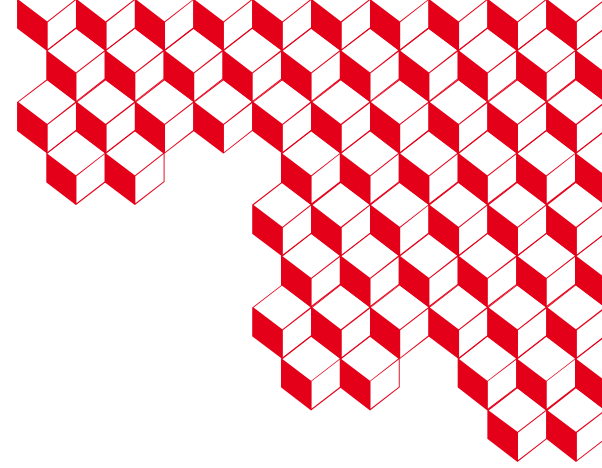




iresne



# **Modelling the corrosion product transfer in fusion systems: the OSCAR-Fusion code**

F. Dacquait, C. Cherpin, F. Broutin, C. Chalons

CEA/DES/IRESNE/DTN/SMTA/LMCT Laboratory for Mastering Contamination and Chemistry of Coolants and Tritium – Cadarache

*IAEA Technical Meeting on Compatibility Between Coolants and Materials for Fusion Facilities and Advanced Fission Reactors*

*30 Oct - 3 Nov 2023 | Vienna (Austria)*

# Outline

- 1. OSCAR code**
- 2. Challenges related to the interactions between coolants and materials**
- 3. Conclusion**

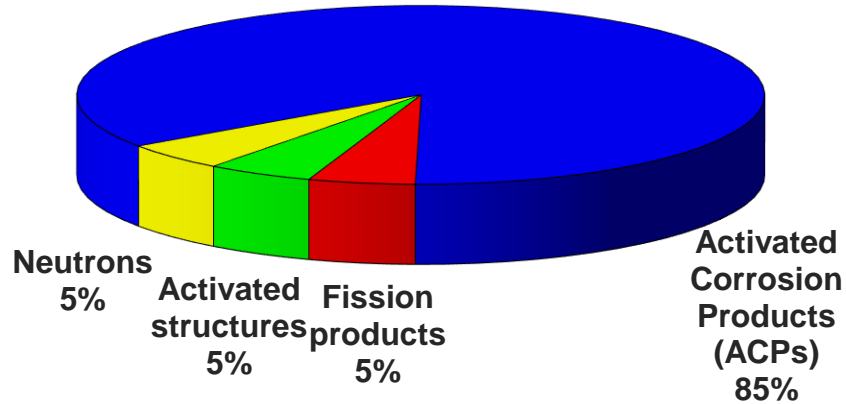




# 1 ■ OSCAR code

# Principle and stakes

## Collective dose for operation and maintenance of PWRs

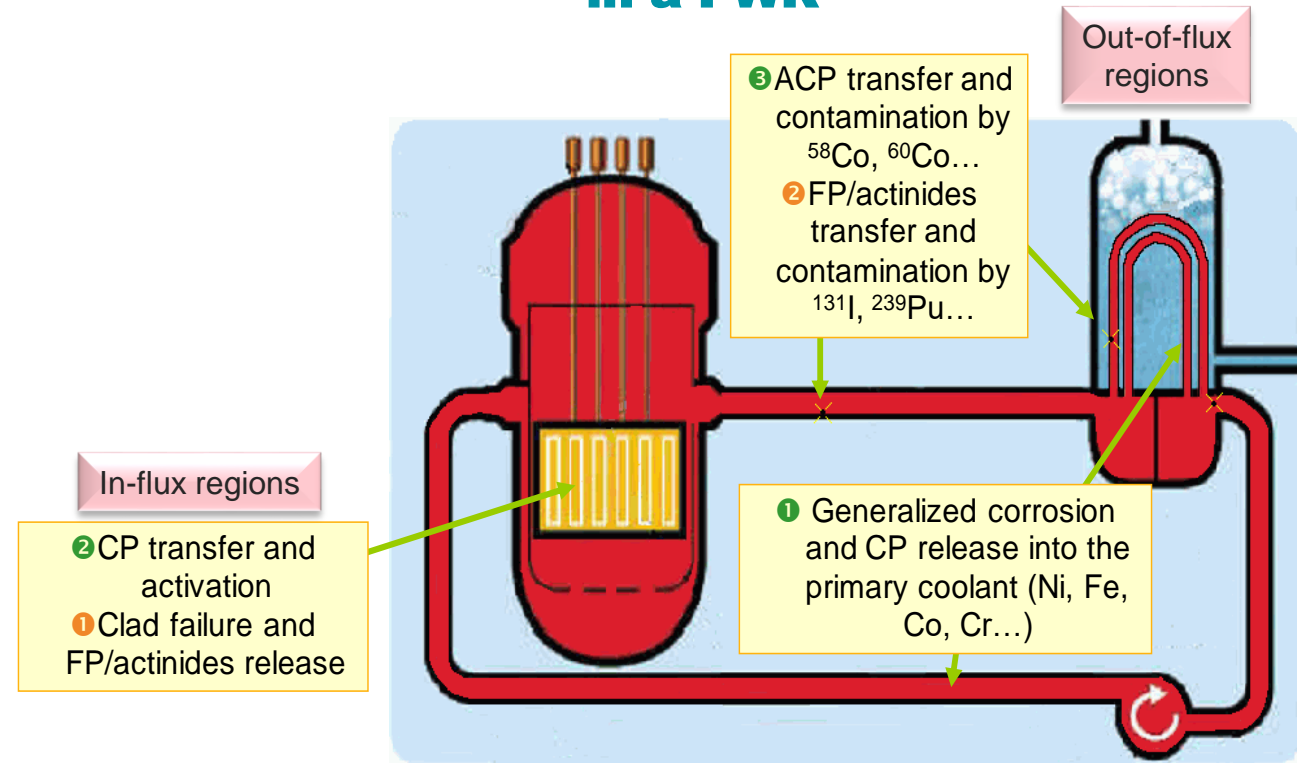


For other types of reactor, especially fusion reactors, mainly due to ACPs as well (fundamental safety function for DEMO)

## Industrial stakes:

- **Radioprotection:** Reduction of Occupational Radiation Exposure (ORE)
- **Environment:** Minimization of release/waste – Optimization of dismantling process
- **Availability:** Optimization of reactor operation
- **Safety:** Source term in case of accident/incident

## Principle of contamination transfer in a PWR



# OSCAR Outil de Simulation de la Contamination en Réacteur tool for Simulating Contamination in Reactors

- Simulation of contamination transfer in nuclear reactor systems during power operation and shutdown (PWR: 20 → 350 °C - reducing/oxidizing - acid/alkaline)
  - Calculation of masses/activities of CPs, ACPs, Coolant Activation Products (CAPs), Actinides and FPs (AFPs) in solid, liquid and gaseous phases of nuclear circuits as a function of time (normal operation over several decades and transient events of a few minutes/seconds within a reasonable calculation time)
  - Development of calculation codes for PWRs since 1970's by CEA in collaboration with EDF and Framatome:
    - **PACTOLE** (ACPs) and **PROFIP** (AFPs) codes
    - **OSCAR** code since 2008 (merger of PACTOLE and PROFIP)
      - Modular code (easy evolving tool / C++ / Linux OS)
      - Input/output Graphical User friendly Interfaces (Salome/Python/QT)
  - Validation based on an extensive and world-unique OPEX:
    - >430 contamination expertise assessments in 76 different PWR units: EMECC campaigns ( $\gamma$  measurements of ACP surface activities)
  - Application to **SFRs**: **OSCAR-Na** (2012) → see [C. Latge's presentation](#)
  - Application to **Fusion reactors**: **PACTOLE-ITER** (1995) → **PACTITER** (1998) (see [L. Di Pace's presentation](#)) → **OSCAR-Fusion** (2012)

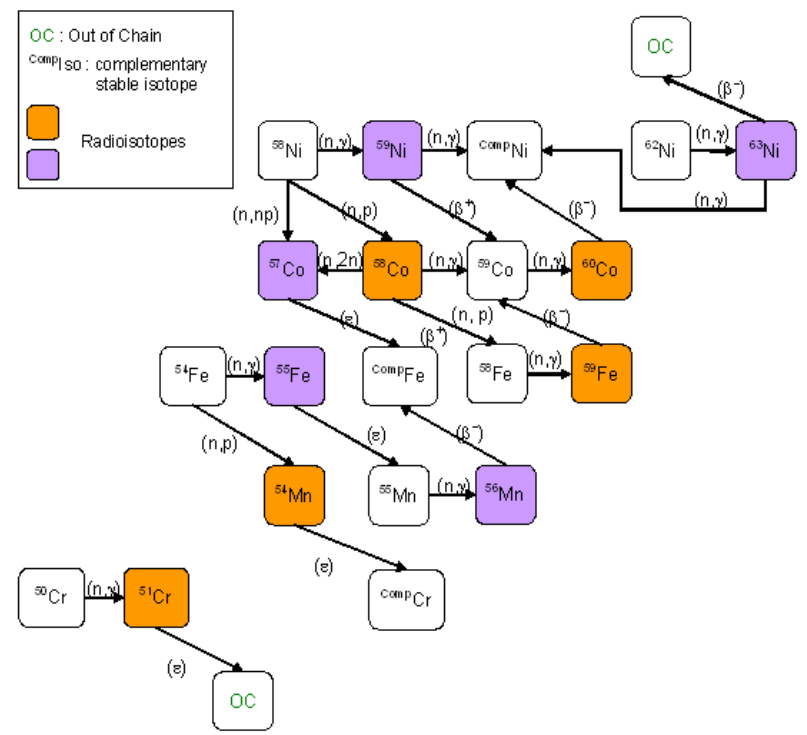


EMECC measurement of a hot leg of a PWR RCS

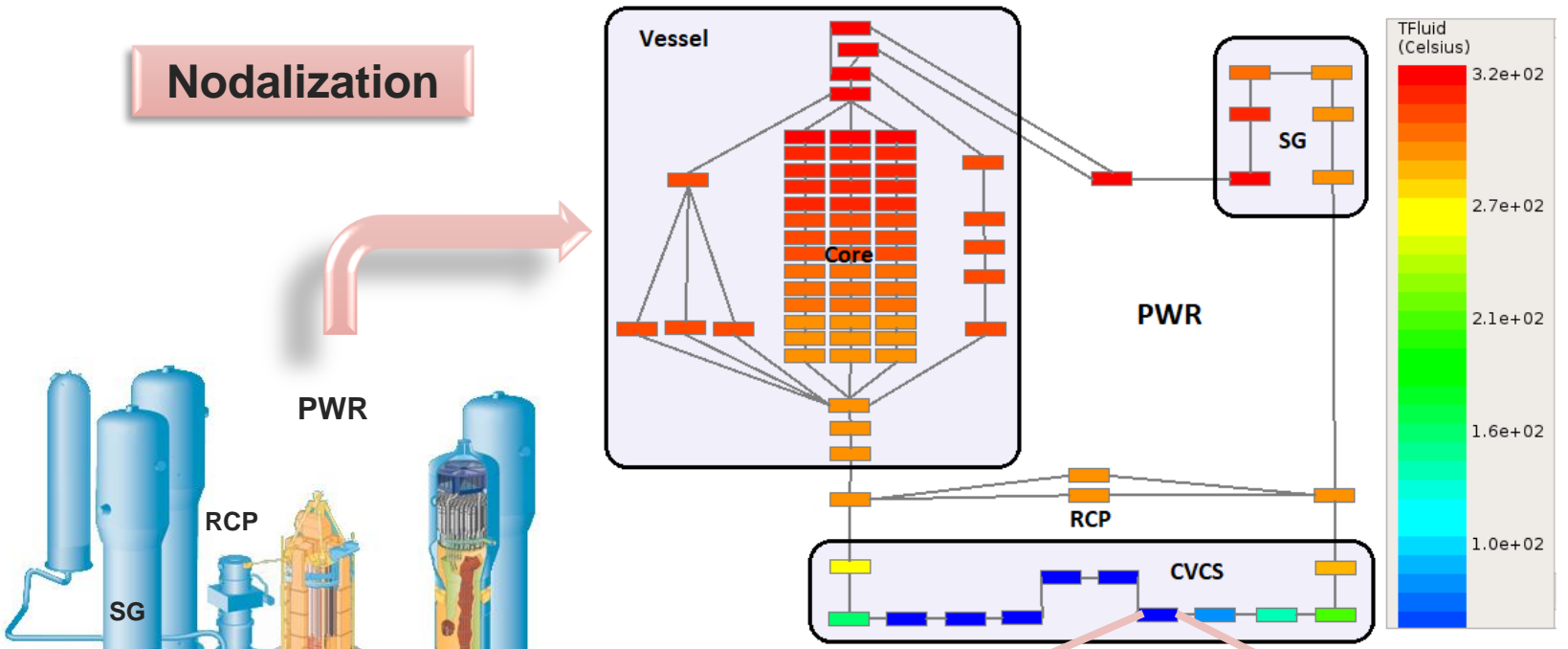
# OSCAR modelling for ACPs

## Elements/Isotopes

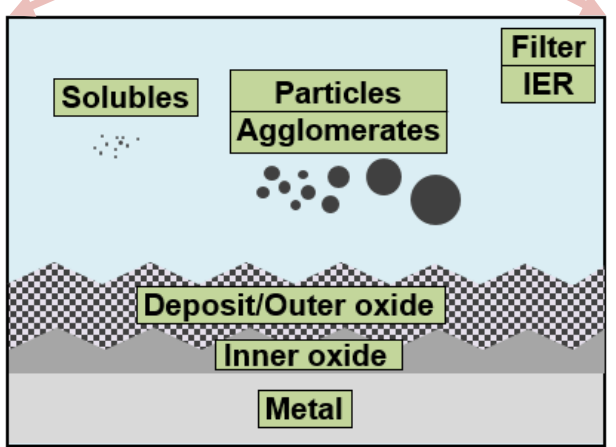
Co, Cr, **Cu**, Fe, Mn, Ni  
Ag, Sb, Zn, Zr



## Nodalization

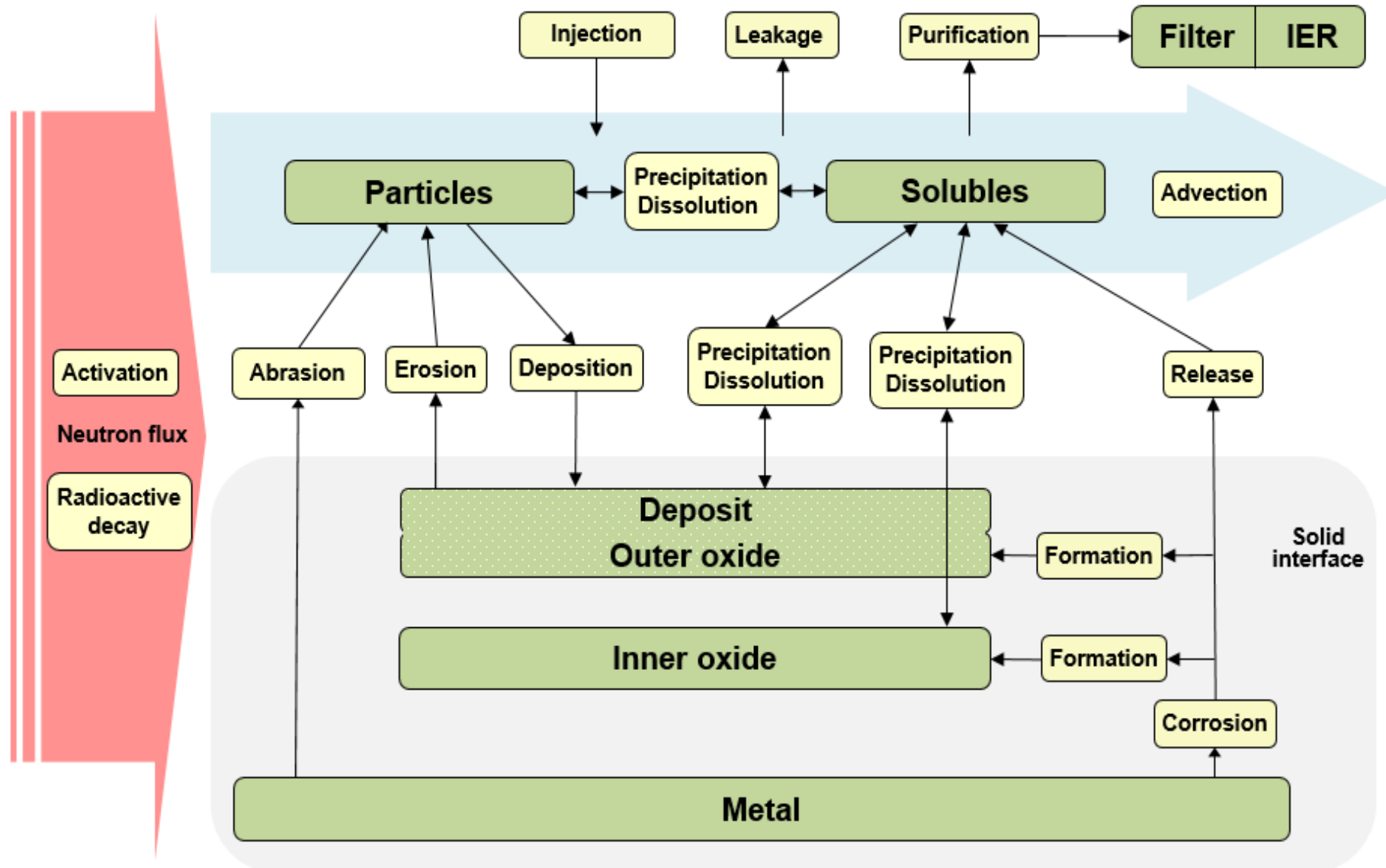


## Media



# OSCAR modelling

## Transfer mechanisms



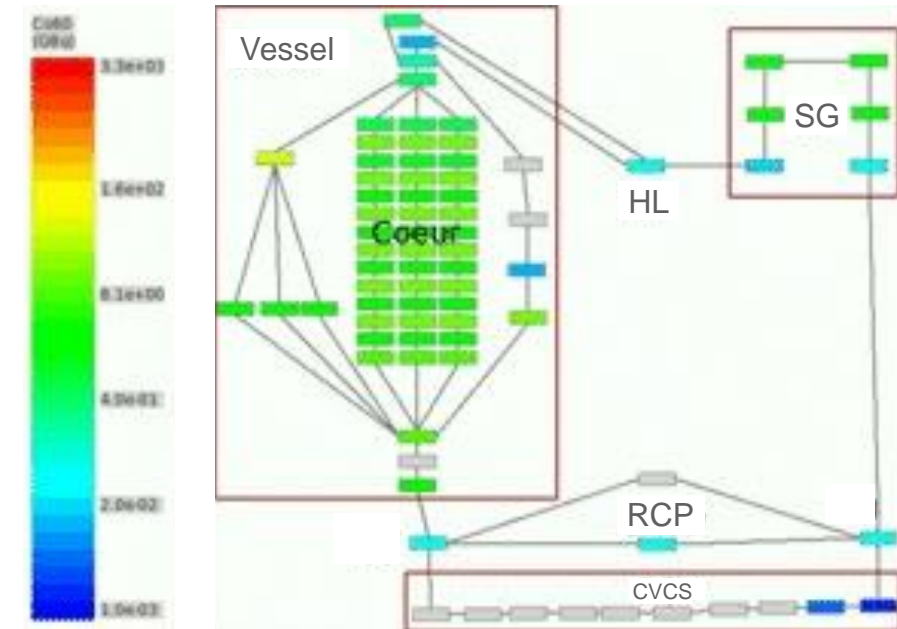
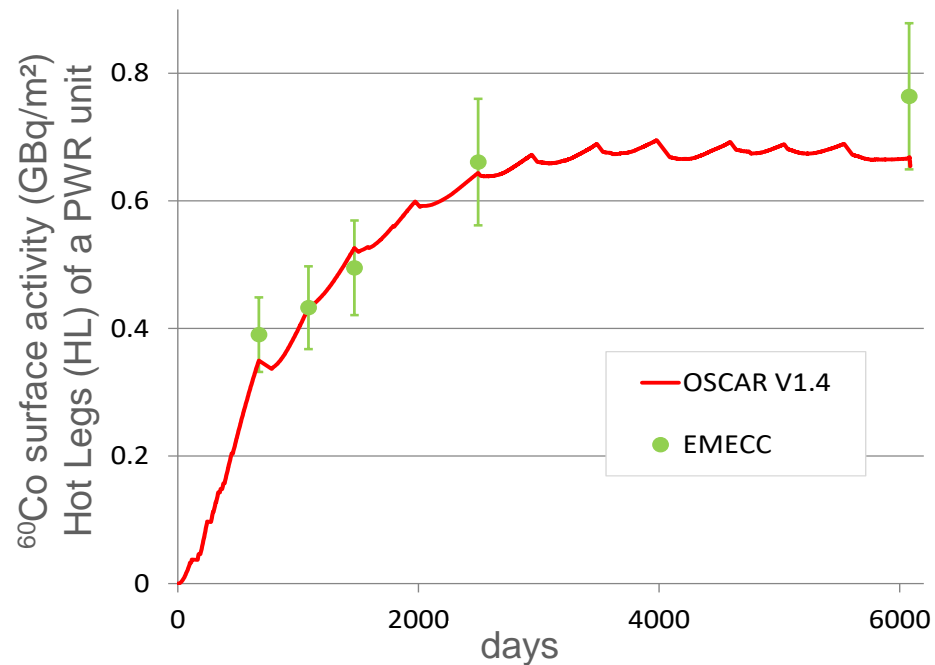
# OSCAR modelling

## Solver

$$\frac{\partial m^i}{\partial t} + (\dot{m}_{out}^i - \dot{m}_{in}^i) = \sum_{sources} J_m^i - \sum_{sinks} J_m^i$$

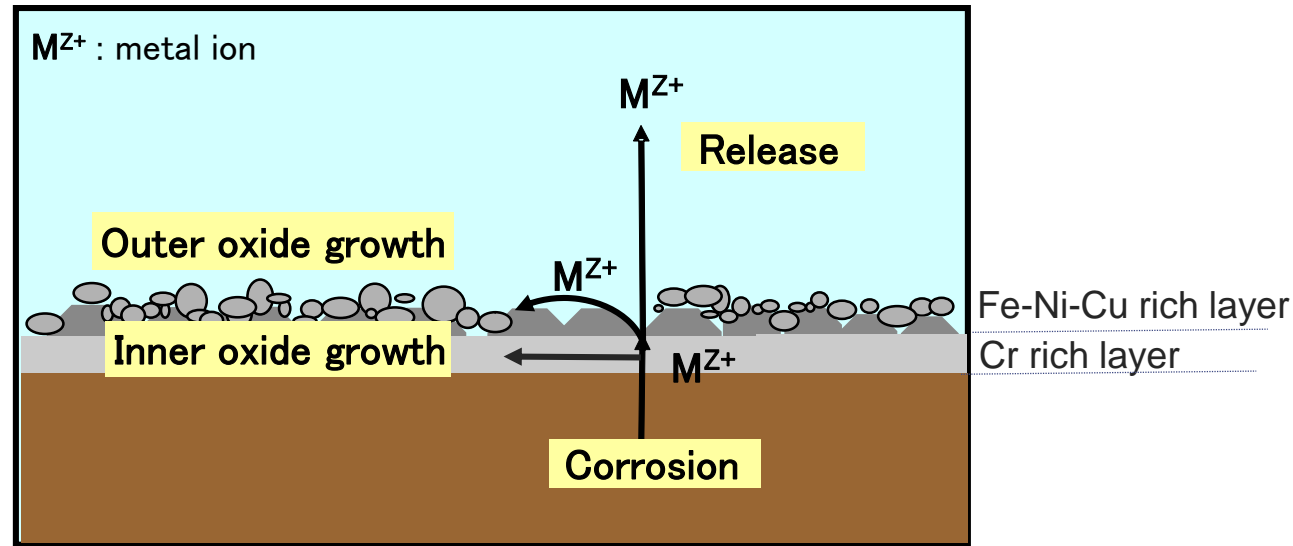
Applications of OSCAR-Fusion:  
see D. Carloni's presentation (ITER IBED PHTS)  
and C. Gasparrini's presentation (DTT)

## Results





# OSCAR modelling - Corrosion-Release



➤ Corrosion/release rates :  $J_{cor/rel}^i = f_{met}^i S_w V_{cor/rel}$

- Empirical laws (material, chemistry, temperature, time)
- User data (time power law, time logarithmic law, constant value per stage)

# OSCAR modelling - Dissolution/Precipitation

➤ Dissolution/precipitation rate: 
$$J_{dissol/precip}^i = \frac{S_{dissol/precip}}{R_{solid-fluid}^{elt} + R_{wall-bulk}^{fluid}} \left| f_{solid}^{i,elt} C_{eq}^{elt} - f_{fluid}^{i,elt} C^{elt} \right|$$

- $S_{dissol/precip}$  : dissolution/precipitation surface
- $R_{solid-fluid}^{elt}$  : transfer resistance of element  $elt$  at interface solid-fluid  $\left( = 1 / \frac{M_{elt}}{C_{eq}^{elt}} \sum_n \alpha_n \nu_n k_n (10^{-pH})^{\mu_{H^+,n}} (p_{H_2/O_2})^{\mu_{H_2/O_2,n}} \right)$
- $R_{wall-bulk}^{fluid}$  : transfer resistance in fluid between wall and bulk  $\left( = 1/h \text{ for Dep/OE-Part. or } = L_{dep}/D_{dep}^{elt} + 1/h \text{ for IO} \right)$
- $C_{eq}^{elt}$  : equilibrium concentration of element  $elt$
- $C^{elt}$  : bulk concentration of element  $elt$
- $f_{solid/fluid}^{i,elt}$  : isotopic mass fractions of isotope  $i$  of element  $elt$  in solid/fluid

*Equilibrium concentrations and composition of solid solution:*

- Calculated by PHREEQCEA (OSCAR chemistry module) (version of PHREEQC code extended to 350 °C) and its thermodynamic database developed by CEA-Saclay → **For more information, ask M. Roy**
- Depend on chemical conditions (pH, redox), bulk/wall temperature and masses of each medium in each region

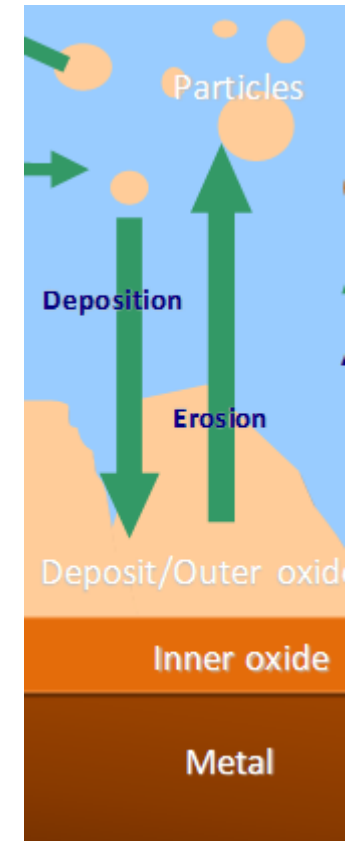
# OSCAR modelling – Erosion & Deposition

➤ Erosion rate:  $J_{erosion}^i = f_{dep}^i m_{eros} E_{eros} / \Psi_{eros}$

- $m_{eros}$  : mass of the deposit that can be eroded
- $E_{eros}$  : erosion coefficient (based on Cleaver & Yates model)
- $\Psi_{eros}$  : erosion resistance

➤ Deposition rate:  $J_{deposition}^i = f_{part}^i S_w v_{depos} C_{part}$

- $v_{depos}$  : deposition velocity taken into account laminar and turbulent (Beal model) diffusion, sedimentation, thermophoresis, boiling deposition, flow disturbances
- $C_{part}$  : particle concentration



# OSCAR Input/Output

## Input data file (.xml)

- Geometry and Thermal-hydraulic ( $D_H$ ,  $S_w$ , flow rate,  $T_{wall/bulk}$ ,  $\rho$ ,  $Ra$ , porosity, tortuosity)
- Material (initial composition and thickness,  $\rho$ ,  $Ra$ , porosity, tortuosity)
- Neutronic (nuclear power fraction & activation rates)
- Operating data ( $P_n$ ,  $B$ ,  $Li$ ,  $H_2$ ,  $O_2 = f(t)$ , Zn injection rate)
- Purification efficiency
- Dose rate coefficients
- Replacement of components

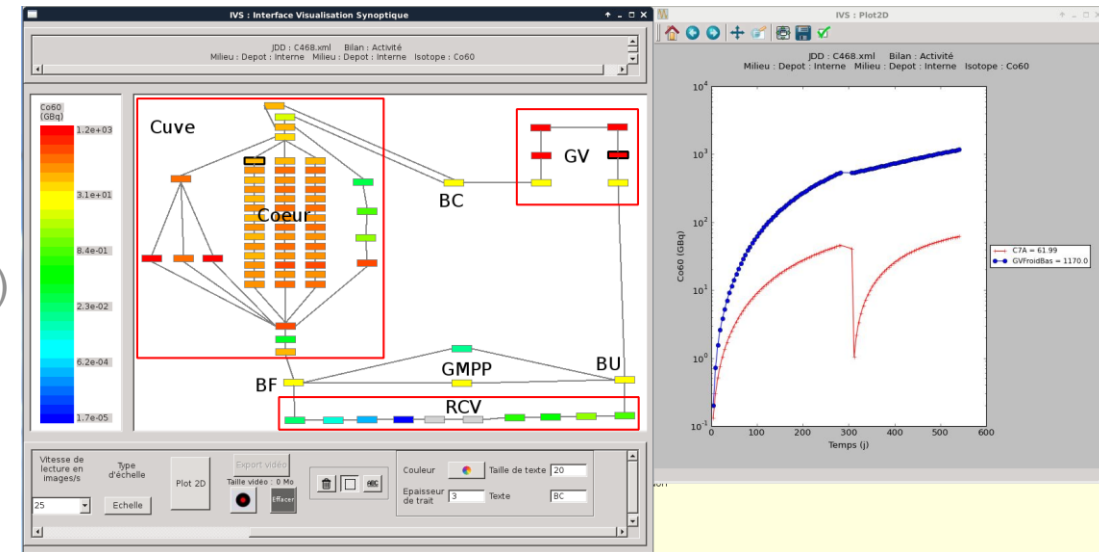
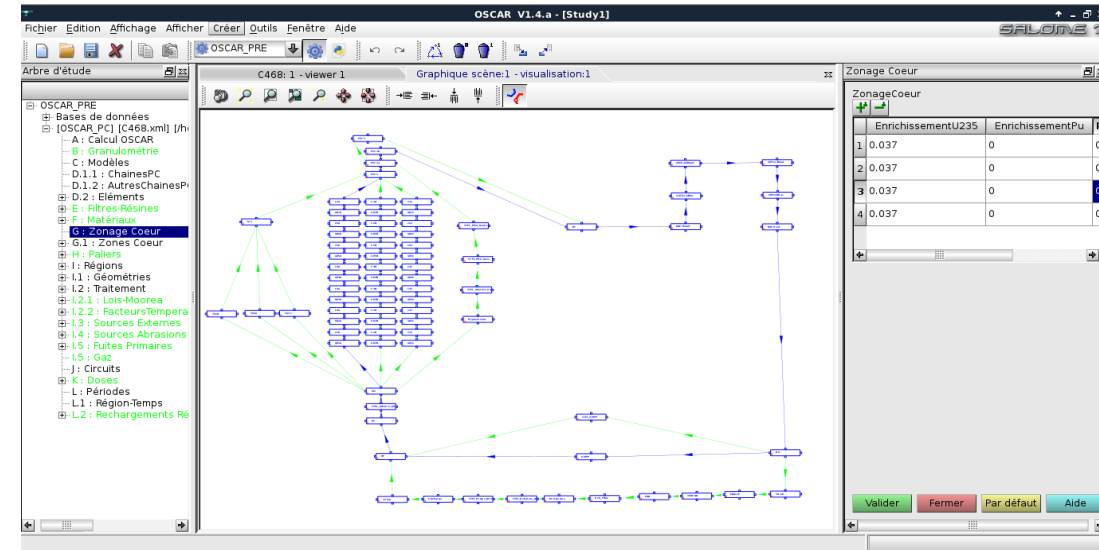
### Input GUI



## Output files (.csv)

- Masses of isotopes/elements (kg)
- Activities of radioisotopes (Bq, Bq/m<sup>2</sup>, Bq/m<sup>3</sup>, Bq/kg)
- Dose rates (Sv/h)
- Transfer mass rates (corrosion, dissolution, deposition... rates in kg/s)
- Chemical data (pH, pe, Ceq, speciation...)
- Thermal-hydraulic data (density, viscosity, Re, Sc...)
- Region characteristics (thicknesses of IO & Dep/OE)
- Operating data

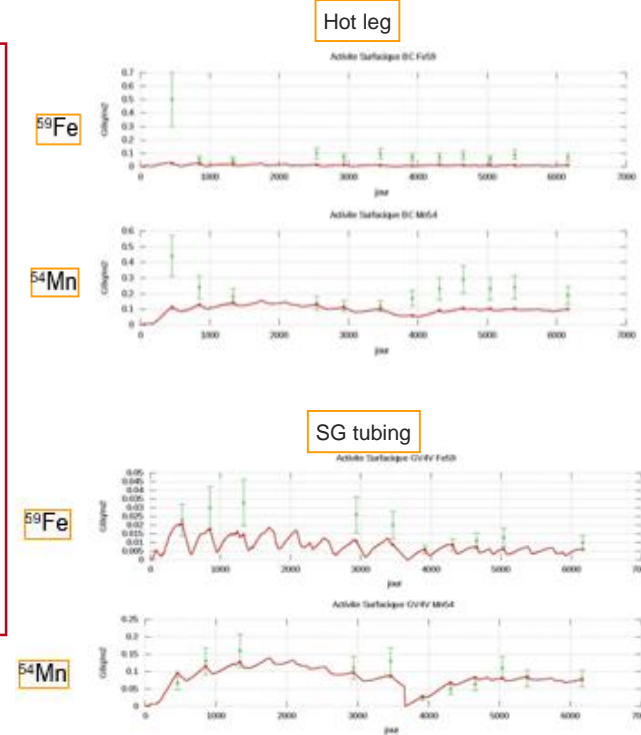
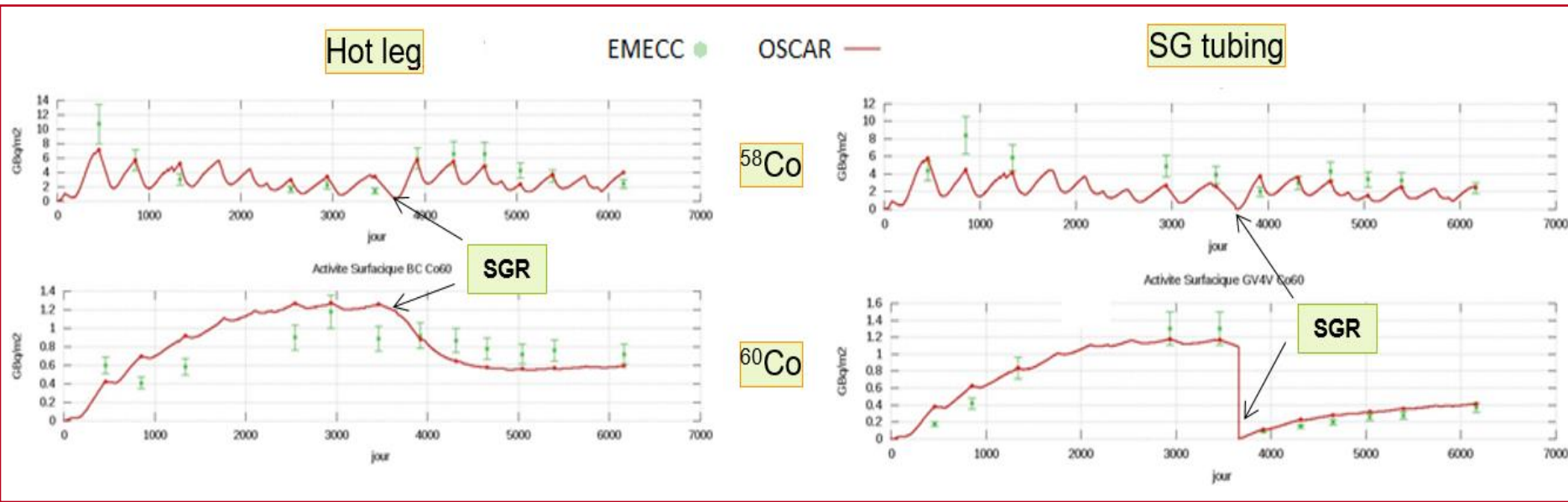
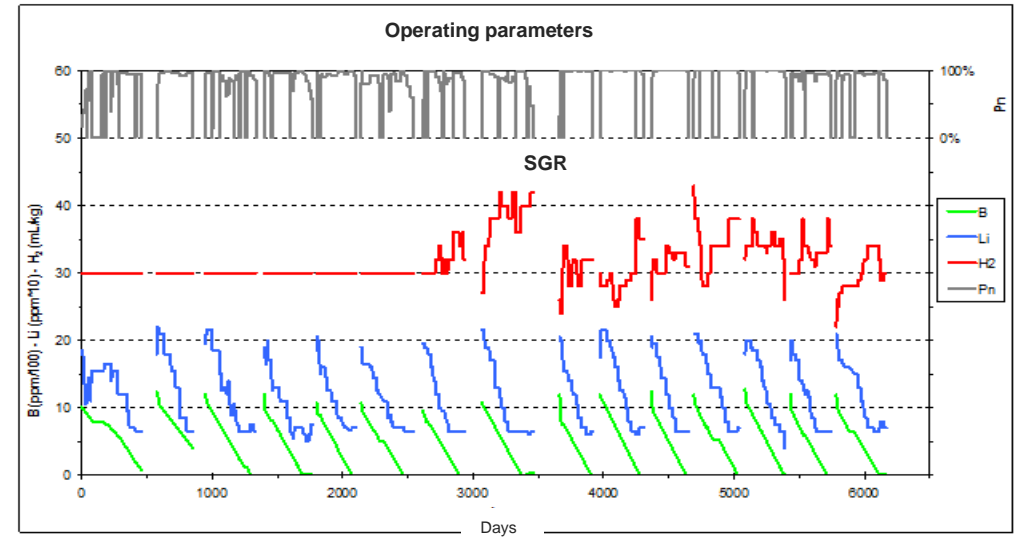
### Output GUI



# OSCAR validation

Validation based on operational feedback from various PWRs (EMECC campaigns), e.g.:

- 900 MWe PWR
- Steam Generator Replacement during cycle 8 outage  
SG tubing 600MA → 690TT
- First 15 cycles simulated



**Good agreement between OSCAR and EMECC**



# **2 ■ Challenges related to the interactions between coolants and materials**

# Challenges related to the interactions between coolants and materials

- Similarities between PHTS conditions in fusion reactors and PWRs (e.g; WCLL-BB PHTS of DEMO  $\cong$  RCS of PWRs)  $\rightarrow$  **OSCAR-Fusion** leverages from the validation of **OSCAR** for PWRs (fission-fusion synergy)

- But distinct characteristics for fusion reactors:

- Materials: RAFM steels (Eurofer, F82H...) and Cu-base alloys
- Higher thermal flux (up to 10-20 MW/m<sup>2</sup> for DEMO)

$\rightarrow$  Subcooled nucleate boiling ( $T_{\text{wall}} > T_{\text{sat}} + \Delta T_{\text{sat}}$ )

$\rightarrow$  Higher deposition rate (deposition by vaporization, deposition by trapping, enrichment = models already in **OSCAR**)

$\rightarrow$  Thicker deposit

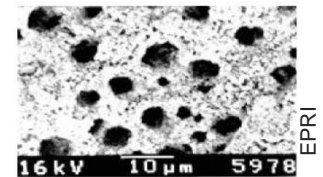
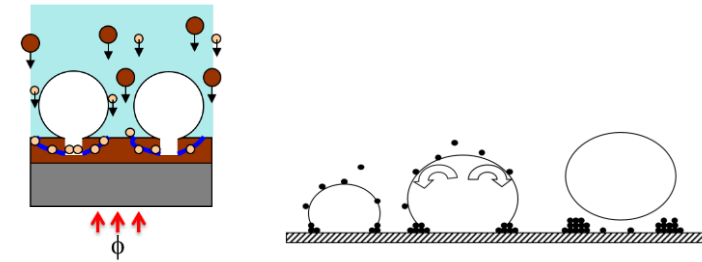
$\rightarrow$  More formation of ACPs and higher  $T_{\text{mat}}$

$\rightarrow$  Probably more out-of-flux contamination and localized corrosion (called CILC in PWRs - Crud-Induced Localized Corrosion)

$\rightarrow$  Experiments to be conducted to measure corrosion rate of RAFM steels (different manufacturing processes) and Cu-base alloys up to  $T_{\text{sat}}$  (350 °C for Eurofer), even higher

NB: Higher corrosion rate at 300 °C for Eurofer (low Cr content) compared to stainless steel

(e.g. see [M. Molinari's presentation](#))



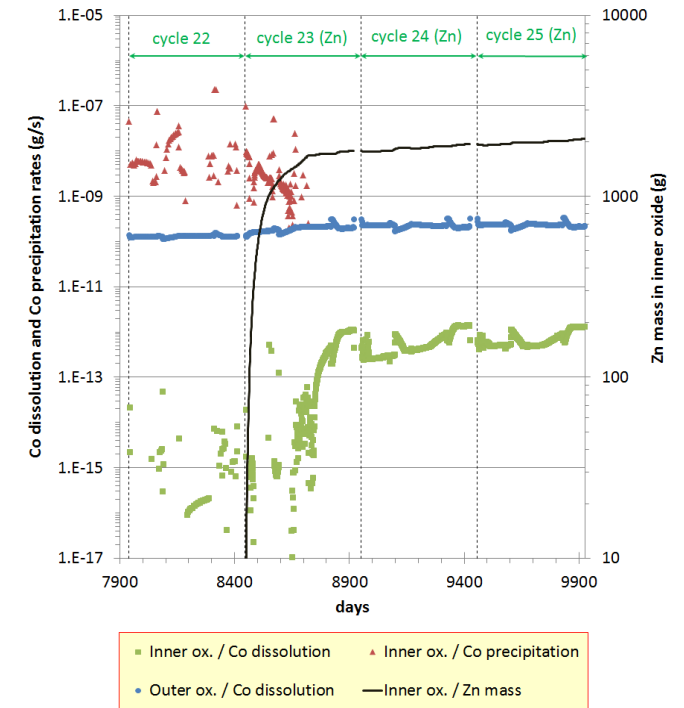
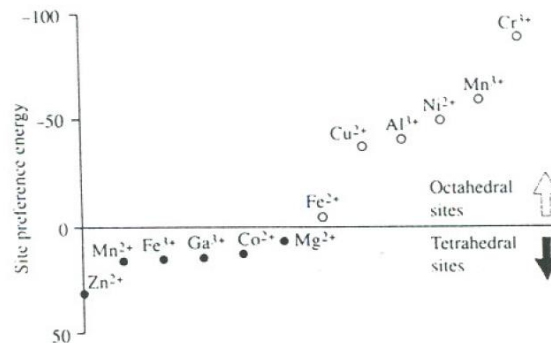
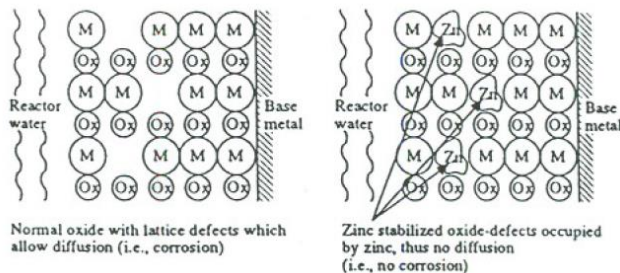
# Challenges related to the interactions between coolant and materials

- Colloidal behaviour in PHTSs of fusion reactors to be studied → see C. Cherpin's presentation
- Higher neutron flux (14 MeV)
  - Recoil energy for reaction products
  - Release of ACPs
  - Model to be introduced in **OSCAR-Fusion**, as was the case with **PACTITER**
- Plasma pulsed mode + High neutron flux (high radiolysis) + High flow velocity (~14 m/s for CuCrZr)
  - Cyclic slightly oxidizing environment despite H<sub>2</sub> addition (to be studied)
  - Flow Assisted Corrosion (FAC) [Obitz, 2016]
  - Effects of erosion-corrosion can be taken into account in **OSCAR-Fusion**
  - Probably more out-of-flux contamination and impact on material integrity



# Challenges related to the interactions between coolant and materials

- Intense magnetic field
  - Probably impact on corrosion and deposition of ferrimagnetic particles (ferrites)
  - Study currently being conducted by **M. Molinari** (La Sapienza University) for **OSCAR-Fusion**
- Benefits of injecting Zn in PHTSs?
  - Under certain conditions in PWRs, reducing corrosion and  $^{60}\text{Co}$  incorporation into chromites
  - Probably not for Eurofer and Cu-base alloys (low Cr content)
  - Experiments to be conducted (Zn impact on corrosion rate)
  - Zn simulated using **OSCAR**





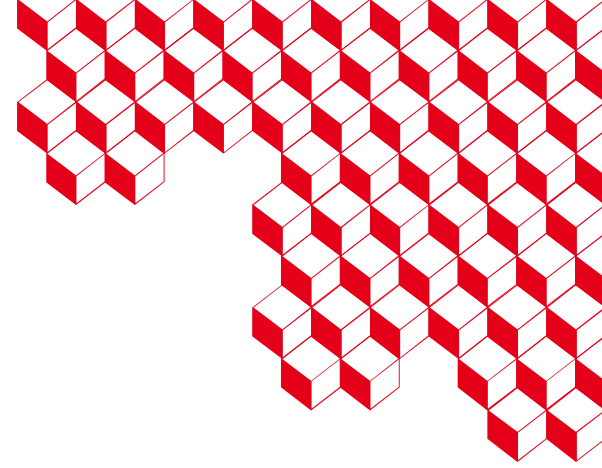
# 3 ■ Conclusion



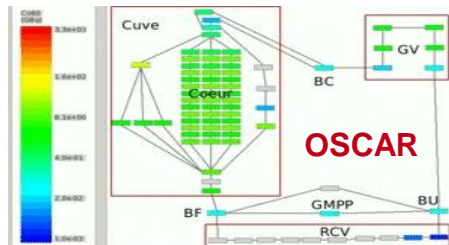
- **OSCAR**: Tool for simulating contamination in primary and auxiliary systems of PWRs
- **OSCAR-Na**: **OSCAR** version for SFRs
- **OSCAR-Fusion**: **OSCAR** version for TCWS PHTSs of ITER and DEMO
- **OSCAR-Fusion** benefits from validation of **OSCAR** in a wide range of PWR conditions
- Distinct features of fusion reactors → Challenges related to the interactions between coolants and materials (RAFM steels and Cu-base alloys) to be addressed:
  - Subcooled nucleate boiling
  - High neutron flux
  - High coolant velocity
  - Plasma pulsed operation
  - Magnetic field
  - Colloidal transport
- Perspectives
  - **OSCAR-LiPb** (for DEMO – EUROfusion task)
  - **OSCAR-SMR** (for Nuward)
  - **OSCAR-MSR**



iresne



**Thank you**



**CEA CADARACHE**

13108 St-Paul lez Durance Cedex

France

frederic.dacquait@cea.fr

+ 33 4 42 25 75 74