

# 2022 IAEA SMR ARIS Booklet on Advances in SMR Technology Developments

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# OUTLINE

## □ Characteristics and Attributes of SMR

- Major SMR technology lines
- Key attributes
- Salient design characteristics

## □ Advances on SMR Technology

- Global map of development
- Deployment horizon & Stage of development
- Designs of each category
- Examples of Fuel Cycle Approach and Waste Management Plan adopted by SMR Designs

# Small Modular Reactors (SMRs)

Advanced Reactors that produce typically up to 300 MWe, built in factories and transported as Modules to sites for Installation as demand arises.



**LARGE, CONVENTIONAL REACTOR**  
700+ MW(e)



**SMALL MODULAR REACTOR**  
Up to 300 MW(e)



**MICROREACTOR**  
Up to ~10 MW(e)

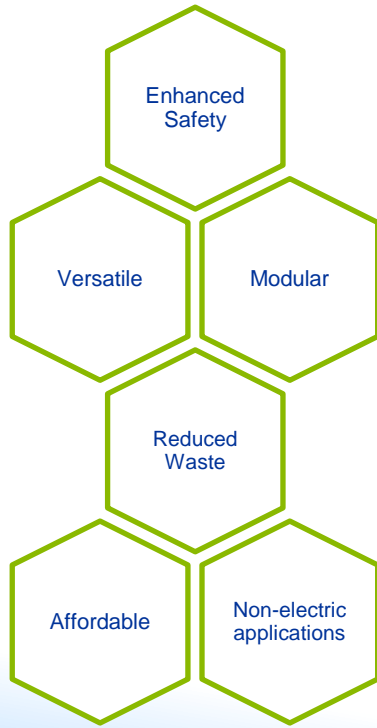


**Small:** in size, comparing to traditional reactors.

**Modular:** factory-manufactured, installed onsite.

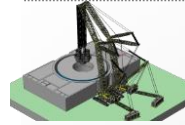
**Reactor:** energy generation via nuclear fission.

# Key attributes of SMRs



## Economic

- Lower Upfront capital cost
- Economy of serial production



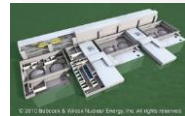
## Modularization

- Multi-module
- Modular Construction



## Flexible Application

- Remote regions
- Small grids

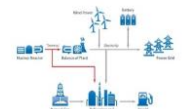


## Smaller footprint

- Reduced Emergency planning zone



## Replacement for aging fossil-fired plants



## Potential Hybrid Energy System

Better Affordability

Shorter construction time

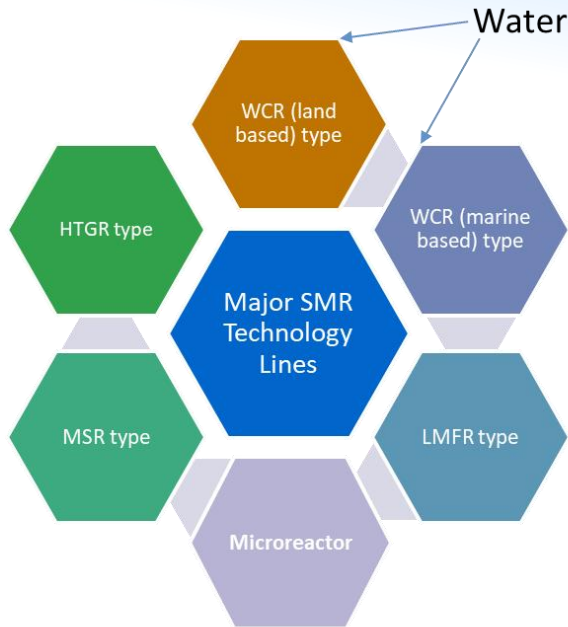
Wider range of Users

Site flexibility

Reduced CO<sub>2</sub> production

Integration with Renewables

# Major SMR technology lines



## Microreactor (U.S. DOE Glossary)

**Compact reactors** that will be small enough to transport by truck and could help solve energy challenges in a number of areas. They are capable of producing 1-20 Megawatts of thermal energy used directly as heat or converted to electric power.

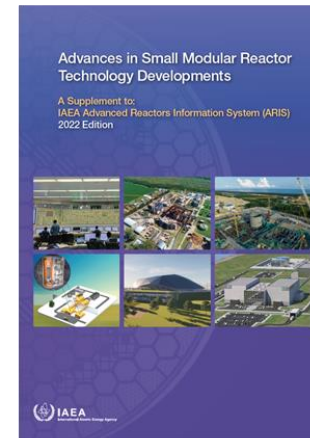
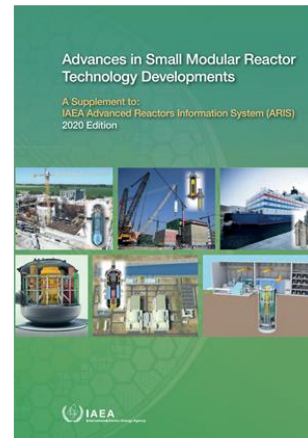
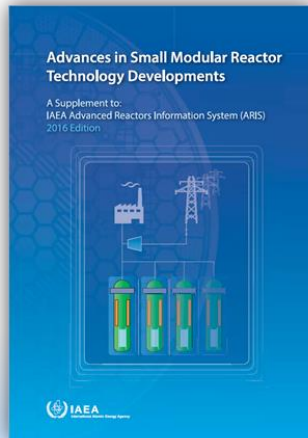
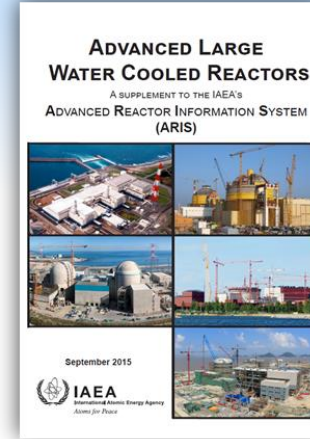
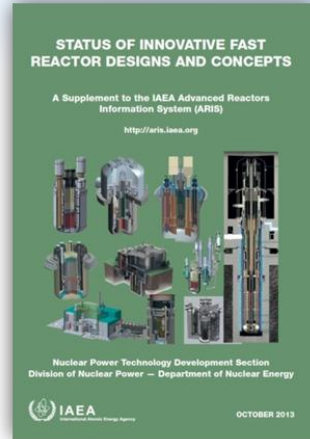
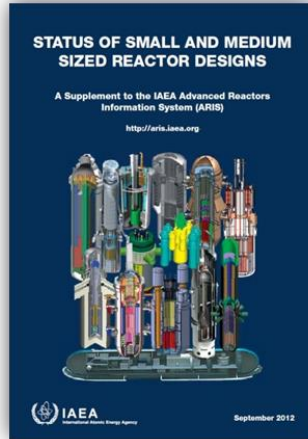
Ref. [What is a Nuclear Microreactor? | Department of Energy](#)

## Other Descriptions of Microreactor

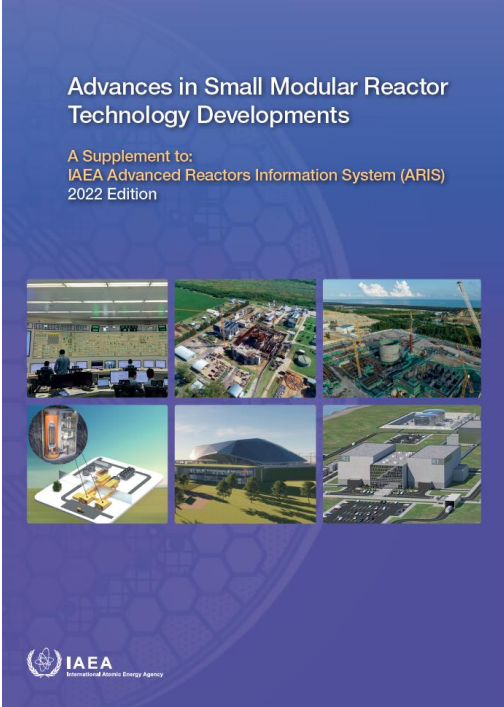
- 1) **A subset of small modular reactors** of 1-20 MWe capacity;
- 2) Advanced reactors of power up to 10MWe, **with common features** including modularity, passive safety, flexibility, simpler designs, more factory-based manufacturing possibilities, reduced site construction time and easier and more cost-effective reproduction, etc.



Notes: Commonly seen but lacking consensus.



↑ 2022 edition to be publicly released soon.



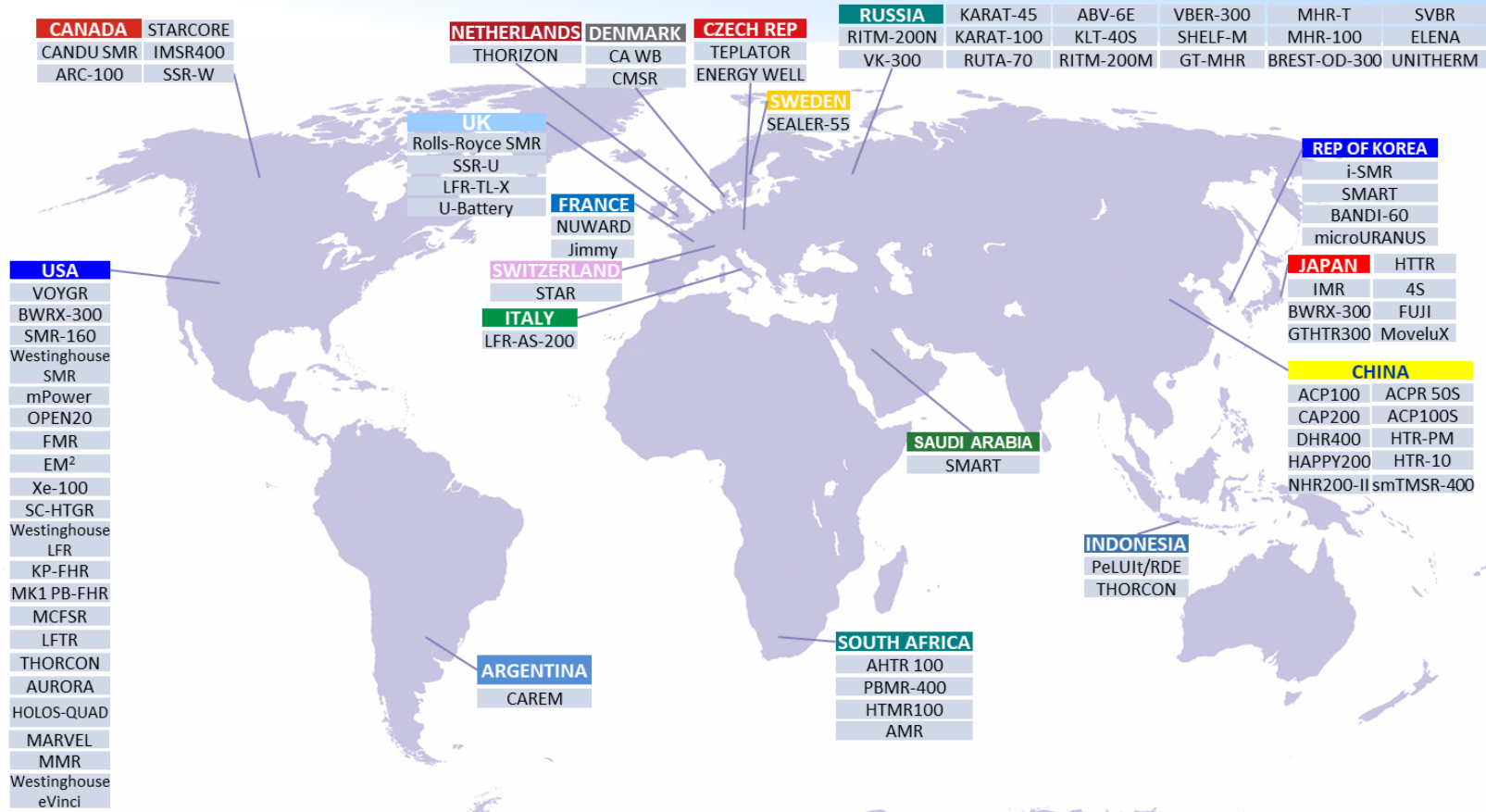
IAEA SMR Booklet, 2022 Edition	
Number of reactor designs:	<b>83</b>
Member states involved:	<b>18</b>
Reactor types	<ol style="list-style-type: none"> <li>1.1. Water-cooled Land Based – <b>25</b></li> <li>1.2. Water-cooled Marine Based – <b>8</b></li> <li>2. High temperature Gas-cooled – <b>17</b>, including <b>3</b> HTGR-type test reactors</li> <li>3. Liquid Metal-cooled Fast Neutron Spectrum – <b>8</b></li> <li>4. Molten Salt – <b>13</b></li> <li>5. Microreactors – <b>12</b></li> </ol>
Distinguishing features	<ul style="list-style-type: none"> <li>• New annexes on <b>economic challenges, decommissioning, and experimental testing for design verification and validation</b></li> <li>• Insightful annexes with various charts and tables</li> </ul>
Status	Finished, submitted for publication.
Downloadable version	Coming soon.

The 2022 IAEA SMR ARIS Booklet is a biennial publication as a supplement to the IAEA Advanced Reactor Information System (ARIS) Database. It provides a brief yet comprehensive design description of 83 different reactor designs. The 2022 version is an updated version of the 2020 booklet. It includes 11 more designs and a more comprehensive set of annexes.

# Global map of SMR Technology Development

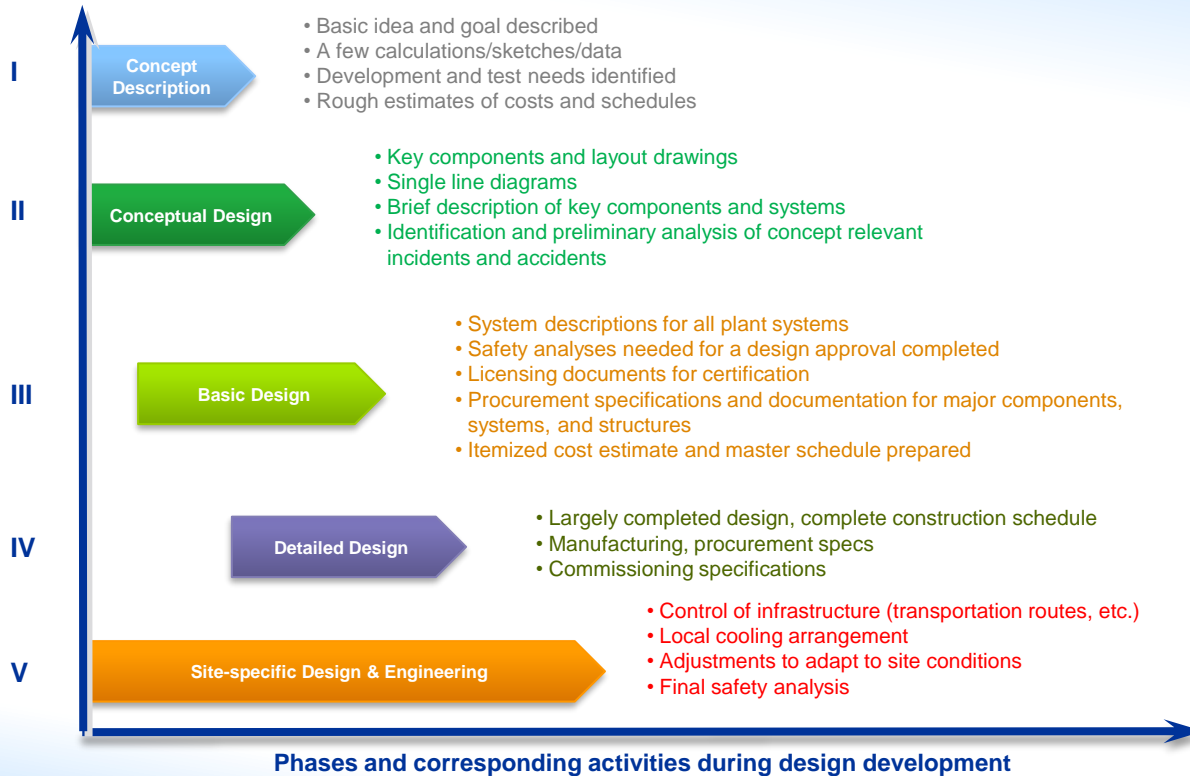


IAEA

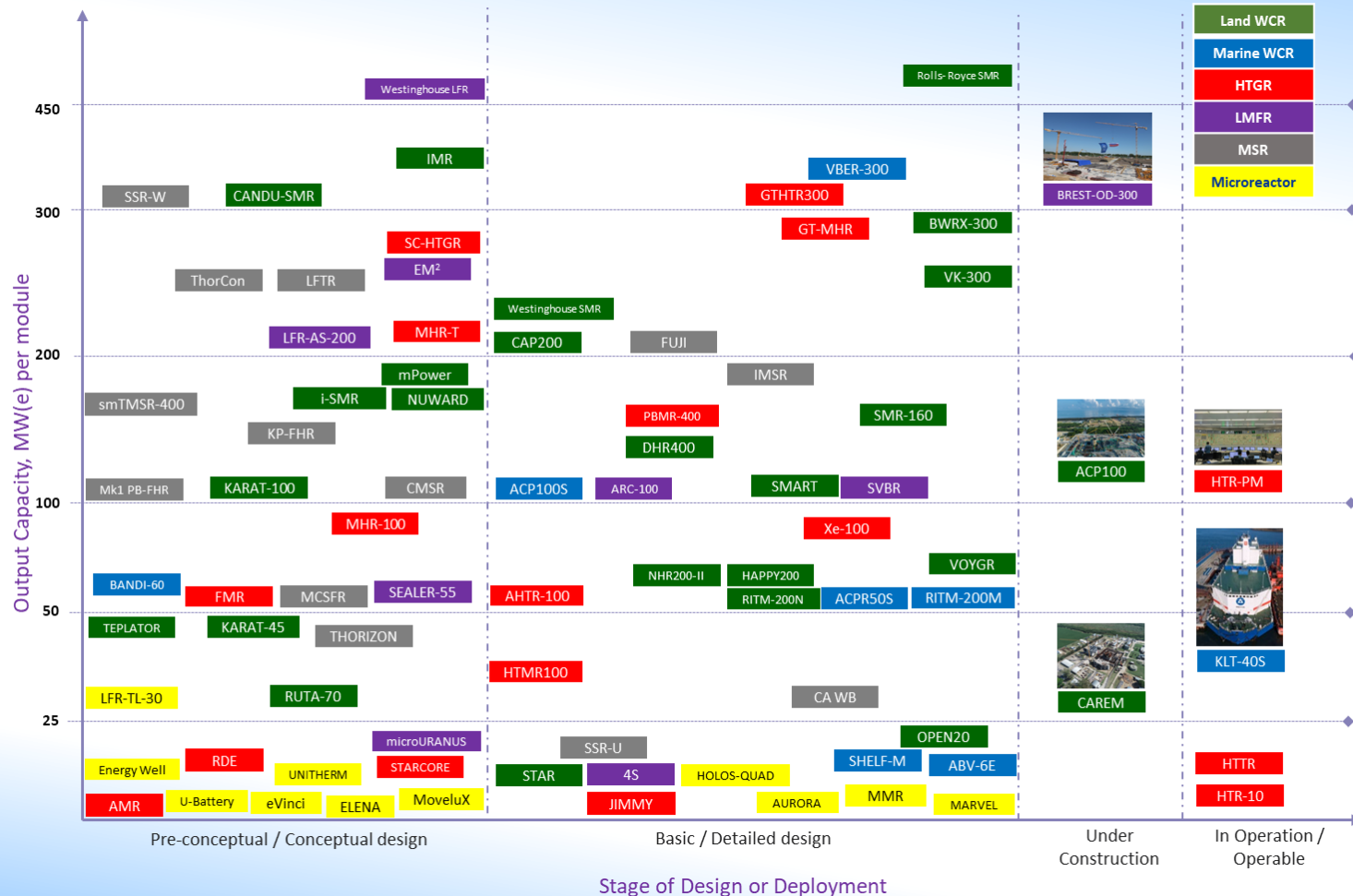




# Design development phases

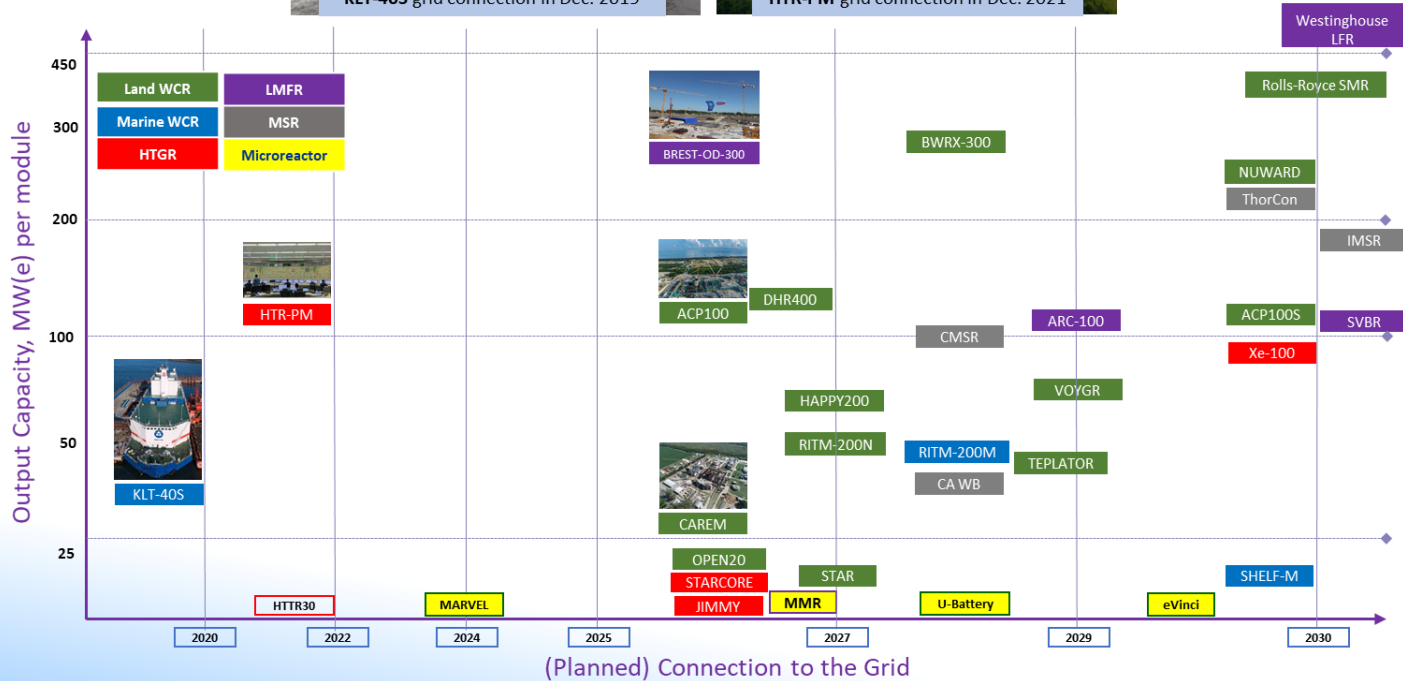


# Stage of development or deployment of SMRs

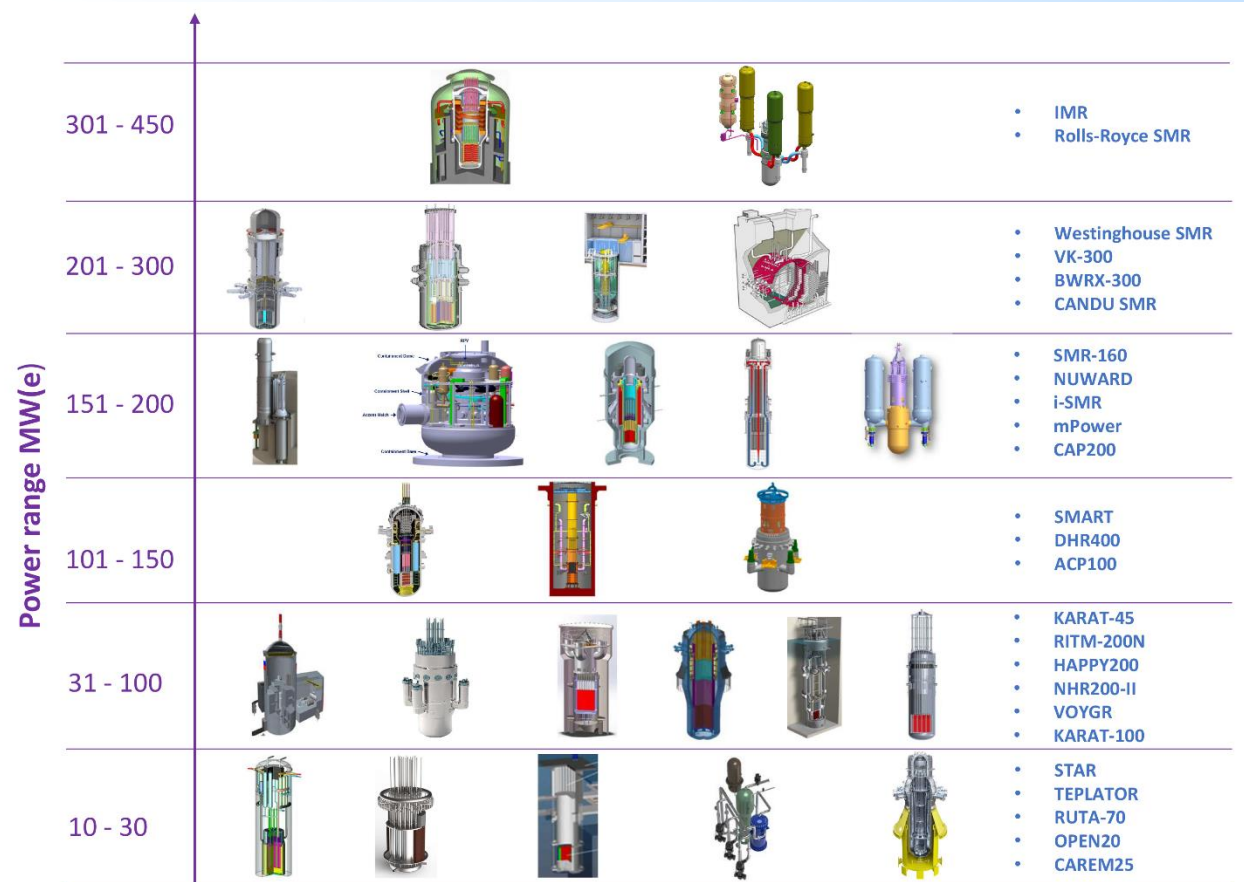


# Stage of development or deployment of SMRs

The Forerunners: 2 in operation, 3 under construction. More target at deployment by 2030

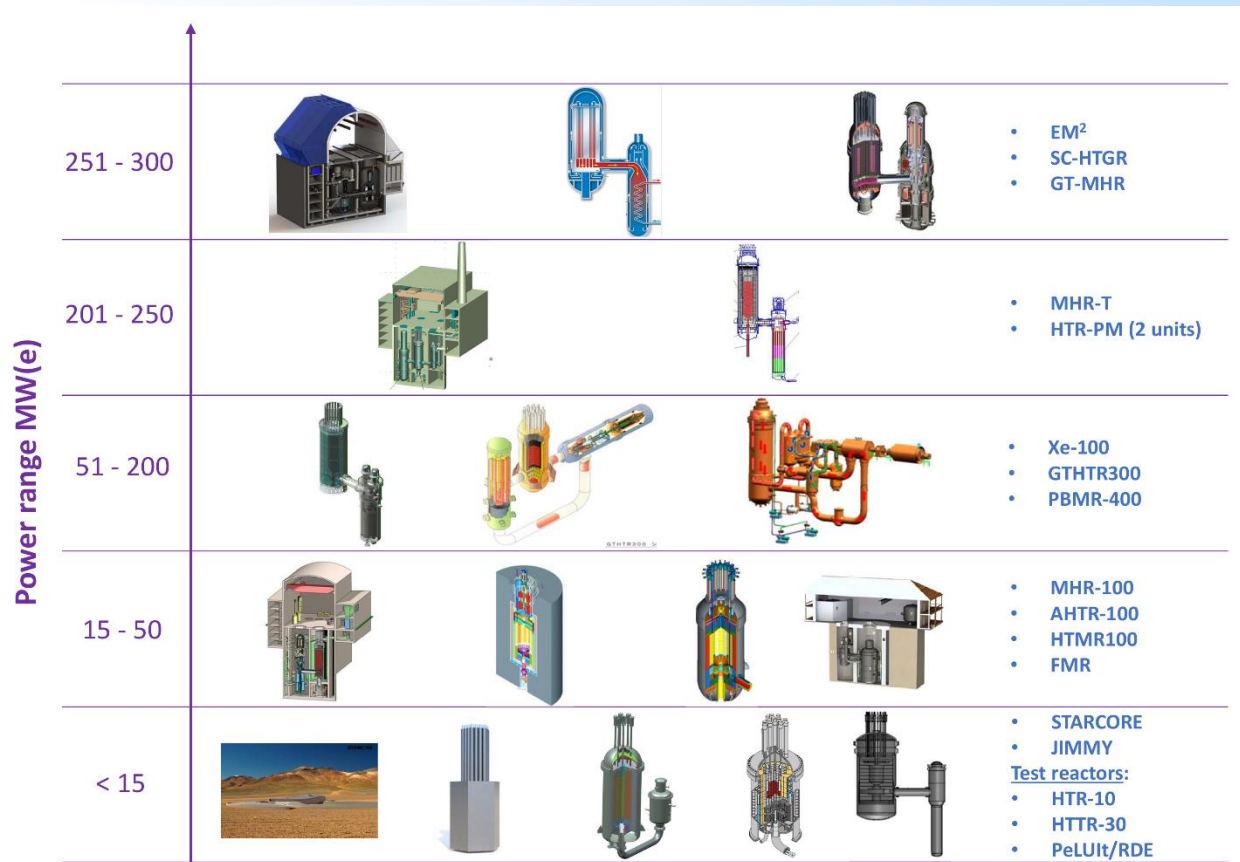


# Power range of SMRs of each category



Water-cooled reactors (land-based)

# Power range of SMRs of each category



High temperature gas-cooled reactors

# Specific characteristics and applications of microreactors

- Inherent and passive safety features
- Substantially lower upfront capital costs
- Much smaller footprints, reduced-sized or even eliminated EPZ
- Rapid deployability from modularity (even an entire reactor)
- Spent fuel smaller in size (probably easier to manage)
- Scalability, Resiliency, Self-regulating
- Potential to operate in island-mode & to black-start
- High transportability from mobility
- Long refueling interval

Powering secure micro-grids for critical infrastructure

Powering remote off-grid areas

Restoring power quickly

Seawater desalination

Naval: powering nuclear submarines and UUV

Space: powering spacecraft, supporting manned exploration

Replacement for diesel gensets

Remote mining operations: offer reliable power source

Hydrogen production: feasible for HTGR type

Integration with renewables: e.g. integrating solar panels

# Power range of SMRs of each category



Microreactors

# Comparison of Main Characteristics among Some HTGR-type SMR Designs

	HTR-PM	GTHTR300	GT-MHR	HTMR100	Xe-100	SC-HTGR	EM <sup>2</sup>
Country of Origin	China	Japan	Russian Federation	South Africa	USA	USA	USA
Design organization(s)	INET, Tsinghua University	JAEA	JSC "Afrikantov OKBM"	STL Nuclear (Pty) Ltd.	X-energy, LLC	Framatome Inc.	General Atomics
Reactor type	Modular pebble bed HTGR	Prismatic HTGR	Modular Helium Reactor	Pebble-bed HTGR	Modular HTGR	Prismatic HTGR	Modular high temperature gas-cooled fast reactor
Fuel materials	TRISO spherical elements with coated particle fuel	UO <sub>2</sub> TRISO ceramic coated particle	Coated particle fuel in compacts, hexagonal prism graphite blocks	TRISO particles in pebbles; LEU/Th	UCO TRISO/pebbles	UCO TRISO particle fuel in hexagonal graphite blocks	UC pellet / hexagon
Coolant	Helium	Helium	Helium	Helium	Helium	Helium	Helium
Moderator	Graphite	Graphite	Graphite	Graphite	Graphite	Graphite	N/A
Thermal output, MW(t)	2 x 250	< 600	600	100	200	625	500
Electrical output, MW(e)	210	100 - 300	288	35	82.5	272	265
Core inlet temp., °C	250	587 - 633	490	250	260	325	550
Core outlet temp., °C	750	850 - 950	850	750	750	750	850
Enrichment, %	8.5	14	14-18% LEU or WPu	10%	15.5	14.5 (avg) 18.5 (max)	~14.5 (LEU)
Core Discharge Burnup (GWd/ton)	90	120	100-720 (depends on fuel type)	80 - 90	165	165	~130
Refuelling cycle, months	Online refuelling	48	25	Online fuel loading	Online fuel loading	½ core replaced every 18 months	360
Reactivity control	Control rods	Control rods	Control rods	Control rods in the reflector	Control rods	Control rods	Control rods
Reactor Vessel's height/diameter, (m)	25 / 5.7 (inner)	23 / 8	29 / 8.2	15.7 / 5.6	16.4 / 4.88	24 / 8.5	12.5 / 4.6
Design status	In operation	Basic design	Preliminary Design completed	Basic Design	Basic Design	Preliminary Design	Conceptual design



# Fuel Cycle Approach adopted by HTGR-type SMRs

- Open fuel cycle: The majority.  
e.g., HTR-PM will adopt close fuel cycle in the future.
- Longer refuelling cycle, e.g., GTHTR300, GT-MHR
- Fuel enrichment: all in the range of (5 – 19.75)%
- Use of Th cycle and/or Disposition of Pu: e.g., GTHTR300 (applicable), MHR-100 (possible), PBMR<sup>®</sup>-400 (flexible), EM<sup>2</sup> (capable).
- Use of Spent Fuel as Fuel: e.g., GTHTR300 (applicable), EM2 (feasible)

- Volume Reduction and Conditioning:

Coated particle separation from graphite will reduce volume by up to a factor of 100.

- Waste Processing:

- Low and intermediate level waste from plant operation will be conditioned by different process technologies.

- Possible graphite recycling,  $^{14}\text{C}$  separation process

- Storage Approach, Spent Fuel Pool Cooling Mechanism:

- With higher thermal efficiencies the radiotoxicity and decay heat will be lessened by 50% for HTGRs as compared to LWRs.

- Dry storage with natural convection after short material active cooling.

- Facilities for long-term storage of spent fuel and solid radwaste are in the NPP complex

- Spent Fuel Take-back Option: to date not considered in HTGRs.



# IAEA

International Atomic Energy Agency  
*Atoms for Peace and Development*



8 December 1953



1 to 23 October 1957



11 December 1957



1959



10 December 2005



1958 to 1979



23 August 1979

## *Thank you for your attention!*

**For inquiries, please contact:**

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