



Technical Meeting on Long-Pulse
Operation of Fusion Devices
Vienna, November 14-16th, 2022

LONG PULSE OPERATION IN A TUNGSTEN ENVIRONMENT : ACHIEVEMENTS AND WORK PLAN FOR WEST

P. Maget on behalf of the WEST Team

*Special thanks to P. Manas, Y. Corre, J. Gaspar, J. Gunn,
as well as C. Bourdelle, A. Ekedahl, M. Goniche, J. Hillairet, X.
Litaudon, Th. Loarer, Y. Peysson, E. Tsitron*

A comprehensive workplan for ITER PFU characterisation and the way to long pulse operation in a metallic environment

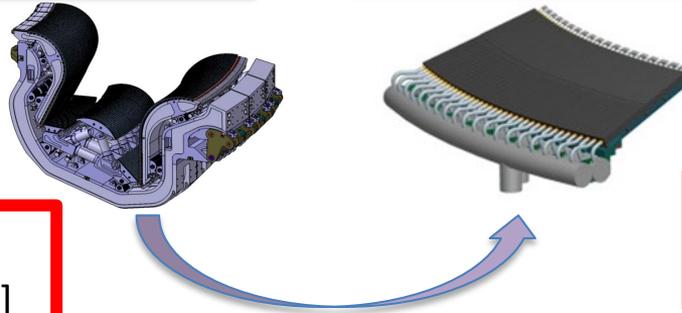
Manufacturing & validation of ITER Plasma Facing Units (PFU)
[talk by M. Firdaouss on Wednesday]

**WEST
W Environment in
Steady-state Tokamak**

**ITER PFU under
operational constraints**

ITER divertor cassette

WEST ITER-like divertor



Ageing & structure evolution
[talk by E. Bernard on Wednesday]

**Scenario developments for
Long Pulse Operation**

Control and protection challenges
[talk by Ph. Moreau on Tuesday]

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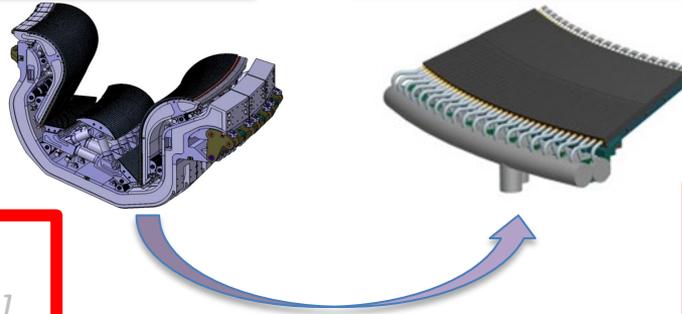
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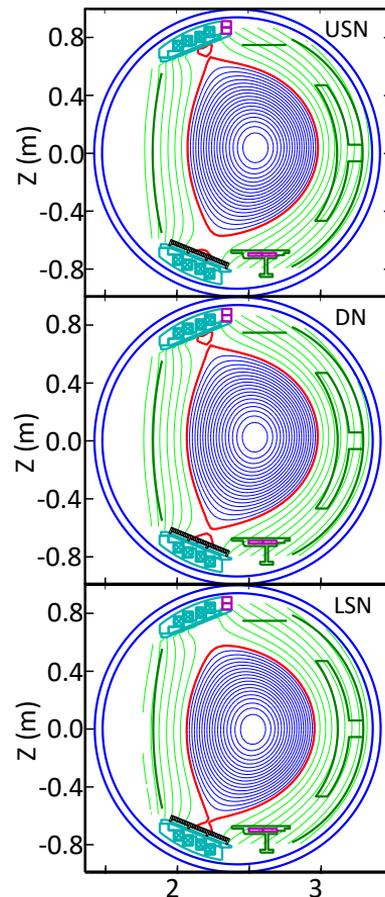
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Control and protection challenges
[talk by Ph. Moreau on Tuesday]



West Tokamak

- ▶ Actively cooled superconducting tokamak
- ▶ Flexible magnetic configuration (LSN, USN, DN)
- ▶ Large current drive capability
- ▶ **Long pulse operation → 1000 s**

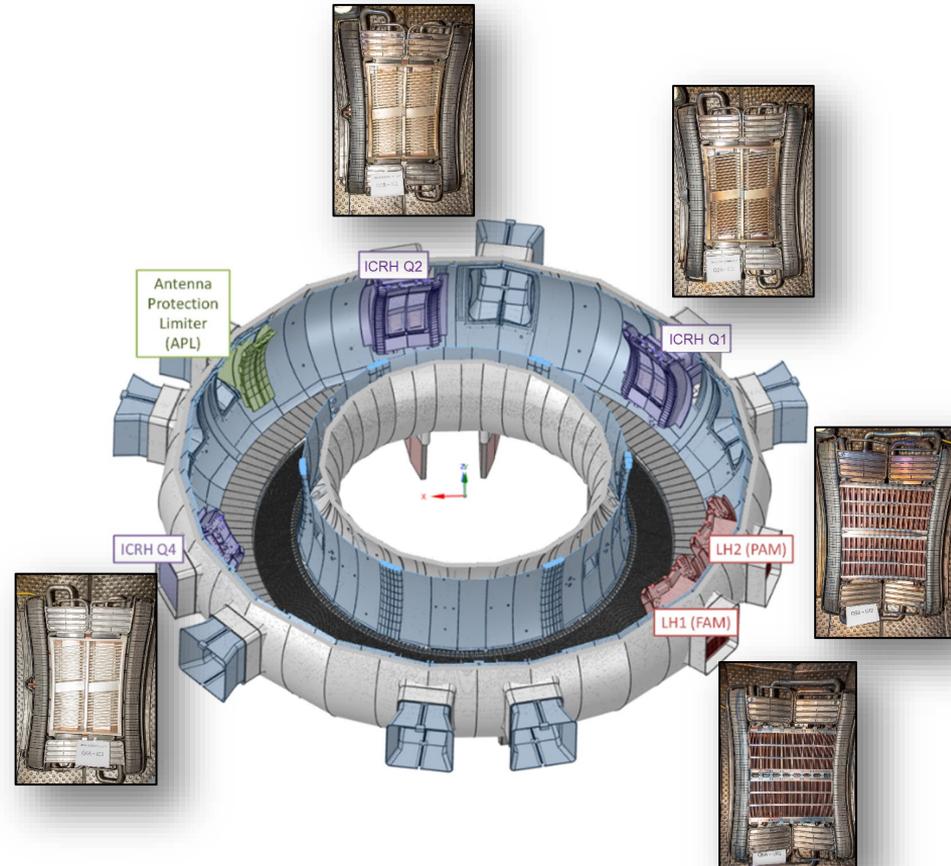
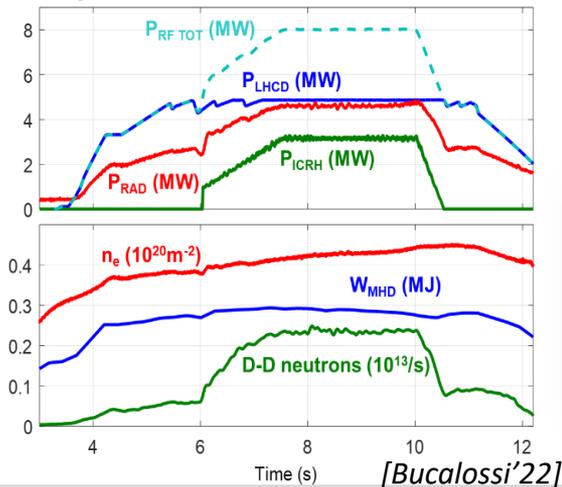


I_p ($q_{95} \sim 2.5$)	1.0 MA
B_ϕ	3.7 T
R	2.5 m
a	0.5 m
A	5-6
Max κ	1.35
δ	Up to 0.5
V_p	15 m ³
P_{ICRH}	9 MW
P_{LHCD}	7 MW
P_{ECCD}	3 MW

16 MW of Radio Frequency power

- ▶ 9 MW/30s of ICRH power, or 3 MW/1000s
 - ✓ Three actively cooled ELM-resilient antennas
- ▶ 7 MW of LHCD power @ 3.7 GHz - 1000s
 - ✓ Two actively cooled launchers
 - LH1 (FAM) - Fully Active Multi-junction
 - LH2 (PAM) - Passive Active Multi-junction
- ▶ 3 MW of ECCD power @ 105 GHz- 1000s
 - Staged installation from 2023

**8 MW RF power
coupled to plasmas
in W environment**



Pulse duration and injected energy

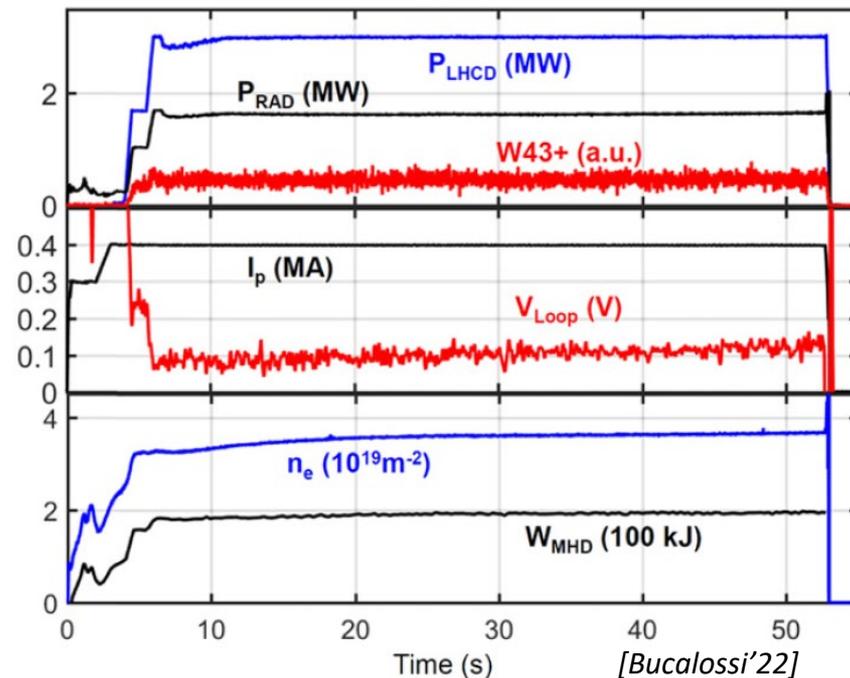
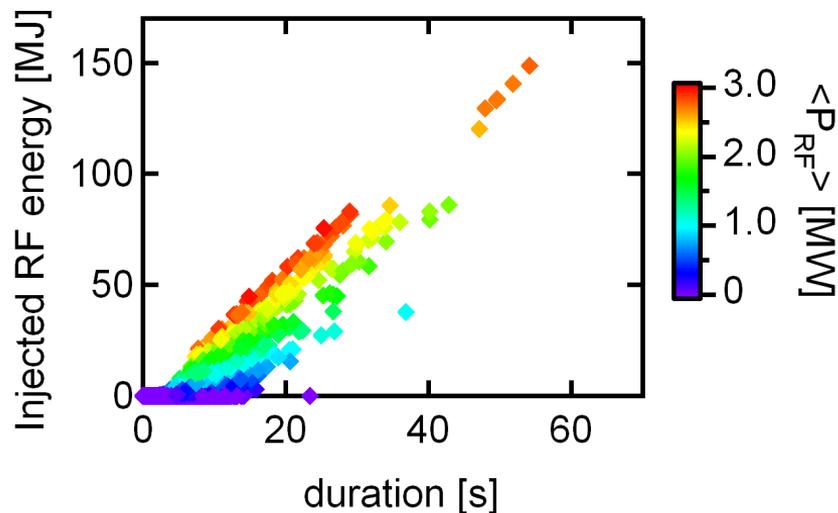
► Pulse length extended to ~ 1 min

- Steady-state obtained on most quantities (slow drift of loop voltage associated with density landing to reference)
- Next step : Coil current optimization & feedback control of loop voltage on LH power

#55787

► Injected energy up to 150 MJ

- With ~3 MW of LH power on average



[Bucalossi'22]

Non-inductive operation

$$(I_p^2 f_{Gr})^{max} \sim \eta_{CD} P_{tot}^{max} / (1 - f_{bs})$$

CD efficiency

Access larger
 I_p & f_{Gr}

MHD avoidance

Current profile
control

Impurity
control

Minimize W
radiation

Power to PFU &
H-mode regime

Address PFU
limits

Machine protection

Remain within
safety
constraints

Overview based on actual WEST database & 0D integrated simulations with METIS*

* [Artaud'18]

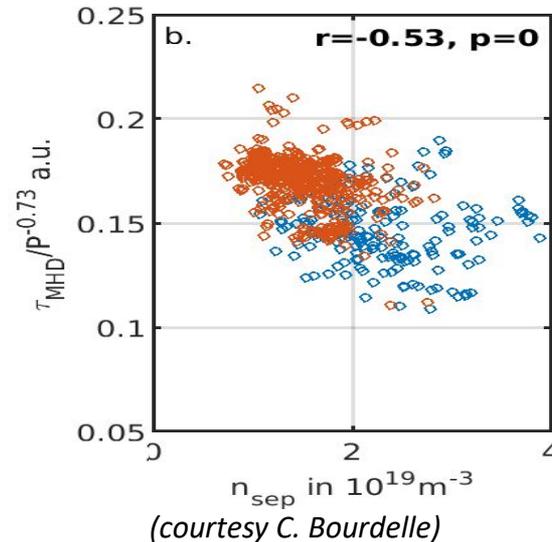
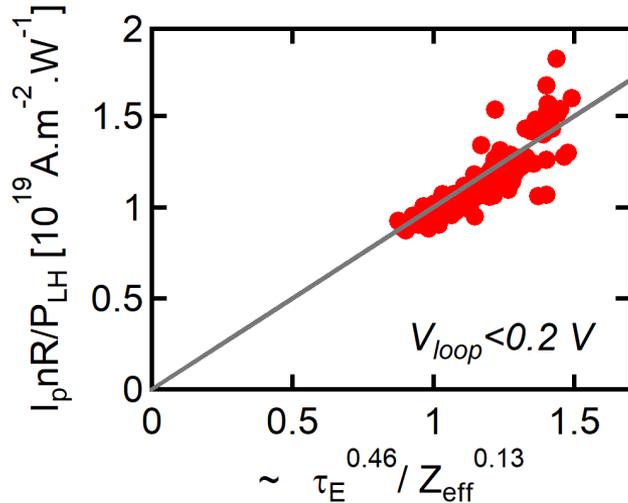
Current Drive efficiency

► Understanding the physics of LH CD efficiency is a key research area

- Estimated at $\eta_{CD} \sim 0.7 \times 10^{19} \text{ A.m}^{-2}.\text{W}^{-1}$ in L-mode
- An empirical scaling in $\tau_E^{0.46}$ [Goniche'05] is consistent with measurements on WEST

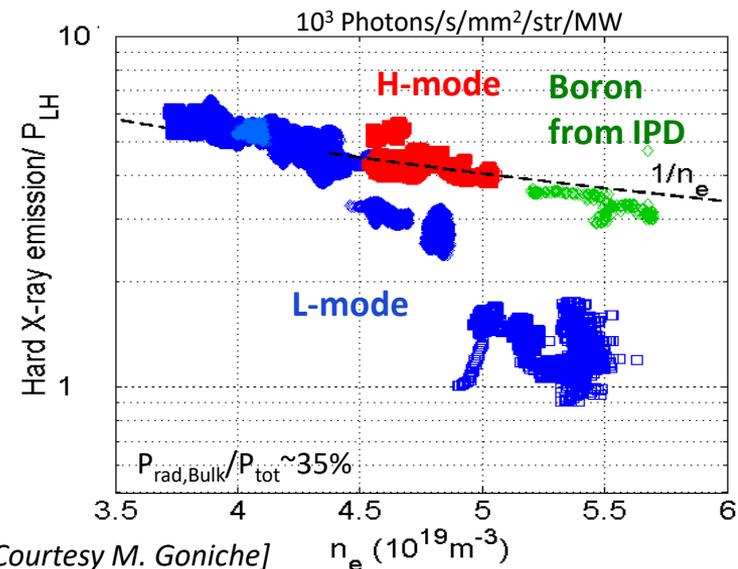
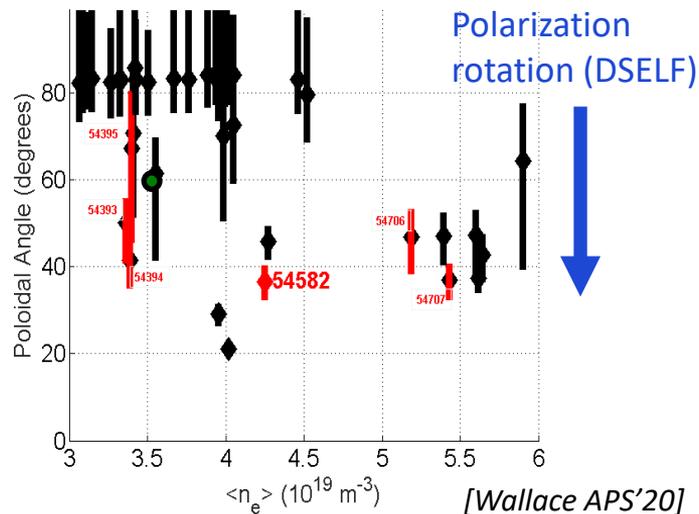
► Correlations between edge & core quantities tends to hide physics mechanisms involved at the edge

- Confinement time is also connected to SOL physics [Ryter'21, Bourdelle'22]
- Evidences that LH coupling & CD is impacted by SOL physics [Wallace'10, Pericoli-Ridolfini'11, Madi'15, Baeck'20]



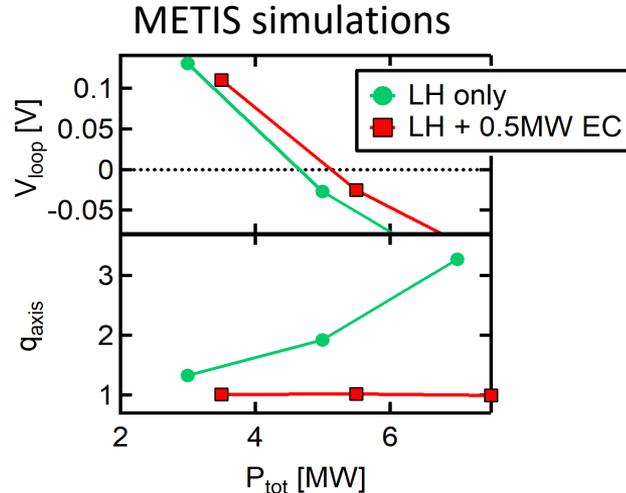
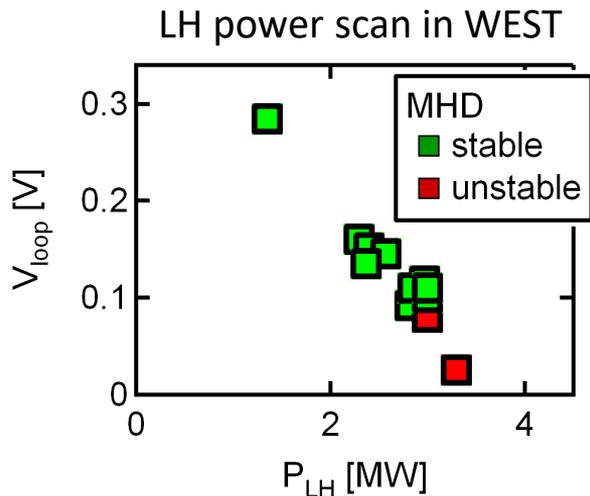
Current Drive efficiency

- ▶ **Indications of degraded CD efficiency at high density from spectroscopy & HXR measurements**
 - Spectroscopy in the SOL (DSELF, ORNL): rotation of the wave vector at high density [Martin'19, Lau APS'21]
 - Consistent with the fall of HXR signals
- ▶ **Preliminary investigations suggest actuators for improved CD efficiency**
 - in H-mode $\eta_{CD} \sim 0.75 \times 10^{19} \text{ A.m}^{-2}.\text{W}^{-1}$ (+ 7%) & with light impurity seeding via IPD (PPPL)
 - Similar observations on EAST with Lithium coating [Goniche'17]



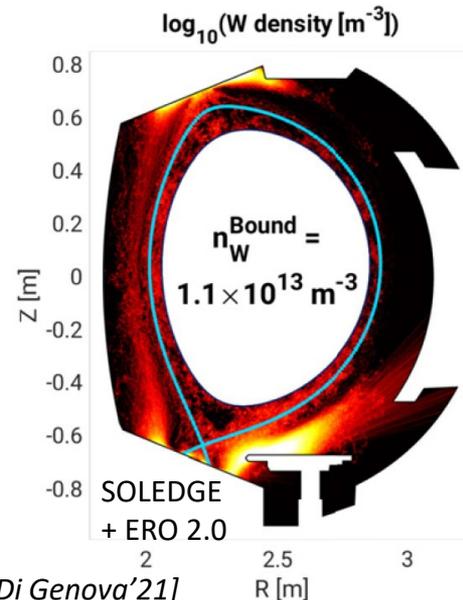
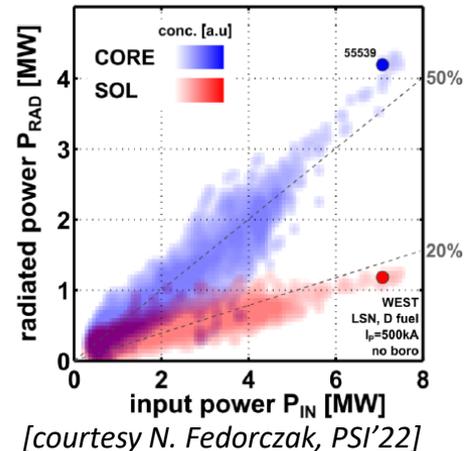
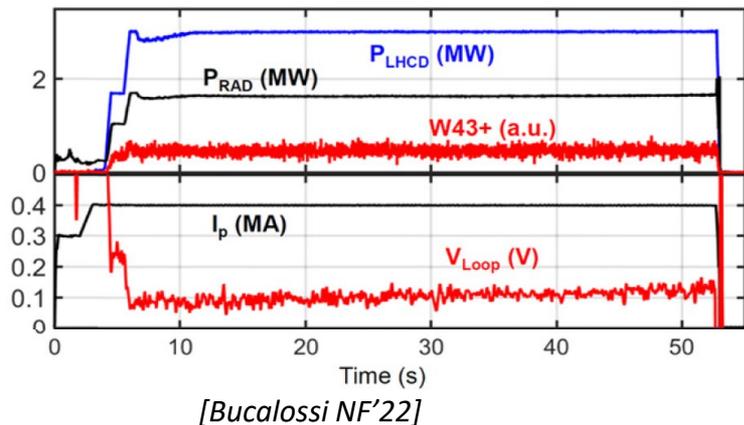
Non-inductive current profile prone to MHD limits

- ▶ LHCD deposition is off-axis and drives hollow current profile [Peysson'20, Wongrach'21, Ostuni'22]
 - MHD instability triggering at low V_{loop} , reminiscent of Double-Tearing Mode issue on Tore Supra [Maget'05]
 - Consistent with q-reversal in integrated simulations with METIS
 - Improved stability expected from ICRH contribution [Dumont'14]
- ▶ A robust (monotonic-q) scenario requires current profile control actuators
 - Parallel refractive index of LH
 - Co-ECCD in the plasma core : 0.5 MW are sufficient to prevent q-profile inversion



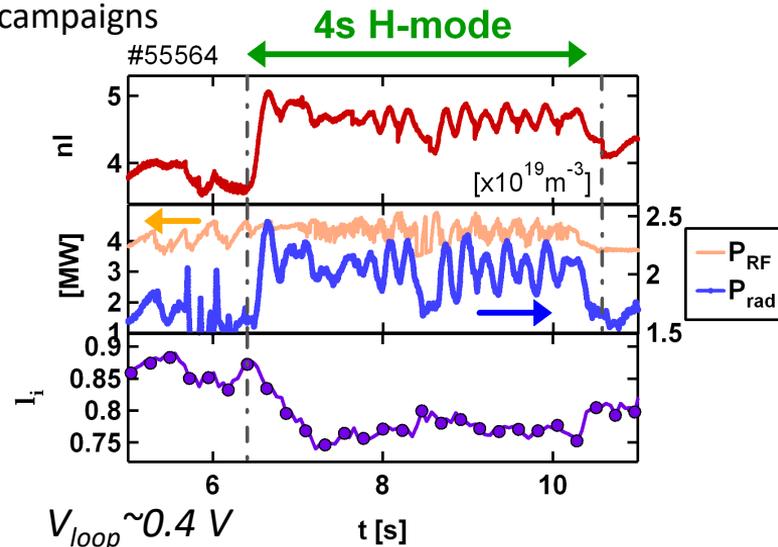
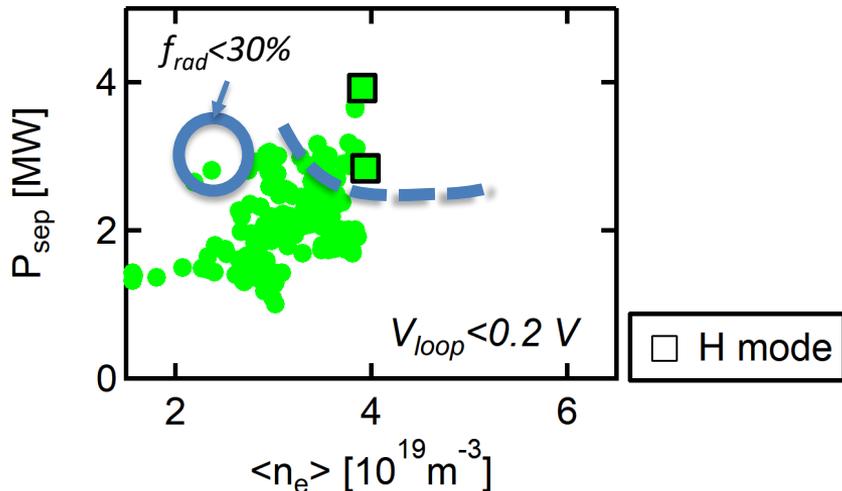
Steady tungsten concentration in long pulses

- ▶ **No tungsten peaking (except during radiative collapses [Ostuni'22])**
 - Consistent with torque-free plasma condition [Yang'20]
- ▶ **Core radiated fraction around 50% far from boronisation**
 - Robust trend independent on density or input power [Fedorczyk, PSI'22]
- ▶ **Opening the operational space for H-mode access & SOL power increase**
 - Better understand & mitigate W sources [Di Genova'21] : role of vessel conditioning
 - Investigate X-point radiator regime
 - First results on IPD conditioning (PPPL procurement) [Bodner'22, Gallo APS'22]



PFU testing requires high power transferred to Scrape-Off Layer

- ▶ **Power crossing separatrix : up to 4 MW achieved at low V_{loop}**
- ▶ **H-mode transition : key for enhancing heat flux on PFUs**
 - Threshold consistent with Martin's scaling [Martin'08]
 - Observed only above $\langle n_e \rangle \sim 4 \times 10^{19} \text{m}^{-3}$: reminiscent of low density branch limit [Ryter'14, Solano'22]
- ▶ **4s H-mode obtained at $V_{loop} \sim 0.4 \text{ V}$**
 - Pedestal formation / density increase : more radiative losses and oscillating regime [Vermare'22]
 - Exploration of H-mode regime at larger input power in coming campaigns



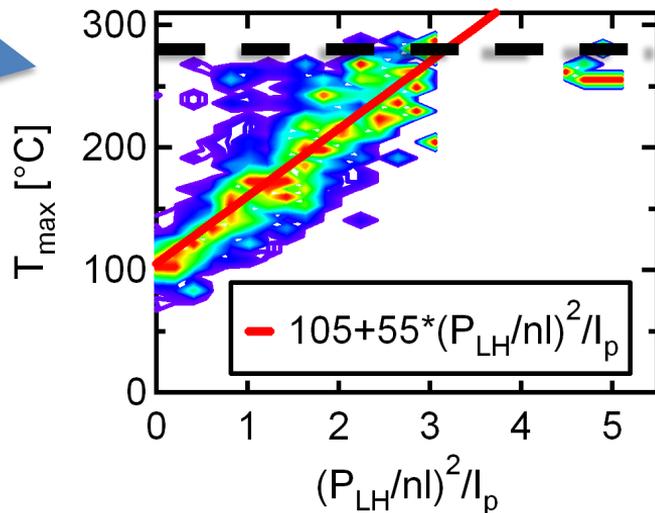
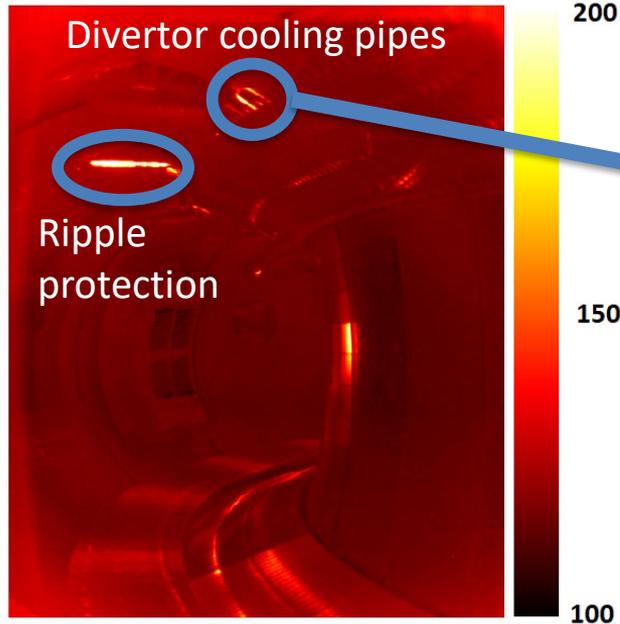
Real-time Infra Red machine protection (see talk by Ph. Moreau on Tuesday)

► Vessel protected against electron ripple losses

- Limit based on calorimetry evaluation $P_R^{lim} \sim P_{LH}^{1.85} n_I^{-2.2} I_p^{1.5}$

► Upper divertor cooling pipes : apparent temperature increases as well with LH power

- But different scaling with plasma current, as $T^{max} \sim P_{LH}^2 n_I^{-2} I_p^{-1}$: to be refined in high power domain (larger P_{LH})

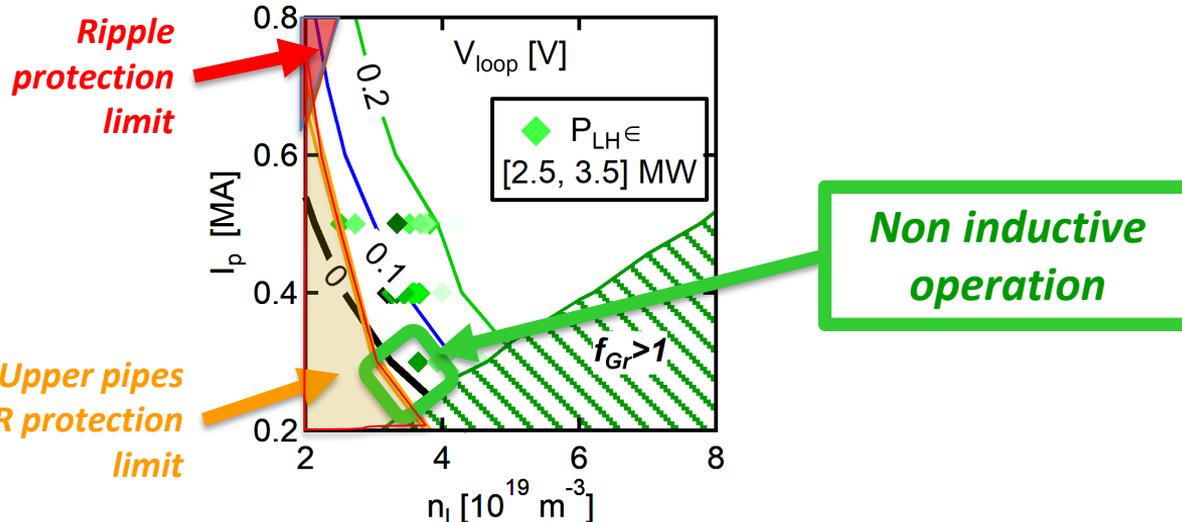


Real-time IR limitation at 275°C (T_{BB}) in place

Integrating all constraints in OD simulations with METIS at $f_{\text{rad,bulk}} = 40\%$

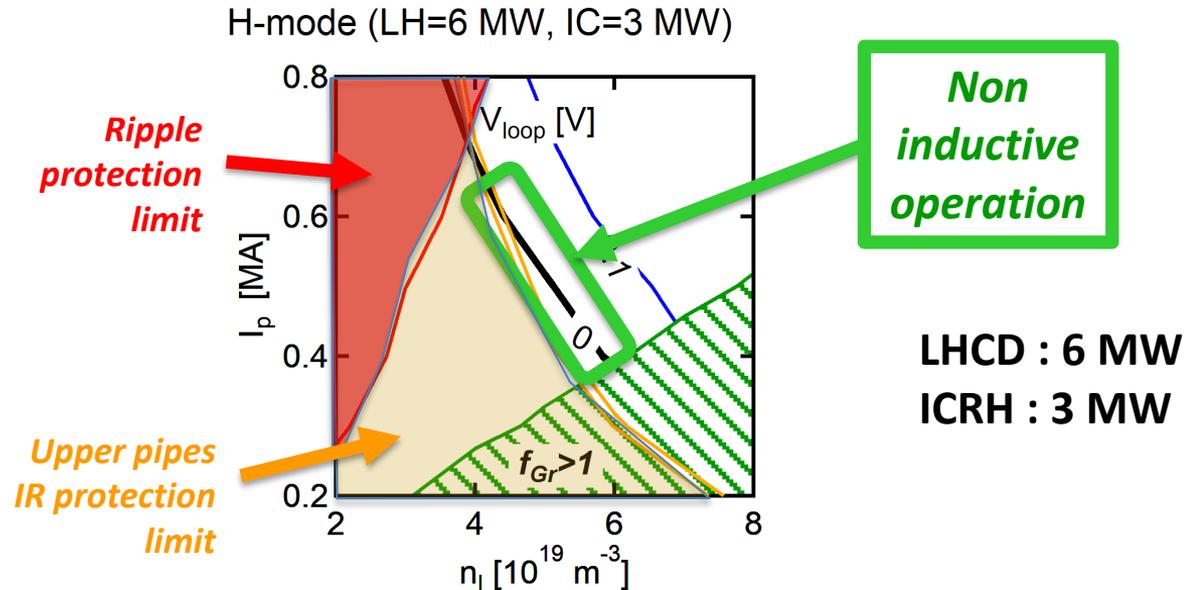
- ▶ Projection in L-mode and H-mode
- ▶ Non-inductive discharges in L-mode for 3 MW of RF power
 - Diagram consistent with present WEST data
 - Scaling on upper pipes temperature : density should be above a minimum $\sim 3 \times 10^{19} \text{m}^{-3}$

L-mode (LH=3 MW)



Integrating all constraints in OD simulations with METIS at $f_{\text{rad,bulk}} = 40\%$

- ▶ Projection in L-mode and H-mode
- ▶ Non-inductive discharges in L-mode for 3 MW of RF power
- ▶ Non-inductive discharges in H-mode for 9 MW of RF power :
 - Operational window opens in H-mode thanks to bootstrap current (~ 2) & CD efficiency increase



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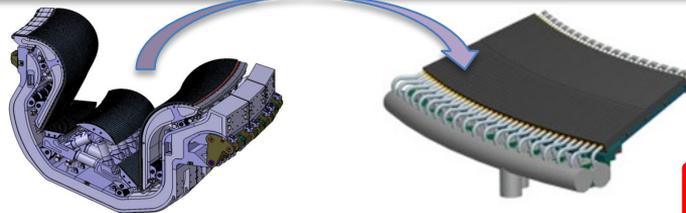
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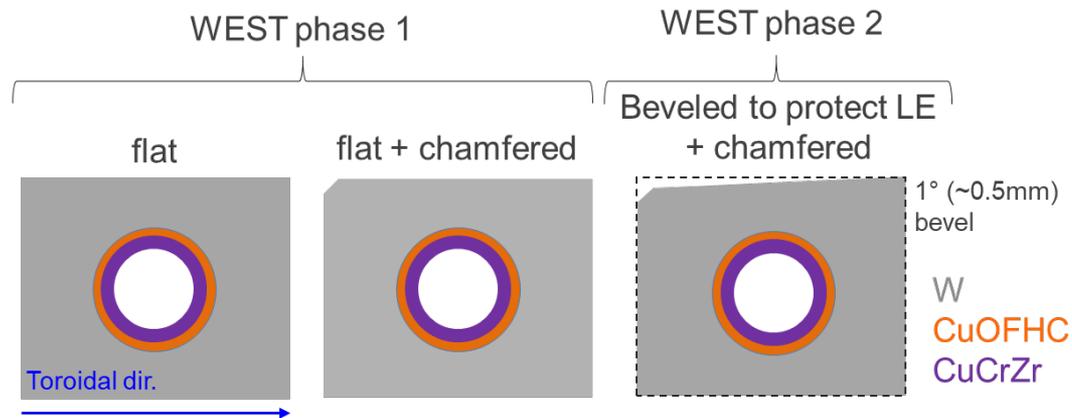
PFU shaping

High heat flux

Melting

High Fluence

Erosion / redeposition



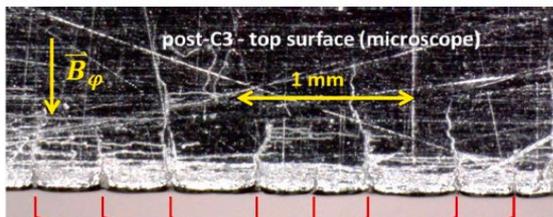
ITER PFU design: surface shaping and Optical Hot Spots

► Shaping : need for beveled PFU in ITER [Gunn'17,Gunn'19]

- Cracks on misaligned leading edges : induced by disruption on WEST (thermo-mechanical analysis [Durif'22])
- Beveled PFU : demonstration of protected leading edges (IR) [Grosjean'20] (post-mortem to be done)

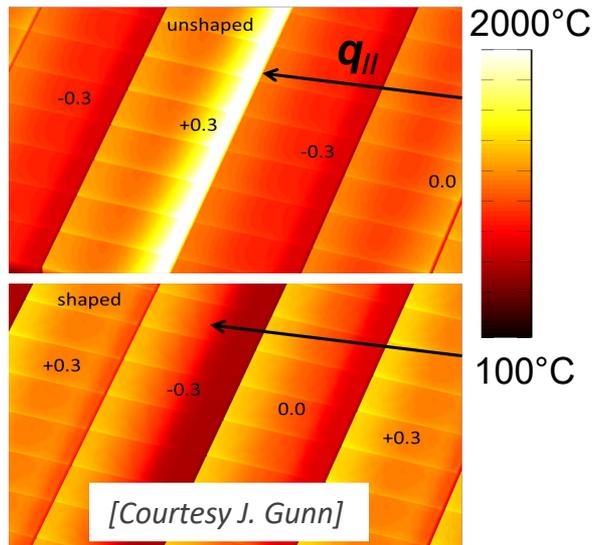
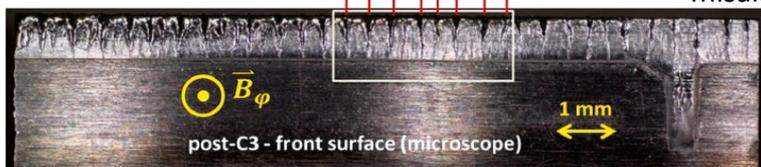
► Optical Hot Spots:

- Post-mortem evidence [Diez'21,Gunn'21]
- Risk expected during large ELMs [Gunn'17]

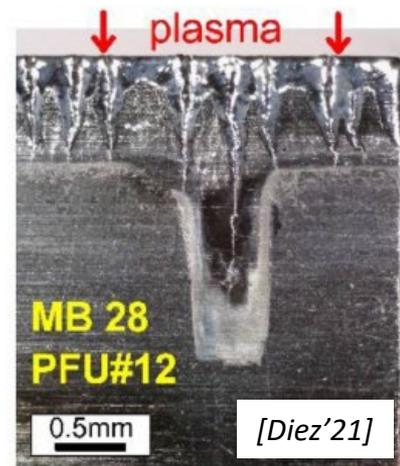


Leading edge
misalignment

[Gunn'21]



OHS (tor. gaps)



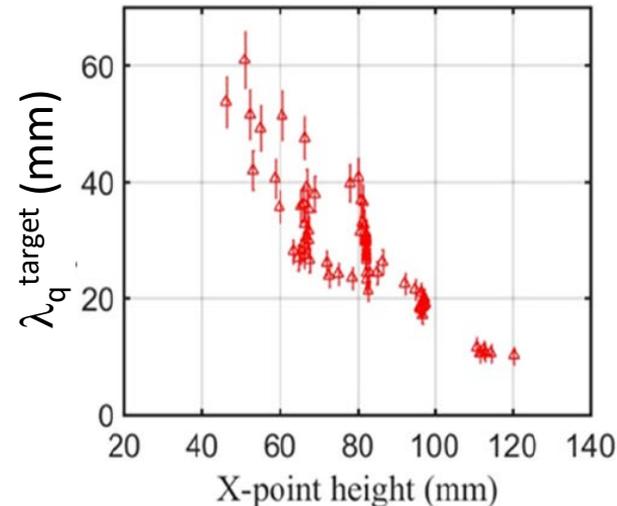
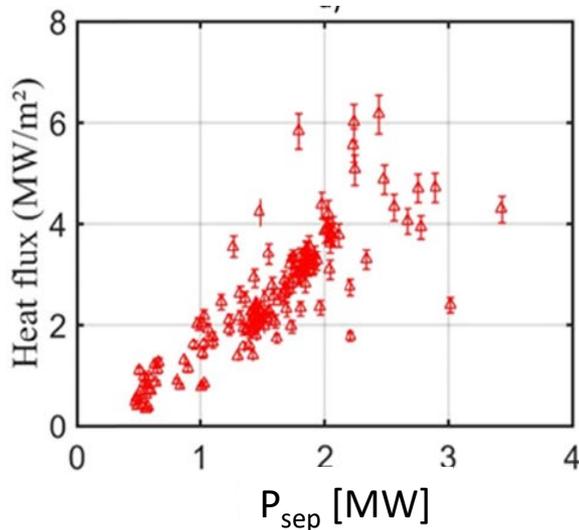
High heat flux on the ITER PFU, controlled by X-point height (magnetic compression)

► Goal 10 MW/m² (ITER steady-state) at reach:

- High X-point height and increased P_{sep} required, H-mode will help ($\lambda_q/2$ expected)

► Scaling law for λ_q at mid-plane from IR measurements

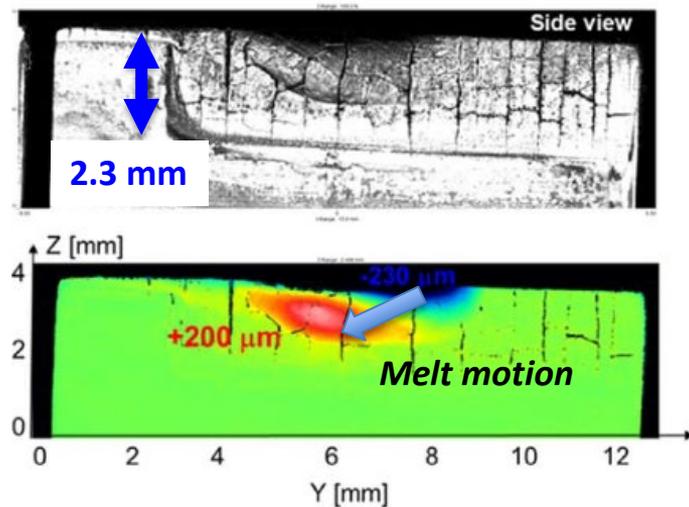
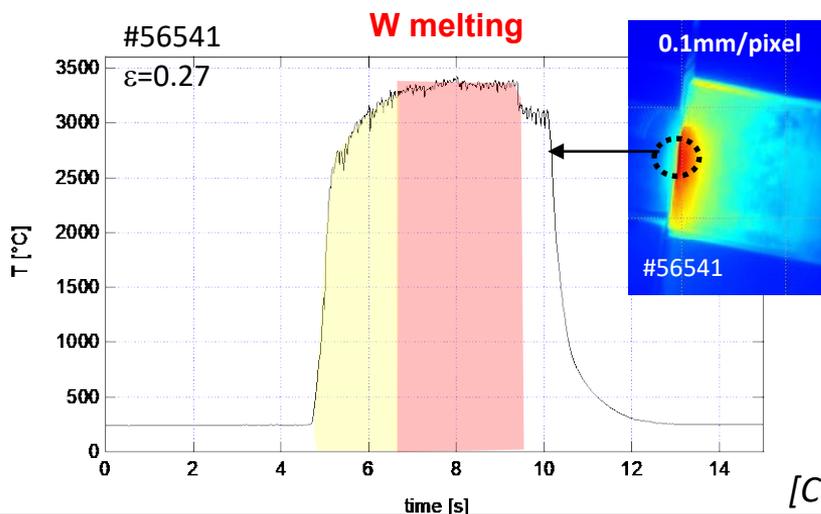
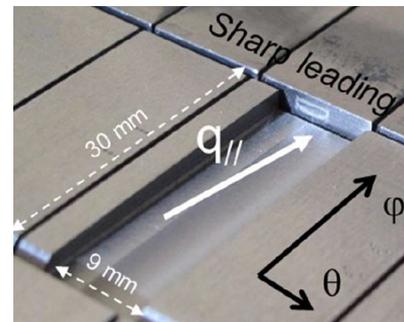
- Consistent with existing IR-based L-mode scaling [Scarabosio'13]
- Discrepancy between λ_q obtained from IR and other diagnostics to be understood



[Gaspar NF'21]
[Bucalossi NF'22]

Melting of actively cooled tungsten monoblocks

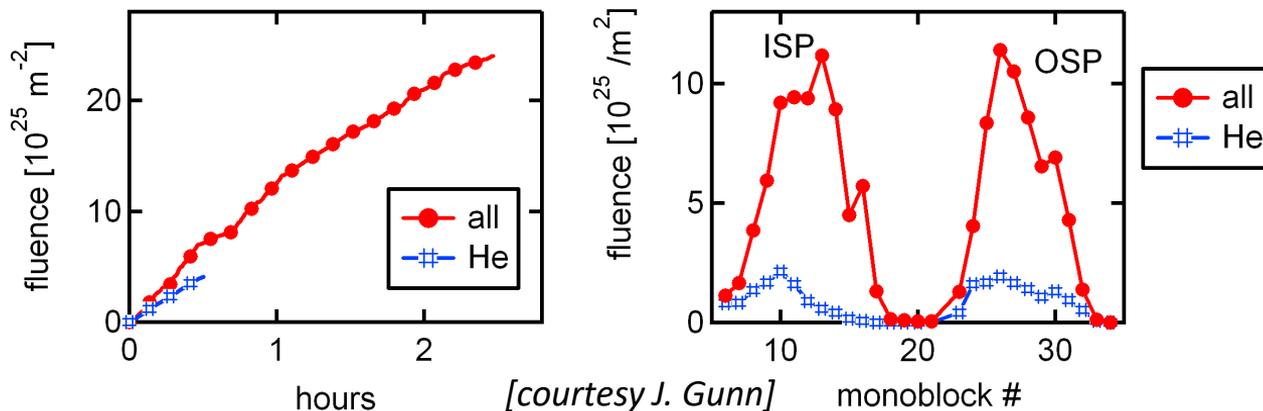
- ▶ **First sustained melting on actively cooled PFU (5 MW / 5 s of LH power)**
 - High X-point and machined groove at the Outer Strike Point : $q_{//} > 100 \text{ MW/m}^2$
 - Shallow melting, melt motion (JxB) driven by Thermo-ionic emission
 - Agreement with MEMOS simulation [Ratynskaia'22]
- ▶ **No impact on radiated fraction & plasma operation**
 - More intense melting experiments in preparation



[Corre, Phys. Script. '21]

High cumulated fluence in He and D: investigating W surface modification

- ▶ **Helium campaign : a fluence $\sim 2 \times 10^{25} \text{m}^{-2}$ has been achieved on tungsten inertial PFU (30' of plasma)**
 - No sign of macroscopic W surface modification (fuzz formation conditions reached)
 - Post-mortem analysis on-going (surface, He bubbles)
- ▶ **Total fluence (from 1st measurements) $\sim 2.5 \times 10^{26} \text{m}^{-2}$ (2h 30' of plasma, 7h from 1st WEST plasma)**
 - Represents $\sim 25\%$ of an ITER pulse ($\sim 10^{27} \text{m}^{-2}$)
 - Symmetric between ISP and OSP : remains to be understood
- ▶ **ITER fluence in both He and D are foreseen**
 - Reach the fluence of ~ 1 ITER pulse
 - Pre-characterization and Post-mortem for in-depth ageing studies

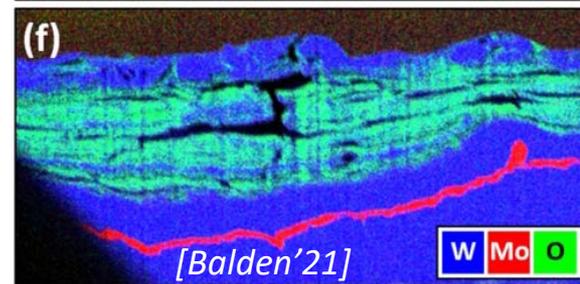
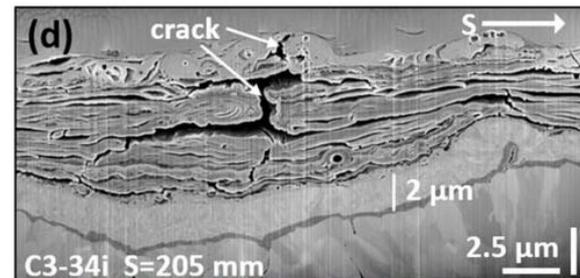
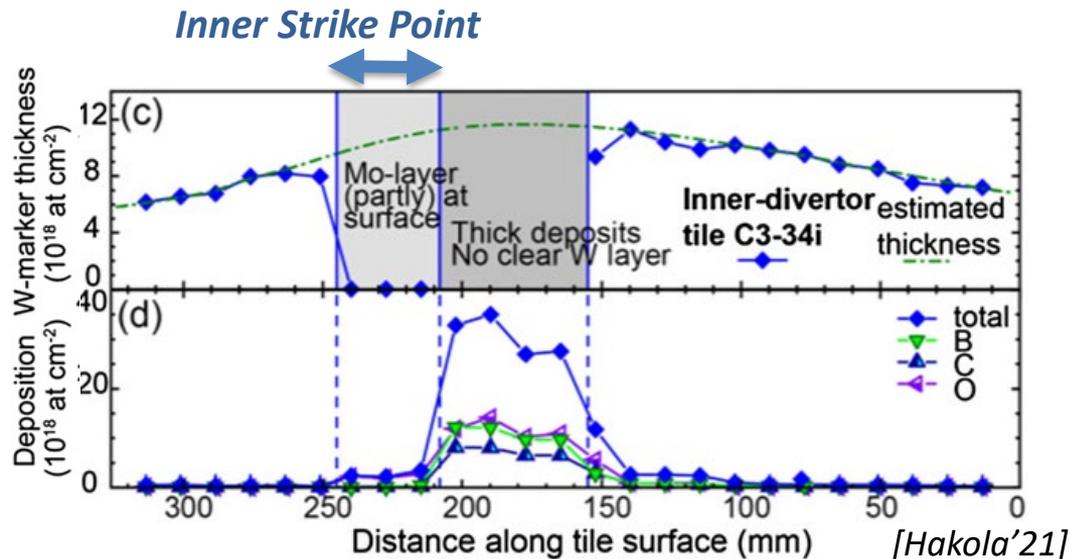


Erosion / redeposition patterns

- ▶ Erosion evaluated as ~ 0.1 nm/s at Strike Points in L-mode (as in AUG and W7-X) [Balden'21]
- ▶ Thick deposits (W, B, C, O) outside erosion area [Hakola'21, Balden'21, Martin'21]

Perspectives for long pulse operation

- ▶ Evolution of deposited layers : flakes formation & ejection ? [Pégourié'09]



First steps towards a suitable scenario for ITER PFU testing on Long Pulse Operation

- ▶ **Integrated scenario development towards low loop voltage, high power to SOL**
- ▶ **CD efficiency to be probed vs SOL characteristics in H-mode, IPD, low-Z seeding conditions**
- ▶ **MHD stability (hollow current profile) can be mitigated with a limited ECCD power**
 - ECCD will be a key player to manage current profile control (installed late 2023)
- ▶ **Radiative fraction robustly around 50%**
 - On-going work : Mitigate W sources / Vessel conditioning / Impurity Powder Dropper
 - However H-mode threshold accessible : a domain to be explored in future campaigns
- ▶ **Vessel protection limits integrated in the preparation of Long Pulse Operation**
 - Synergy effect between LHCD and ECCD can be exploited to mitigate IR vessel protection
 - H-mode regime opens the operational space

First results on ITER PFU characterization in WEST

- ▶ **PFU shaping mandatory to avoid premature W-cracking and damages**
 - → 0.5mm toroidal bevel on ITER
- ▶ **Optical Hot Spots : post-mortem evidence in the absence of shaping**
- ▶ **Demonstration that large heat flux accessible at high X-point height**
 - 10 MW/m² can be reached in WEST even in L-mode
 - Multiple diagnostic evaluation of λ_q : shows differences that remain to be investigated
- ▶ **First melting experiments show good agreement with modeling**
 - Shallow melting observed → deeper melting pool foreseen
 - no impact on plasma operation (P_{rad})
- ▶ **Helium fluence experiments showed no W fuzz formation**
 - ISP / OSP symmetry is measured
- ▶ **Deuterium & He fluence exp. planned to reach ~ITER fluence**
 - evolution of deposited layers is a key aspect to investigate

- [Balden'21] Balden M. et al, Phys. Scr. 96 (2021) 124020
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- [Yang'20] Yang X. et al, Nucl. Fusion 60 (2020) 086012

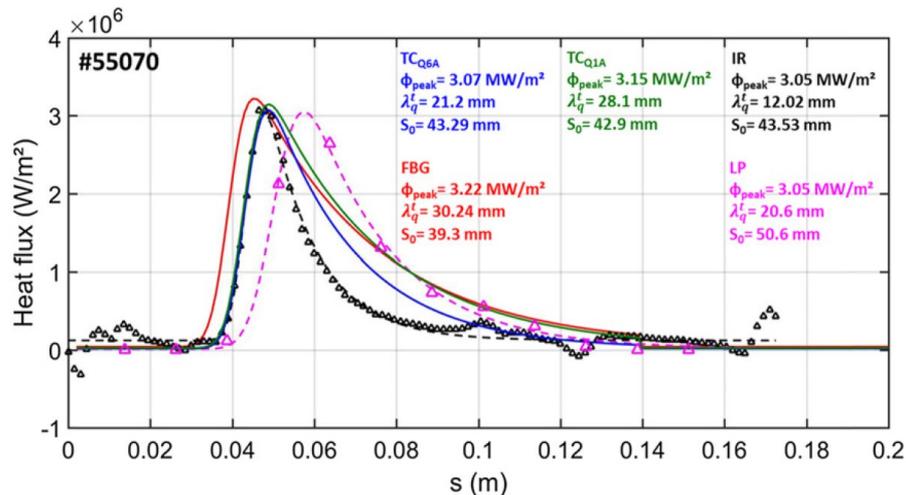


Thank you for your attention



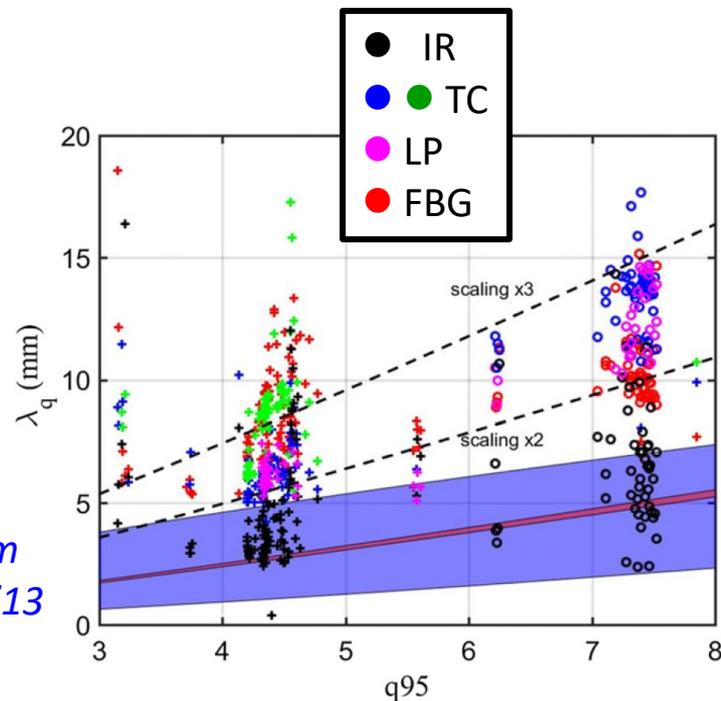
Discrepancy between λ_q evaluation from different diagnostics

- ▶ IR, Langmuir probes, Thermocouples, FBG
- ▶ IR evaluation consistent with previous IR-based scaling [Scarabosio'13]



[Gaspar NF'21]

Scaling from
Scarabosio'13





End back-up slides

