



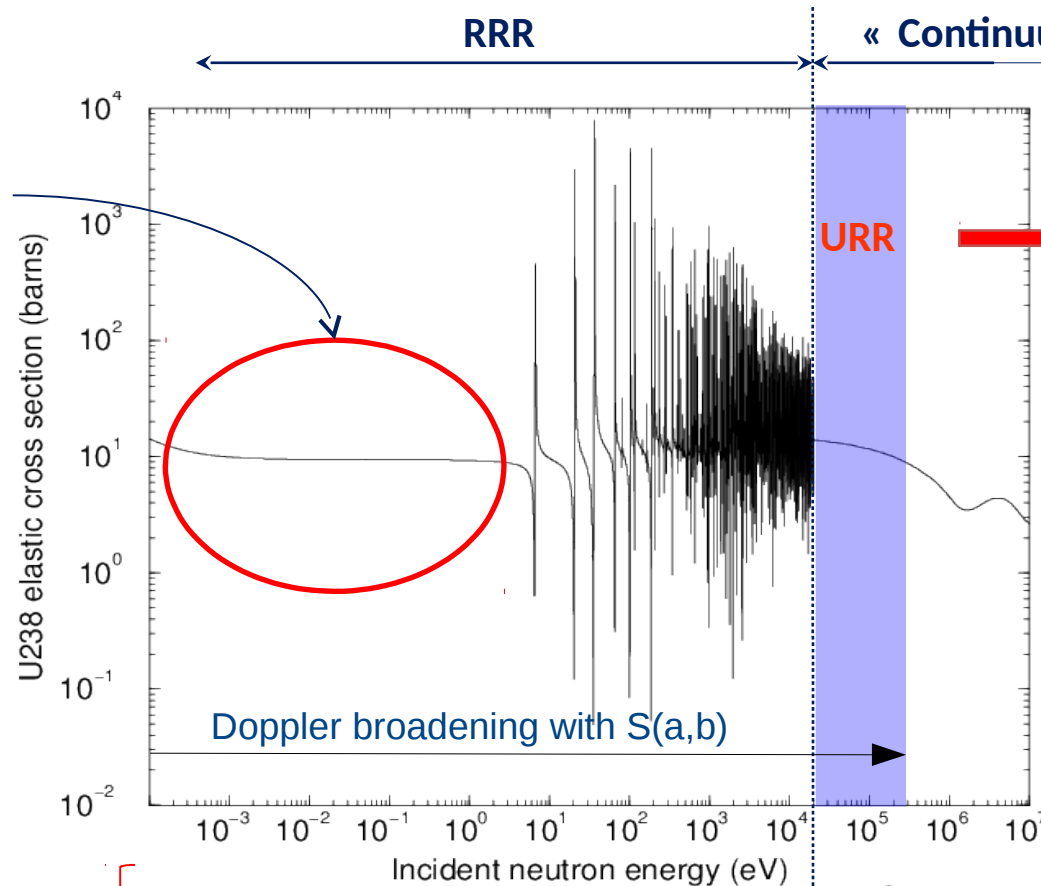
# From nuclear model parameters to neutronic simulation with the CONRAD and LAST codes

DE LA RECHERCHE À L'INDUSTRIE

**Consultancy Meeting on model code output & application nuclear data form structure  
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CINELmodel  
Thermal  
Scattering Law



Average resonance  
parameters  
calculated with the  
URR option of TALYS

↓  
Probability Tables

+ for fission reaction  
PFNS model  
Neutron multiplicity model  
(n,gf) model  
Energy dependent  
penetration factor model  
...

Conrad

ESTIMA methos

SPRT method

**The Library and Stochastic Transport (LAST) code** is a Monte-Carlo neutron transport code developed at CEA Cadarache. It consists in C++ modules for intensive-computation functions, all led by a python wrapping. It handles Open-MPI parallel computation to improve calculation efficiency and reduce the calculation time. A few basic scores are available, such as  $k_{eff}$ , neutron flux and adjoint flux (sensitivity calculations, perturbation theory). The  $k_{eff}$  calculation accuracy has been numerically validated against the CEA reference code TRIPOLI-4

**The LAST code is coupled with the nuclear data evaluation code CONRAD.** Therefore, both standard ENDF-6 files and CONRAD theoretical files (containing nuclear model parameters and the associated covariance matrix) can be used to perform simulations. The coupling of CONRAD and LAST ensures the consistency among different nuclear data.

+ in the case where some evaluated data are preferred rather than the CONRAD theoretical calculation, LAST can merge the desired ENDF data with the CONRAD calculation.

**Unresolved Resonance Range (URR)** : limitation due to the width fluctuation correction in NJOY

**Thermal Scattering Law (TSL)** : how to handle huge  $S(\alpha, \beta)$  covariance matrix ?

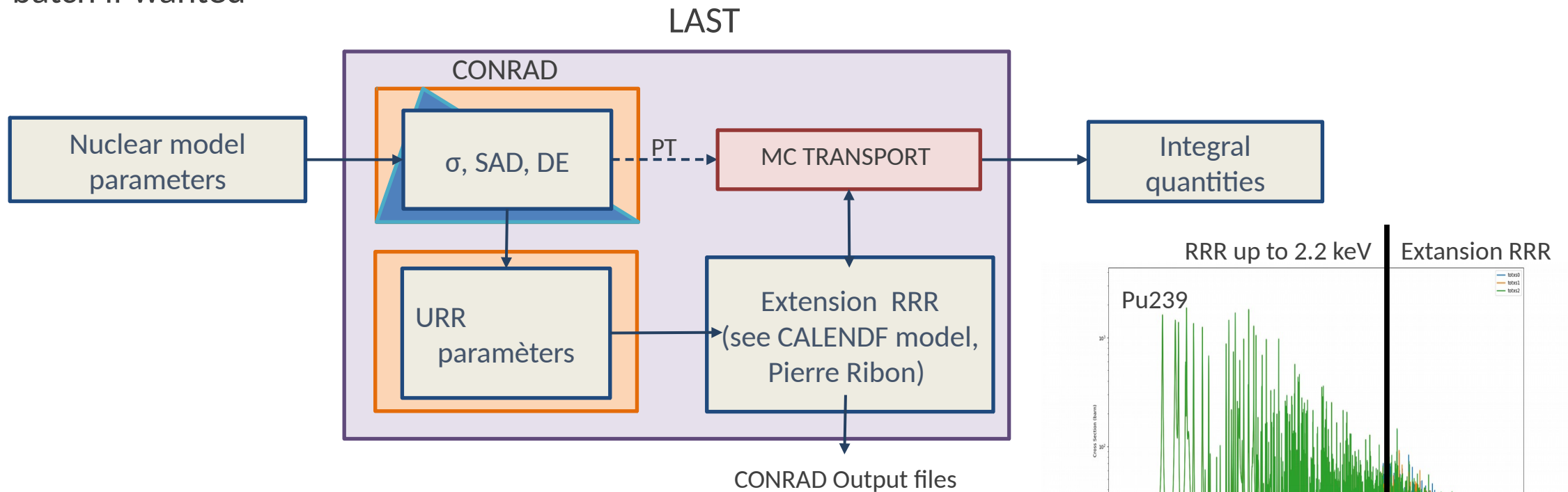
**Scattering Angular Distribution (SAD)** : correlations between SAD and neutron cross sections

**Fission models** : progress in the resonance range with (n,gf) reaction, Class-II states,  $P_f(E)$

**Direct capture in the RRR** : imaginary part of the infinite radius as in REFIT (not yet implemented in CONRAD, not discussed here)

...

Random extension of the RRR by using URR parameters (+average parameter statistical laws) at each batch if wanted



- No URR format limitation
- No RRR/URR cross correlation issues
- No Probability Tables (PT)

Molecular Dynamic parameters and covariance matrix

**Table 5.** Relative uncertainties and correlation matrix between the CAB model parameters after the marginalization.

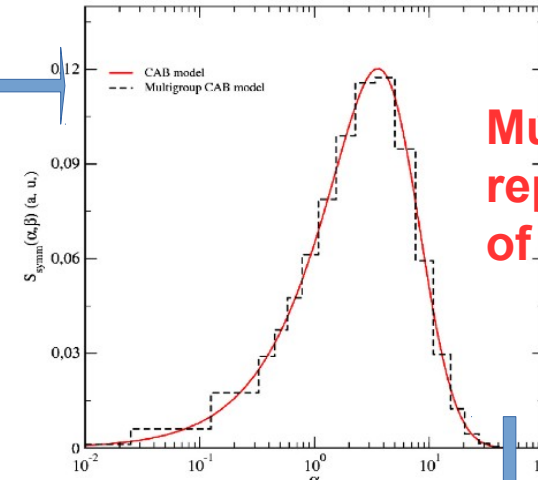
Parameter	Value	Relative uncertainties	Correlation matrix								
$\sigma_0$ (nm)	0.31644	2.3%	100	-77	93	69	33	-18	-64	83	-14
$\sigma_{OH}$ (nm)	0.7749	14.6%		100	-71	-98	-85	53	97	-54	-32
$q_H$ (e <sup>-</sup> )	0.5564	3.2%			100	59	28	-2	-60	81	-18
$d_{OH}$ (nm)	0.09419	6.3%				100	89	-63	-96	44	38
$D_{OH}$ (km <sup>2</sup> /mol)	432.581	6.2%					100	-63	-88	6	57
$\beta_{OH}$ (1/nm)	22.87	4.2%						100	51	-11	-28
$\theta_{OH}$ (°)	107.4	6.4%							100	-45	-9
$k_\theta$ (kJ/mol/rad <sup>2</sup> )	367.81	3.8%								100	-14
$d_{OM}$ (nm)	0.13288	2.7%									100

10 direct calculations only

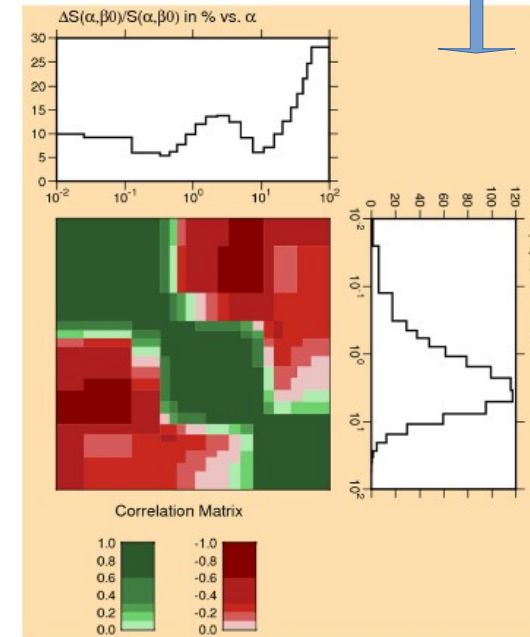
**Table 7.** Example of uncertainties on the reactivity (MOX configuration at room temperature) in pcm due to the nuclear data. The contribution of <sup>1</sup>H in H<sub>2</sub>O comes from the present work. The other contributions were calculated with the covariance data base COMAC [32] developed at the CEA of Cadarache.

Isotopes	(n,f)	Capture	(n,n)	(n,n')	(n,xn)	$\nu_{tot}$	$\chi_{fast}$	$\chi_{th}$	Total
<sup>1</sup> H in H <sub>2</sub> O		46	110						119
<sup>10</sup> B		8							8
<sup>16</sup> O		114	24	4					117
<sup>90</sup> Zr		11	24	7					27
<sup>91</sup> Zr		13	16	4					21
<sup>92</sup> Zr		8	22	4					24
<sup>94</sup> Zr		2	59	3					59
<sup>96</sup> Zr		2	13	1					14
<sup>235</sup> U	2	6	3	1		5		4	9
<sup>238</sup> U	114	88	80	-60	25	35	12		160
<sup>238</sup> Pu	1	70	-20	1		9	1		67
<sup>239</sup> Pu	278	371	26	5		57	0	126	484
<sup>240</sup> Pu	42	178	-16	-5	1	2	9		182
<sup>241</sup> Pu	108	96	8			88	58		179
<sup>242</sup> Pu	3	131	10	2		2	1		131
<sup>241</sup> Am	-3	47	2	29		1			47
Total	322	475	156	-59	25	111	60	126	619

Uncertainty in pcm on keff (UOX fuel)



Multigroup representation of the S(alpha,beta)



Multigroup S(alpha,beta) covariance matrix

Problems to get positive definite matrix



## Optical Model Parameters and covariance matrix

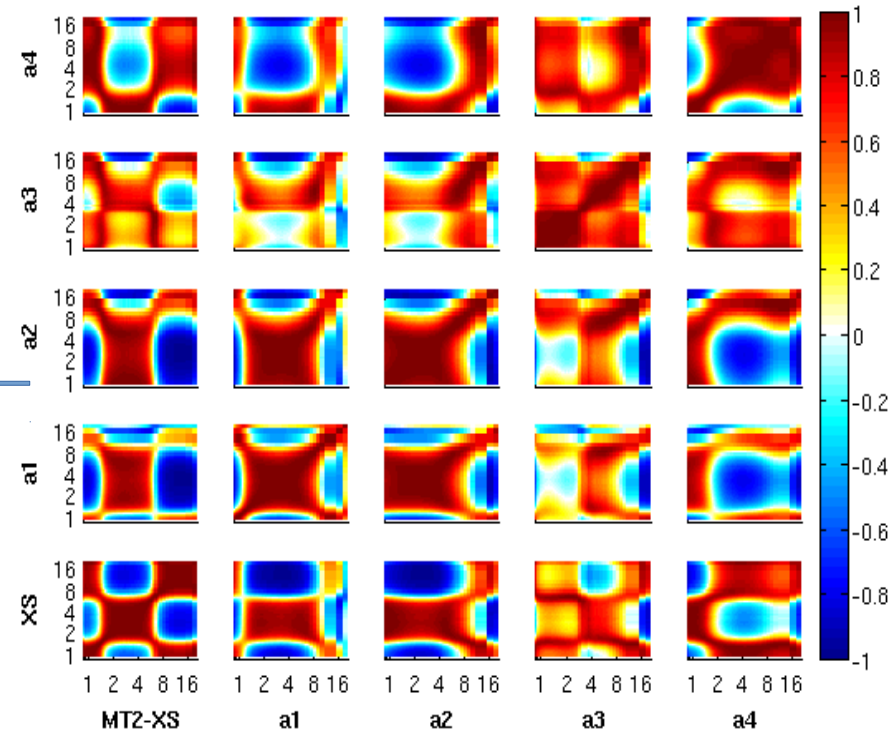
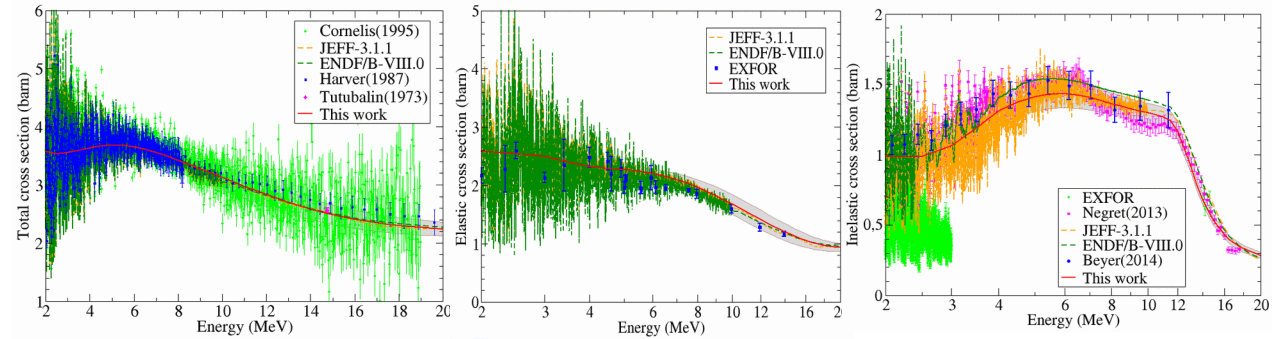
Table 1. Model parameters and the corresponding uncertainty ( $1\sigma$ ) and the correlation matrix for  $n+^{56}\text{Fe}$  reaction\*

Parameter	$A_{S0}$ (MeV)	$V_{HF}$ (MeV)	$a_0$ (fm)	$r_0$ (fm)	$T(^{56}\text{Fe})$ (MeV)
Value	15.126	92.627	0.6032	1.224	1.352
Uncertainty	1.966	9.554	0.0551	0.019	0.118
Correlation matrix	1.000				
	-0.486	1.000			
	-0.851	0.312	1.000		
	0.325	-0.808	-0.307	1.000	
	0.487	0.104	-0.575	-0.414	1.000

Use perturbation formalism

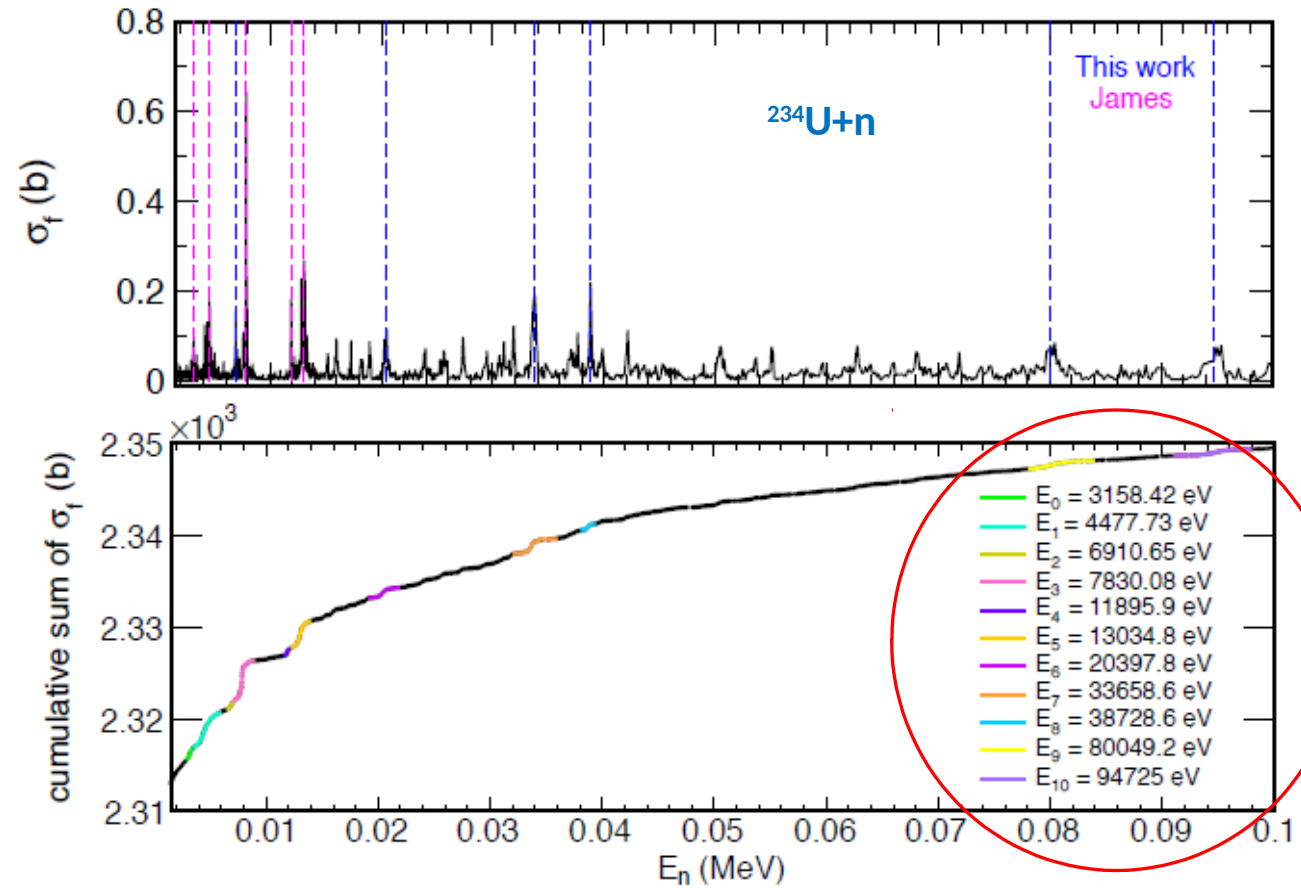
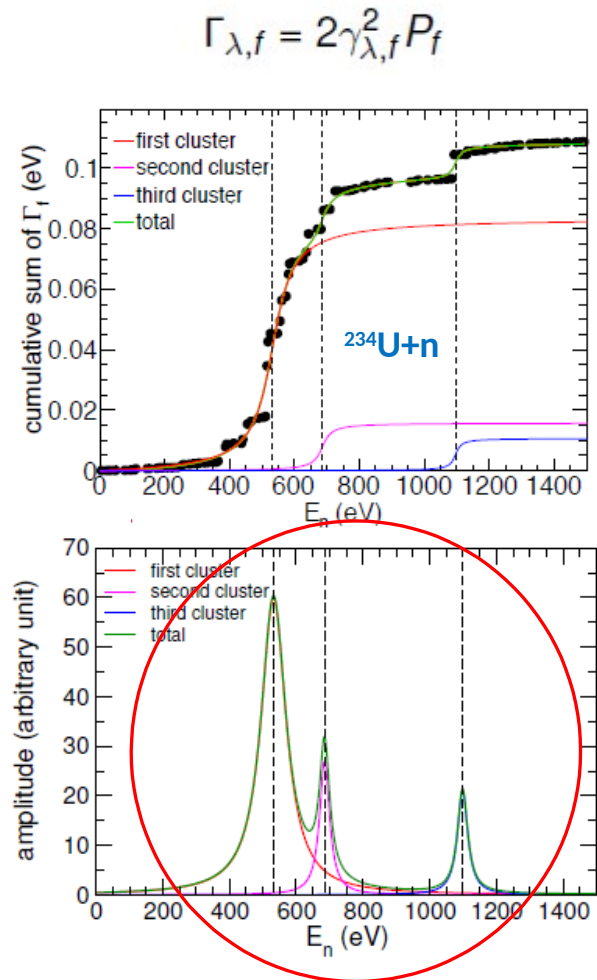
Table 5. Uncertainty (in pcm) concerning  $k_{eff}$  from the OMP parameters and the correlation between AD and XS deduced from  $k_{eff}$ .

Benchmark	Perturbation	$\Delta k_{eff}$	$\Delta k_{eff}/k_{eff}$	$\Delta\rho$
Thermal	AD	46.8±2.0	49.0±2.1	51.2±2.2
	XS	70.8±1.7	74.1±1.8	77.5±1.9
	AD&XS	67.9±1.7	71.0±1.7	74.3±1.8
	Correlation		-0.3924±0.0454	
Fast	AD	95.7±2.0	94.1±2.0	92.5±2.0
	XS	172.7±1.9	169.9±1.9	167.1±1.9
	AD&XS	71.7±2.2	70.5±2.2	69.3±2.2
	Correlation		-1.0243±0.0165*	



Correlation matrix between SAD and first-four orders Legendre coefficients

Keff uncertainties (thermal and fast benchmarks) and correlations between SAD and XS



Class-II states included in the RRR and URR analysis : **How to store this information in nuclear data files ?**





**Merci de votre attention**