

Divertor Detachment In Negative-Triangularity PI 103 **Configurations In The TCV Tokamak**

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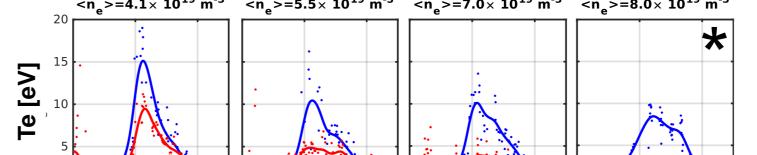
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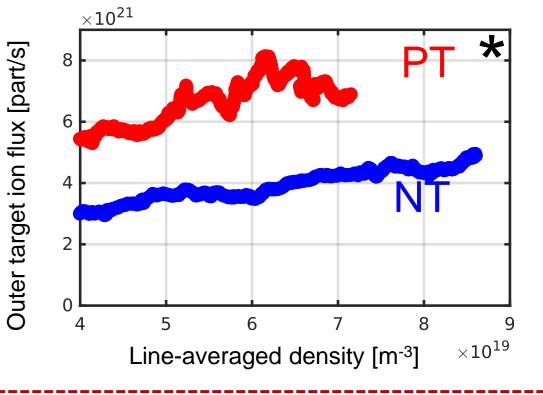
Introduction & Motivation

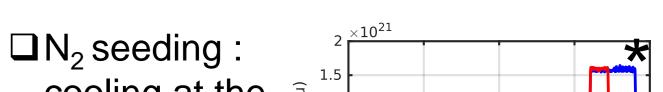
- □ To achieve good fusion performance, operation of future tokamaks is typically envisionned in **H-Mode** (High-Confinement) :
- \rightarrow ELMs (Edge Localized Modes), periodic relaxation of the pressure pedestal
- \rightarrow Large heat/particle-flux towards divertor: ELMs must be avoided
- □ Strategies :
- \rightarrow Mitigate the ELMs (suppress or reduce amplitude), try to buffer their impact on the divertor (« buffering »)
- \rightarrow Explore naturally ELM-free operation scenarios :

Outer target measurements □ Jsat roll-over at *outer* target observed in PT part/s] plasmas, not in the NT discharges \Box Reduction of OSP T_e and heat flux PT, not for NT □ At ISP, low Te < 10 eV in both cases, higher Jsat target peak in PT **Increasing line-averaged density** $< n_e > = 4.1 \times 10^{19} \text{ m}^{-3}$ $< n_e > = 5.5 \times 10^{19} \text{ m}^{-3}$ $< n_e > = 7.0 \times 10^{19} \text{ m}^{-3}$ $< n_e > = 8.0 \times 10^{19} \text{ m}^{-3}$



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ELM-free H-Mode High-performance L-Mode (Low-confinement mode, no transport barriers)

Negative Triangularity: a route to high-performance L-Modes ?

□ In recent years, renewed interest for negative-triangularity (NT) plasmas, with H-Mode grade confinement achieved in L-Mode **plasmas** (TCV, DIII-D) [1,2,3,4]

- □ From the power exhaust point of view :
 - \blacktriangleright No H-Mode \rightarrow no ELM mitigation required
 - \blacktriangleright No H-Mode \rightarrow No power threshold Allow increased core radiation \rightarrow lower power to the divertor
 - Strike-points at higher major radius Heat flux spread over wider area
 - \blacktriangleright No H-Mode \rightarrow Higher Scrape-Off Layer width ?

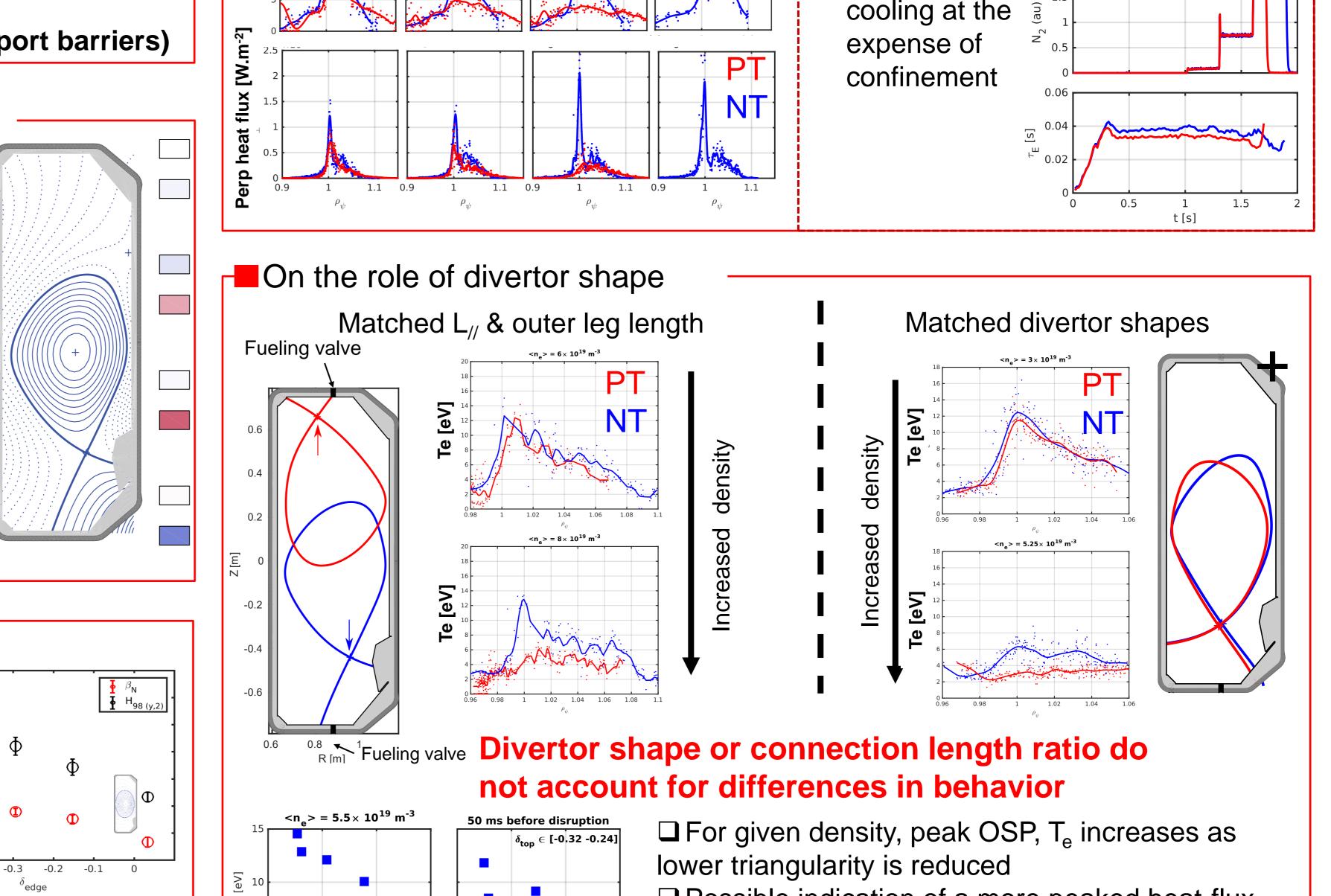
Operation in detached conditions still desirable

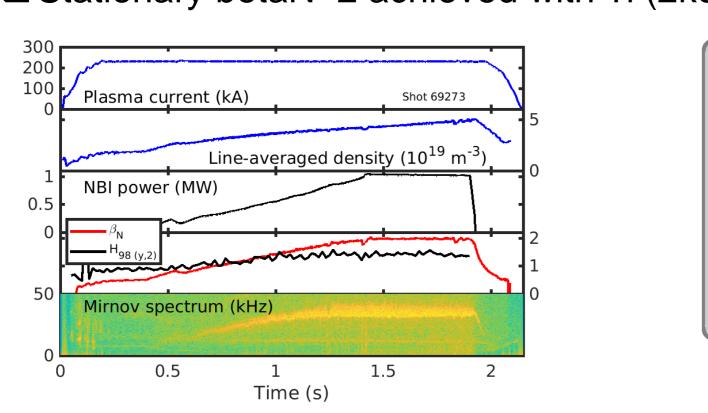
Confinement improvement in NT, a quick overview [2]

□ In ohmic, limited discharges, confinement increase with decreasing triangularity \rightarrow No saturation observed (Also true in diverted plasmas) [2]

□ High-performance (betaN=2.7) achieved in L-Mode NT

□ Stationary betaN=2 achieved with Ti (2keV) > Te (1.3keV)





□ H-Mode power threshold higher in **NT** compared to **PT** \rightarrow Larger L-Mode operation window

□ No significant difference found for current or L-Mode density limits

Plasma pressure (kPa) Neg. triang. L-mode P=0.4 MW, $\beta_{\rm N}$ =2.7 20 TER-like baselin 10 H-mode scenario $P=1.4 \text{ MW}, \beta_{N}=1.4$ Shot 64770, 0.9-1.4 s 0.8 δ < 0 δ > 0 Line-averaged density [m Dalpha [au] NBH Power [MW] 1.5

NT

Neutral pressure

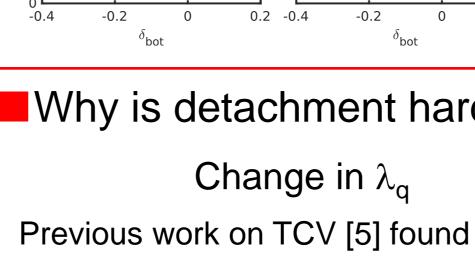
measurement

1.1 **D**

-0.4

Investigating power exhaust in NT plasma : experimental setup

- \Box L-Mode, I_P = 220kA, Ohmic only.
- \Box Configuration mirrored around R₀=0.88m
- NT (Negative Triangularity) : $\delta_{top} \approx -0.30$, $\delta_{bot} \approx -0.27$
- PT (Positive Triangularity) : $\delta_{top} \approx 0.27$, $\delta_{bot} \approx 0.29$
- □ *Enhanced confinement* observed in NT discharge



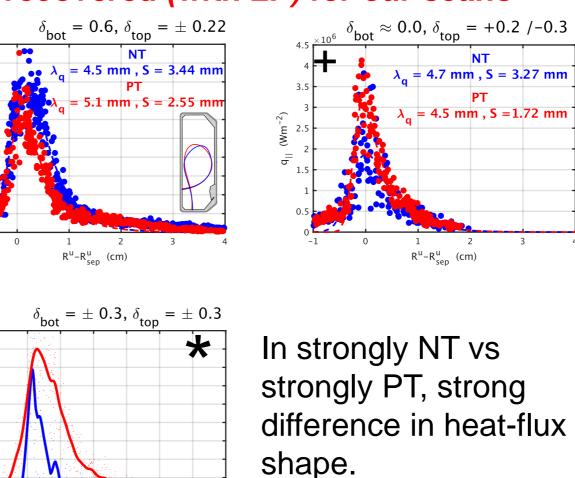
□ Possible indication of a more peaked heat-flux (smaller λ_{q})?

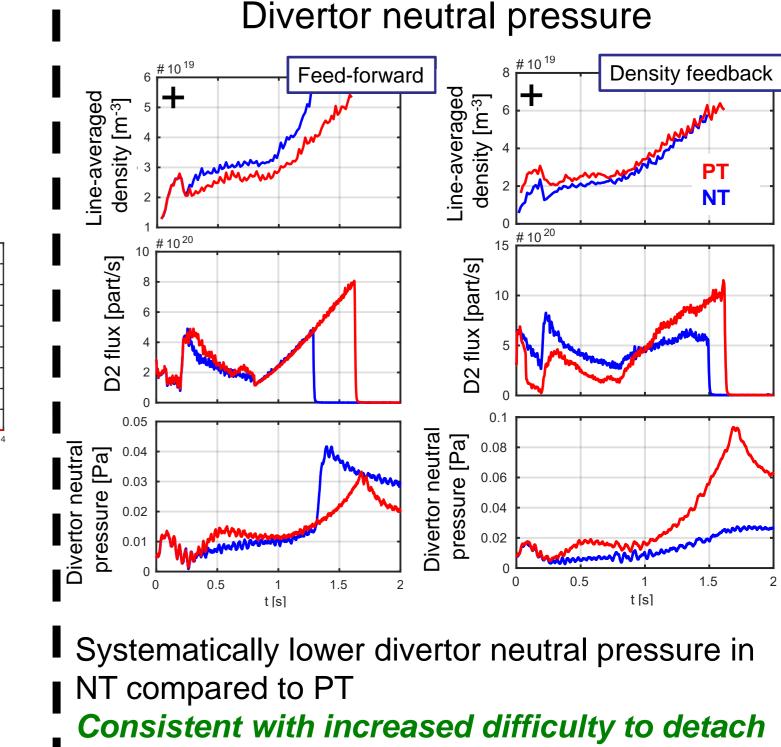
□ At ISP (for matched geometries), similar behavior, with low Te < 10 eV from the start

Why is detachment harder to achieve ?

-**-**-

Previous work on TCV [5] found lower λ_{α} in discharges with neg. top. δ , keeping bot. δ constant (IR measurements) Not recovered (with LP) for our scans

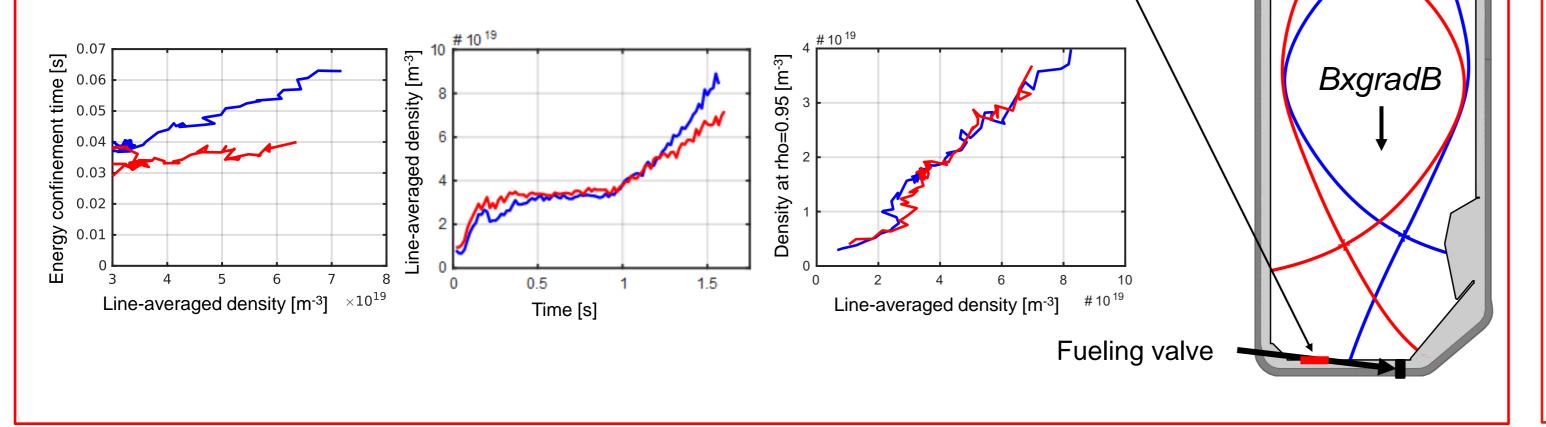




Conclusions & Perspectives

 $R^{u} - R^{u}_{sen}$ (cm)

Describe and a set of a set o



operation, associated with several advantages for power exhaust: No ELMs, no need for $P_{sep} > P_{LH}$

• Experiments on TCV have demonstrated the **enhanced confinement associated** with Negative Triangularity, but power exhaust remains challenging (harder to detach)

□ Perspective & future work :

• Further investigations of the λq scaling

More peaked in NT,

but integral smaller

- Extension to high-power (NBH/ECRH) NT, comparison to PT H-Mode
- Exploration of NT with divertor baffles (enhanced detachment access)

[5] M. Faitsch et al, Plasma Phys. Control. Fusion 60 045010 (2018).

[1] Y. Camenen et al, Nucl. Fusion 47 510 (2007) [2] S, Coda et al, Plasma Phys. Control. Fusion 64 014004 (2021) [3] M. E. Austin et al, Phys. Rev. Lett. 122, 115001 (2019). [4] A. Marinoni et al, Reviews of Modern Plasma Physics 5, 6 (2021)

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