

Technical Meeting on the Development and Application of Open-Source Modelling and Simulation Tools for Nuclear Reactors

Tuesday, 21 June 2022

Poster Session: Poster Session - POLIMI (13:30 - 15:00)

[id] title	presenter	board
[35] MULTI-SCALE COMPUTATIONAL ANALYSIS TO PREDICT THE IRRADIATION-INDUCED CHANGE IN ENGINEERING PROPERTIES OF FUSION REACTOR MATERIALS.	MOHAMED KUNJU, Salahudeen	
[29] Verification & Validation process in the open-source TRUST/TrioCFD Platform	Dr DARONA, Julie	
[37] Investigating efficient coupling of Finite Element Analysis with Machine Learning to optimise a laboratory experiment.	LEWIS, Rhydian	
[48] VERIFICATION OF MULTIPHYSICS COUPLING TECHNIQUES FOR MODELING OF MOLTEN SALT REACTORS	Dr GROTH-JENSEN, Jacob	
[40] Open source tools in support of multiphysics simulation for fusion	DAVIS, Andrew	
[27] Validation of an OpenFOAM gas distribution solver for containment analysis	STEWERING, Jörn	
[6] Active coding assignments in nuclear reactor modelling	DEMAZIÈRE, Christophe	
[12] Open source neutron noise modelling	DEMAZIÈRE, Christophe	
[20] THE GEN-FOAM MULTI-PHYSICS SOLVER FOR REACTOR ANALYSIS: STATUS AND ONGOING DEVELOPMENTS	FIORINA, carlo	
[8] Development and maintenance of OpenFOAM_RCS for German nuclear safety research related to the reactor cooling system	Dr LEHNIGK, Ronald	
[10] MODIFICATION OF OPEN SOURCE MONTE CARLO CODE OPENMC TO INCLUDE TIME-DEPENDENCE IN A FISSILE SYSTEM INCLUDING INDIVIDUAL DELAYED NEUTRON PRECURSORS	ROMERO-BARRIENTOS, Jaime	
[5] FOUNDATIONS FOR A FISSION BATTERY DIGITAL TWIN	BROWNING, Jeren	
[47] VANGARD: A GPU-BASED HIGH-SPEED PINWISE NODAL CORE ANALYSIS CODE	JEON, SEOYOON	
[31] "DEVELOPMENT OF CODES FOR CALCULATING THE CORROSION OF STEEL CLADDING OF FUEL ELEMENTS WITH MIXED NITRIDE FUEL, TAKING INTO ACCOUNT THE SEGREGATION OF IMPURITIES AT GRAIN BOUNDARIES"	IVANOV, ALEXANDER	
[50] Development of an open source tool for water hammer and fluid-structure interaction simulation in nuclear reactor systems	ROCHA, Marcelo	
[51] Enabling techniques within OpenMC for dynamic models of molten salt reactors	Mr KNUDSEN, Erik	
[16] DEVELOPMENT OF ACCESSIBLE CODES FOR NUCLEAR SAFETY ANALYSIS IN THAILAND	Mr VECHGAMA, Wasin	
[4] Development of Post Processor for VSOP'94 Output	PURWANINGSIH, Anik	
[32] MFront: an open-source code generation tool for the rigorous management of material knowledge in the PLEIADES and MAP platforms	HELPER, Thomas	

[42] AN OPEN-SOURCE DATA-CENTRIC REPRESENTATION OF THE FAST FLUX TEST FACILITY ISOTHERMAL BENCHMARK CORE	TOURAN, Nicholas	
[41] A concept of fluid dynamic code for molten salt reactor analysis with Open FOAM	Dr KHAKIM, Azizul	