

CAPABILITIES OF IFMIF-DONES AND STATUS OF THE PROJECT

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IFMIF-DONES is the EU proposed research infrastructure to irradiate the materials to be used in a fusion reactor. The facility would provide a unique neutron source of energy spectrum and flux level representative of those expected for the first wall of the future fusion reactors

Engineering design

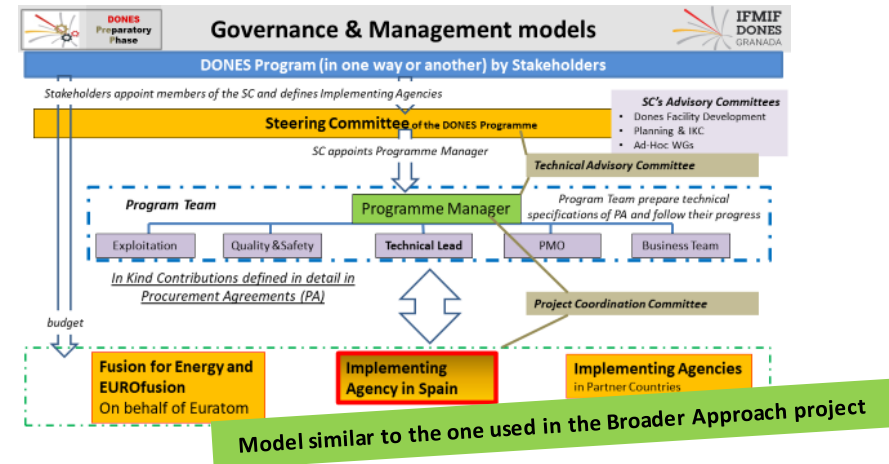
Implementation

Engineering: WPENS Timeline

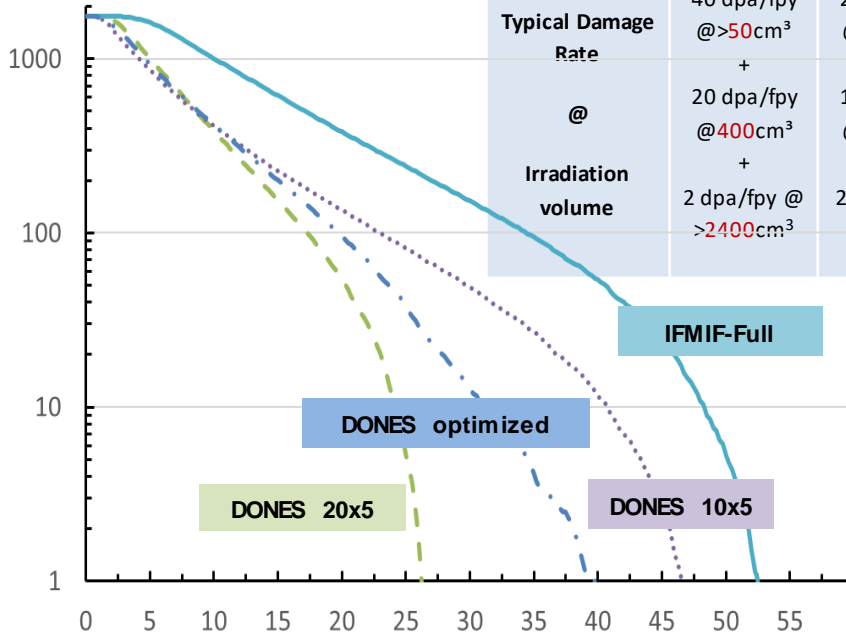


- Developed for IFMIF (and a generic site) up to 2013 in the framework of IEA and BA (IFMIF/EVEDA)
- Developed for IFMIF-DONES from 2015 in the framework of the ENS WP of EUROfusion

- A complete planning of the project has been prepared (Project Plan, Time Schedule, Cost Estimate, Risk Analysis, Quality, Project Lifecycle, Licensing...)
- Proposals for Project implementation, governance and management, sharing of contributions... have also been agreed

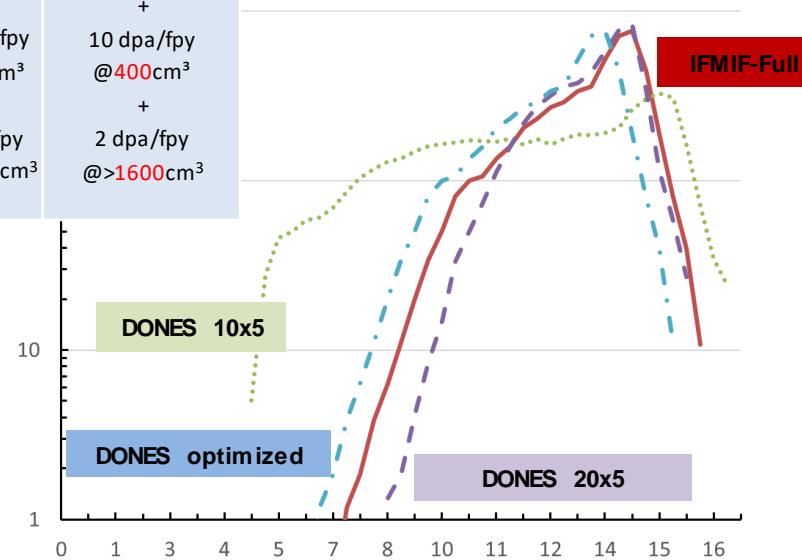


Available Irradiation Volume (cm³) vs DPA (dpa/fpy)



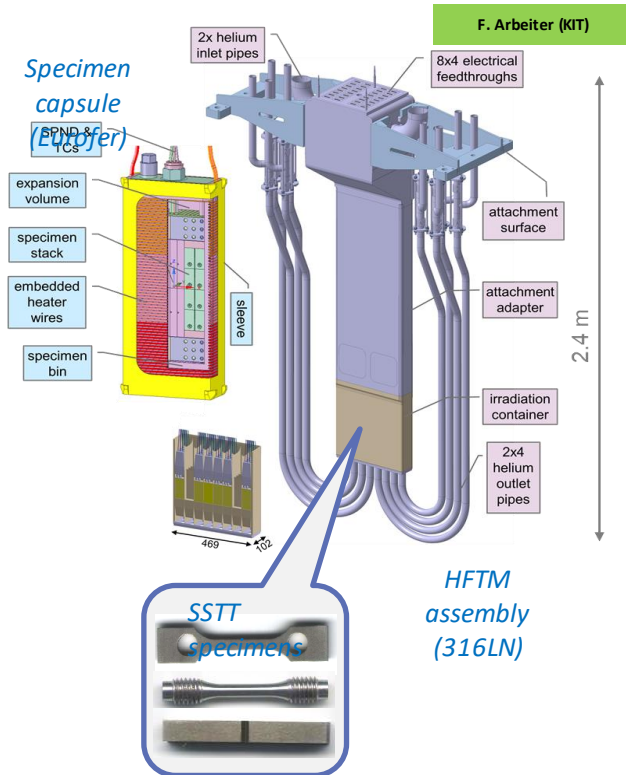
| | IFMIF-Full | IFMIF-DONES | | |
|----------------------|-----------------------------------|----------------------------------|-----------------------------------|-------------------------------------|
| Beam footprint | 20 x 5 cm ² | 20 x 5 cm ² | 10 x 5 cm ² | Optimized (15 x 5 cm ²) |
| Typical Damage Rate | 40 dpa/fpy @ >50 cm ³ | 20 dpa/fpy @ >50 cm ³ | 20 dpa/fpy @ >130 cm ³ | 20 dpa/fpy @ >90 cm ³ |
| @ Irradiation volume | 20 dpa/fpy @ 400 cm ³ | 10 dpa/fpy @ 400 cm ³ | 10 dpa/fpy @ 400 cm ³ | 10 dpa/fpy @ 400 cm ³ |
| | 2 dpa/fpy @ >2400 cm ³ | 2 dpa/fpy @ 1800 cm ³ | 2 dpa/fpy @ >1500 cm ³ | 2 dpa/fpy @ >1600 cm ³ |

Available Irradiation Volume (cm³) vs He/dpa ratio (HE appm/dpa)



Objective: to irradiate a large volume of SSTT samples in the high flux region of DONES
First-to-be-installed irradiation experiment (critical path).

- Heating: Nuclear **2.3 W/g peak**, 17 kW tot., 1.5 kWe per capsule
- Cooled by **low pressure helium gas** (0.3MPa, 50°C), **Sodium** heat transfer filler
- Lifetime: 1year / 2.5 years (**53 dpa**). Body made from **316LN** (acc. RCC-MRx)
- Masses: Total 680 kg, 40 kg irradiation capsules with specimens



Steel irradiation

- 12–25 dpa/fpy in 306 cm³ (~ 850 specimens)
- 13 appm He / dpa, 53 appm H / dpa.
- 250 – 550 °C, sodium immersed specimens

Copper irradiation (divertor heat sink)

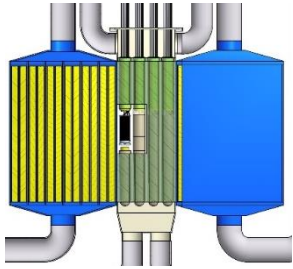
- 5–30 dpa/fpy
- 6–8 appm He/dpa is (~DEMO), 48–50 appm(H)/dpa (~1.4x DEMO)
- >100°C, helium immersed specimens

Tungsten irradiation (armor)

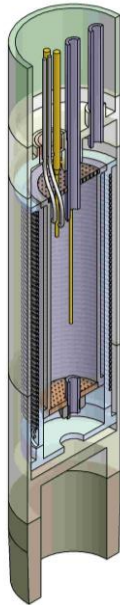
- Up to 800°C, assisted by self-heating
- 8x20 cm³ (cylindrical HT capsules)
- 1–3 dpa/fpy in W
- 9–10 appm He / dpa, (2x of DEMO), 20–29 appm H / fpy, (3x of DEMO)

Other materials or other irradiation modules can be also installed (sequentially or simultaneously to the HFTM)

A “catalogue” of possible irradiation modules has been developed and preliminary conceptual designs are available for some of them:

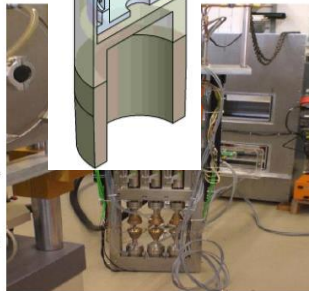
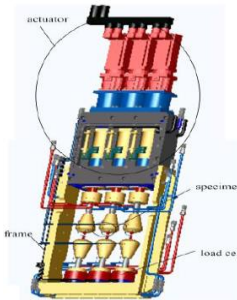


**In-situ ceram. breeder testing
KIT concept**

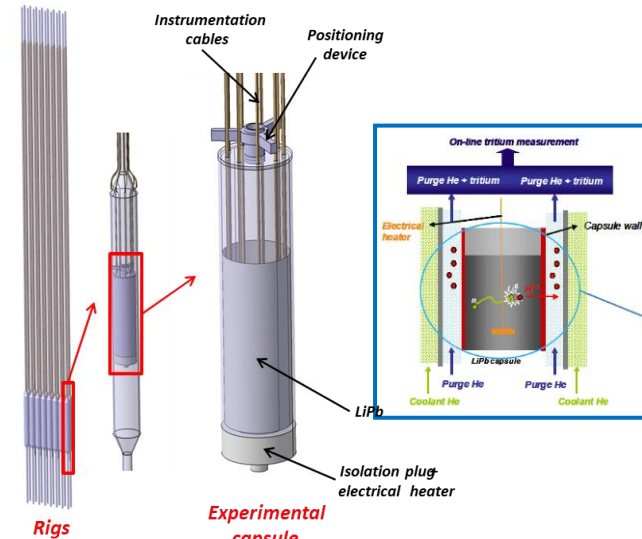


- In-situ Creep Fatigue
- In-Situ Tritium Release
- Liquid breeder validation Module
- Diagnostics irradiation
- “Model Blanket Module”
- Tritium permeation / coatings
- Corrosion, IASCC
- Superconducting magnets
- Radioisotope production
- ...

**In-situ creep-fatigue
EPFL concept**



In-situ liquid breeder testing, CIEMAT concept



Several capsules with Pb-Li eutectic inside

**IFMIF-DONES Program is expected to start
Construction Phase in short time**