



Self-Consistent Energy Normalization for Quasistatic Reactor Calculations

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Introduction

- Reactor feedback calculations require neutron flux distributions that are normalized to the specific power level of the reactor.
- Detailed power normalization methods rely on nuclide-specific energy release data in ENDF-format.
 - ENDF MT 458 (energy release per fission) and MT 300 (KERMA)
- Nuclear data is used to estimate the rate of energy release within the reactor.
 - At steady-state: energy deposition rate = energy release rate
 - Non-critical systems use quasistatic approximation to force a steady-state solution.
 - Neutron production rate is artificially modified by a factor $1/k_{\text{eff}}$.
 - Production adjustment results in a significant effect on energy deposition.
 - Conservation of energy release/deposition is not satisfied under this approximation.

Background

- In 2013, Griesheimer and Stedry proposed a generalized framework for detailed energy deposition during radiation transport.
 - Four different levels of energy deposition fidelity were proposed
 - 1: Constant eV per Fission
 - 2: Constant indirect energy release
 - 3: Local photon deposition
 - 4: Fully coupled neutron/photon transport
 - Modes 3 and 4 include explicit neutron energy transport.
 - In these modes, an “energy rebalance factor” ($\tau = k_{\text{eff}}$) was used to balance neutron energy release and deposition for non-critical systems.
- In 2019, Serpent adopted a similar framework for detailed energy deposition treatments.

Overview of Current Work

- Extensive testing of the 2013 energy deposition framework showed unexpected results for some cases.
 - Calculations produced negative indirect energy release rates for some fast-spectrum systems.
- Original energy rebalance factor definition ($\tau = k_{\text{eff}}$) is not exact
 - Original definition does not account for the incident energy of neutrons causing fission.
 - Additional questions uncovered regarding the application of the energy rebalance factor to exothermic (non-fission) reactions.
- This work establishes a self-consistent method for calculating energy deposition in non-critical systems
 - Complete (exact) definition for the energy rebalance factor.

Energy Balance Formulation

- Begin with the premise that energy release must equal energy deposition + leakage for a reactor in steady-state (or quasi-static conditions).

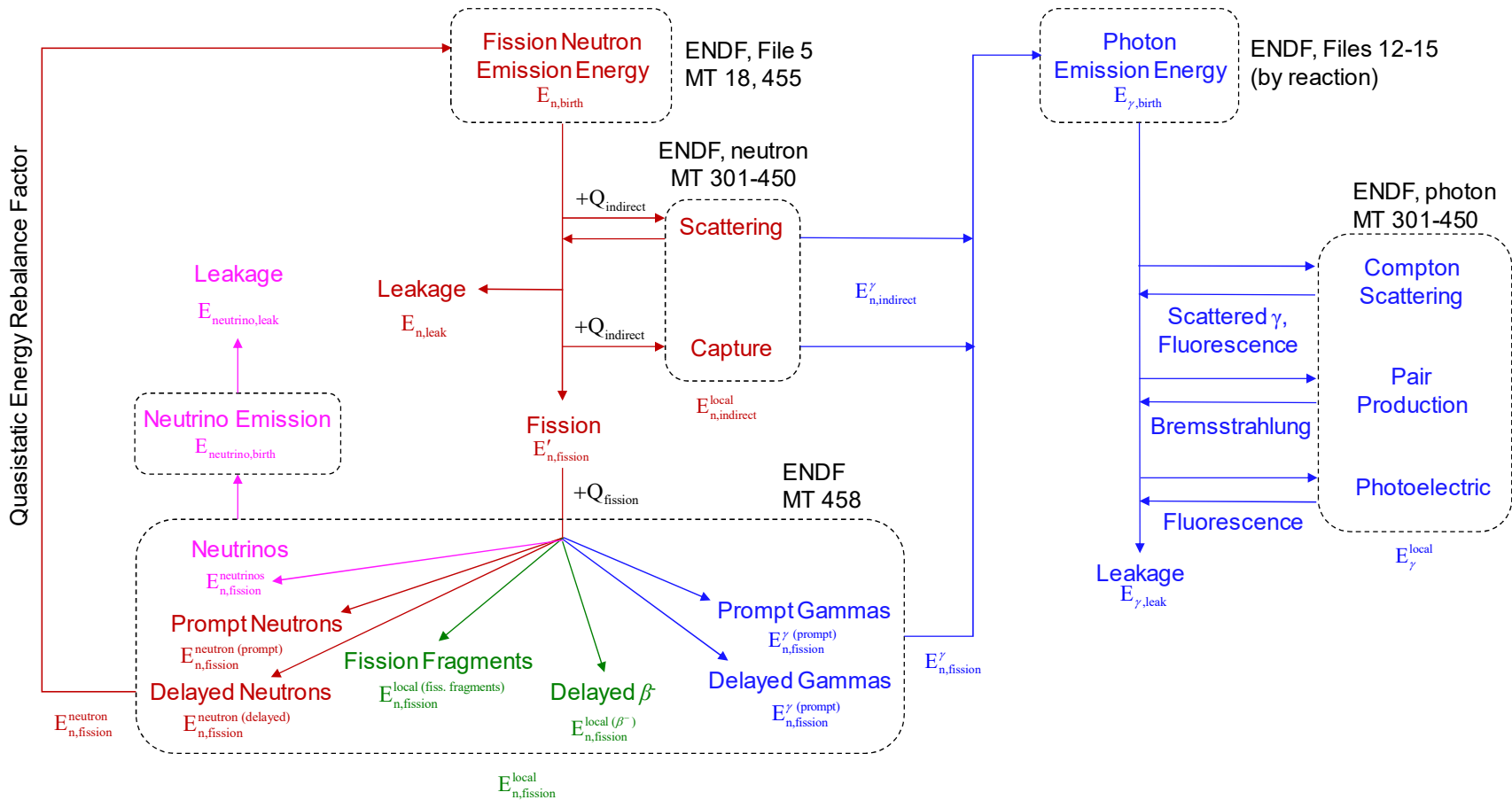
$$E_{n,birth} + Q_{indirect} + Q_{fission} = (E_{n,fission}^{total} + E_{n,indirect}^{total}) + E_{n,leakage}$$

- Q and E values represent total energy release and transfer, respectively, integrated over space and energy for specified reaction types.
- Break the master energy balance equation into “local” and “non-local” energy deposition terms to represent the effects of secondary radiation transport.

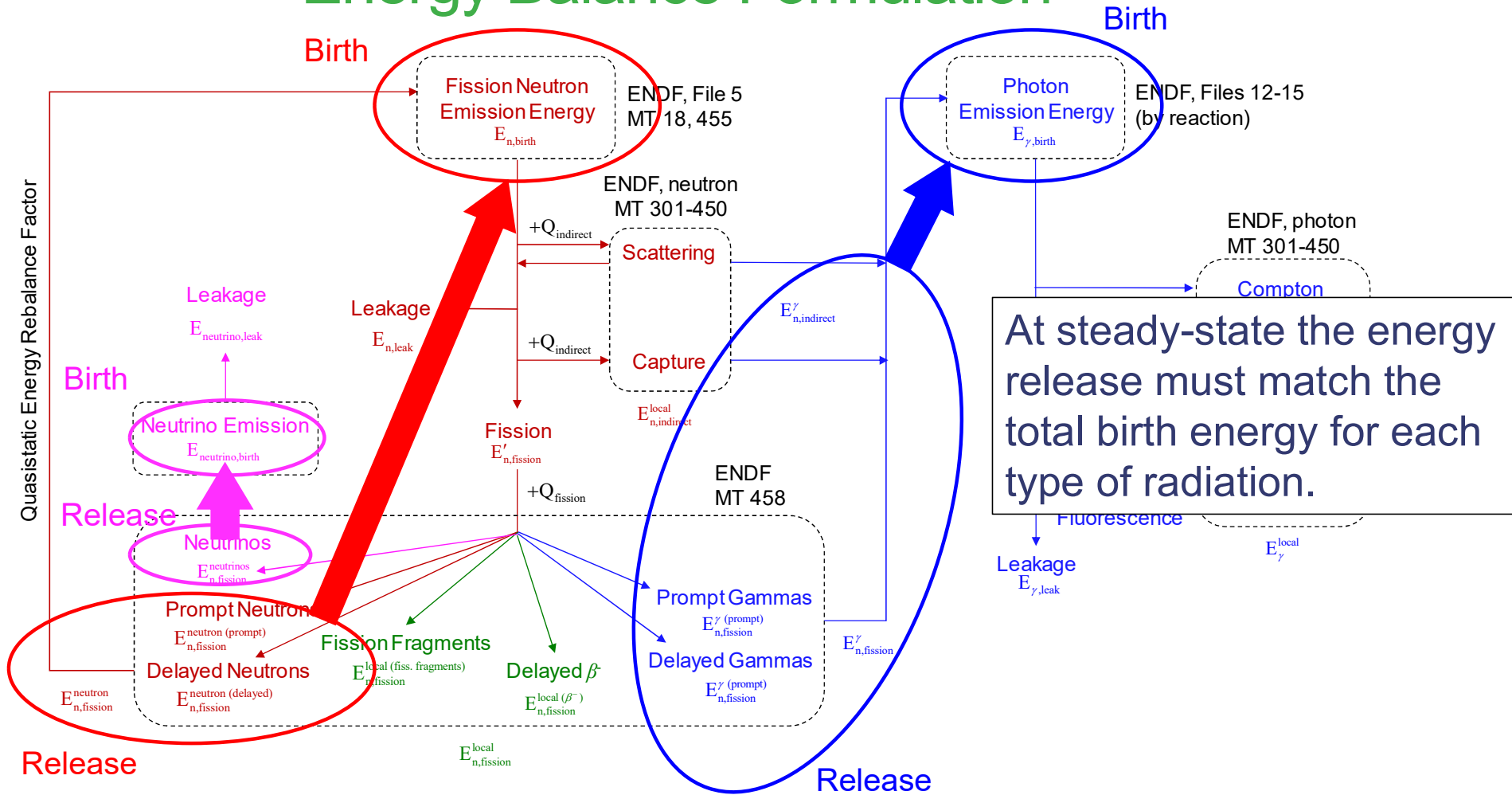
$$E_{n,birth} + Q_{indirect} + Q_{fission} = (E_{n,fission}^{local} + E_{n,fission}^{neutron} + E_{n,fission}^{neutrino} + E_{n,fission}^{\gamma}) + (E_{n,indirect}^{local} + E_{n,indirect}^{\gamma}) + E_{n,leakage}$$

- Variable superscripts denote energy form following reaction.
- Auxiliary energy balance relationships for photons and neutrinos are illustrated on the next slide.

Energy Balance Formulation



Energy Balance Formulation



Quasistatic Energy Balance Formulation

- At steady-state, energy equations for each radiation type are coupled by balancing energy release and energy emission.

$$E_{n,birth} = E_{n,fission}^{neutron}, \quad E_{neutrino,birth} = E_{n,fission}^{neutrino}, \quad \text{and} \quad E_{\gamma,birth} = E_{n,fission}^{\gamma} + E_{n,indirect}^{\gamma}$$

- Neutron energy balance is not preserved under the quasi-static approximation (i.e., $1/k_{eff}$ factor applied to ν).

$$E_{n,birth} = \left\langle \bar{E}_n(E'_n) \frac{1}{k_{eff}} \nu \Sigma_f(\vec{r}, E'_n) \phi_n(\vec{r}, E'_n) \right\rangle_{E'_n, \vec{r}} \neq E_{n,fission}^{neutron} = \left\langle h_{n,fission}^{neutron}(E'_n) \Sigma_f(\vec{r}, E'_n) \phi_n(\vec{r}, E'_n) \right\rangle_{E'_n, \vec{r}}$$

where

E'_n Incident energy of neutron causing fission

$\bar{E}_n(E'_n)$ Avg. fission neutron birth energy

$h_{n,fission}^{neutron}(E'_n)$ Avg. energy released as neutrons during fission.



Quasistatic Energy Rebalance Factor

- Compensate for quasistatic energy deposition imbalance by adjusting the neutron weight for energy deposition and leakage reactions.
 - **Transport Energy:** True energy of neutron used for xs lookups
 - **Deposition Energy:** Adjusted neutron energy used for deposition/leakage
 - **Energy Rebalance Factor (τ_n):** Ratio of Deposition to Transport Energy
- To enforce balance between neutron energy release/deposition, define:

$$\tau_n = \frac{E_{n,\text{fission}}^{\text{neutron}}}{E_{n,\text{birth}}} = \frac{\left\langle h_{n,\text{fission}}^{\text{neutron}}(\tau_n, E'_n) \Sigma_f(\vec{r}, E'_n) \phi_n(\vec{r}, E'_n) \right\rangle_{E'_n, \vec{r}}}{\frac{1}{k_{\text{eff}}} \left\langle \int_0^\infty \bar{E}_n(E'_n) \nu \Sigma_f(\vec{r}, E'_n) \phi_n(\vec{r}, E'_n) \right\rangle_{E'_n, \vec{r}}}$$

Neutron Energy Release Per Fission

- The 2013 definition for the energy rebalance factor assumed that

$$\frac{\text{Fission neutron energy release per neutron}}{h_{n,\text{fission}}^{\text{neutron}}(E'_n)} = \bar{E}_n(E'_n) \quad \text{Average fission neutron birth energy}$$

- This assumption yields $\tau_n = k_{\text{eff}}$, but the assumption is **not correct**
- Per ENDF, $h_{n,\text{fission}}^{\text{neutron}}(E'_n)$ includes the energy of the neutron causing fission.

$$h_{n,\text{fission}}^{\text{neutron}}(\tau_n, E'_n) = h_{n,\text{fission}}^{\text{neutron}}(1, E'_n) + E'_n(\tau_n - 1).$$

- Note that E'_n is the neutron deposition energy, which leads to the dependence of $h_{n,\text{fission}}^{\text{neutron}}(\tau_n, E'_n)$ on the rebalance factor τ_n .

Corrected Energy Rebalance Factor

- Substituting definition for $h_{n,\text{fission}}^{\text{neutron}}(\tau_n, E'_n)$ into the implicit energy rebalance formula and solving for τ_n gives

$$\tau_n = \frac{\langle h_{n,\text{fission}}^{\text{neutron}}(E'_n) \Sigma_f(\vec{r}, E'_n) \phi_n(\vec{r}, E'_n) \rangle_{E'_n, \vec{r}} - \langle E'_n \Sigma_f(\vec{r}, E'_n) \phi_n(\vec{r}, E'_n) \rangle_{E'_n, \vec{r}}}{\frac{1}{k_{\text{eff}}} \langle \int_0^\infty \bar{E}_n(E'_n) \nu \Sigma_f(\vec{r}, E'_n) \phi_n(\vec{r}, E'_n) \rangle_{E'_n, \vec{r}} - \langle E'_n \Sigma_f(\vec{r}, E'_n) \phi_n(\vec{r}, E'_n) \rangle_{E'_n, \vec{r}}}$$

- Note that τ_n approaches k_{eff} if E'_n is small
 - For thermal-spectrum reactors this is a reasonable assumption.
 - Not accurate for reactors where a significant fraction of fissions are caused by above-thermal neutrons.

Corrected Energy Rebalance Factor

- Substituting definition for $h_{n,\text{fission}}^{\text{neutron}}(\tau_n, E'_n)$ into the implicit energy rebalance formula and solving for τ_n gives

Fission energy released as neutrons

Total energy of neutrons causing fission events

$$\tau_n = \frac{\langle h_{n,\text{fission}}^{\text{neutron}}(E'_n) \Sigma_f(\vec{r}, E'_n) \phi_n(\vec{r}, E'_n) \rangle_{E'_n, \vec{r}} - \langle E'_n \Sigma_f(\vec{r}, E'_n) \phi_n(\vec{r}, E'_n) \rangle_{E'_n, \vec{r}}}{\frac{1}{k_{\text{eff}}} \langle \int_0^\infty \bar{E}_n(E'_n) \nu \Sigma_f(\vec{r}, E'_n) \phi_n(\vec{r}, E'_n) \rangle_{E'_n, \vec{r}} - \langle E'_n \Sigma_f(\vec{r}, E'_n) \phi_n(\vec{r}, E'_n) \rangle_{E'_n, \vec{r}}}$$

Total birth energy of fission neutrons

- Note that τ_n approaches k_{eff} if E'_n is small
 - For thermal-spectrum reactors this is a reasonable assumption.
 - Not accurate for reactors where a significant fraction of fissions are caused by above-thermal neutrons.

Quasistatic Energy Balance Revisited

- Enforcing balance between neutron energy release and deposition allows simplification of the master energy balance equation.

$$\cancel{E_{n,birth}} + Q_{indirect} + Q_{fission} = \left(E_{n,fission}^{local} + \cancel{E_{n,fission}^{neutron}} + E_{n,fission}^{neutrino} + E_{n,fission}^{\gamma} \right) + \left(E_{n,indirect}^{local} + E_{n,indirect}^{\gamma} \right) + E_{n,leakage}$$

- Illustrates that neutron energy present in a (quasi)static system remains constant over time.
- Note that $Q_{fission}$ and $Q_{indirect}$ energy release rates are preserved on a per reaction basis, regardless of k_{eff} .
 - However, the total energy release rate will change with respect to k_{eff} , due to changes in the capture/fission rate.

Local Heating Edits

- The energy rebalance factor, τ_n , is applied to neutron leakage and all reactions involving energy deposition or transfer.
- For indirect (non-fission) reactions the modification is given by:

$$E_{n,\text{indirect}}^{\text{total}} = \left\langle \phi_n(\vec{r}, E'_n) \sum_i^{\text{nucs}} \sum_j^{\text{rxns}} (\tau_n E'_n + Q_{i,j}(E'_n) - \tau_n \bar{E}_{n,i,j}(E'_n)) \Sigma_{i,j}(\vec{r}, E'_n) \right\rangle_{E'_n, \vec{r}}$$

- Note that the rebalance factor is not applied to the reaction Q -values.
- In ENDF, energy deposition is represented by the KERMA pseudoreaction cross section, which is defined as:

$$K_{n,\text{indirect}}^{\text{total}}(E'_n) = \sum_i^{\text{nucs}} \sum_j^{\text{rxns}} (E'_n + Q_{i,j}(E'_n) - \bar{E}_{n,i,j}(E'_n)) \Sigma_{i,j}(\vec{r}, E'_n),$$

Local Heating Edits

- The energy rebalance factor, τ_n , is applied to neutron leakage and all reactions involving energy deposition or transfer.
- For indirect (non-fission) reactions the modification is given by:

$$E_{n,\text{indirect}}^{\text{total}} = \left\langle \phi_n(\vec{r}, E'_n) \sum_i \sum_j^{\text{nucs rxns}} (\tau_n E'_n + Q_{i,j}(E'_n) - \tau_n \bar{E}_{n,i,j}(E'_n)) \Sigma_{i,j}(\vec{r}, E'_n) \right\rangle_{E'_n, \vec{r}}$$

- No

NOTICE:

$$E_{n,\text{indirect}}^{\text{total}} \neq \left\langle \phi_n(\vec{r}, E'_n) \tau_n K_{n,\text{indirect}}^{\text{total}}(E'_n) \right\rangle_{E'_n, \vec{r}}$$

- In E
- cross

The R.H.S **incorrectly** applies the τ_n factor to all reaction Q -values!

reaction

$$K_{n,\text{indirect}}^{\text{total}}(E'_n) = \sum_i \sum_j^{\text{nucs rxns}} (E'_n + Q_{i,j}(E'_n) - \bar{E}_{n,i,j}(E'_n)) \Sigma_{i,j}(\vec{r}, E'_n),$$

KERMA Inconsistency

- In KERMA data there is no way to know how the incident neutron energy is apportioned among deposition and secondary neutrons exiting the reaction.
- Many codes assume that KERMA deposition is scaled by τ_n :

$$E_{n,\text{indirect}}^{\text{total}} \approx \left\langle \phi_n(\vec{r}, E'_n) \tau_n K_{n,\text{indirect}}^{\text{total}}(E'_n) \right\rangle_{E'_n, \vec{r}}$$

- This approximation unphysically applies τ_n to all (non-fission) reaction Q -values.
 - Violates philosophy of preserving reaction Q -values.
 - Typically a small effect – indirect energy release accounts for <1% of reactor power.
- Resolving inconsistency will require changes to KERMA calculation and ENDF format.
 - Adopt a common KERMA format that explicitly accounts for incident neutron energy, similar to the format of the ENDF MT 458 fission energy release data.

Numerical Results

- An analytical reference model was used to quantify the effects of the quasistatic approximation on energy release/deposition.
 - Monoenergetic 2.0 MeV neutrons in an infinite, homogeneous mixture of two nuclides.
 - Fuel – Purely fissioning, $\sigma_t = \sigma_f = 1.0$, $\nu = 2.0$ n/fiss., $Q_{\text{fission}} = 200$ MeV/fiss.
 - Poison – Purely absorbing, $\sigma_t = \sigma_a = 1.0$, $Q_{\text{absorption}} = 1.0$ MeV/abs.
 - Number density of fuel N_f is constant at 1.0 atoms/(barn·cm)
 - Number density of poison N_p was adjusted from 0.5 to 1.5 atoms/(barn·cm).

Fission energy release by category for fictitious, purely-fissioning fuel nuclide.

Category	Nominal Energy Rel. ($h(0)$) [MeV/fiss]	Energy-Dependent Energy Rel. ($h(E')$) [MeV/fiss]
Fission Fragments (h^{FR})	169.0	$h^{\text{FR}}(0)$
Prompt Neutrons [†] (h^{neutron})	1.386	$h^{\text{neutron}}(0) + 1.307 \times E' \times (\tau_n - 1)$
Prompt Gammas [†] (h^γ)	12.15	$h^\gamma(0) - 0.075 \times E'$
Beta Particles (h^β)	7.15	$h^\beta(0) - 0.075 \times E'$
Neutrinos (h^{neutrino})	10.2	$h^{\text{neutrino}}(0) - 0.010 \times E'$

[†]Delayed neutron and photon release is assumed to be equal to zero.

Numerical Results

- Analytical solutions for k_{eff} and energy release/deposition as a function of poison number density.

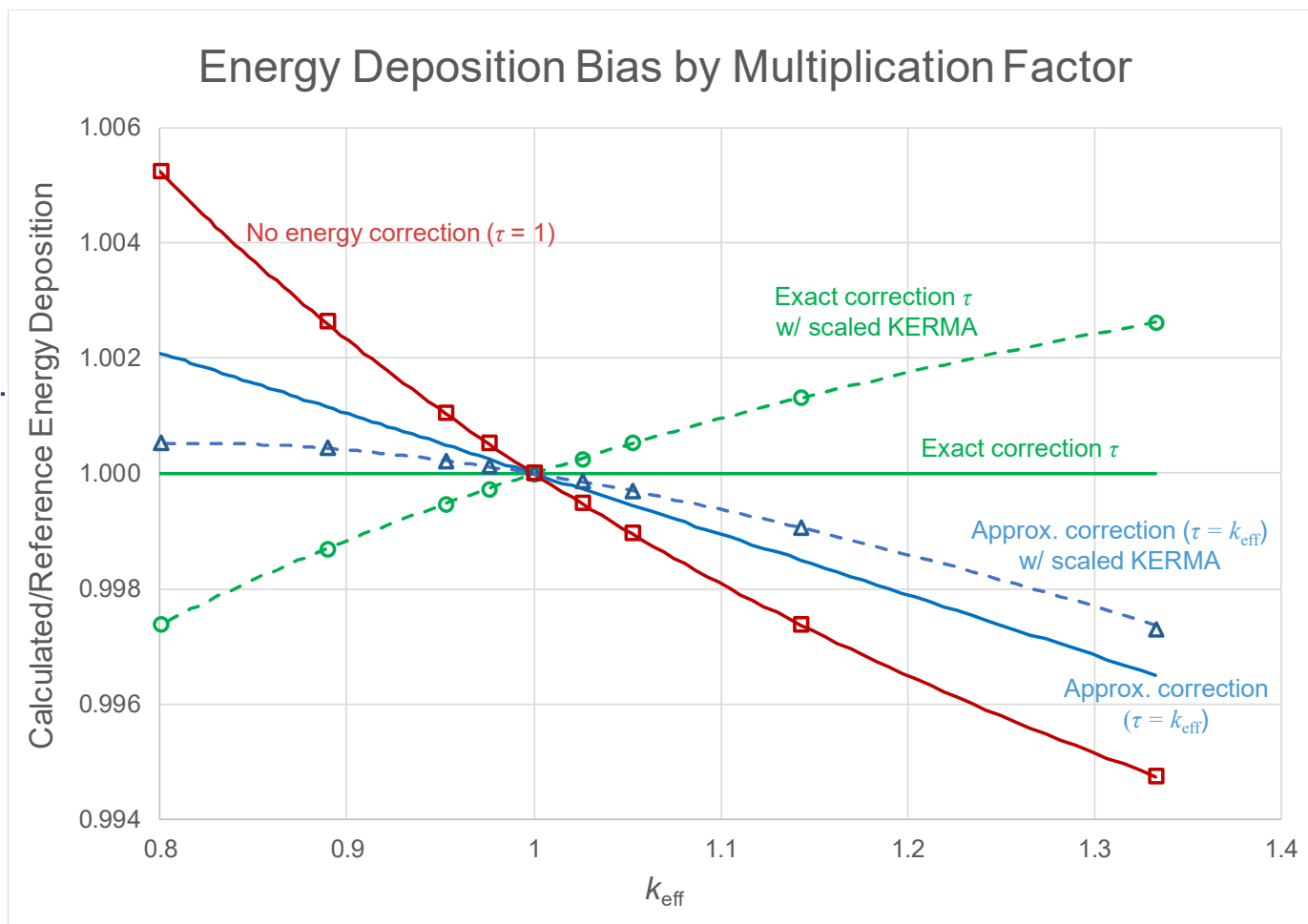
$$k_{\text{eff}} = \frac{N_F \nu \sigma_f}{N_F \sigma_f + N_P \sigma_a} = \frac{2}{1 + N_P}$$

$$\frac{E_{\text{n,deposited}}^{\text{total}}}{\text{fission}} = 190 + N_P \frac{\text{MeV}}{\text{fission}}$$

- Theoretical behavior of three energy renormalization schemes considered
 - No correction ($\tau = 1$), approximate correction ($\tau = k_{\text{eff}}$), exact correction
 - Effects of scaled KERMA values also considered.
- Theoretical results validated by calculating energy deposition with the MC21 Monte Carlo transport solver.
 - 100 batches (10 discard) of 10,000 histories
 - Manually created nuclear data library.
 - Scaled KERMA values only.

Energy Deposition Bias

Plot illustrates the ratio between computed energy deposition (for each correction factor) and the analytical reference solution for energy deposition.



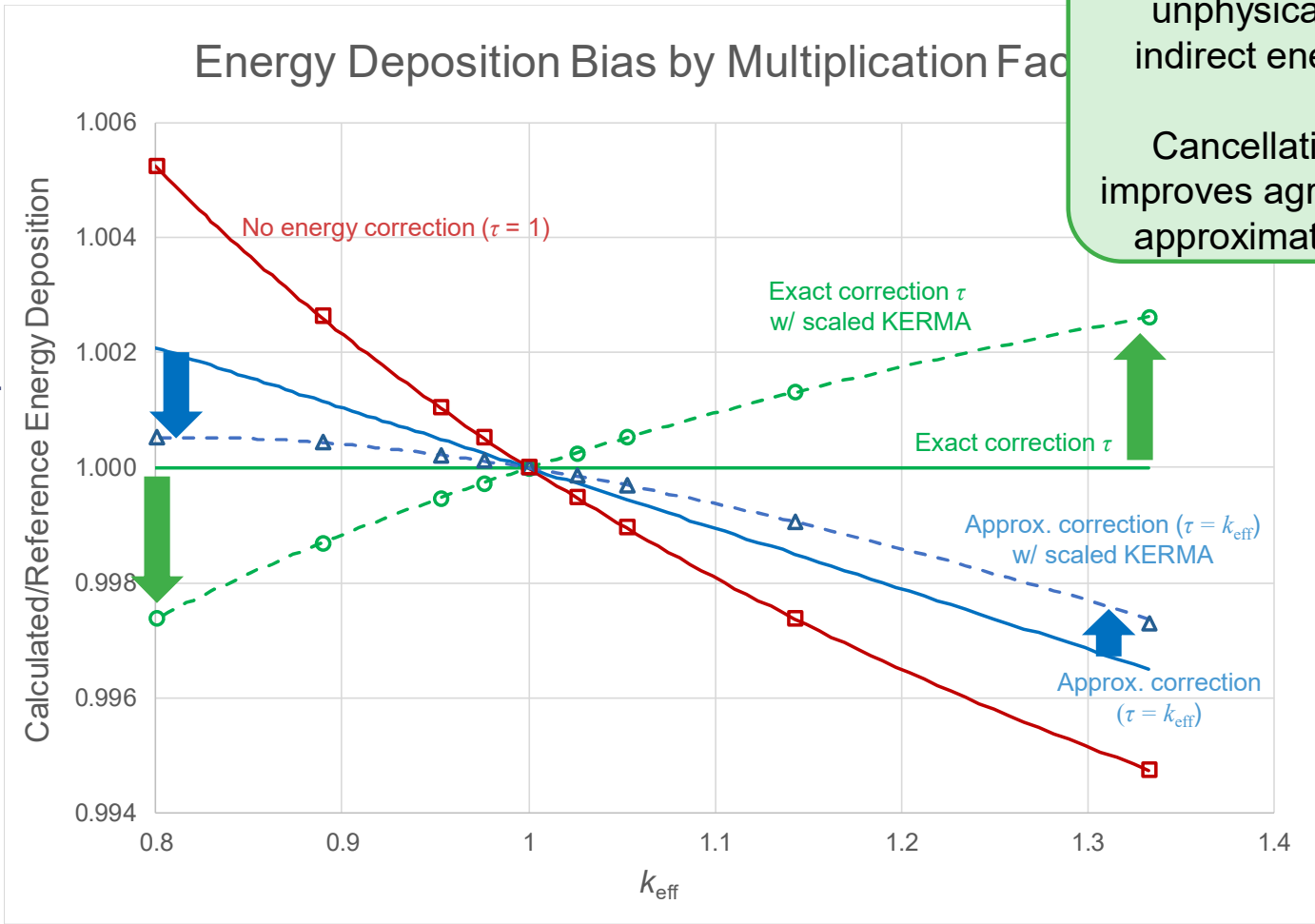
Lines – Theoretical
Markers – MC21

Energy Deposition Bias

KERMA scaling unphysically changes indirect energy release.

Cancellation of errors improves agreement for the approximate correction.

Plot illustrates the ratio between computed energy deposition (for each correction factor) and the analytical reference solution for energy deposition.

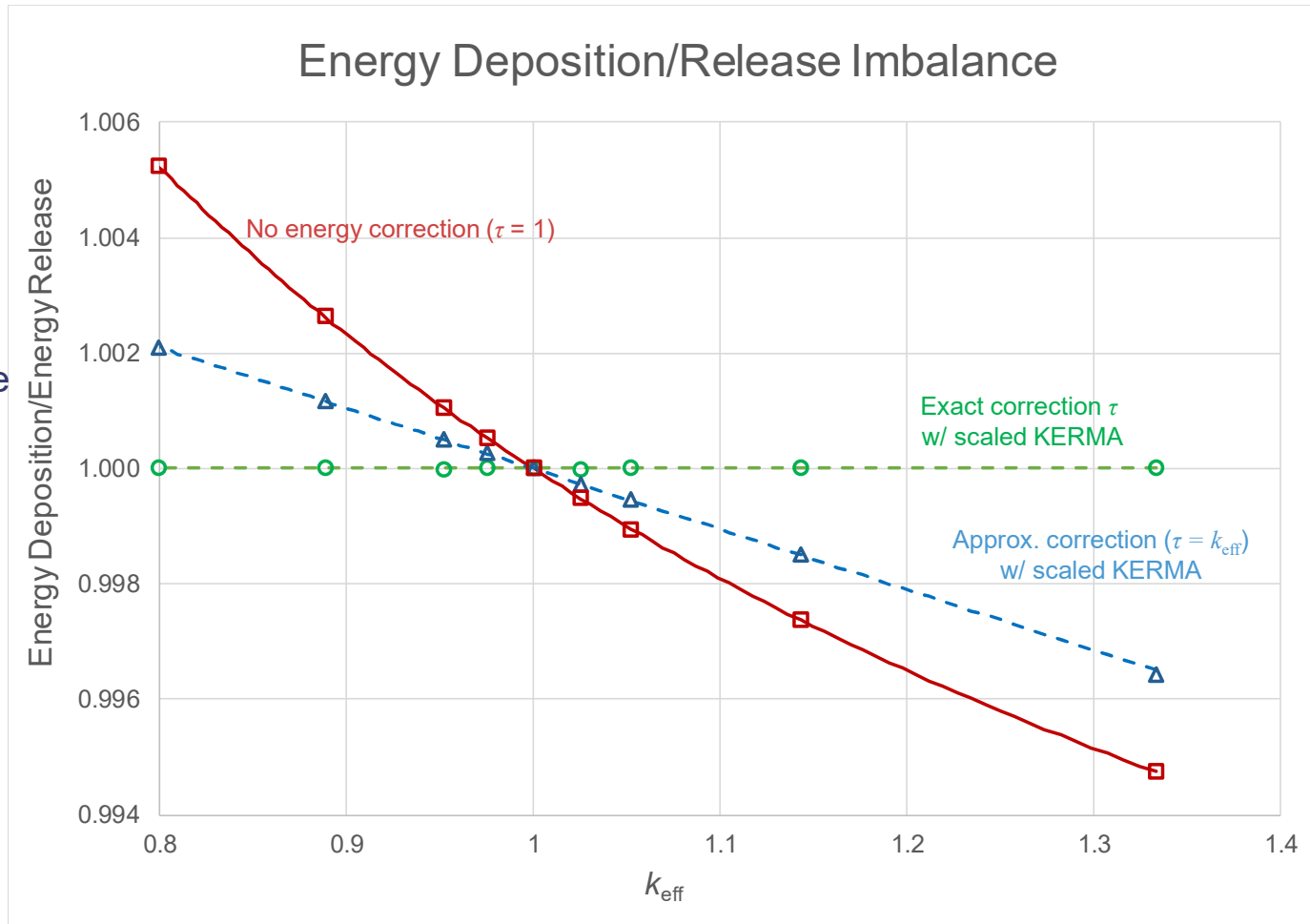


Lines – Theoretical
Markers – MC21

Energy Deposition/Release Imbalance

Plot illustrates the ratio of energy deposition to energy release for each correction factor.

Values other than 1.0 indicate an imbalance between energy release and deposition.



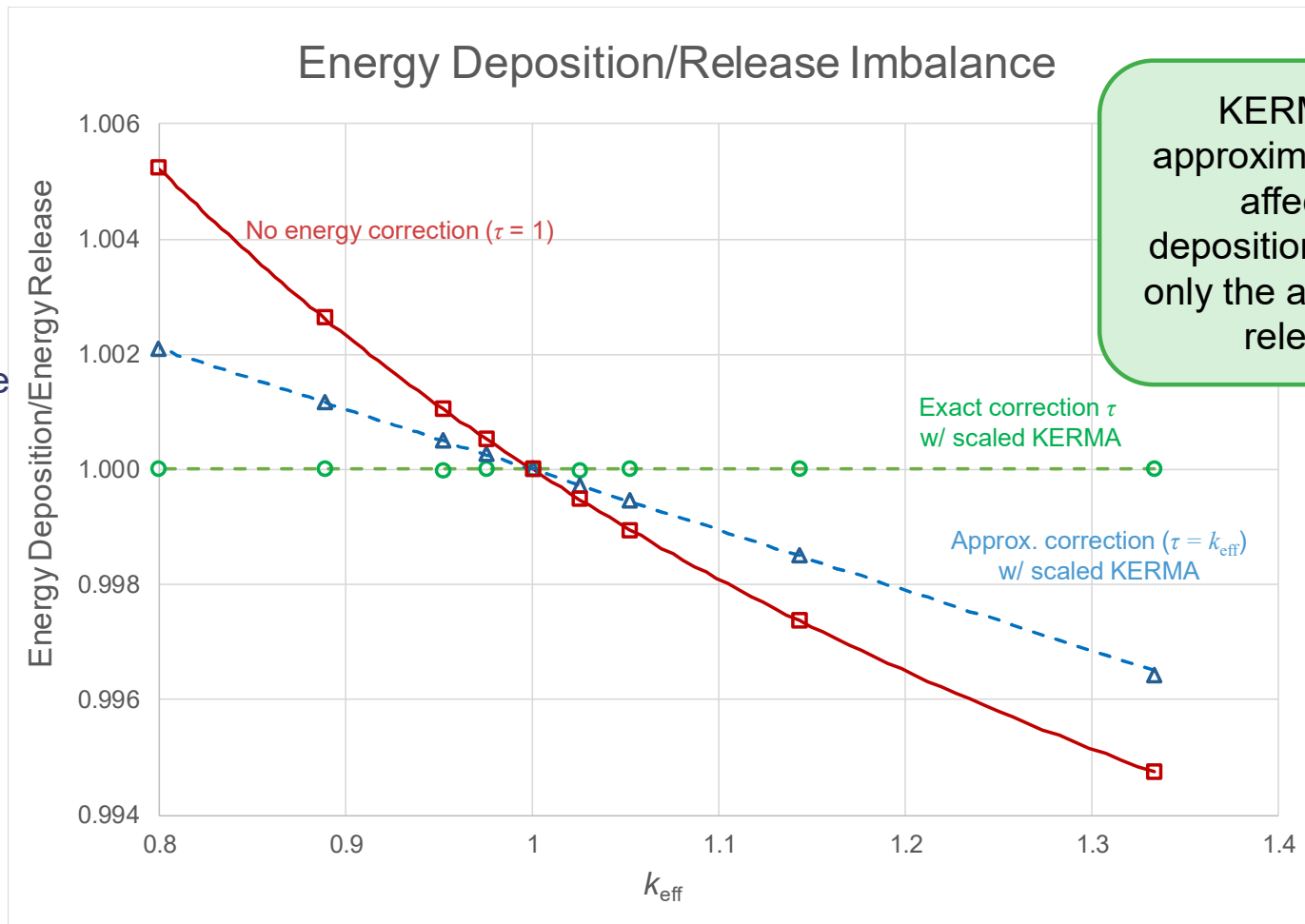
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Values other than 1.0 indicate an imbalance between energy release and deposition.

Lines – Theoretical
Markers – MC21



KERMA scaling approximation does not affect energy deposition/release ratio, only the absolute energy release rate.

Conclusions

- In non-critical fission systems, the use of the quasistatic approximation leads to an imbalance between energy release and deposition.
- In this work, we formally quantified this imbalance and derived an exact technique for enforcing energy release/deposition balance.
 - Technique involves applying an energy rebalance factor to local energy deposition, transfer, or leakage reactions.
 - Numerical results for an analytical energy deposition problem confirm the effectiveness of the technique.
- Limitations in the KERMA data format still prevents conservation of indirect energy release in most applications.
 - Consider improvements to KERMA data representation in ENDF.

Questions?