lower energies (20-200 MeV)

...and perspectives

-  Further campaign with ^{235}U with the aim to:
 - improve the accuracy (increase the statistics)
 - reach a higher neutron-energy value (FFs already detected up to 1 GeV)
 - measure simultaneously cross section and FFs angular distribution
-  Extend the study to other isotopes



Thank you for your attention

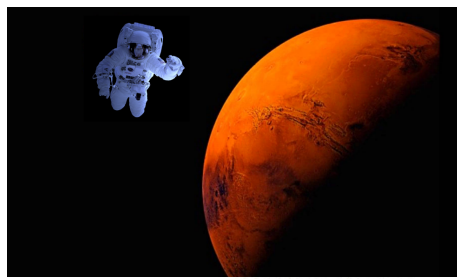
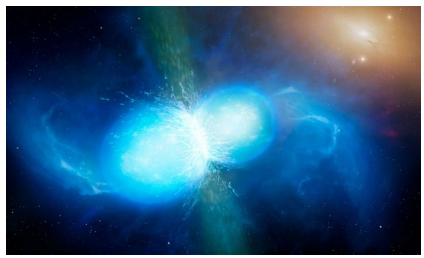
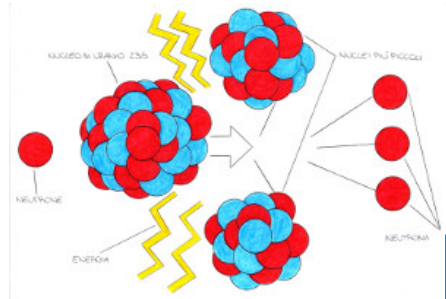
Motivations



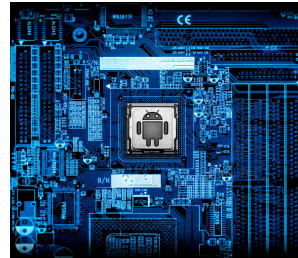
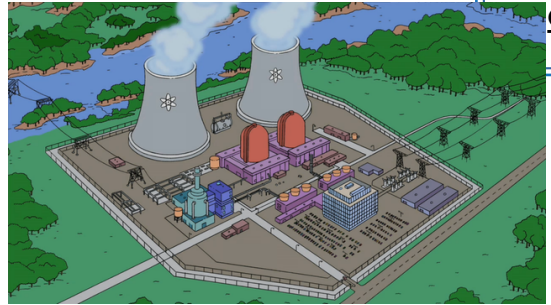
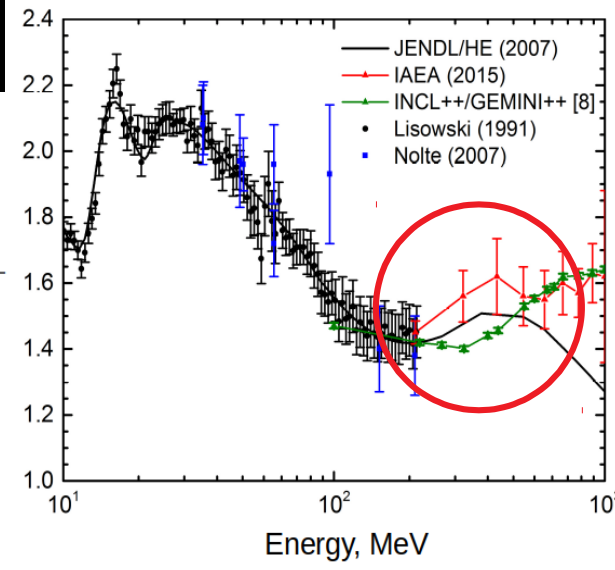
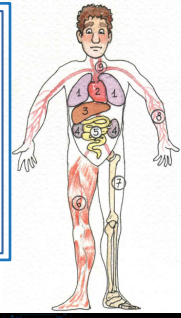
INDC International Nuclear Data Committee

“...Our analysis indicates that the new absolute measurements of the neutron induced fission cross section (e.g. relative to n-p scattering) on Uranium, Bismuth, Lead and Plutonium have the highest priority in establishing neutron induced fission reaction standard above 200 MeV...”

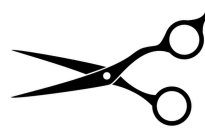
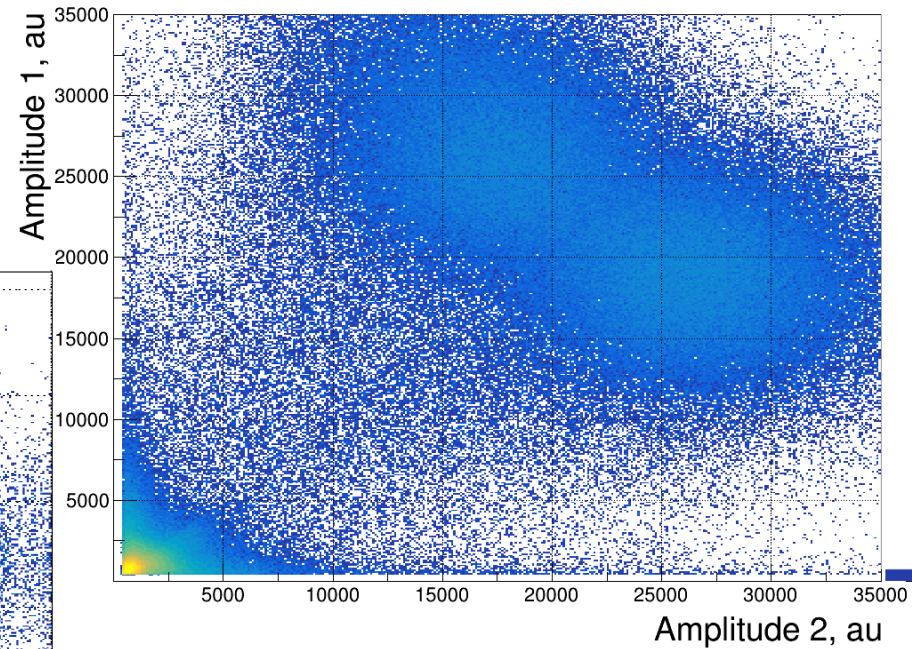
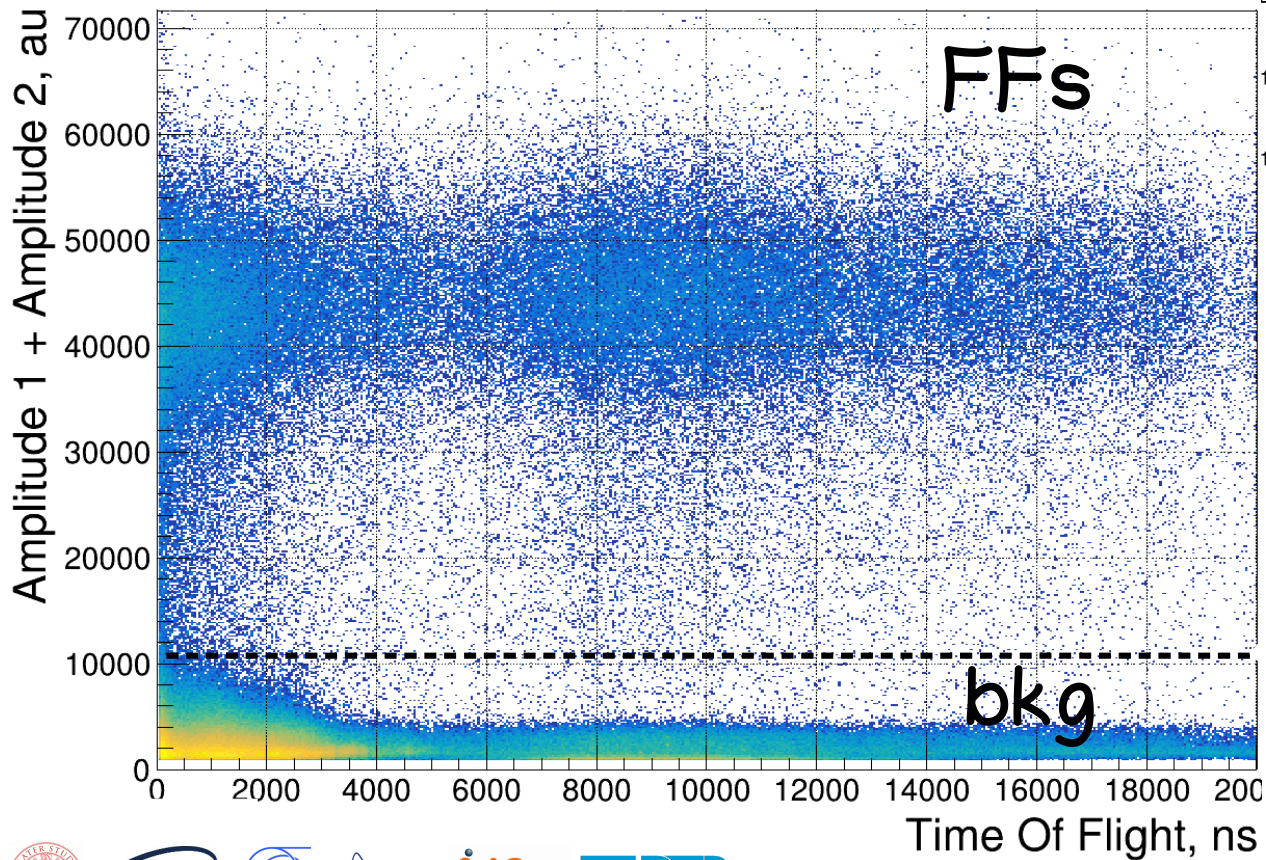
(INDC(NDS)-0681 Distr. ST/J/G/NM, IAEA 2015)



$^{235}\text{U}(n,f)$ is one of the most significant cross-section standards at 0.025 eV and [0.15-200] MeV **BUT** there are **no experimental data above 200 MeV**



Fission events signature



Polyethylene samples

Characterization

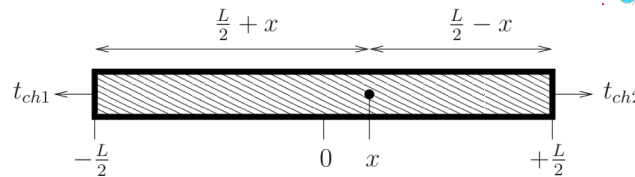
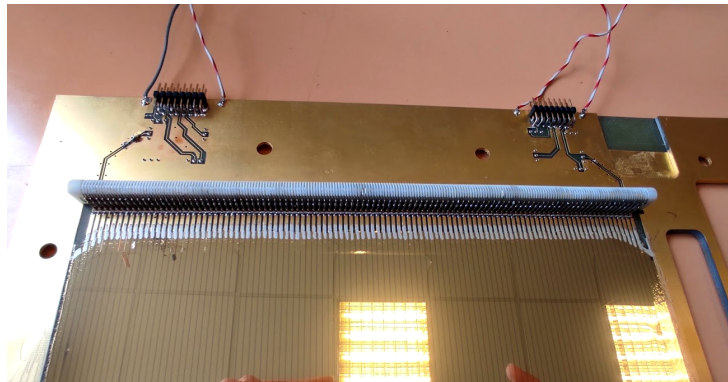
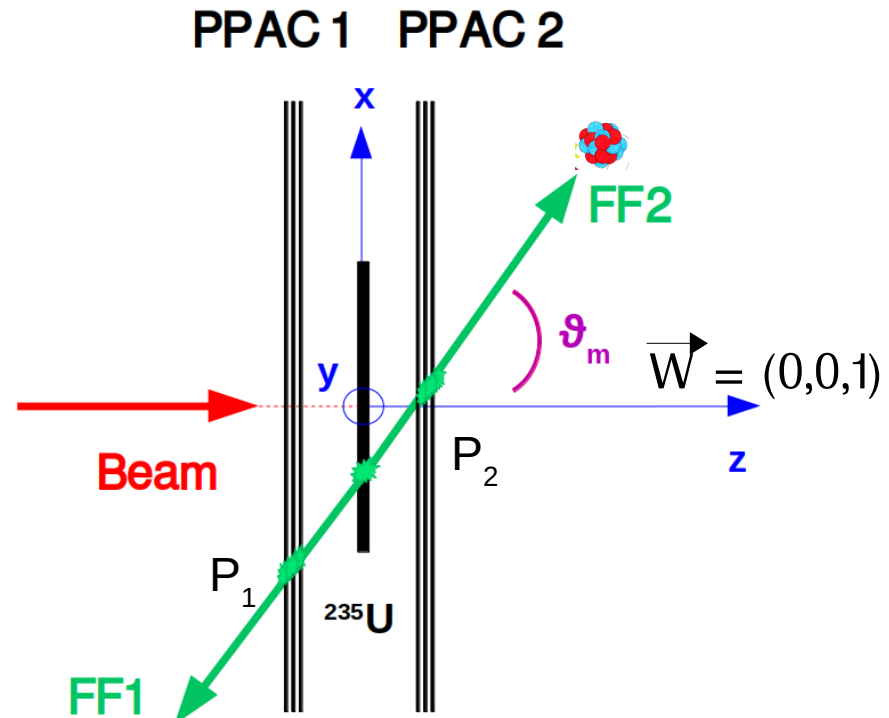
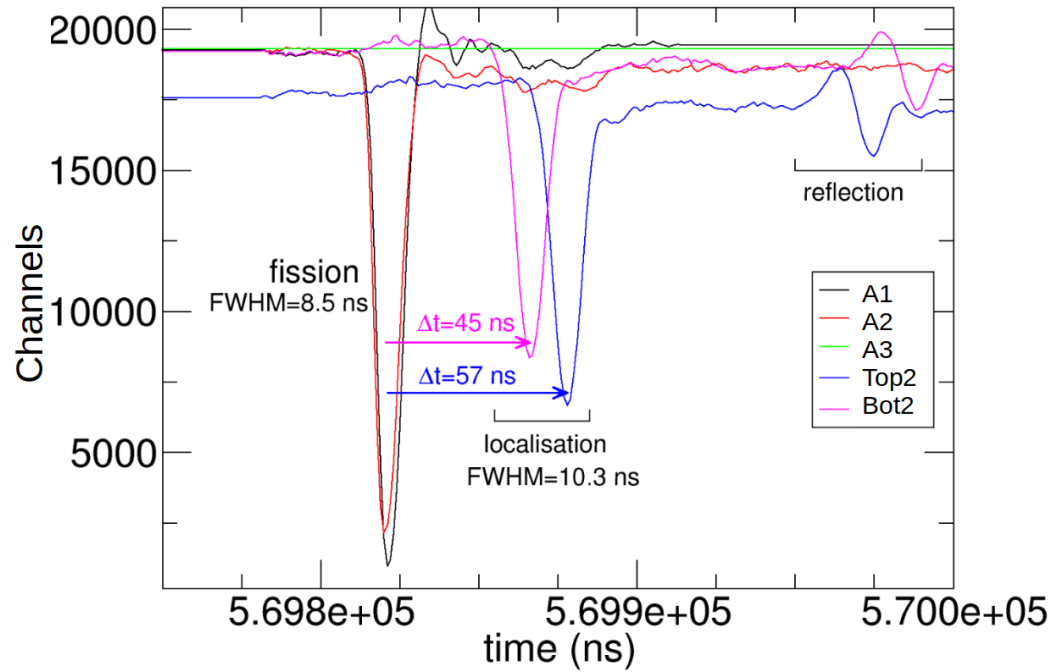
- mass density from hydrostatic weighing (PTB)
- thickness: precision measurement of the profile (PTB)
- uncertainty on the areal density: 0.2-0.6%

- H/C ratio via combustion analysis, two measurements (Forschungszentrum Jülich, TU Braunschweig): 1.98(3) and 2.00(3)
- In the simulations: assumed nominal stichometry H/C=2

Sample	Thickness (mm)	Density g/cm ³	Areal density g/cm ²	(rel. unc.)
PE 1mm	1.025(4)	0.9534(20)	0.0978(4)	(0.4%)
PE 2mm	1.824(11)	0.9555(20)	0.1743(11)	(0.6%)
PE 5mm	4.925(4)	0.9597(20)	0.4726(11)	(0.2%)
C 0.5mm	0.500(4)	1.7749(27)	0.0887(8)	(0.9%)
C 1mm	1.000(5)	1.7364(86)	0.1736(12)	(0.7%)
C 2.5mm	2.500(4)	1.7512(32)	0.4378(11)	(0.3%)



Fission fragment trajectory



$$(t_{ch1} - t_o) + (t_{ch2} - t_o) = DLT$$

$$P_1 = (x_1, y_1, z_1)$$

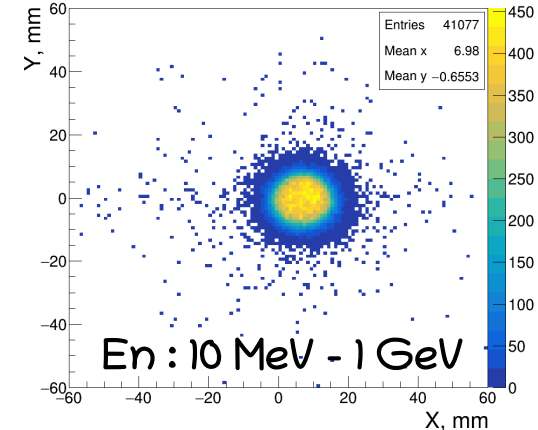
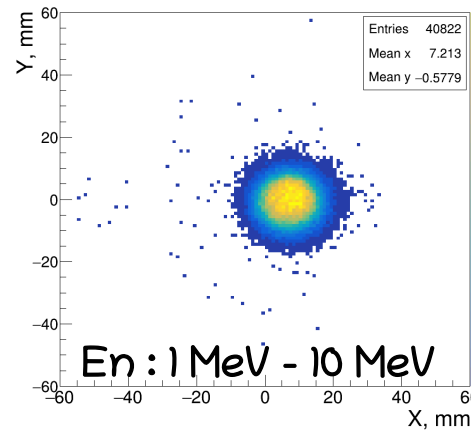
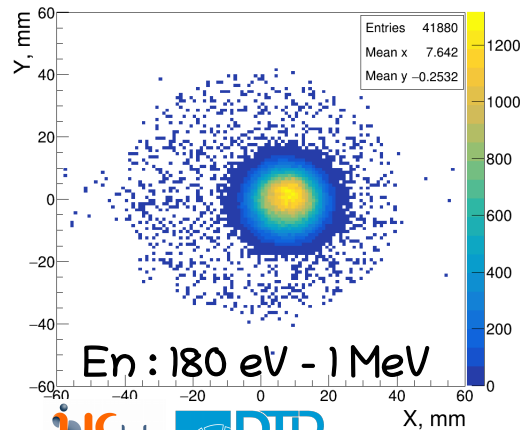
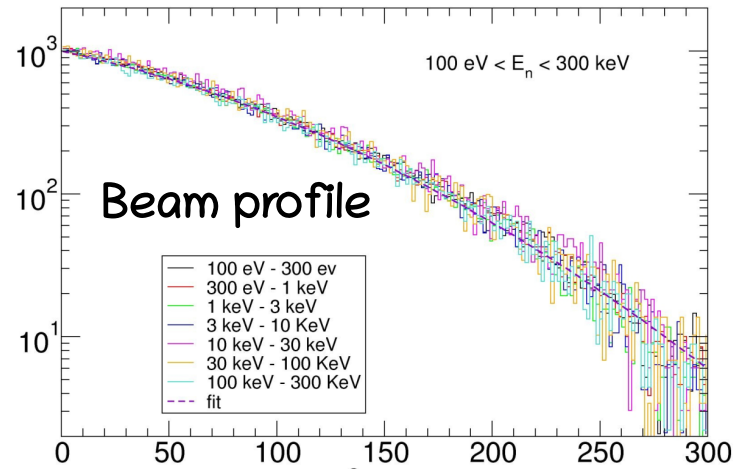
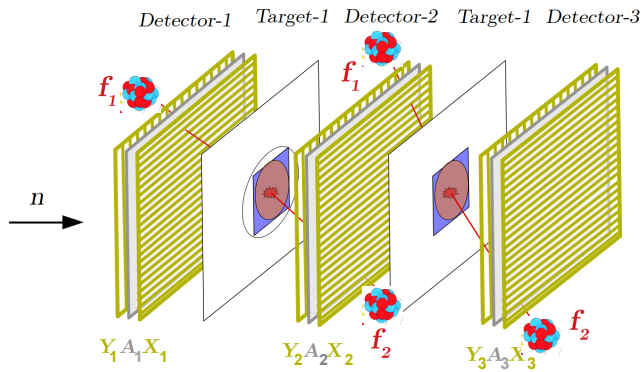
$$P_2 = (x_2, y_2, z_2)$$

$$\cos \vartheta = \frac{\vec{V} \cdot \vec{W}}{|\vec{V}| |\vec{W}|}$$

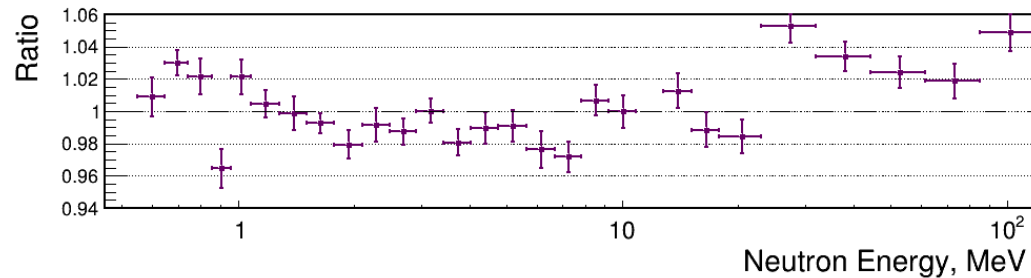
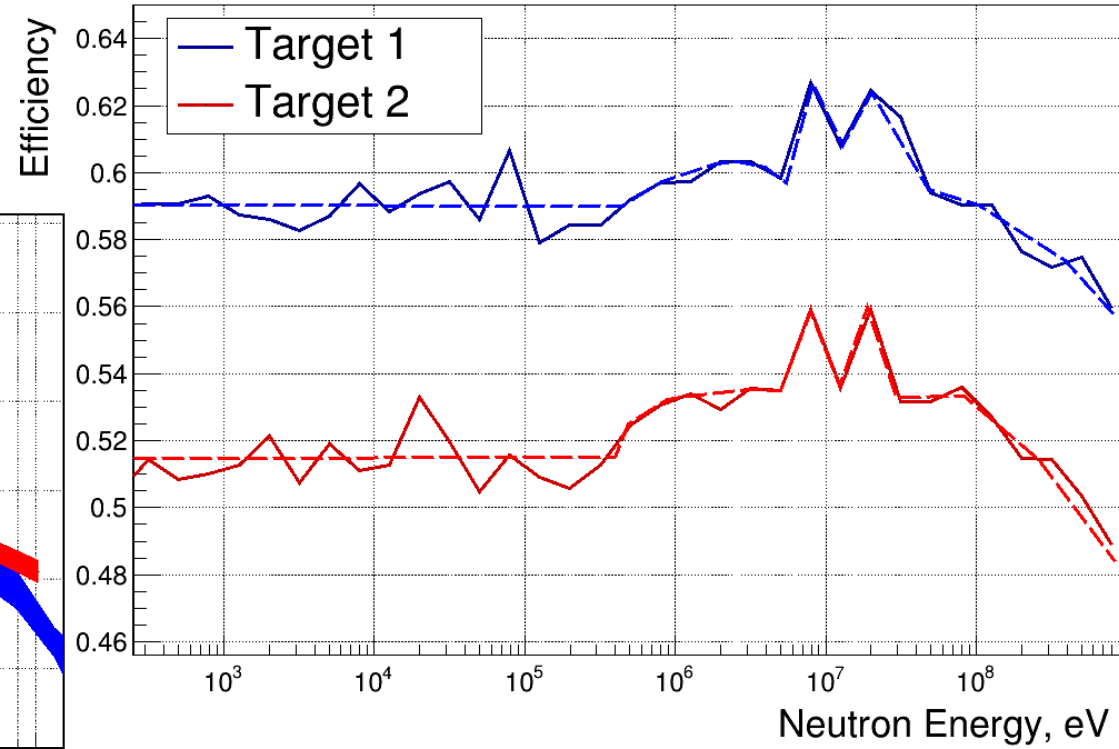
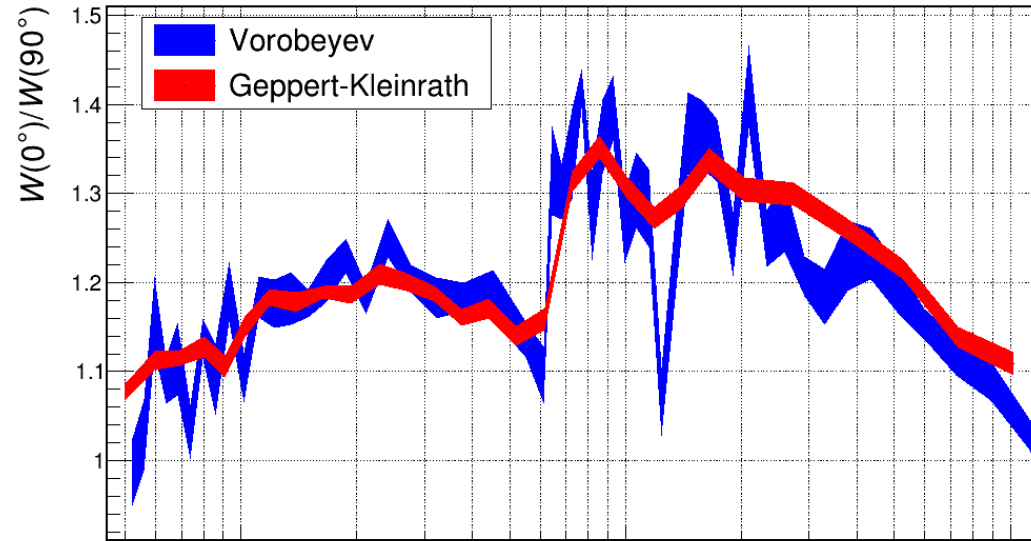
Fission fragment trajectory

The strips, which are read at both ends of the delay line, allow a localisation of the FFs impact positions

→ the reproduction of the hitting point of the neutrons in the sample



PPAC Efficiency



^{235}U Sample characterisation

Coulomb scattering

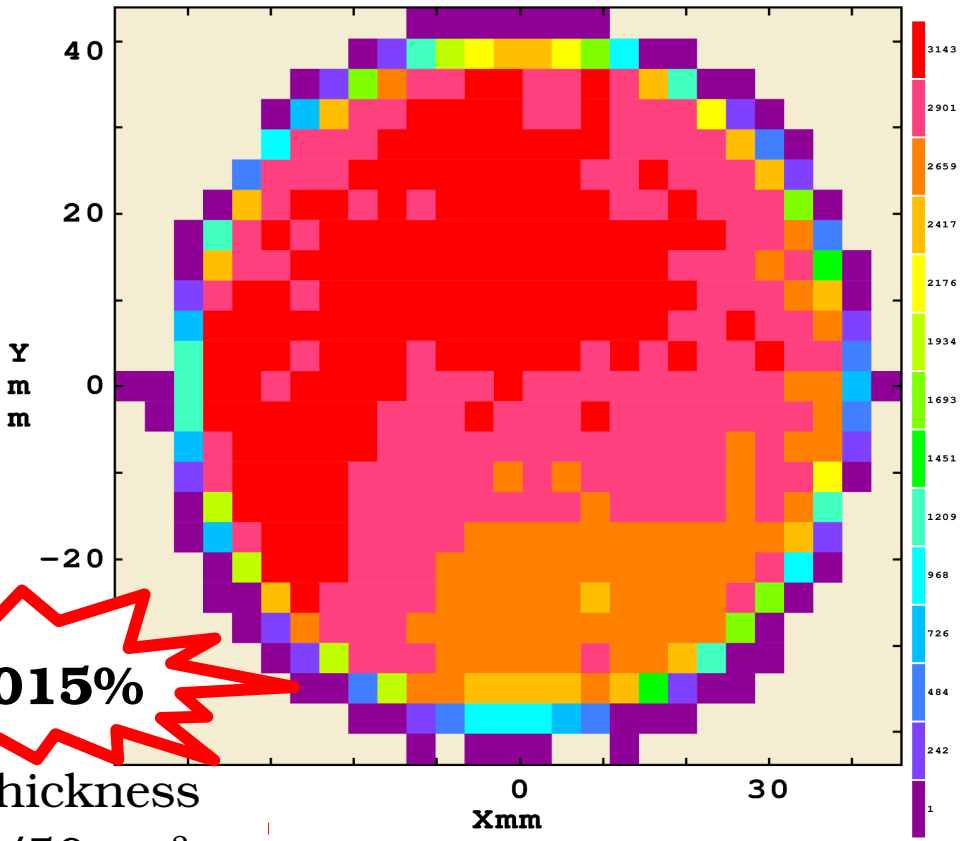
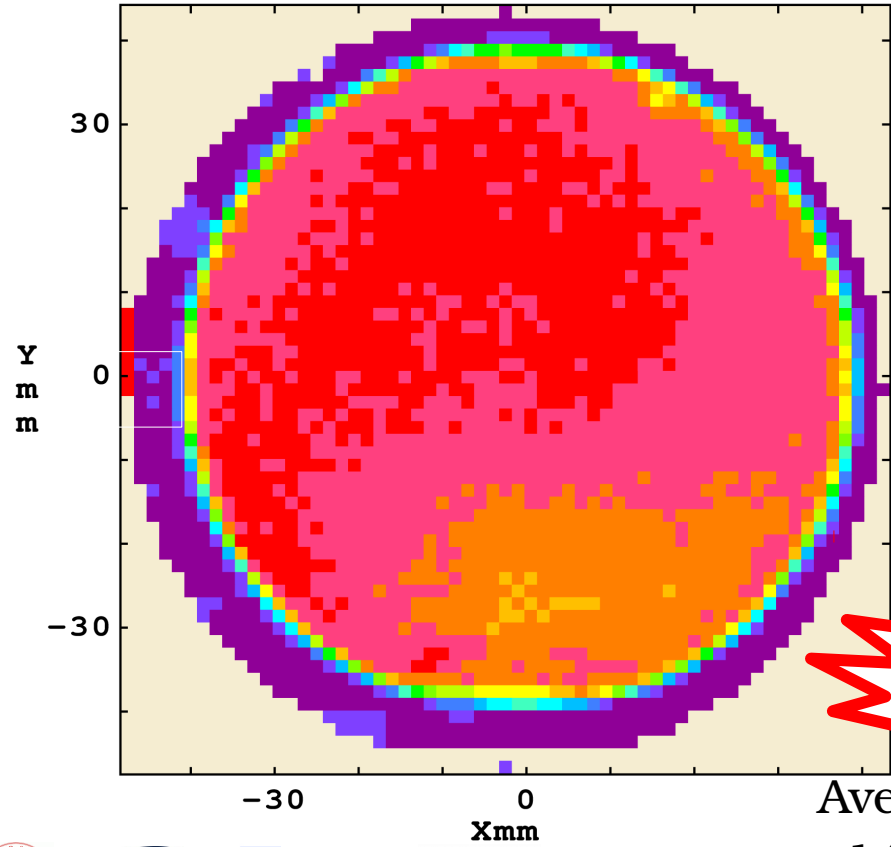
α activity \rightarrow ^{234}U

Proton beam

$^{234}\text{U}/^{235}\text{U} = 0.007472$

Spatial Resolution = 1.5 mm

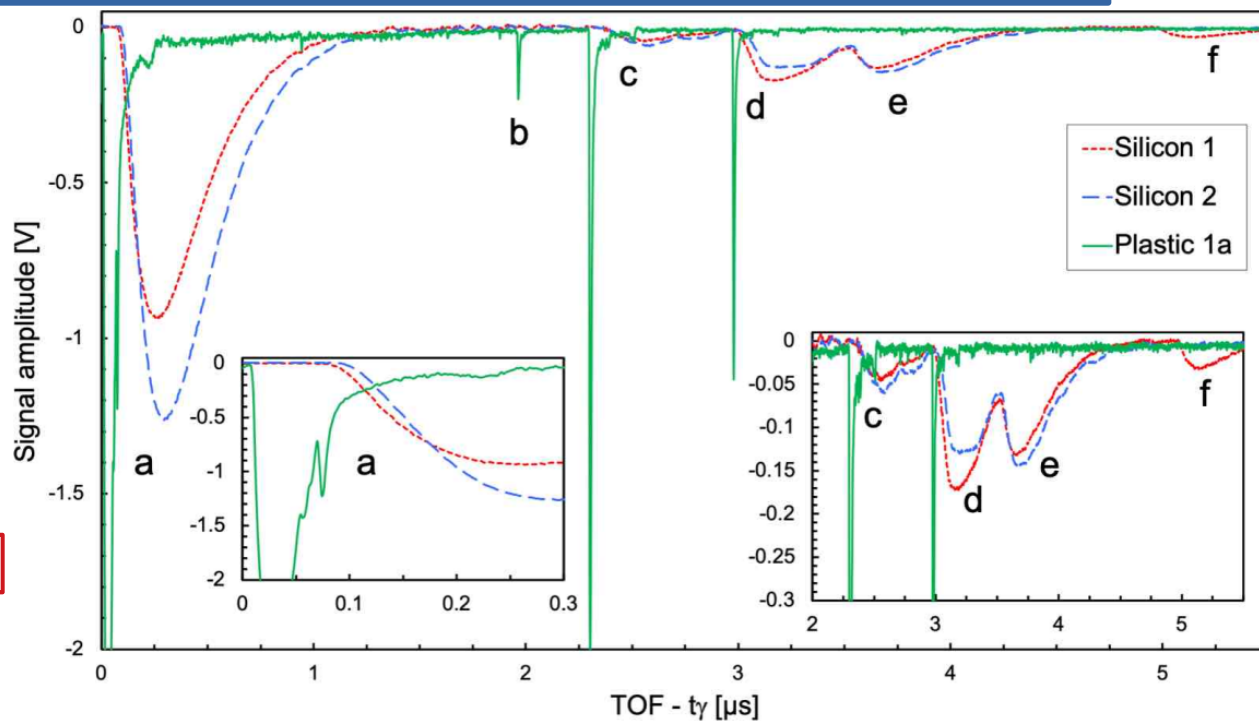
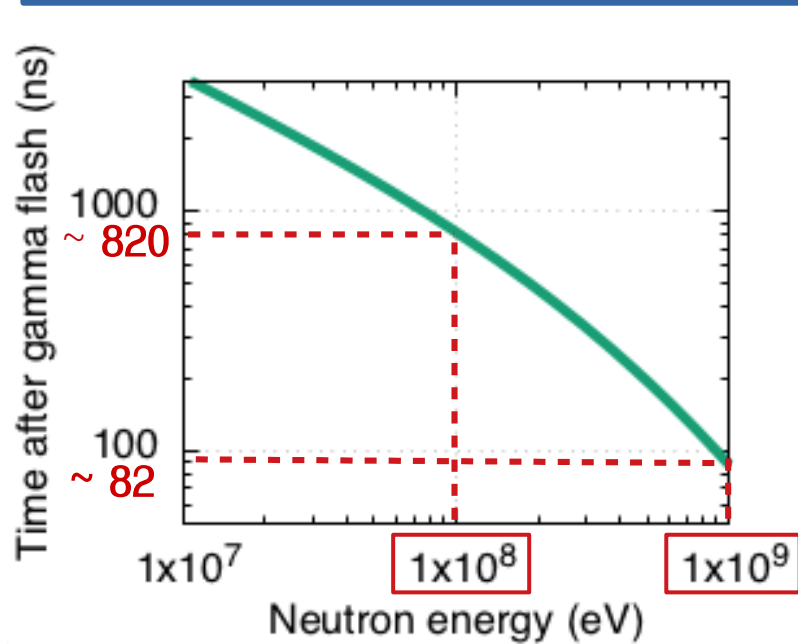
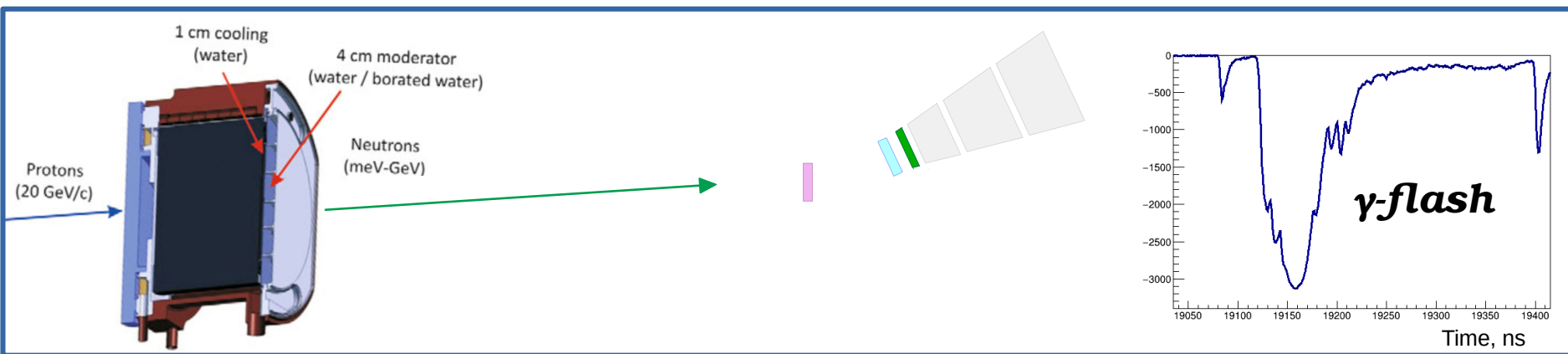
Spatial Resolution = 3.5 mm



~ +1.015%

Average thickness
14.0 mg/50 cm²

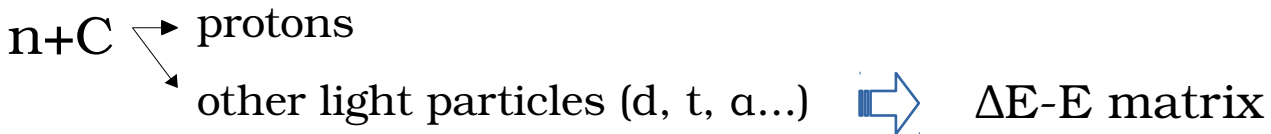
Proton Recoil Telescope



Monte Carlo simulation - RPT

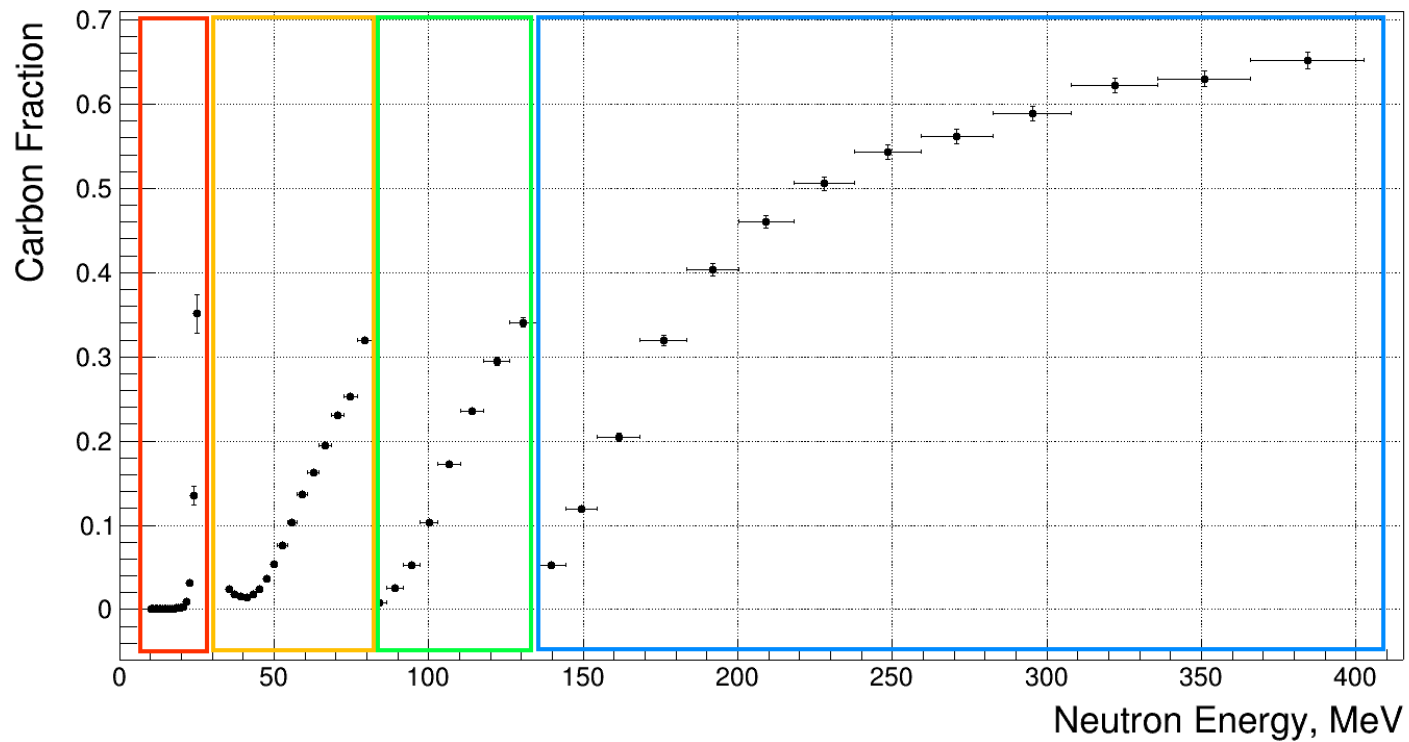
...but the sample was C_2H_4

Study of the background components:

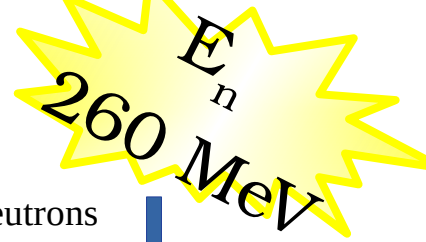


$$\frac{\text{protons from } n + C}{\text{total protons from the target}}$$

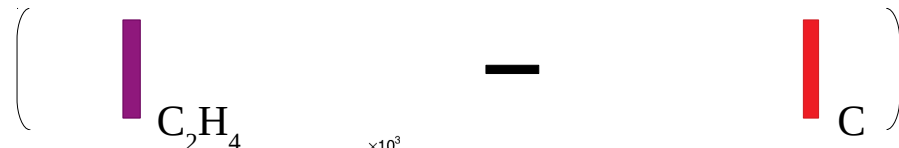
Segmentation of the proton recoil telescope



Neutron flux extraction

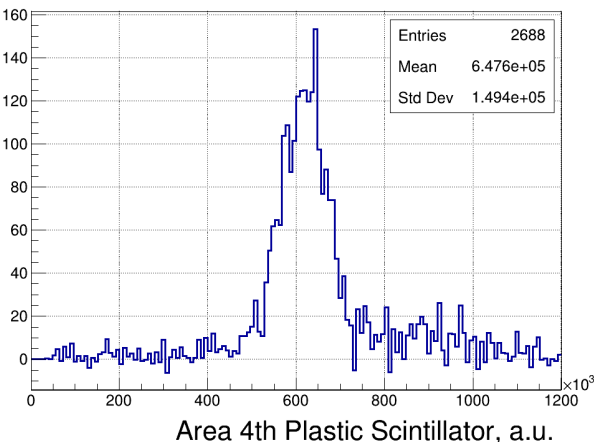
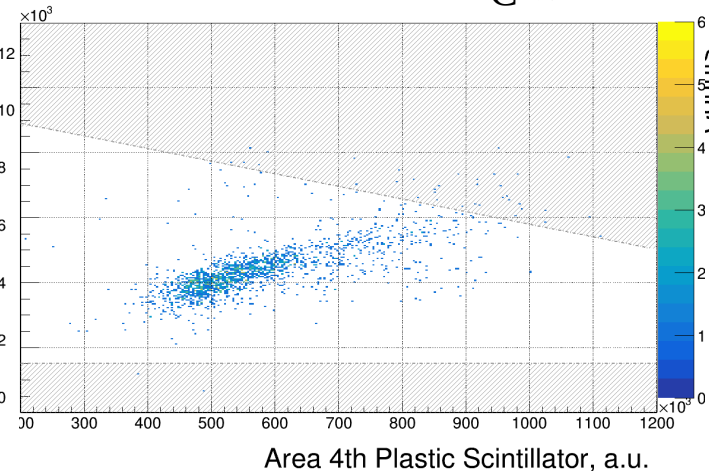
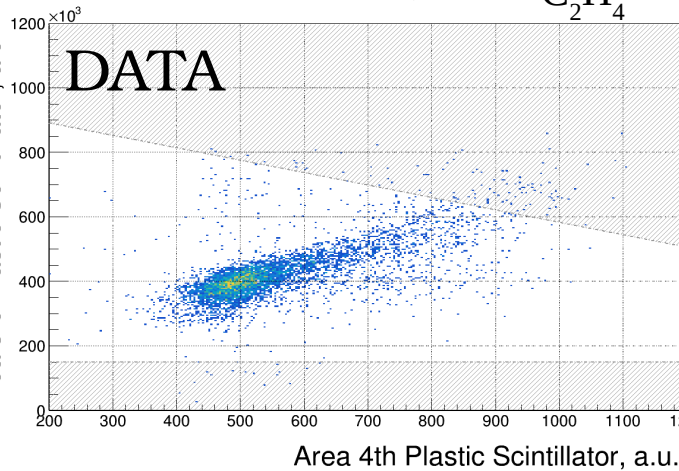


neutrons
→

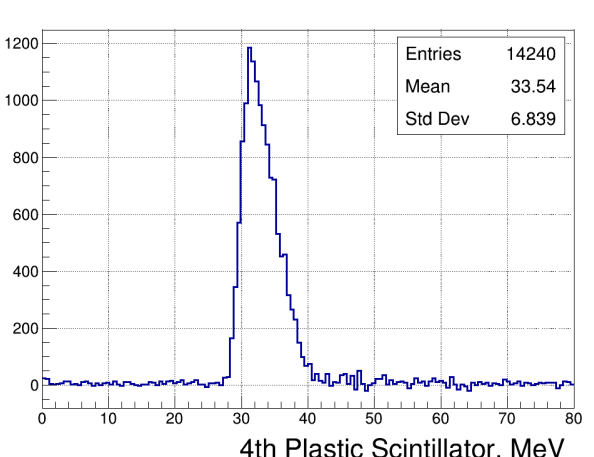
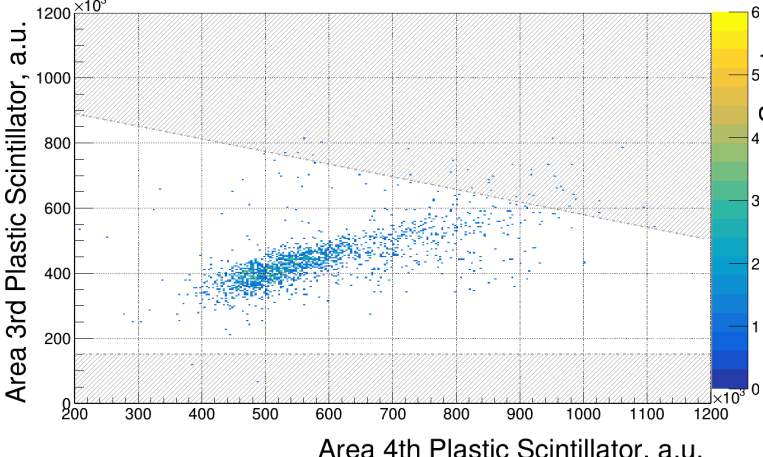
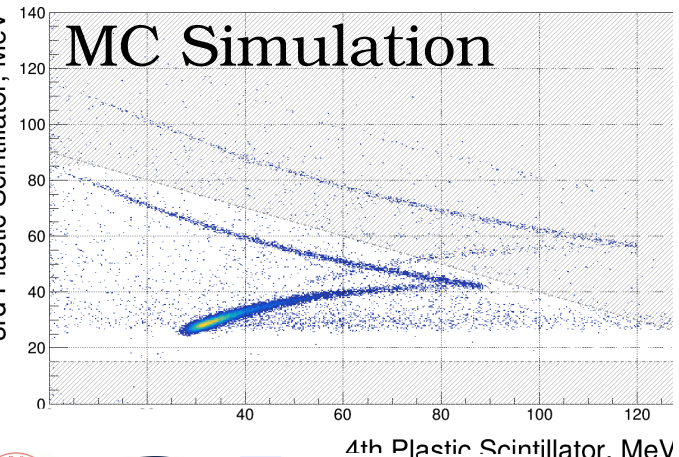


neutrons
→

Area 3rd Plastic Scintillator, a.u.



3rd Plastic Scintillator, MeV



Neutron flux extraction

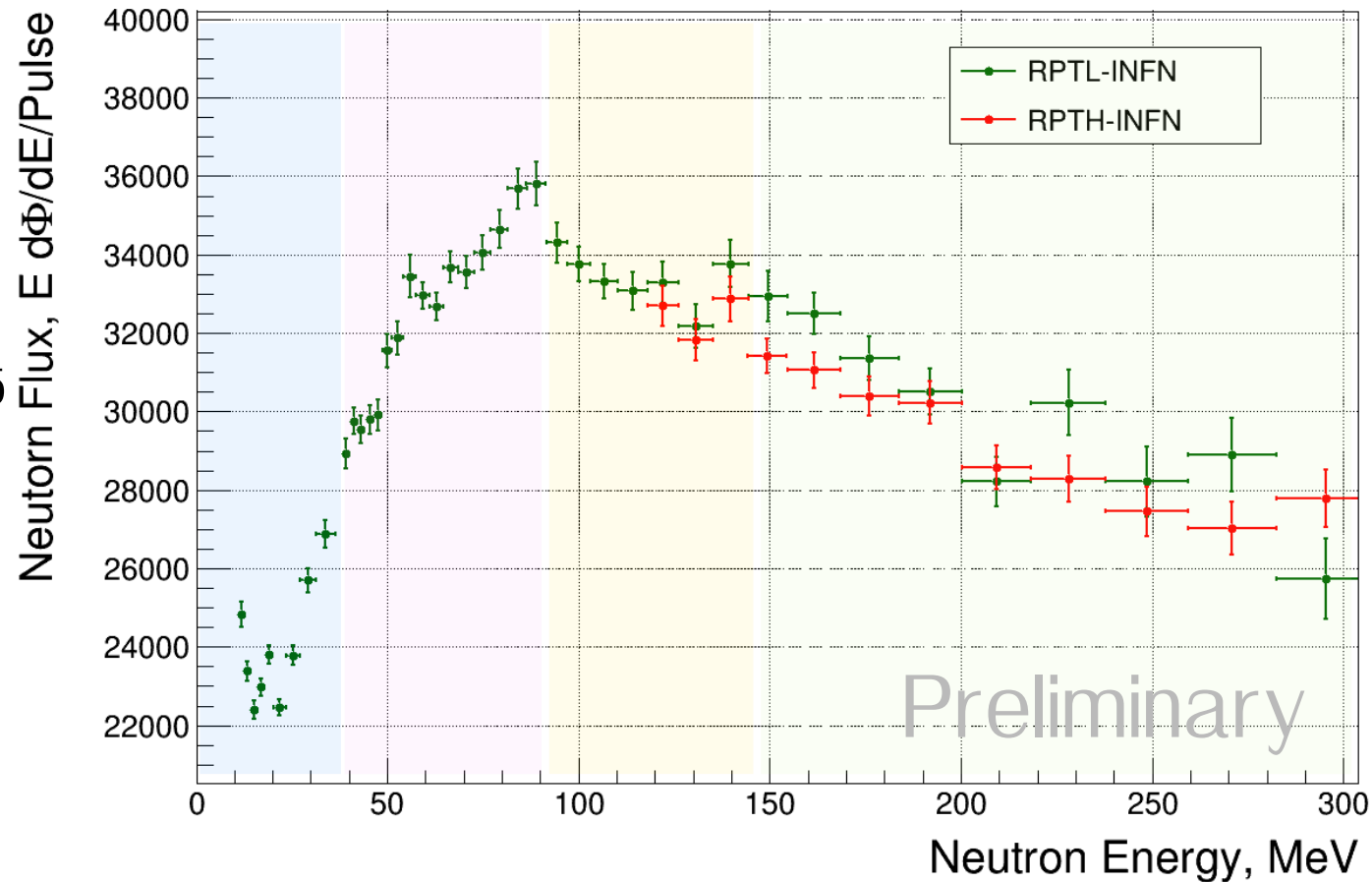
Segmentation + Coincidences

Coinc Silicon 1-2: PE1 + PE2
12-30 MeV

Coinc Scint 1-2: PE2
37-90 MeV

Coinc Scint 1-2-3: PE2 + PE5
91-145 MeV

Coinc Scint 1-2-3-4: PE5
>146 MeV



Neutron flux extraction

