

Neutron-induced capture-tofission cross section ratio measured at LANSCE

CNR-24

Esther Leal Cidoncha (elealcid@lanl.gov)

Presented by Ingrid Knapova

July 8-12, 2024



LA-UR-24-26643

Motivation

- There is a need of reduced uncertainties for applications in nuclear technology, including the design of advanced reactors.
- Also, uncertainties in the capture cross section impact our understanding of the criticality of U/Pu systems and transmutation rates.
- Isotopes involved in the Th-U and U-Pu fuel cycles of interest for these studies.
 - ²³⁵U, ²³³U and ²³⁹Pu measured at LANSCE.
- The capture-to-fission ratio measurement eliminates the systematic uncertainties derived from:
 - Neutron flux, self-shielding and sample mass.



Figure 1 Illustration of the thorium fuel cycle where ²³³U plays a role.



Motivation

- Experimental cross section data in the literature are sometimes scarce and measured decades ago with the detecting technologies available at the time (detectors, neutron flux and electronics).
- New data are constantly required to update the nuclear data evaluations.
- For some isotopes, the fission rate is considerable compared to capture. ъ Good discrimination 10-between γ -rays coming ENDF/B-VIII.0 JEFF-3.3 from capture and fission is JENDL-5 required. New measurements proposed at LANL combining a fission 10 detector and DANCE. ъ Hopkins (1962) Weston (1968) 10⁻² Berthomieux 1 (2007) The ²³³U measurement was the first Berthomieux 2 (2007) ND measurement funded at LANSCE 10^{2} 10^{3} 10^{4} 10^{5} 10^{6} by NCSP! 10 E_n (eV) Figure 2 ²³³U capture-to-fission ratio from the literature



LANSCE facility

- Neutrons produced by proton spallation on a W target.
- DANCE:
 - Mark-III spallation target.
 - Flight path 14 (20 m).







Time-of-flight measurements





Detectors

DANCE (Detector for Advanced Neutron Capture Experiments)

- 4πBaF₂ γ-ray calorimeter composed by 160 crystals with an inner cavity of 17 cm radius [1].
- Used to measure neutron capture cross section data on small quantities of radioactive isotopes. Single γ -ray detection efficiency of 85%.
- We can measure En, Esum, Ecl, and Mcl, providing more information than with C6D6 detectors.
- A LiH ball is placed inside around the sample to absorb scattered neutrons.





Figure 4 ²³³U DANCE sample (left), DANCE + LiH ball (center) and DANCE picture (right)



Detectors

NEUANCE (**NEU**tron detector array at d**ANCE**)

- Neutron detector array that consists in 21 stilbene crystals arranged in a cylindrical geometry around the beam pipe [2].
- Used to detect neutrons coming from fission and determine by coincidence with DANCE, the fission γrays.
- NEUANCE detects neutrons with energies above 500 keV (fission neutrons have these energies), therefore low energy scattered neutrons that are below this threshold are discriminated.
- Single fission neutron efficiency of 12.5%.
- Possibility to use a thick target -> higher statistics/lower measuring time required.
- NEUANCE can also detect γ -rays.



Figure 5 NEUANCE instrument

[2] M. Jandel et al. Nuclear Inst. and Methods in Physics Research, A 882 (2018) 105-113.



Detectors

PPAC (Parallel Plate Avalanche Counter)

- 4π detection range [3].
- Used to detect Fission Fragments (FF) and determine by coincidence with DANCE, the fission γ-rays.
- Need to use a thin sample to achieve a high fission fragment detection efficiency -> lower statistics/larger measuring time required.
- Charged particle detection efficiency of ~65%.





Figure 7 PPAC for DANCE

Figure 6 PPAC geometry

[3] M. Jandel et al. PRL **109**, 202506 (2012).



DANCE calibrations

- Intrinsic radioactivity of BaF₂ used to calibrate the DANCE crystals.
- Using the Alpha-decay chain of the ²²⁶Ra present in the BaF₂.





Fission tagging process

- Search for coincidences between the two detectors.
- The DANCE γ-rays in coincidence with the fission particles (FFs/ neutrons) are tagged as fission gammas.
- The purpose of tagging is to define the shape of the fission γ-ray spectrum that can be subtracted from the total spectrum.



Figure 9 Fission tagging process



Background studies

The background varies with the neutron energy, therefore it is subtracted per En bin.



Figure 10 Background subtraction process for ²³⁹Pu analysis

The capture-to-fission ratio is given by:

$$\alpha(E_n) \equiv \frac{\sigma_{\gamma}(E_n)}{\sigma_f(E_n)}$$



The capture-to-fission ratio is given by:

$$\alpha(E_n) \equiv \frac{\sigma_{\gamma}(E_n)}{\sigma_f(E_n)}$$

We measure a number of events Ci as a function of the neutron energy associated with process i (fission or capture):

$$C_i(E_n) = \epsilon_i Y_i(E_n)$$



The capture-to-fission ratio is given by:

$$\alpha(E_n) \equiv \frac{\sigma_{\gamma}(E_n)}{\sigma_f(E_n)}$$

We measure a number of events Ci as a function of the neutron energy associated with process i (fission or capture):

$$C_i(E_n) = \epsilon_i Y_i(E_n)$$

and

$$Y_i(E_n) = \sigma_i(E_n) N \phi_n(E_n)$$



The capture-to-fission ratio is given by:

$$\alpha(E_n) \equiv \frac{\sigma_{\gamma}(E_n)}{\sigma_f(E_n)}$$

We measure a number of events Ci as a function of the neutron energy associated with process i (fission or capture):

$$C_i(E_n) = \epsilon_i Y_i(E_n)$$

and

$$Y_i(E_n) = \sigma_i(E_n) N \phi_n(E_n)$$

$$\frac{C_{\gamma}(E_n)}{C_f(E_n)} = \frac{\epsilon_{\gamma}Y_{\gamma}(E_n)}{\epsilon_fY_f(E_n)}$$



The capture-to-fission ratio is given by:

$$\alpha(E_n) \equiv \frac{\sigma_{\gamma}(E_n)}{\sigma_f(E_n)}$$

We measure a number of events Ci as a function of the neutron energy associated with process i (fission or capture):

$$C_i(E_n) = \epsilon_i Y_i(E_n)$$

and

$$Y_i(E_n) = \sigma_i(E_n) N \phi_n(E_n)$$

$$\frac{C_{\gamma}(E_n)}{C_f(E_n)} = \frac{\epsilon_{\gamma}Y_{\gamma}(E_n)}{\epsilon_f Y_f(E_n)}$$
$$= \frac{\epsilon_{\gamma}\sigma_{\gamma}(E_n)N\phi_n(E_n)}{\epsilon_f\sigma_f(E_n)N\phi_n(E_n)}$$



The capture-to-fission ratio is given by:

$$\alpha(E_n) \equiv \frac{\sigma_{\gamma}(E_n)}{\sigma_f(E_n)}$$

We measure a number of events Ci as a function of the neutron energy associated with process i (fission or capture):

$$C_i(E_n) = \epsilon_i Y_i(E_n)$$

and

$$Y_i(E_n) = \sigma_i(E_n) N \phi_n(E_n)$$

$$\frac{C_{\gamma}(E_n)}{C_f(E_n)} = \frac{\epsilon_{\gamma} Y_{\gamma}(E_n)}{\epsilon_f Y_f(E_n)}$$
$$= \frac{\epsilon_{\gamma} \sigma_{\gamma}(E_n) \mathcal{N} \phi_n(E_n)}{\epsilon_f \sigma_f(E_n) \mathcal{N} \phi_n(E_n)}$$



The capture-to-fission ratio is given by:

$$\alpha(E_n) \equiv \frac{\sigma_{\gamma}(E_n)}{\sigma_f(E_n)}$$

We measure a number of events Ci as a function of the neutron energy associated with process i (fission or capture):

$$C_i(E_n) = \epsilon_i Y_i(E_n)$$

and

$$Y_i(E_n) = \sigma_i(E_n) N \phi_n(E_n)$$

$$\frac{C_{\gamma}(E_n)}{C_f(E_n)} = \frac{\epsilon_{\gamma} Y_{\gamma}(E_n)}{\epsilon_f Y_f(E_n)}$$
$$= \frac{\epsilon_{\gamma} \sigma_{\gamma}(E_n) \mathcal{N} \phi_n(E_n)}{\epsilon_f \sigma_f(E_n) \mathcal{N} \phi_n(E_n)}$$
$$= \frac{\epsilon_{\gamma} \sigma_{\gamma}(E_n)}{\epsilon_f \sigma_f(E_n)}$$



The capture-to-fission ratio is given by:

$$\alpha(E_n) \equiv \frac{\sigma_{\gamma}(E_n)}{\sigma_f(E_n)}$$

We measure a number of events Ci as a function of the neutron energy associated with process i (fission or capture):

$$C_i(E_n) = \epsilon_i Y_i(E_n)$$

and

$$Y_i(E_n) = \sigma_i(E_n) N \phi_n(E_n)$$

$$\frac{C_{\gamma}(E_n)}{C_f(E_n)} = \frac{\epsilon_{\gamma}Y_{\gamma}(E_n)}{\epsilon_f Y_f(E_n)}$$
$$= \frac{\epsilon_{\gamma}\sigma_{\gamma}(E_n)N\phi_n(E_n)}{\epsilon_f\sigma_f(E_n)N\phi_n(E_n)}$$
$$= \frac{\epsilon_{\gamma}\sigma_{\gamma}(E_n)}{\epsilon_f\sigma_f(E_n)}$$
$$= k\frac{\sigma_{\gamma}(E_n)}{\sigma_f(E_n)}$$



The capture-to-fission ratio is given by:

$$\alpha(E_n) \equiv \frac{\sigma_{\gamma}(E_n)}{\sigma_f(E_n)}$$

We measure a number of events Ci as a function of the neutron energy associated with process i (fission or capture):

$$C_i(E_n) = \epsilon_i Y_i(E_n)$$

and

$$Y_i(E_n) = \sigma_i(E_n) N \phi_n(E_n)$$

Therefore, the capture-to-fission ratio can be expressed as:

$$\frac{C_{\gamma}(E_n)}{C_f(E_n)} = \frac{\epsilon_{\gamma}Y_{\gamma}(E_n)}{\epsilon_f Y_f(E_n)}$$
$$= \frac{\epsilon_{\gamma}\sigma_{\gamma}(E_n)N\phi_n(E_n)}{\epsilon_f\sigma_f(E_n)N\phi_n(E_n)}$$
$$= \frac{\epsilon_{\gamma}\sigma_{\gamma}(E_n)}{\epsilon_f\sigma_f(E_n)}$$
$$= k\frac{\sigma_{\gamma}(E_n)}{\sigma_f(E_n)}$$
$$\alpha(E_n) = \frac{1}{k}\frac{C_{\gamma}(E_n)}{C_f(E_n)}$$

Hence:



- Experimental advantages of the **capture-to-fission ratio**:
 - It is much simpler and more reliable to determine experimentally as many of the systematic questions:
 - Sample mass
 - Self-shielding
 - Neutron exposure

will cancel out in an appropriately designed experiment.

 The relative capture cross section can be obtained multiplying the ratio by the evaluated fission cross section.



Measurements at LANSCE

²³⁵U capture-to-fission ratio

- Measured using the DANCE + PPAC setup from (4 eV to 1 MeV).
- Three independent measurements for high precision in capture cross section:
 - Measurement with a thick sample -> to increase statistics at high neutron energies.
 - Measurement with a thin sample inside the PPAC -> for fission tagging.
 - Measurement of the neutron scattering background using a ²⁰⁸Pb sample.
- Targets:
 - Thick target of 26 mg/cm² of 94% enriched ²³⁵U.
 - Thin sample 130 ug/cm² of 99.9% enriched ²³⁵U.
 - Scattering sample of ~120 mg/cm² of 99% enriched ²⁰⁸Pb.
- Results published in [4].

[4] M. Jandel et al., Phys. Rev. Letters 109, 202506 (2012).



²³⁵U relative capture cross section

- The alpha-derived capture cross section was calculated by multiplying the captureto-fission ratio by the ENDF/B-VII.0 fission cross section.
- The broadened cross section was used in the Resolved Resonance Region.



Figure 11 Results of the alpha-derived $^{235}U(n,\gamma)$ cross section



Measurements at LANSCE

²³⁹Pu capture-to-fission ratio

- Measured using the DANCE + PPAC setup from (10 eV to 1.3 MeV) [5] [7].
- Three independent measurements for high precision in capture cross section.
- Targets:
 - Thin 937 ug of 99.97% enriched ²³⁹Pu.
 - Thick ~50 mg of ²³⁹Pu.
 - Scattering 200 mg/cm² ²⁰⁸Pb.



[5] S. Mosby et al. Phys. Rev. C 89, 304610 (2014).





[7] S. Mosby et al. Nucl. Data Sheets 148, (2018) 312-321.

07/09/24 16

²³⁹Pu relative capture cross section

- The alpha-derived capture cross section was calculated by multiplying the captureto-fission ratio by the ENDF/B-VII.1 fission cross section.
- The broadened cross section was used in the Resolved Resonance Region.



Figure 13 Results of the alpha-derived $^{239}Pu(n,\gamma)$ cross section from 10 eV to 1 keV



²³⁹Pu relative capture cross section

- The alpha-derived capture cross section was calculated by multiplying the captureto-fission ratio by the ENDF/B-VII.0 fission cross section.
- The broadened cross section was used in the Resolved Resonance Region.



Figure 14 Results of the alpha-derived $^{239}Pu(n,\gamma)$ cross section from 1 keV to 1.3 MeV



Measurements at LANSCE

²³³U capture-to-fission ratio

- Measured using the DANCE + NEUANCE setup from (0.7 eV to 250 keV).
- Normalization to ENDF/B-VIII.0 broadened cross section ratio in the neutron energy region (8.1-14.7) eV:
- Results published in [7].



[7] E. Leal-Cidoncha et al., Phys. Rev. C 108 014608 (2023)







07/09/24 21

²³³U relative capture cross section

- The alpha-derived capture cross section was calculated by multiplying the captureto-fission ratio by the ENDF/B-VIII.0 fission cross section.
- The broadened cross section was used in the Resolved Resonance Region.



Figure 16 Results of the alpha-derived $^{233}U(n,\gamma)$ cross section



Statistical Model Calculations

- Statistical model calculations were performed by I. Stetcu, T. Kawano and A. Lovell with the CoH3 code [4] from 1 keV to 5 MeV (Energy for which only the first fission chance is involved).
- This code combines the coupled-channels optical model and the statistical Hauser-Feshbach model calculations by performing the Engelbrecht-Weidenmüller transformation of the penetration matrix.
- Different values of the average γray width have been tried by adjusting the M1 γ-ray strength function for the scissors mode.
- Mughabghab gives 40 meV.
- To reproduce the data from Hopkins³ it had to be reduced to 24 meV.
- A smaller value would be needed to reproduce this work.



Figure 17 Statistical model calculations for ²³³U

[4] T. Kawano, Springer Proceedings in Physics 254, 27 (2021)



Conclusions

- The capture-to-fission cross section ratio eliminates the uncertainties associated to the neutron flux, sample mass and self-shielding.
- It can be measured at LANSCE combining DANCE and a fission detector.
- A PPAC and NEUANCE have been used in combination with DANCE to detect FF and fission neutrons.
- Measurements of the ²³⁵U, ²³³U and ²³⁹Pu in the neutron energy region from eV to 100s keV - 1 MeV have been performed in the last years.
- The alpha-derived cross sections have been also calculated multiplying the ratio by the evaluated fission cross section.
- Other detectors could be used in combination with DANCE to tag fission events in the future.



Acknowledgements

Thanks to our collaborators:

Aaron Couture, Evelyn M. Bond, Todd A. Bredeweg, Cathleen Fry, Marian Jandel, Toshihiko Kawano, Amy E. Lovell, Shea Mosby, Gencho Rusev, Ionel Stetcu, and John Ullmann Los Alamos National Laboratory

> Luiz Leal, and Marco Pigni Oak Ridge National Laboratory

