Technical Meeting on Compatibility Between Coolants and Materials for Fusion Facilities and Advanced Fission Reactors



Contribution ID: 82

Type: not specified

Surface Modification Methods of Zriconium Alloy fuel cladding Tube for Mitigation of corrosion product deposit in simulated PWR primary coolant

Thursday, 2 November 2023 14:40 (20 minutes)

Presenter: Mr SHIM, Hee Sang (KAERI)

Session Classification: Compatibility of materials and coolants in nuclear reactors' environment