





FUR PLASMAPHYSIK

Development of the Fenix flight simulator for **DEMO transients scenarios**

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Fenix scope



• Fenix tokamak plasma "flight simulator" [F. Janky et al., FED 2017; E. Fable et al., PPCF 2022]

• Complete for ASDEX Upgrade (AUG), magnetic control in progress for the EU-DEMO [G. Federici et al., NF 2019], with kinetic control fully working

• Goals:

- Check Pulse Schedule (PS) trajectories consistency *including* the full plasma response
- Check if PS satisfies physics goals (H-mode, density, detachment, etc etc)
 - \rightarrow relies on quality of implemented physics models
- Develop/improve the plasma control schemes
- Develop/improve reduced physics models for the plasma dynamics
- Help in scenario development (as it is being used for the EU-DEMO)

•Requirements:

- be fast (as of now, can do a 10s AUG discharge in 3-5 minutes) \rightarrow but has flexibility in choice of models complexity if time not an issue
- be realistic in the plasma model (profile effects, non-linearities)
- embed plasma into control system and machine response
- user friendly (e.g. to be used in the control room)
- modularity but not at all levels (logic of modularity is given by the interfaces)

Fenix structure

• SimulinkTM as "hosting software" \rightarrow PCSSP (ITER platform for the control system) as the embedding framework [M. L. Walker et al., FED 2015]

- Controllers, actuators are modeled in SimulinkTM \rightarrow can be made arbitrarily realistic
- Whatever enters or leaves the plasma \rightarrow only 1 interface via a single S-function "Device"



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I/O between control sys and plasma

- The S-function is an C interface to the ASTRA transport code
- \bullet ASTRA computes the dynamics of the plasma (Grad-Shafr. + circ. Eqs. + transport eqs)
 - D-pellet particle source [1/s] (CPEL1) Astra time [s] (TIME-TSTART) beta (BETP3R(ROC)) Internal inductance (LI3R(ROC)) Pump speed [m^3/s] (CV13) Plasma currrent [A] (~IPL) Loop voltage at plasma boundary [V] (UPL(NA1)) Radiated power [MW] (QRADR(ROC)) 3 NTM trigger [1=ON, 0=OFF] (CDMJ1) Separatrix power [MW] (QTOKR(ROC)) Greenwald fraction (CRAD4) H factor (CMHD2) NTM poloidal number (CDMJ2) Power to divertor [MW/m^2] (CSCL4) 10 Divertor temperature [eV] (CDMW1) 11 L-mode=0 : H-mode=1: I-mode=2 (ZRD15) 12 Tungsten density in divertor [1e19/m^3] (CDJM6) 13 NTM toroidal number (CDMJ3) Tungsten density in SOL [1e19/m^3] (CDJM1) 14 Neutral divertor pressure (CDIM4) 15 Separatrix electron temperature [eV] (CDJM7) 16 NTM seed width [rho_tor] (CDMJ4) Tungsten influx from the limiters (CHE3) 17 NTM width w/a (CDMJ5) 18 NTM amplitude 5000*(w/a)^2 (CDMJ6) 19 7 TS06 [s] (ZRD84) Island position r/a (CDMJ7) 20 Electron temperature profile [keV] 21 Electron density profile [1/m^3] 22 8 ECRH power [W] (CAR32(1:8) [MW]) Z eff profile 23 Tungsten density profile (F4) 24 Coil force R 25 Coil force Z 26 ECRH position (CAR32(9:16)) Ion temperature profile [keV] 27 Ion heat conductivity (XI) 28 Electron heat conductivity (HE) 29 10 NBI power [W] (CAR32(17:24) [MW]) Electron absorbed power [MW/m^3] (PE) 30 Ion absorbed power [MW/m^3] (PI) 31 Fast ion density profile [MW/m^3] (NIBM) 32 11 ICRH power [W] (CAR32(25:26) [MW]) Radiated profile [MW/m^3] (PRAD) 33 Coil currents (CCOIL) 34 Magnetics (GEOM1D) 35 L2 Coil voltges (~ VCOIL) Pumped gas (CAR34) 36 Separatrix contour 37 ASTRA internal crash flag to stop Simulink 38 13 Gas flow (CAR33(1:24)) ASTRA
- Inputs: actuator signals
- Outputs: synthetic diagnostics



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EU-DEMO setup



- EU-DEMO considered has the following parameters:
- R = 8.94 m, a = 2.94 m, B_T = 4.89 T, I_p = 19.07 MA, $\langle n_e \rangle_V \sim 8.5 \ 10^{19} \ m^{-3}$
- $P_{fus} \sim 2 \text{ GW}$, $P_{rad} \sim 250 \text{ MW}$, $P_{aux} \sim 0.50 \text{ MW}$, $H_{98(y,2)} \sim 1$ (standard H-mode)
- Kinetic control (quantity \rightarrow actuator):
 - (electron) Density of the pedestal top (e.g. $x \sim 0.95) \rightarrow$ pellet injector
 - Density at the separatrix \rightarrow DT gas puff from mid-plane valve
 - Power crossing separatrix \rightarrow Xe puff from mid-plane valve
 - Divertor cooling (to be specified) \rightarrow Ar puff from the divertor value
 - Fusion power output \rightarrow central heating or density level
 - MHD events (sawteeth/NTMs) \rightarrow dedicated ECCD launcher(s)
 - others \rightarrow to be discussed
- All these control modes are implemented via PID SISO controllers.
- Magnetic control (of plasma equilibrium and coil currents) is done independently
- For now diagnostics are considered "at will" and "ideal"

Critical aspects of DEMO control

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• Optimization of the diagnostics coverage (being studied inside WPDC) – limited due to several reasons (port access, neutron flux, ...)

• High fusion output scenario works close to some limits (density limit, L-H transition power, detachment regime, peeling-ballooning limit, ...)

• Time scales and coupling between actuators and plasma introduce strong latencies and non-linear response (mostly due to the plasma size and the dominance of the α -power over the auxiliary power)

• Possible failures could lead to catastrophic plasma termination \rightarrow address each one of them and define a "SCRAM" scenario that minimizes damage to internal components

• In general, it is desirable to plan a scenario where the probability of having to use the "SCRAM" scenario, or that leads to a potential disruption, be minimal \rightarrow balance between enough fusion output and safe scenario \rightarrow control the plasma to stay on the "safe" trajectory, with consideration or "shaded area" around the nominal trajectory

Focus of this work



- Evidence the main sources of perturbations during the normal operation, in the plasma flat-top phase
- Describe the way this plasma is controlled
- Show examples when major perturbations happen
- Simulations of ramp-up scenarios
- Considerations on ramp-down modeling

Main sources of perturbation during normal flat-top operation



- 2 Periodic MHD activity (sawteeth) \rightarrow should disappear in DEMO (consideration of hybrid scenario)
- 3 Pellet injection: ideally periodic, but in practice non-periodic as pellet failure rate is > 0% ($\sim 10\%$ realistically)
- 4 Fluctuations in delivered gas puff (not yet included)
- 5 ELMs (Edge Localized Modes) \rightarrow dangerous for the divertor, should be avoided

 \rightarrow For the design of the pellet injection scheme in DEMO, the point 3 has been thoroughly investigated



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Major perturbations

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- Ab-normal events that have been considered:
- 1) Failure of an auxiliary heating system leading to $P_{sep} < P_{LH}$
- 2) W flake enters the plasma (4, 8, 12 mg)
- 3) Unwanted $H \rightarrow L$ transition

Major perturbations



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Major perturbations



2) W flake enters the plasma (4, 8, 12 mg)



Ramp-up scenarios

• The plasma ramp-up serves as both the plasma scenario "roadmap" and also as a mean to optimize some parameters along the operational phase (previous work performed for DEMO by P. Vincenzi and T. Bolzonella)



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Ramp-up scenarios

• The "orange" and "blue" trajectories are the best ones in terms of leading to the smallest excursions

 \bullet The "blue" trajectory has a decrease in loop voltage (UPL) at earlier times \rightarrow best flux consumption



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Preliminary ramp-up with magnetic control



• Added magnetic controllers [work in collaboration with M. Mattei and L. Di Grazia, CREATE Group]



Power flux during ramp-up

- Before "ignition" (that is, ramping-up of the alpha power) \rightarrow power flux to divertor expected to be in the order of ~ 1 MW*T/R, but separatrix density limited to 0.5*Greenwald
- During ignition, separatrix power rises fast, and density as well (but only at pedestal top) \rightarrow separatrix density fixed. Needs to find a good recipe for SOL



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Considerations on the ramp-up

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• It is still unknown how much W will actually enter and stay in the plasma during the ramp-up, but it is estimated that at concentrations > 1.e-5, it will be required to add auxiliary central ECH to avoid radiation collapse of the plasma core, due to the higher central electron temperature present in DEMO with respect to present machines (expected concentrations varying between 1.e-5 to 5.e-5)

• The evolution of the safety factor q can be crucial for the confinement properties during the flat-top phase. In particular, a "freezing" of the evolution of the q-profile due to the use of ECH in the ramp-up, could lead to non-optimal q-profile shape in the flat-top

• The ramp duration vs heating and fueling scheme needs to be optimized to enter into flat-top, e.g. the burning plasma phase, with a satisfactory core confinement

• The role of the impurities for divertor protection during the ramp-up become critical at the L-H transition. Is it possible to perform the DEMO ramp-up in detached L-mode?

• Compatibility with external elements boundaries of operation (e.g. turbines characteristics, cooling system, inertia of elements, etc)

Considerations on the ramp-down



• Similar arguments apply to the ramp-down:

- exit from the burning phase and the interplay with the impurities needs to be studied in a dedicated fashion

- optimization of the ramp to avoid too much excursion of the internal inductance (plasma equilibrium stability issue)

- SCRAM scenario (fast ramp-down to avoid disruptive situation) \rightarrow needs design

- compatibility with constraints on external elements limits (e.g. ramp rate)

Conclusions and outlook



• The Fenix flight simulator, coupling the plasma dynamics to the external control system, allows to predict the evolution of the DEMO scenario including the feedback of the plasma on the actuators and viceversa

Its applications so far have been in determining requirements for the control system, including some elements of realism in the actuators

 → for diagnostics, a study has been carried out on the Prad sensitivity

• Application to transient scenarios are becoming increasingly complex and realistic, towards the full definition of a self-consistent DEMO scenario



THANK YOU