



# Discharge Modeling in EAST Using Bidirectional LSTM

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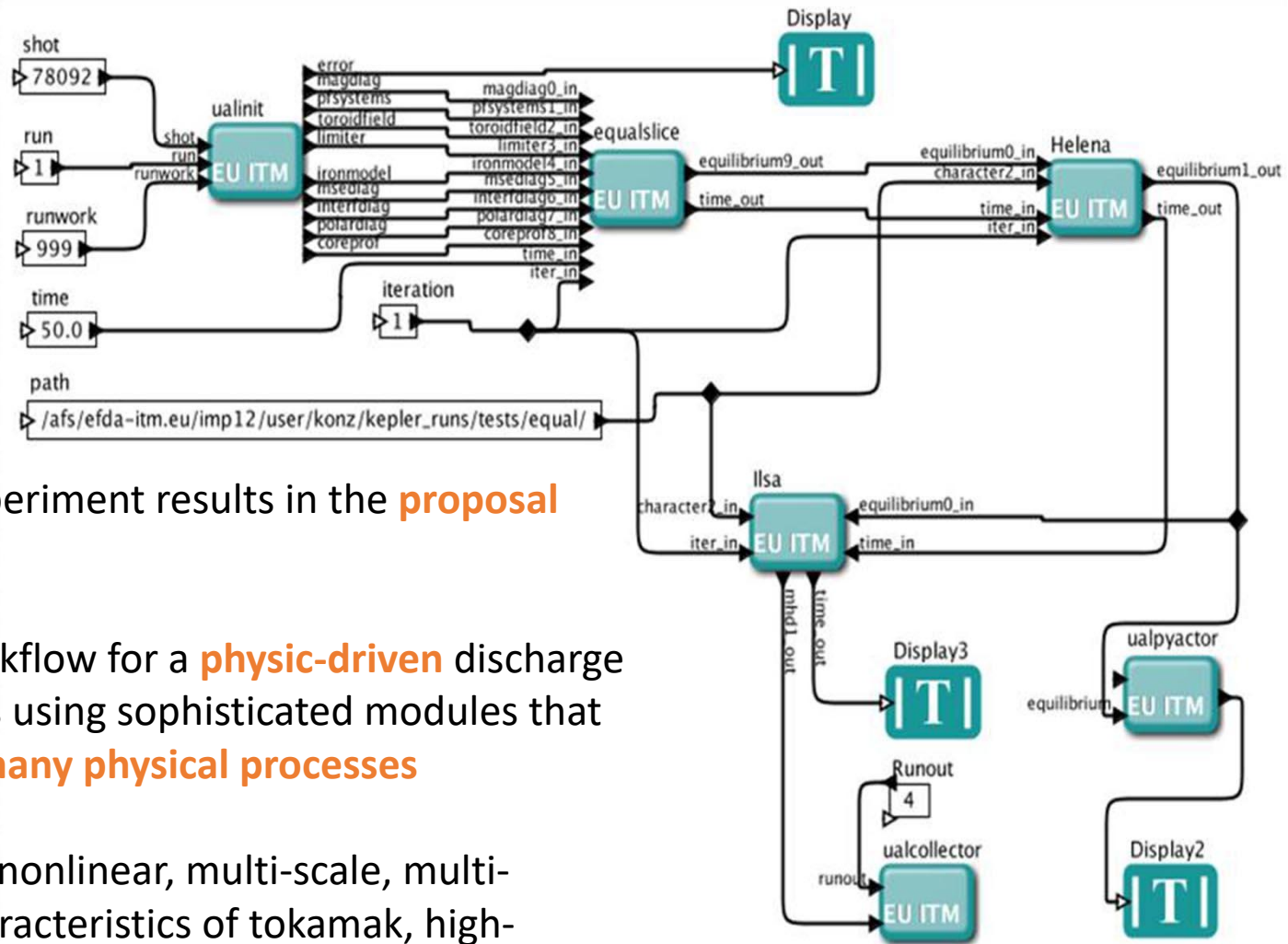


- Background
- Method
- Results
- Conclusions



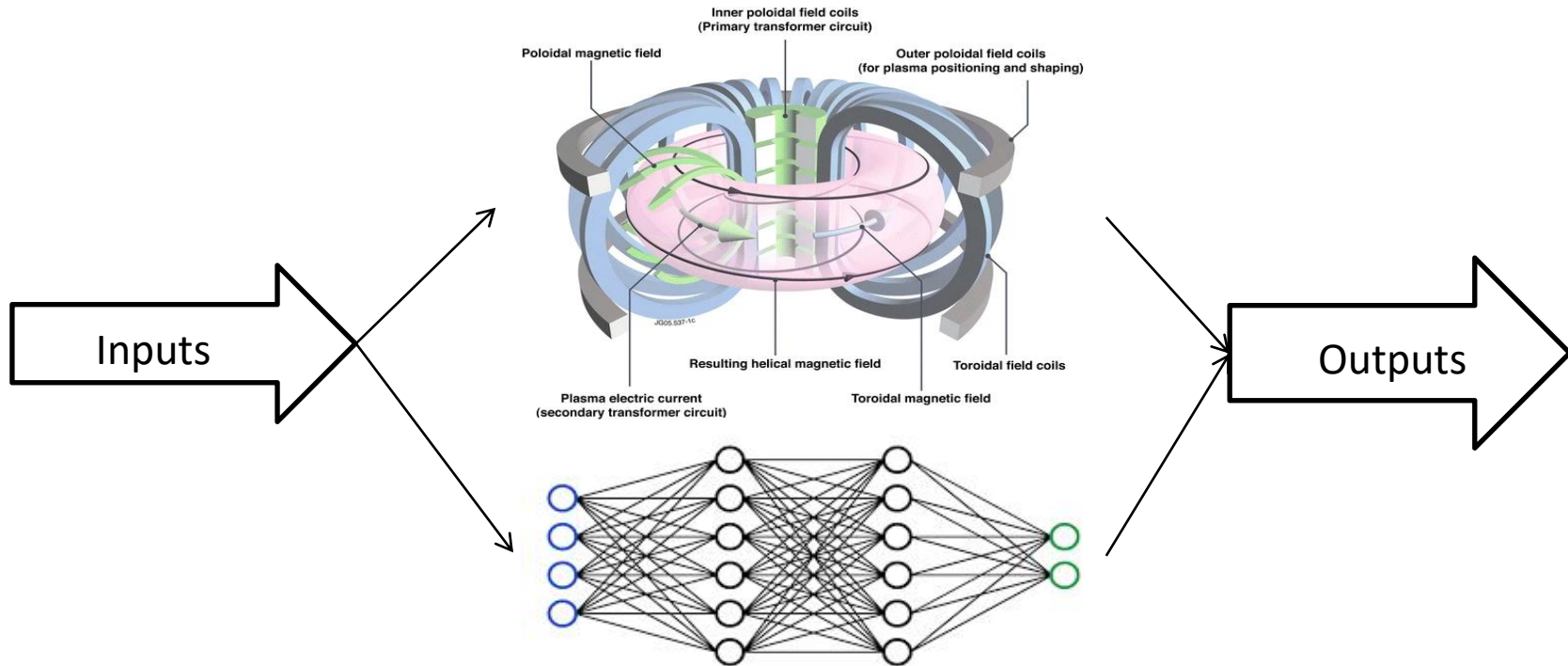
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# Discharge modeling



- Getting experiment results in the **proposal stage**
- Typical workflow for a **physic-driven** discharge modeling is using sophisticated modules that **integrate many physical processes**
- Due to the nonlinear, multi-scale, multi-physics characteristics of tokamak, high-fidelity discharge modeling is still a **great scientific challenge**

Workflow of integrate modeling by G.L. Falchetto et al  
2014 Nucl. Fusion 54 043018



- Divided the tokamak data into **three** categories: **actuator signals** ( NBI, ICRH, etc), **diagnostic signals** ( $W_{mhd}$ ,  $n_e$ , etc.), and **configuration parameters** (position of the poloidal magnetic field (PF) coils, etc.).
- The machine learning discharge modeling can be essentially reduced to a process of mapping **actuator** (input) signals to **diagnostic** (output) signals while the configuration parameters are unchanged.



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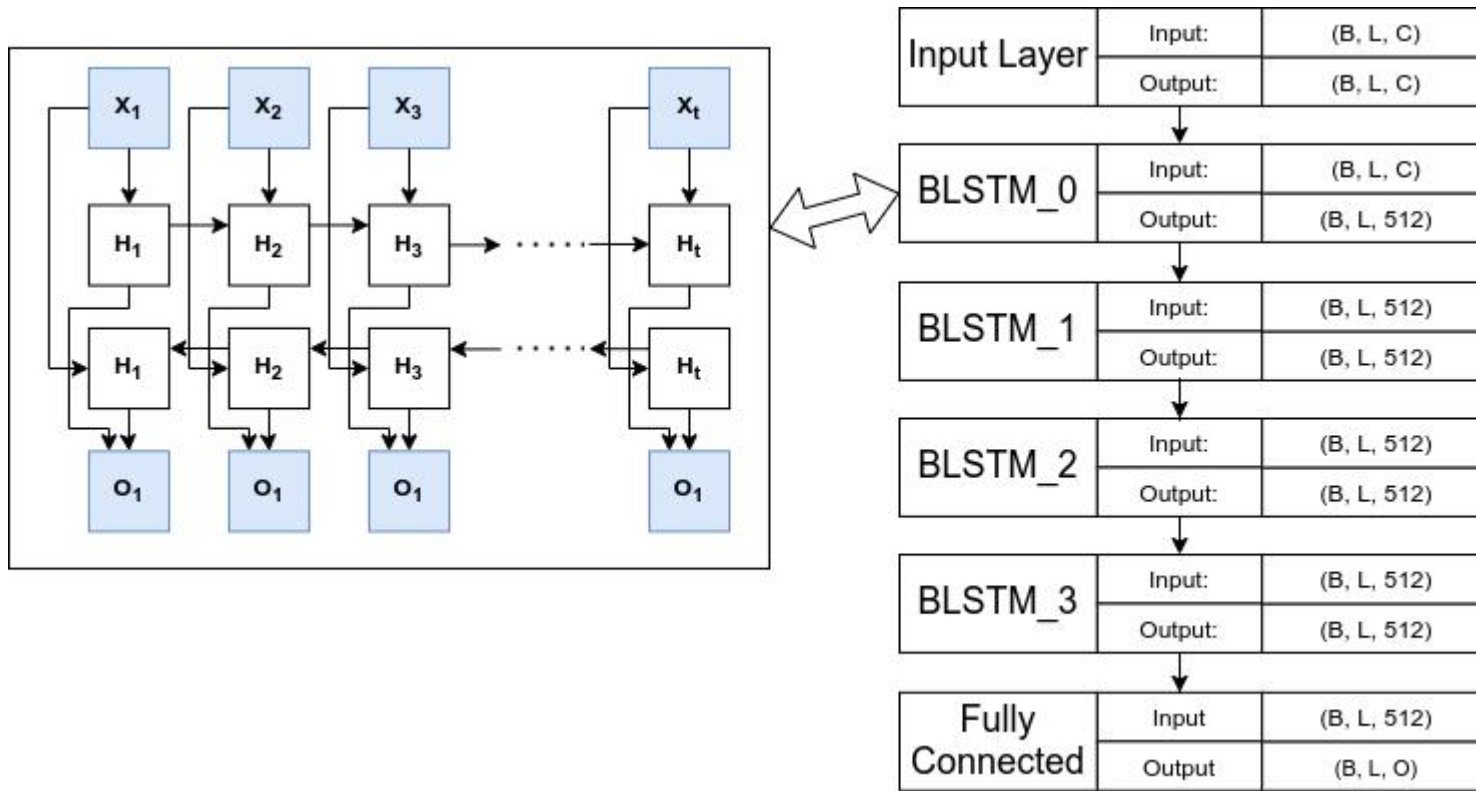


Fig 4. Architecture of BiLSTM

- Reason
  - Discharge modeling is a **offline** modeling task, so the **contextual information** is available and equal vital with past information during the experiment proposal stage
  - The pervious works only using **past** information

Signals	Physics meanings	Unit
<b>Output Signals</b>		
Act. $I_p$	Actual plasma current	A
$n_e$	Electron density	$10^{19} m^{-3}$
$W_{mhd}$	Plasma stored energy	J
$V_{loop}$	Loop voltage	V
$\beta_n$	Normalized beta	dimensionless
$\beta_t$	Toroidal beta	dimensionless
$\beta_p$	Beta poloidal	dimensionless
$\kappa$	Elongation at plasma boundary	dimensionless
$l_i$	Internal inductance	dimensionless
$q_0$	q at magnetic axis	dimensionless
$q_{95}$	q at 95% flux surface	dimensionless
<b>Feedback Signal</b>		
sycic1	In-vessel coil no.1 current	A
<b>Input Signals</b>		
Ref. $I_p$	Reference plasma current	A
PF	Current of Poloidal field (PF) coils	A
$B_{t0}$	Toroidal magnetic field	T
LHW	Power of Lower Hybrid Wave Current Drive and Heating System	kW
NBI	Neutral Beam Injection System	Raw signal
ICRH	Ion Cyclotron Resonance Heating System	Raw signal
ECRH/	Electron Cyclotron Resonance Heating/Current Drive System	Raw signal
ECCD		
GPS	Gas Puffing System	Raw signal
SMBI	Supersonic Molecular Beam Injection	Raw signal
PIS	Pellet Injection System	Raw signal
Ref.	Shape reference	Raw signal
Shape		

- Output signals
  - Eleven key diagnostic signals can be obtained **stably**.
- **Feedback signal**
  - According to the magnetic control logic diagram, the in-vessel coil (IC) must be included.
- Input signals
  - Auxiliary heating system, shape reference, magnetic system, etc.



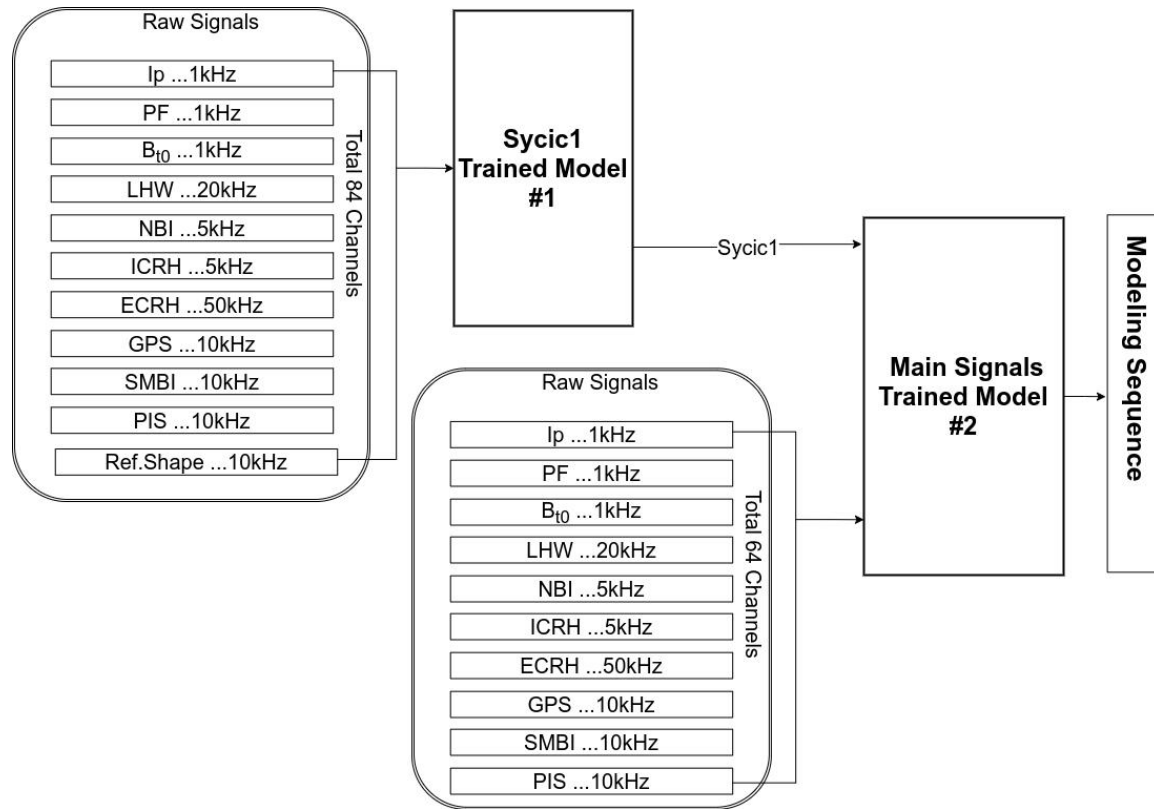


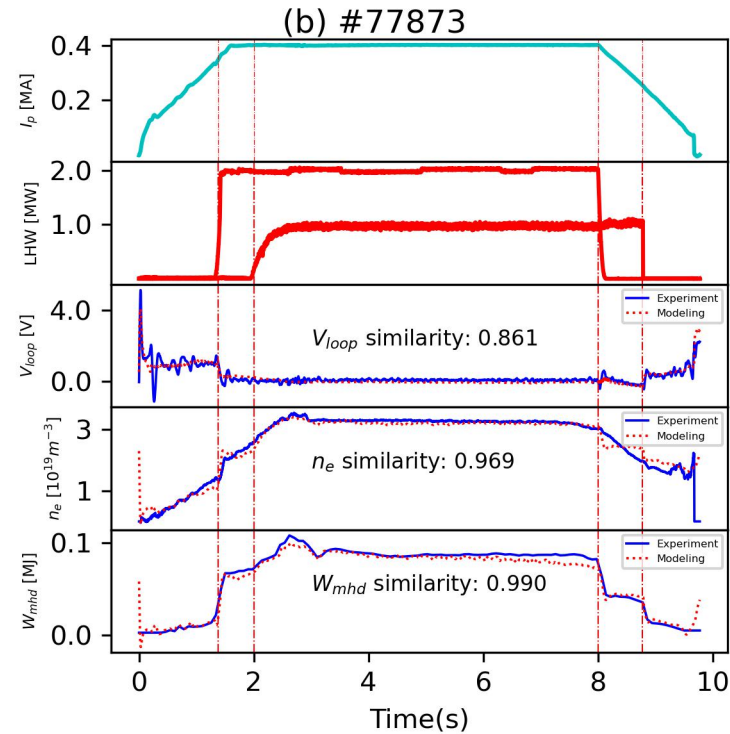
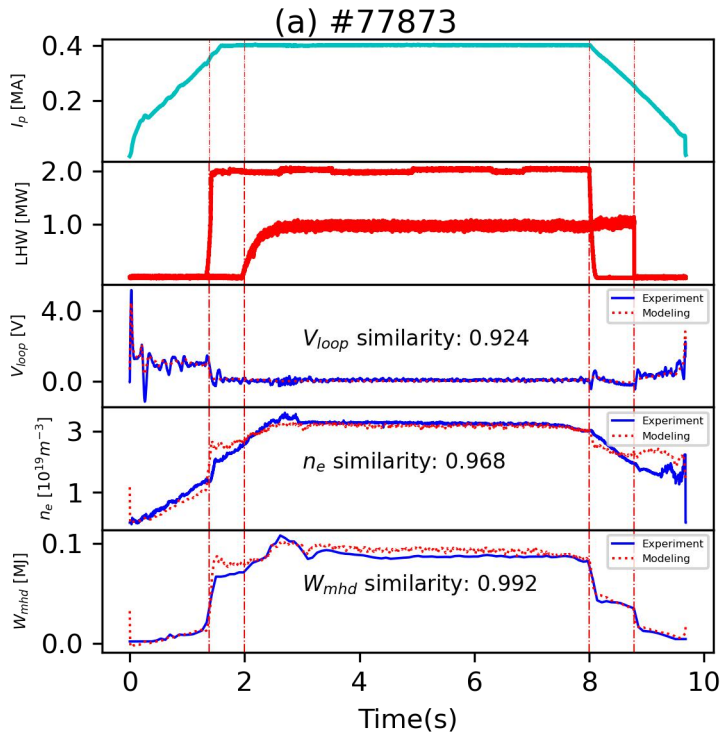
Fig 2. Workflow of the inference

- Feedback signal
  - Modeling “**sycic1**” first and then modeling main diagnostic signals.



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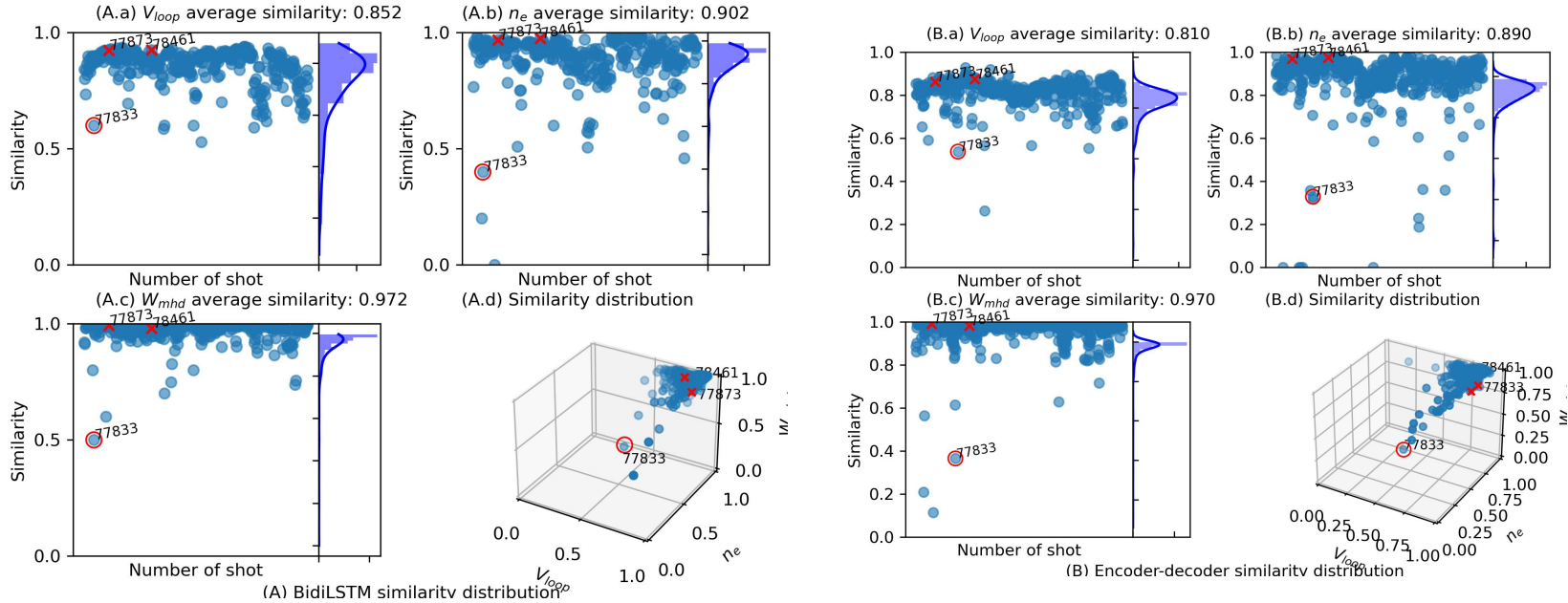
# Comparison-Typical shot



The results of bidirectional LSTM (a) and past information model (b)

- The comparison shows the bidirectional LSTM can get better modeling results of  $V_{loop}$  than model only using the past information even though **not** using adaptive resampling and actual plasma current.
- The BiLSTM is more sensitive to the rising edges of the auxiliary heating signals than past information model.

# Comparison-Distribution

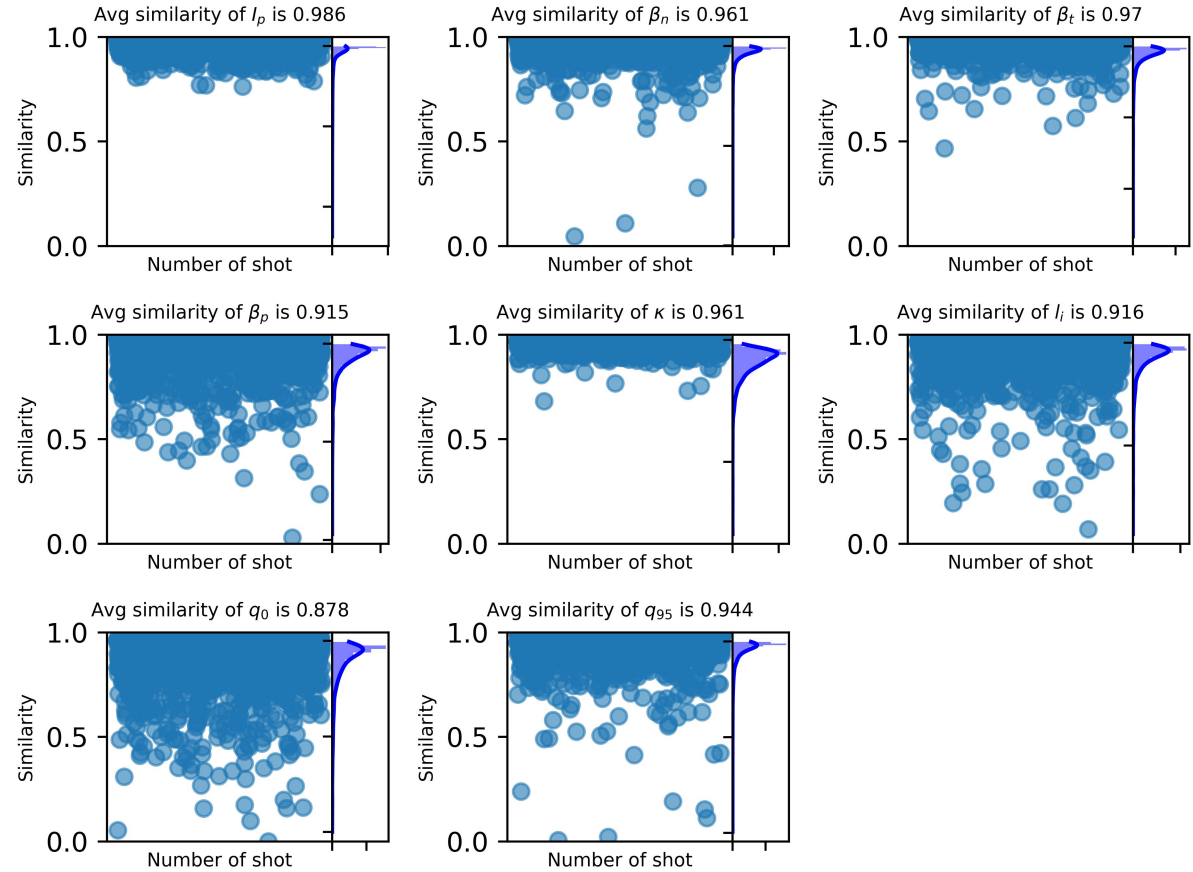
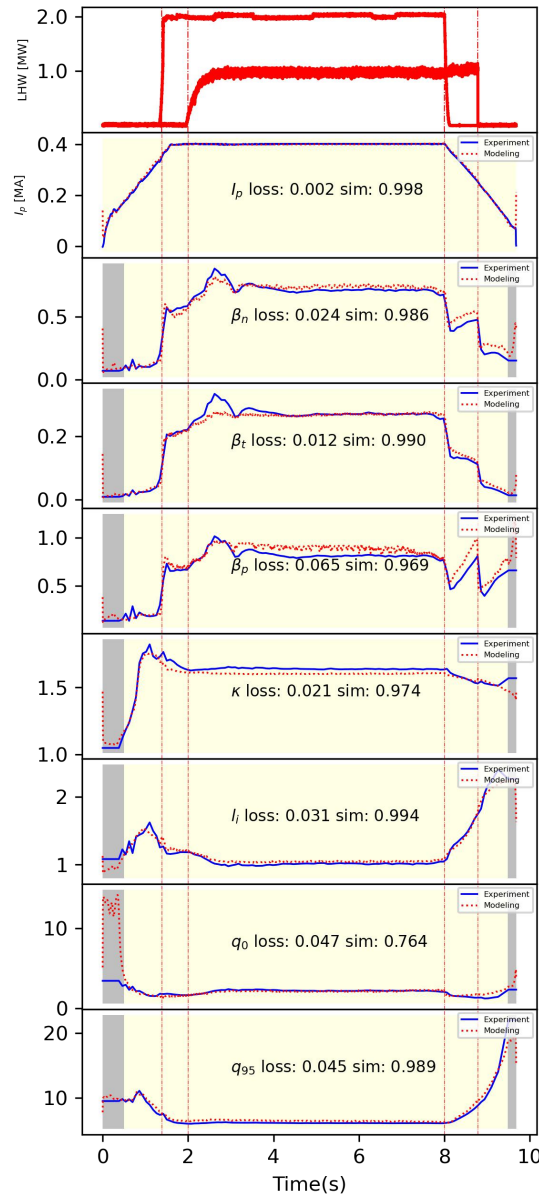


- The similarity of electron density  $n_e$  and loop voltage  $V_{loop}$  is improved by **~1%**, and **~5%**.
- The  $W_{mhd}$  is good enough only using the bidirectional LSTM is not work. We think the reason is the **random variation** of input signals and  $W_{mhd}$  itself.

# Other singles



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- Except for  $q_0$ , the average similarity of other key signals is greater than 90%. And the similarity distribution is concentrated above 90%.
- The reason for the low average similarity of  $q_0$  is that its value is around zero.



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- Providing reference in the experimental **proposal stage**.
  - The electron density  $n_e$ , store energy  $W_{mhd}$ , loop voltage  $V_{loop}$ , actual plasma current  $I_p$ , normalized beta  $\beta_n$ , toroidal beta  $\beta_t$ , beta poloidal  $\beta_p$ , elongation at plasma boundary  $\kappa$ , internal inductance  $l_i$ , q at magnetic axis  $q_0$ , and q at 95% flux surface  $q_{95}$  are predicted in the proposal stage.
  - Except  $V_{loop}$  other signals can be considered well modeling.
  - 1-D profile modeling in the next step
- Providing **accuracy values of whole discharge process** compared to other models.
- Limitations
  - Temporarily unable to predict a discharge curve in real time.
  - Temporarily have not cross-tokamak capacity. (**device dependant**)
  - Temporarily unable to achieve dimensionless.

[https://chgwan.github.io/DataBase/Wan\\_2021\\_IAEA\\_report.pdf](https://chgwan.github.io/DataBase/Wan_2021_IAEA_report.pdf)  
[https://chgwan.github.io/DataBase/draft\\_Proof\\_hi.pdf](https://chgwan.github.io/DataBase/draft_Proof_hi.pdf)

# Thank You!