

Technical Meeting on the Safety Approach for Liquid Metal Cooled Fast Reactors and the Analysis and Modelling of Severe Accidents

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The Simplified Radionuclide Transport (SRT) Code for the Source Term Assessment of Pool-Type Metal Fuel Sodium Fast Reactor Severe Accidents

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Country of origin

USA

Organization

Argonne National Laboratory

Confirmation that the work is original and has not been published anywhere else.

Yes

Topic of the Abstract

Subtrack 2 Analysis and Modelling of Severe Accidents for LMFR

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