

Updates on Multiphysics Endeavors with OpenMC

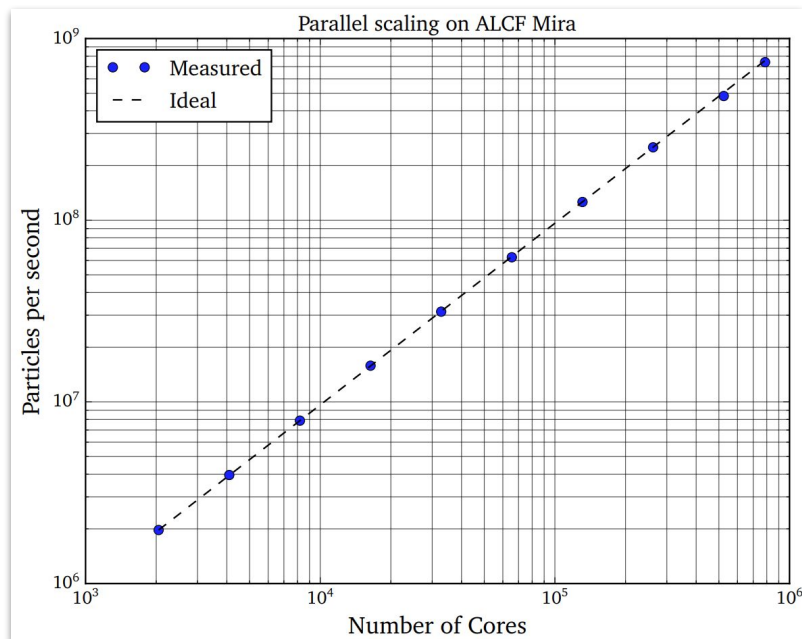
Technical Meeting on the Development and Application of
Open-Source Modelling and Simulation Tools for Nuclear Reactors

June 22nd, 2022

OpenMC

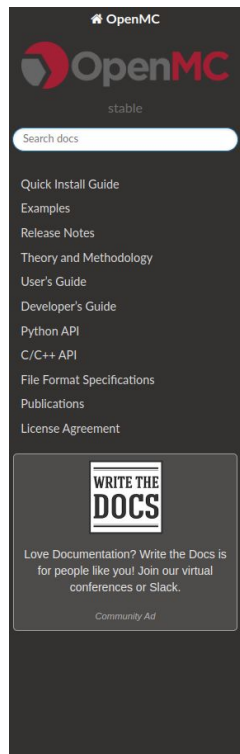
Open-source Monte Carlo Particle Transport

- C/C++ and Python APIs
- Parallel Performance
- Nuclear data interface
- Plotter
- Depletion
- CAD-based geometry
- Community driven
- Technical Committee
 - Paul Romano
 - Ben Forget
 - Adam Nelson
 - Sterling Harper



OpenMC

- Mixed **C++** and **Python** codebase
- **CMake** build system
- Distributed memory parallelism: **MPI**
- Shared memory parallelism: **OpenMP**
- Version control: **git**
- Project management: [GitHub](#)
- CI: [GitHub actions](#)
- Documentation: [Read the Docs](#)



[Docs](#) » The OpenMC Monte Carlo Code

[Edit on GitHub](#)

The OpenMC Monte Carlo Code

OpenMC is a community-developed Monte Carlo neutron and photon transport simulation code. It is capable of performing fixed source, k-eigenvalue, and subcritical multiplication calculations on models built using either a constructive solid geometry or CAD representation. OpenMC supports both continuous-energy and multigroup transport. The continuous-energy particle interaction data is based on a native HDF5 format that can be generated from ACE files produced by NJOY. Parallelism is enabled via a hybrid MPI and OpenMP programming model.

OpenMC was originally developed by members of the [Computational Reactor Physics Group](#) at the [Massachusetts Institute of Technology](#) starting in 2011. Various universities, laboratories, and other organizations now contribute to the development of OpenMC. For more information on OpenMC, feel free to post a message on the [OpenMC Discourse Forum](#).

Recommended publication for citing

Paul K. Romano, Nicholas E. Horelik, Bryan R. Herman, Adam G. Nelson, Benoit Forget, and Kord Smith, "OpenMC: A State-of-the-Art Monte Carlo Code for Research and Development," *Ann. Nucl. Energy*, **82**, 90–97 (2015).

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- [Quick Install Guide](#)
- [Examples](#)
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- [User's Guide](#)
- [Developer's Guide](#)
- [Python API](#)
- [C/C++ API](#)
- [File Format Specifications](#)
- [Publications](#)
- [License Agreement](#)

OpenMC C/C++ API for Multiphysics

- Initialize simulation
- Find cell/material at location (domain mapping)
- Create tallies
- Execute simulation
- Extract tally results
- Set cell temperatures and material densities
- Reset tallies/batches
- Re-running simulation
- Finalize simulation

OpenMC Python C-API

[Great example written by April Novak](#)

```
with openmc.lib.run_in_memory():
    for i in range(n_iterations):
        openmc.lib.reset()

        openmc.lib.run()

        # ---- Multiphysics feedback part ---- #

        # get the total kappa fission computed by OpenMC over the entire domain
        total_kappa_fission = openmc.lib.tallies[1].mean

        # power (W) in each layer of the solid
        q = np.zeros(N)

        for j in range(N):
            q[j] = openmc.lib.tallies[2].mean[j] / total_kappa_fission

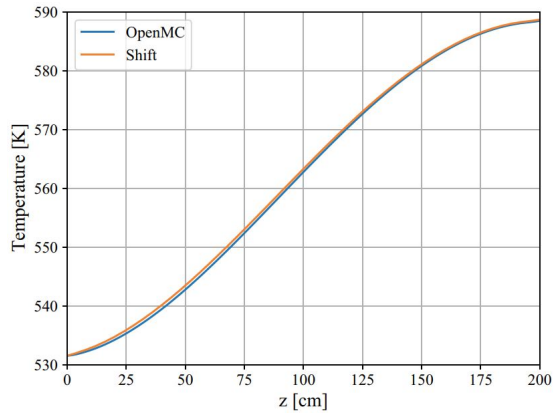
            # to get in units of W, multiply by the total power
            q[j] *= power

            # for greater than the first iteration, relax
            if (i > 0):
                q[j] = (1.0 - alpha) * q_iterations[i - 1][j] + alpha * q[j]

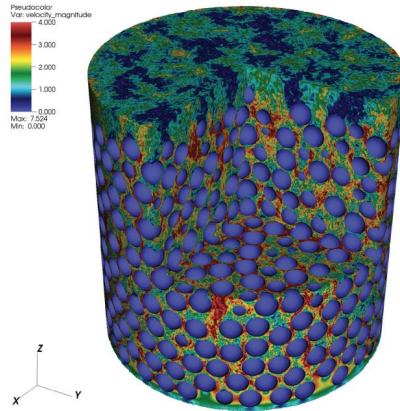
        # compute the fluid temps, fluid densities, and solid temps
        fluid_temps = part3_backend.fluid_temperature(q, T_inlet, N)
        fluid_densities = part3_backend.fluid_density(fluid_temps, N)
        solid_temps = part3_backend.solid_temperature(q, fluid_temps, N, R, Rf, H)
```

Multiphysics Endeavors

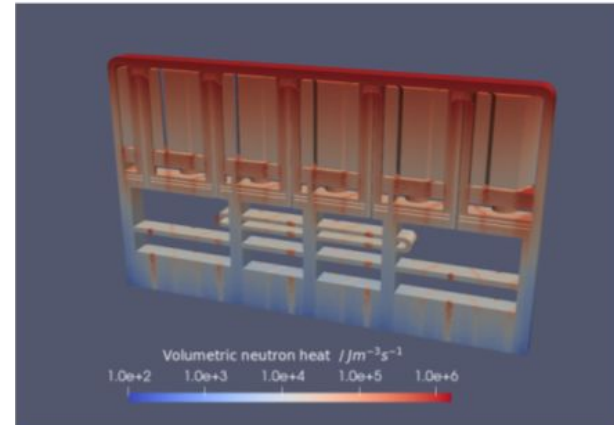
ENRICO



CARDINAL



AURORA



Common Characteristics

- In-memory coupling
- High fidelity
- Highly scalable
- Open-source

ENRICO

Exascale Nuclear Reactor Investigative COde

- DOE Exascale Computing Project
- Collaboration between ANL and ORNL
- Goal:

Demonstration of a full core SMR multiphysics simulation on an exascale platforms

ENRICO: Solvers

[“A Code-Agnostic Driver Application for Coupled Neutronics and Thermal-Hydraulic Simulations”](#)

Romano, Hamilton, et. al.

Neutronics:

OpenMC

SHIFT

Thermal Hydraulics:

Nek5000

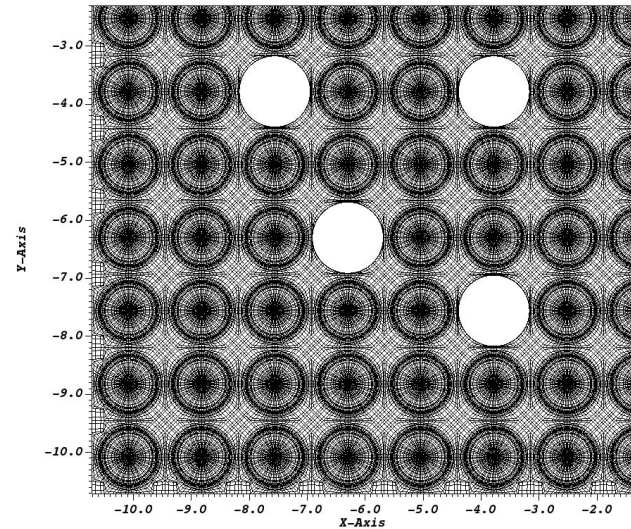
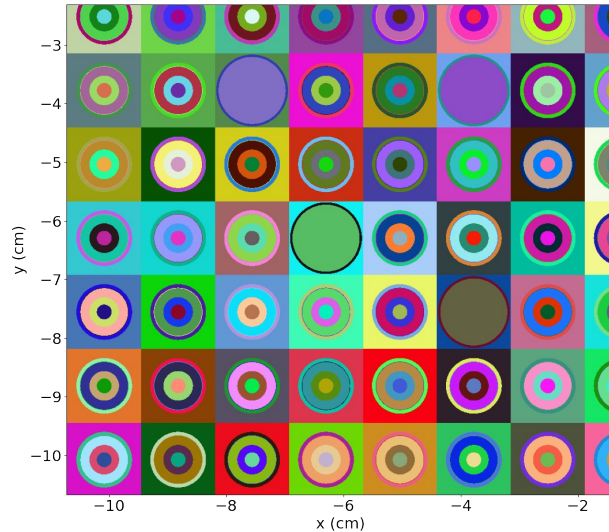
NekRS

TH Surrogate Model

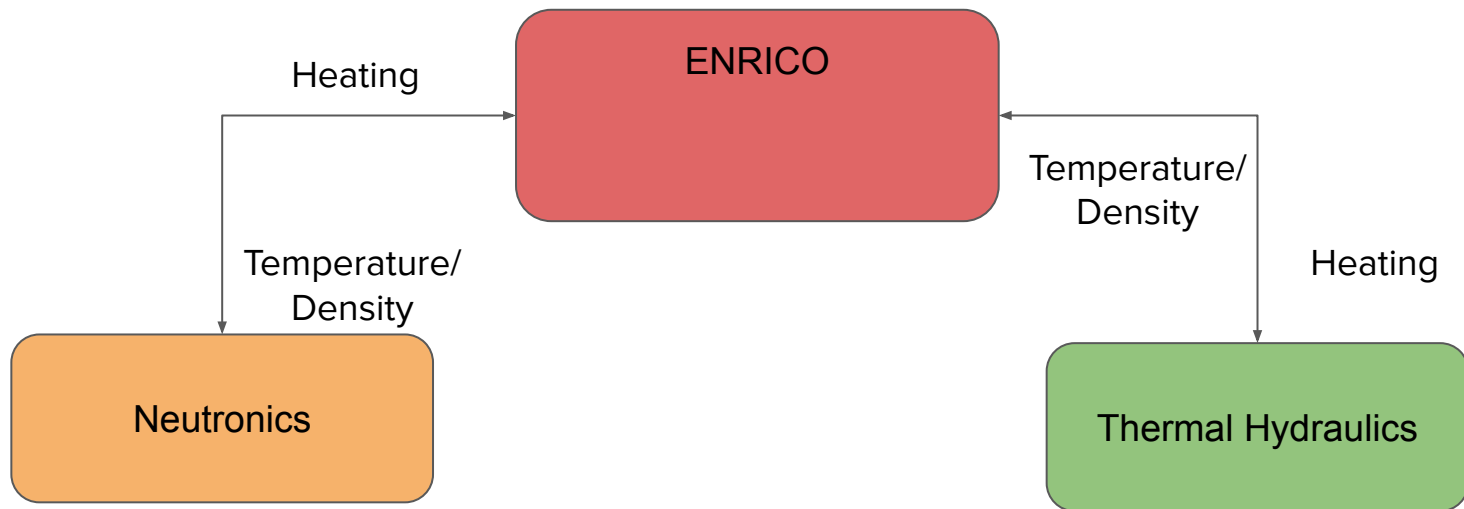
OpenFOAM*

ENRICO: Domain Mapping

- Expected that CSG and TH mesh regions approx. match
- One CSG cell to many TH element mapping (largely automated)

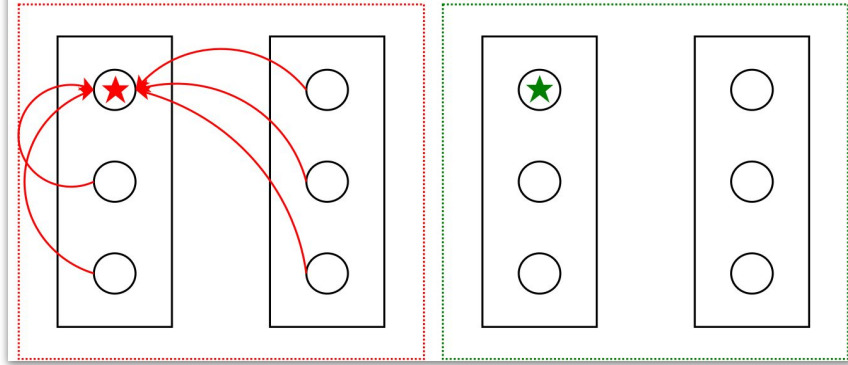


ENRICO: Information Flow

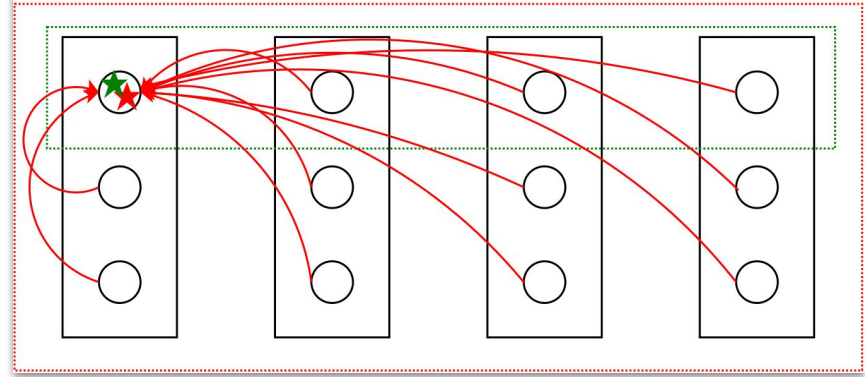


ENRICO: MPI Communication

Disjoint communicators:



Overlapping communicators:



--- TH Comm Boundary

--- Neutronics Comm Boundary

ENRICO: 17 x 17 Assembly

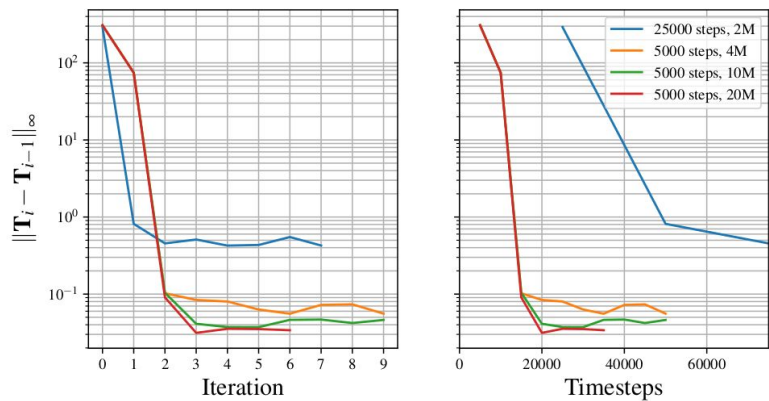
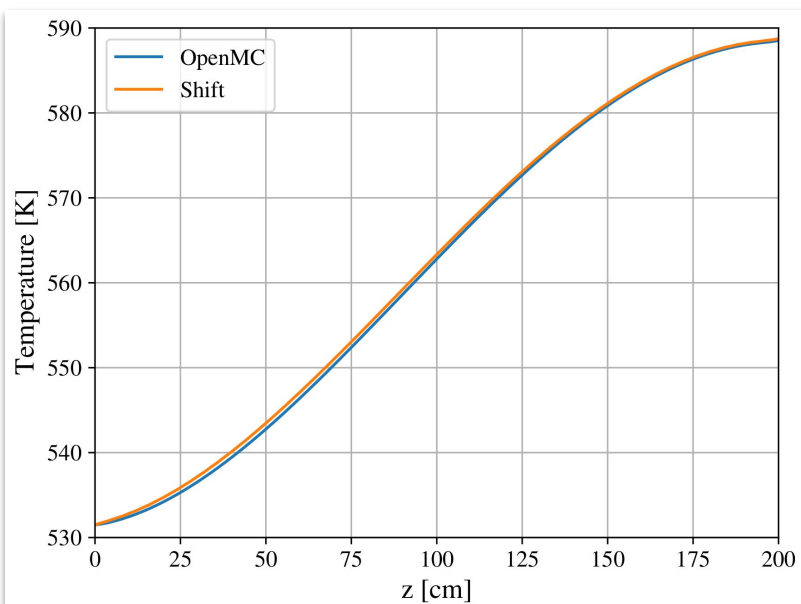


Fig. 6. Convergence of the temperature distribution on the short model as a function of iterations (left) and Nek5000 timesteps (right).

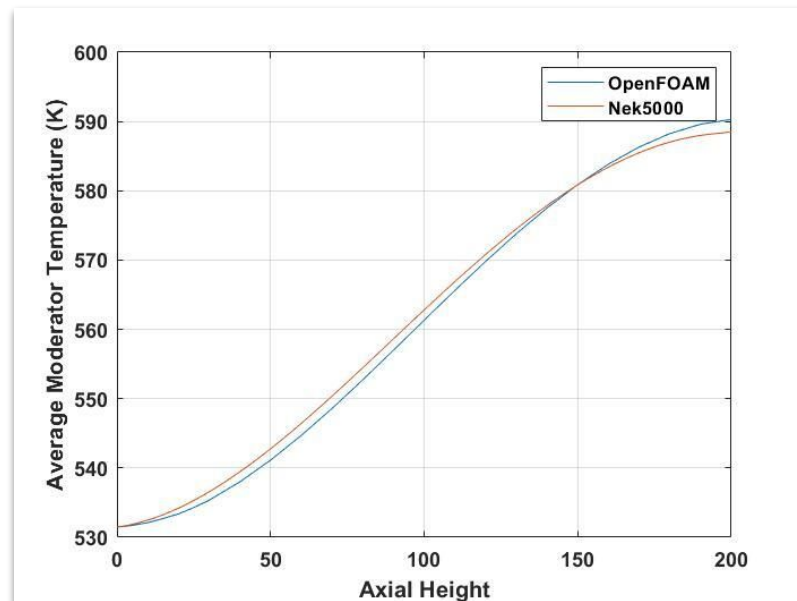
ENRICO: OpenFOAM

“[Coupled OpenFOAM and OpenMC for high-fidelity multiphysics reactor core simulations](#)”

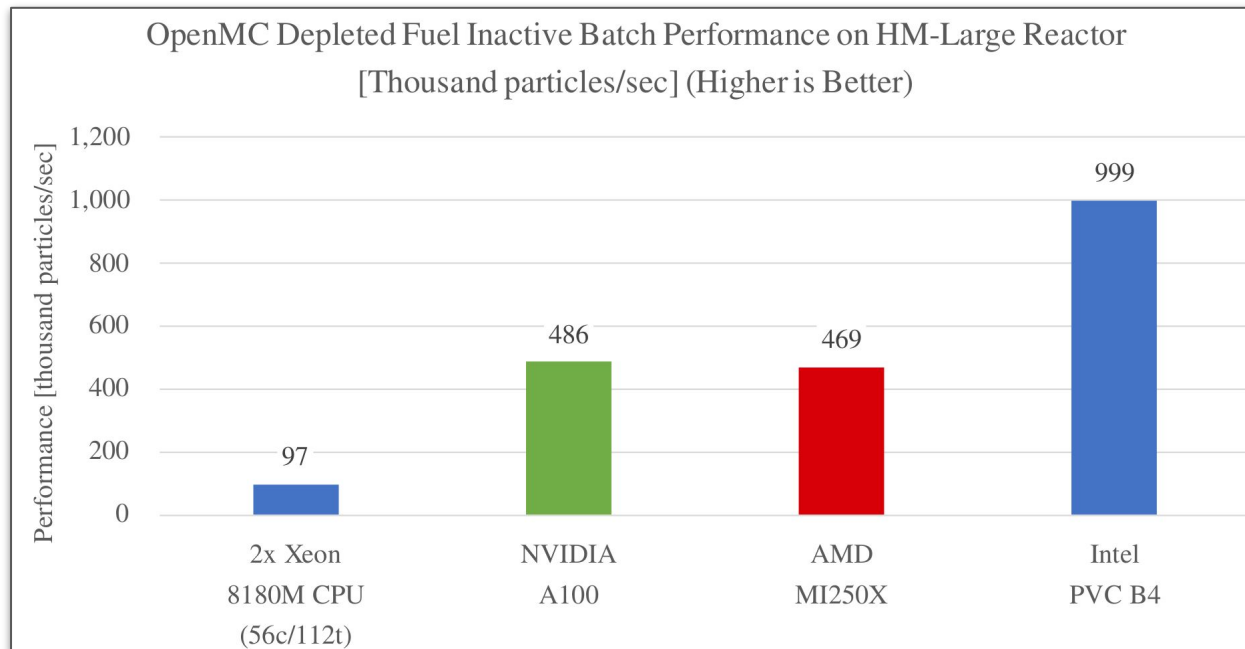
Bullerwell, Hou, et. al.

Table IV. Global Differences in Heat Generation Rate Distributions

Relative RMS Difference	Absolute RMS Difference (W/cm ³)	Max Relative Difference	Max Absolute Difference (W/cm ³)
0.945721%	2.151217	2.349505 %	5.042066



ENRICO: GPU Offloading



[“Toward Portable GPU Acceleration of the OpenMC Monte Carlo Particle Transport Code”](#)

John Tamm et. al.

Cardinal

CARDINAL

- A MOOSE app
- Collaboration between ANL, INL, UIUC, and Penn. State
- Project Lead: April Novak
- Goal:

[Advanced reactor multiphysics simulator](#)

- HTGC-PBR
- SFR
- PGCR

Cardinal: Solvers

[“A Code-Agnostic Driver Application for Coupled Neutronics and Thermal-Hydraulic Simulations”](#)

Romano, Hamilton, et. al.

Neutronics:

OpenMC

Heat Conduction:

MOOSE Heat Conduction

Thermal Hydraulics:

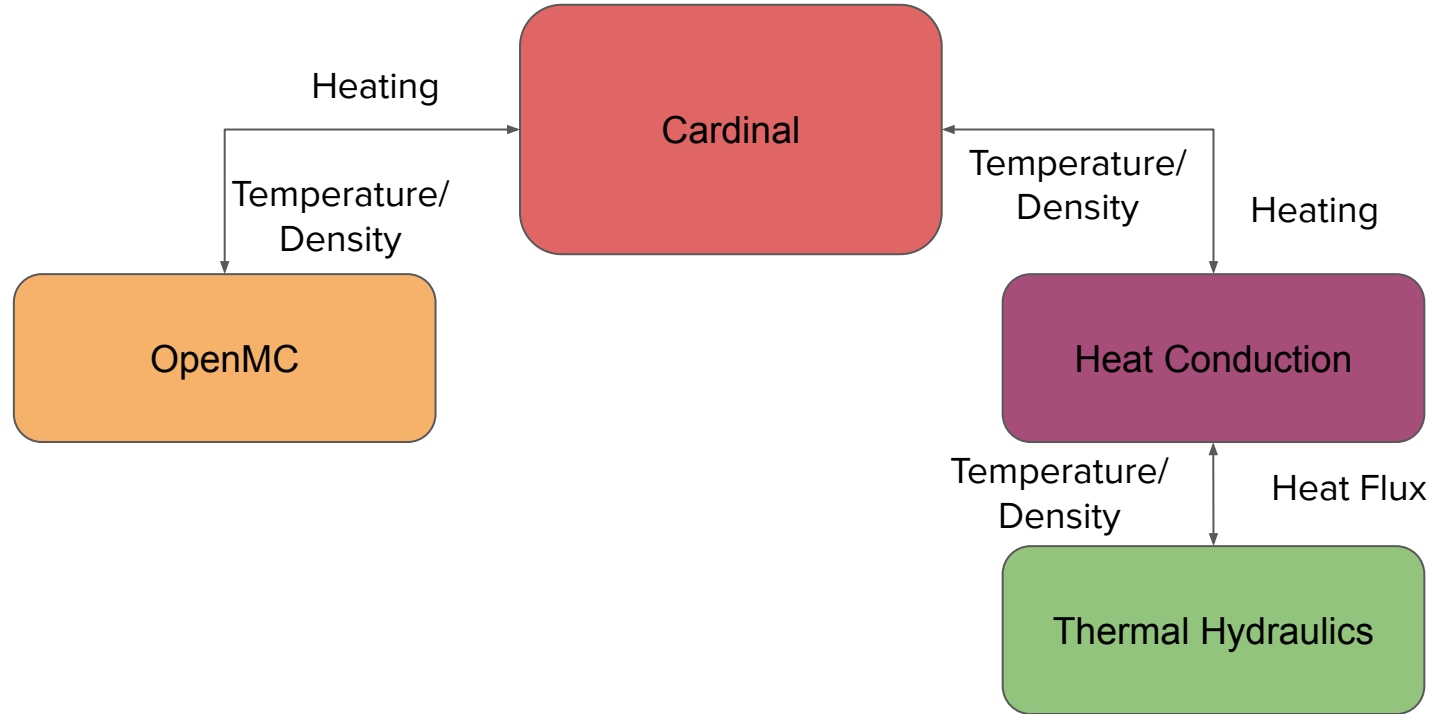
NekRS

MOOSE THM

Other MOOSE Apps:

SAM, Sockeye, BISON, etc.

Cardinal: Information Flow

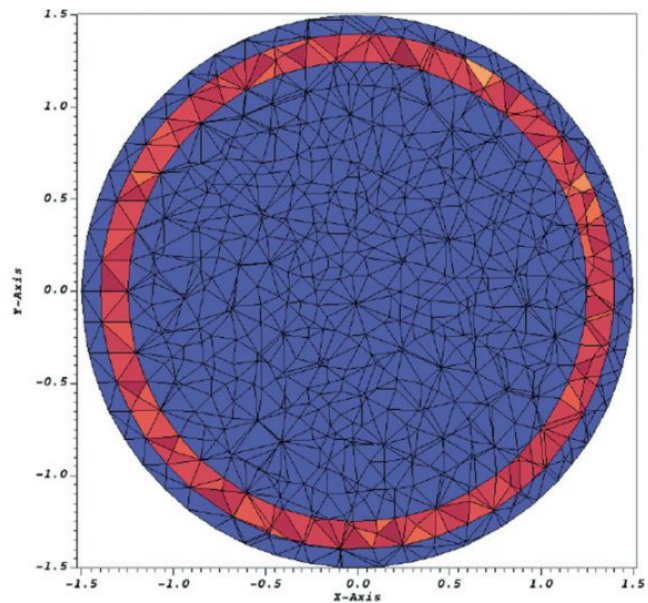
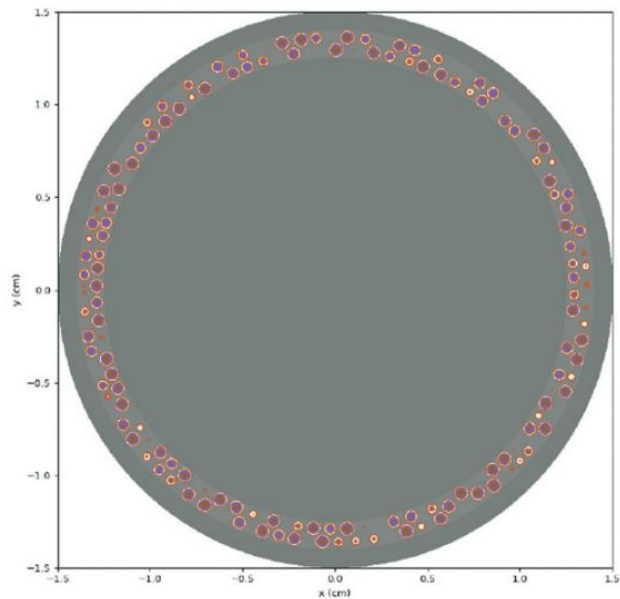


Cardinal: Domain Mapping

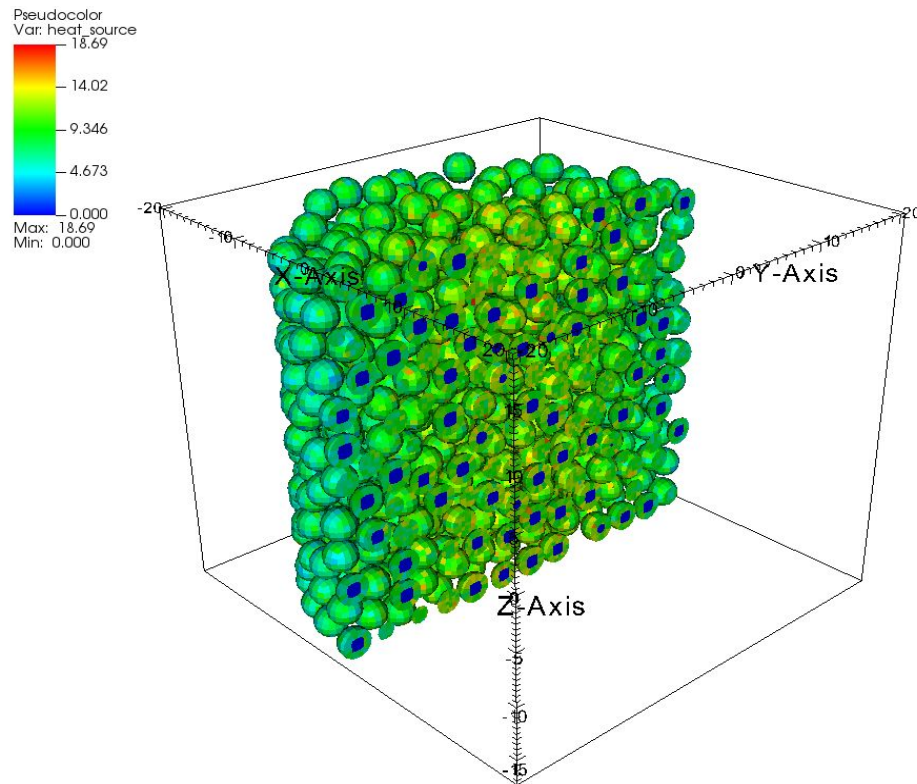
Same as ENRICO – “one neutronics cell to many TH element” approach with automated mapping

- Fluid (T , ρ) vs. solid regions (T) are specified on the MOOSE mesh
- Higher levels in the OpenMC geometry can be specified
- Unstructured mesh tallies to perform one-to-one element transfers

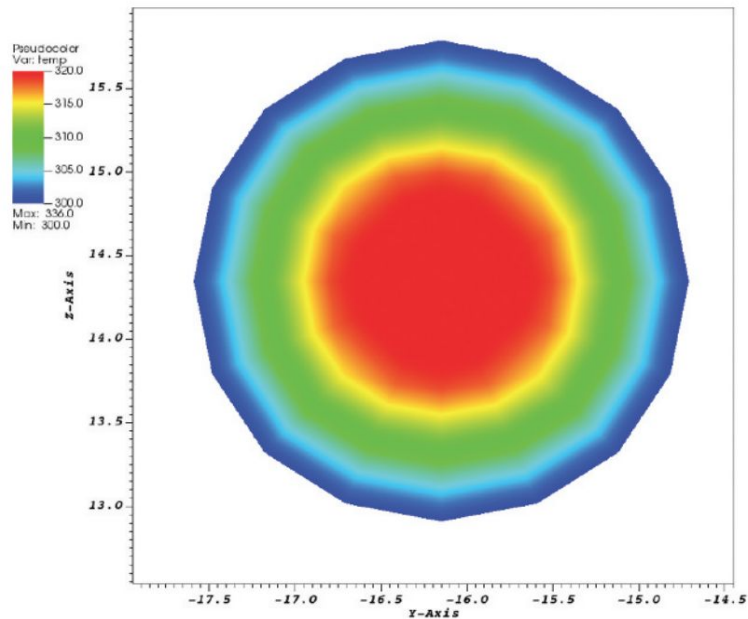
Cardinal: Unstructured Mesh Tallies



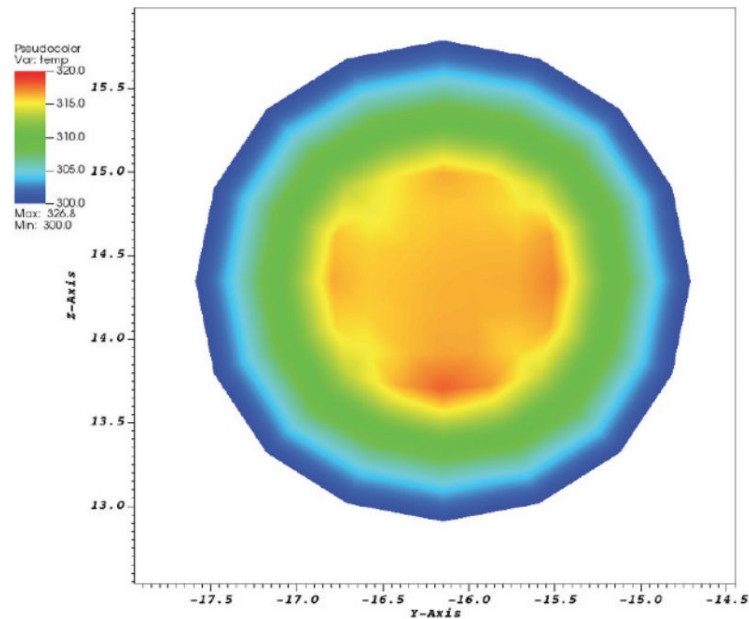
Cardinal: PBR Simulation



Cardinal: Unstructured Mesh Tallies

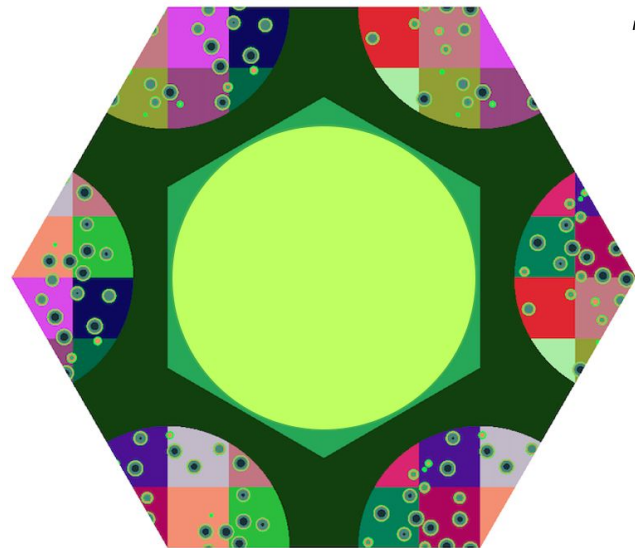


Cell Tally

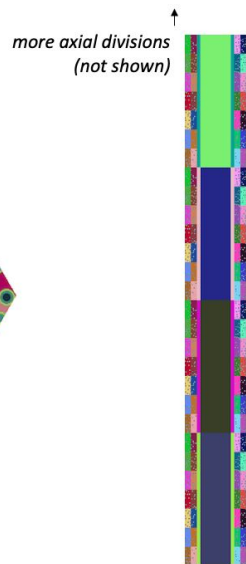


Unstructured Mesh Tally

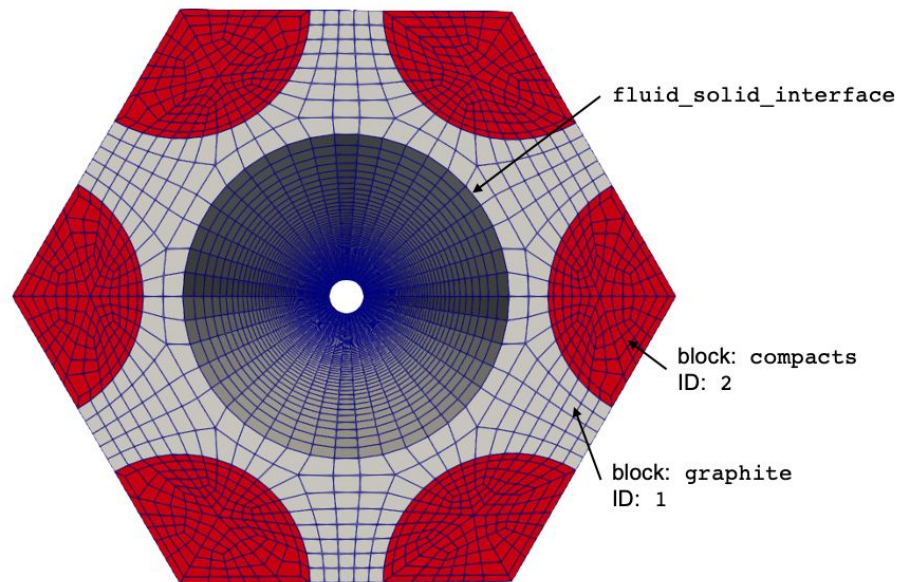
Cardinal: Domain Mapping



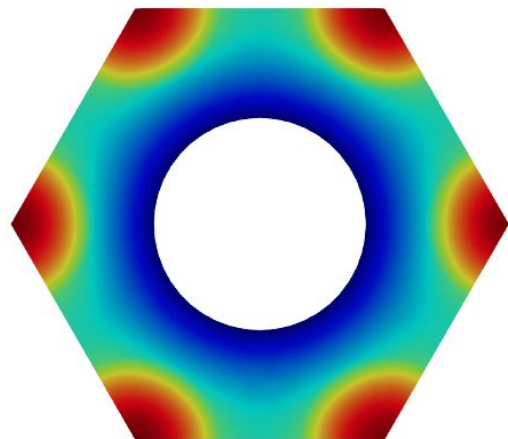
$x - y$ view



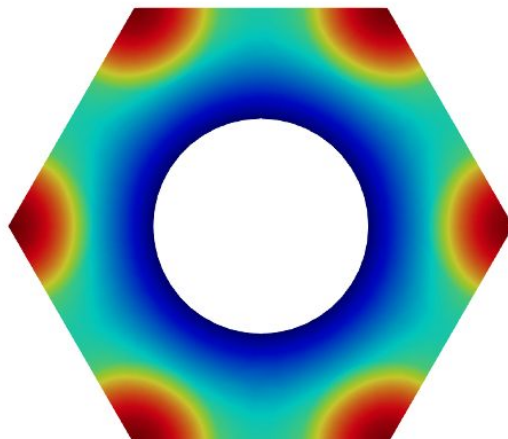
$x - z$ view



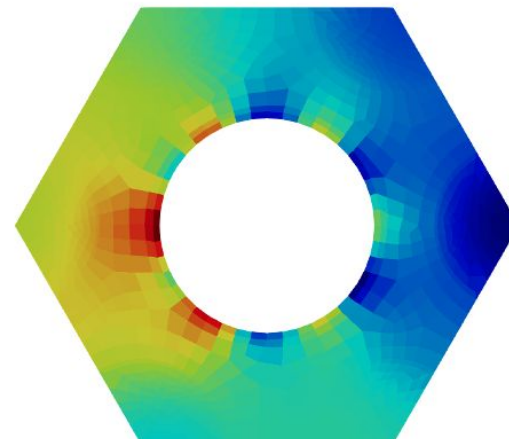
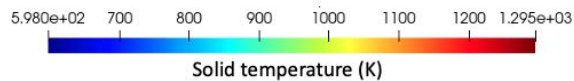
Cardinal: Results



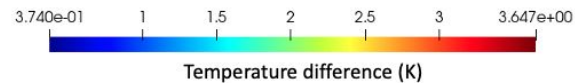
Nek-MOOSE-
OpenMC



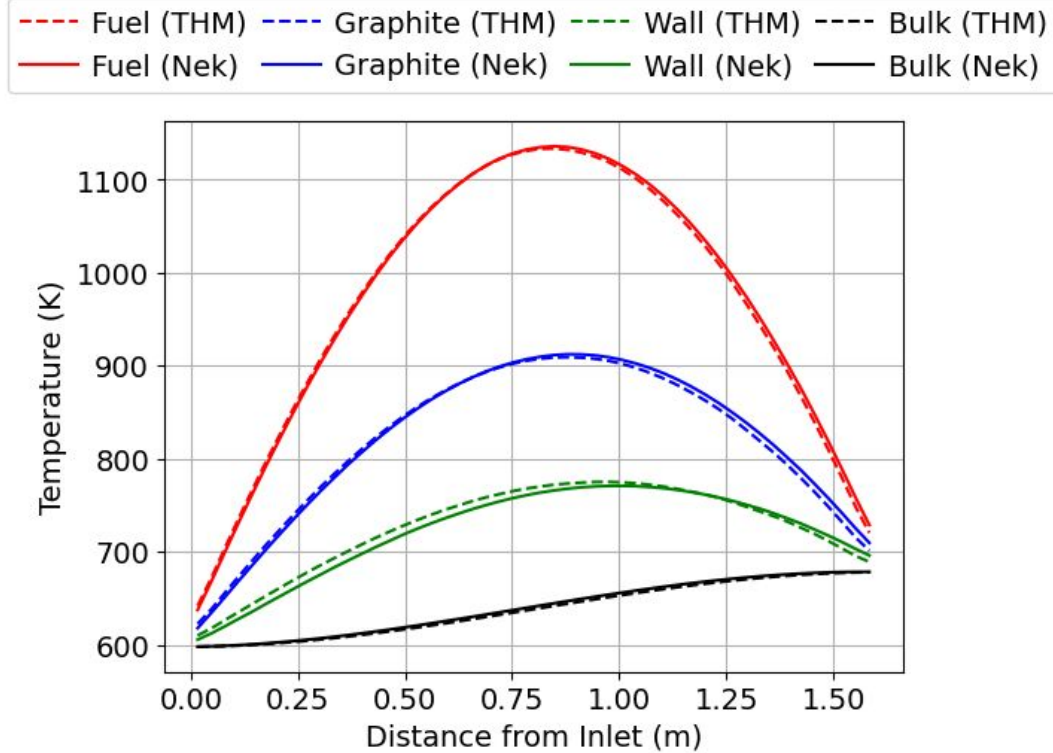
THM-MOOSE-
OpenMC



difference

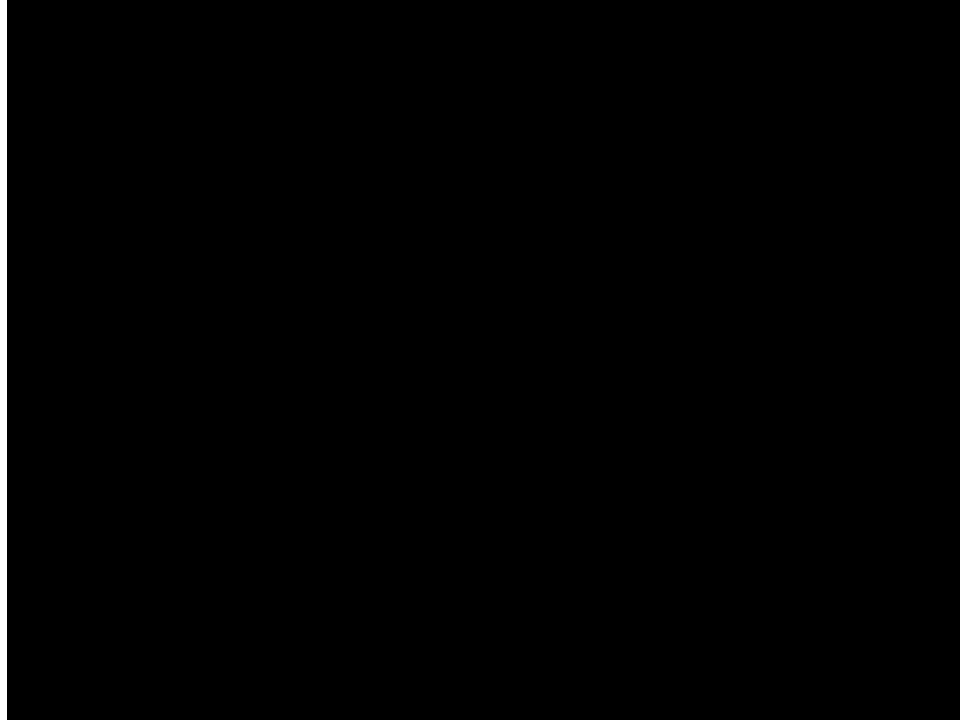


Cardinal: Results



Cardinal: Tracer Simulation

Aaron Huxford
University of Michigan
June 2022



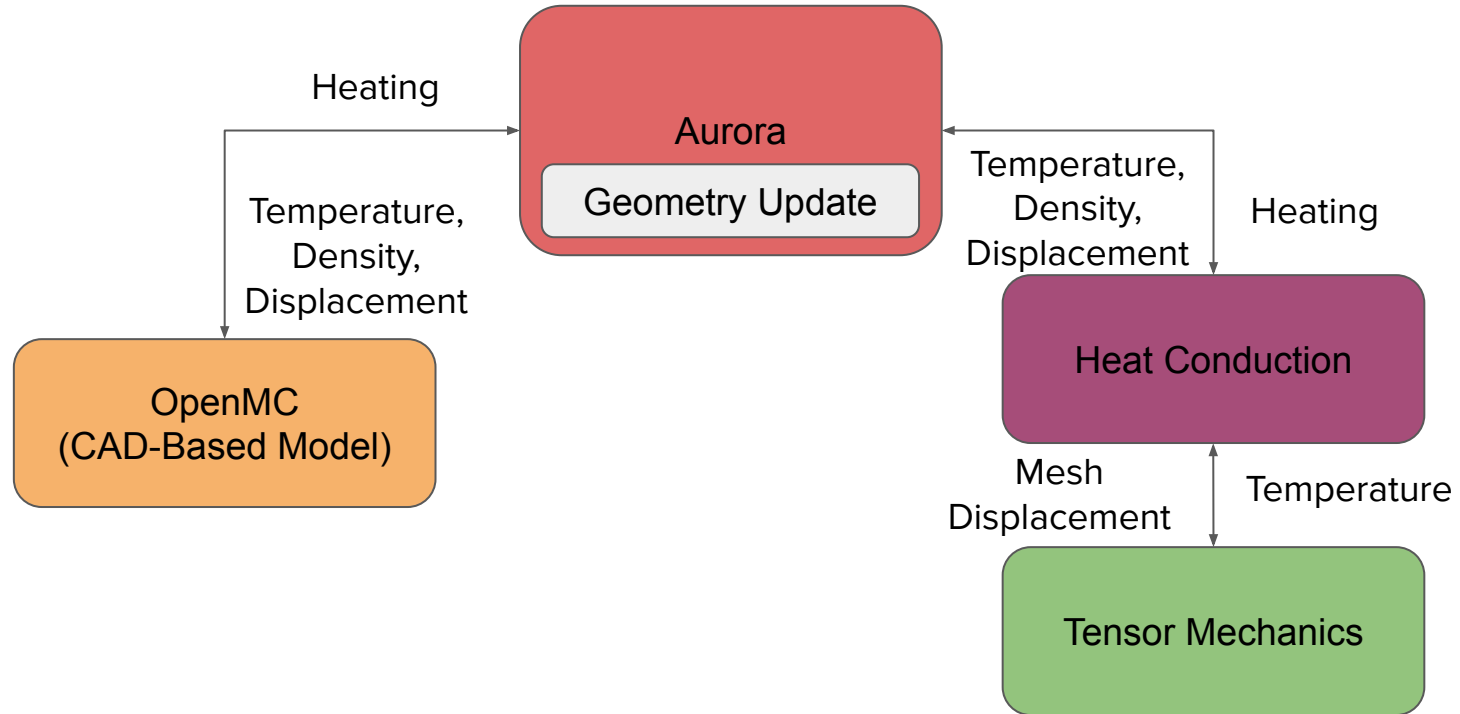
Aurora

AURORA

- A MOOSE app produced by the UKAEA
- Project Leads: Helen Brooks, Andrew Davis
- Goal

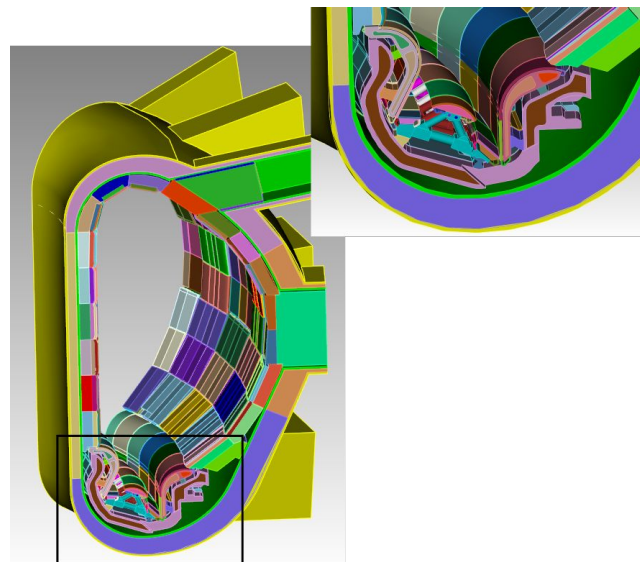
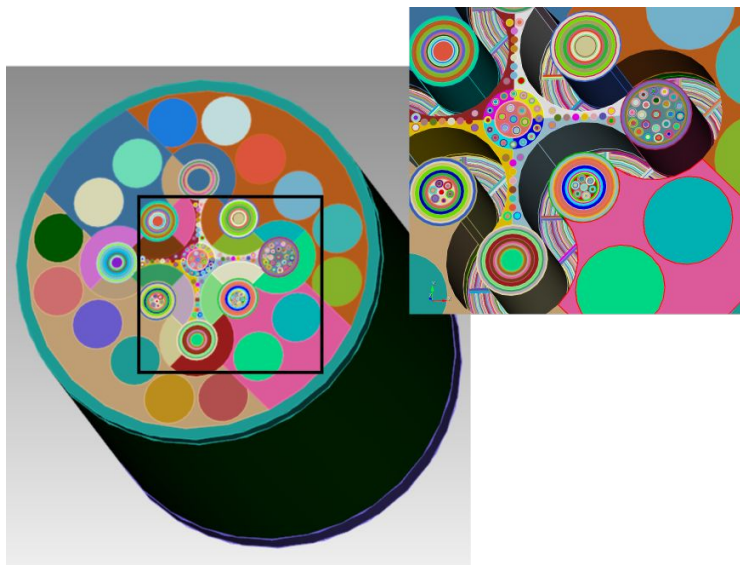
Multiphysics simulation of components in fusion environments

Aurora: Information Flow



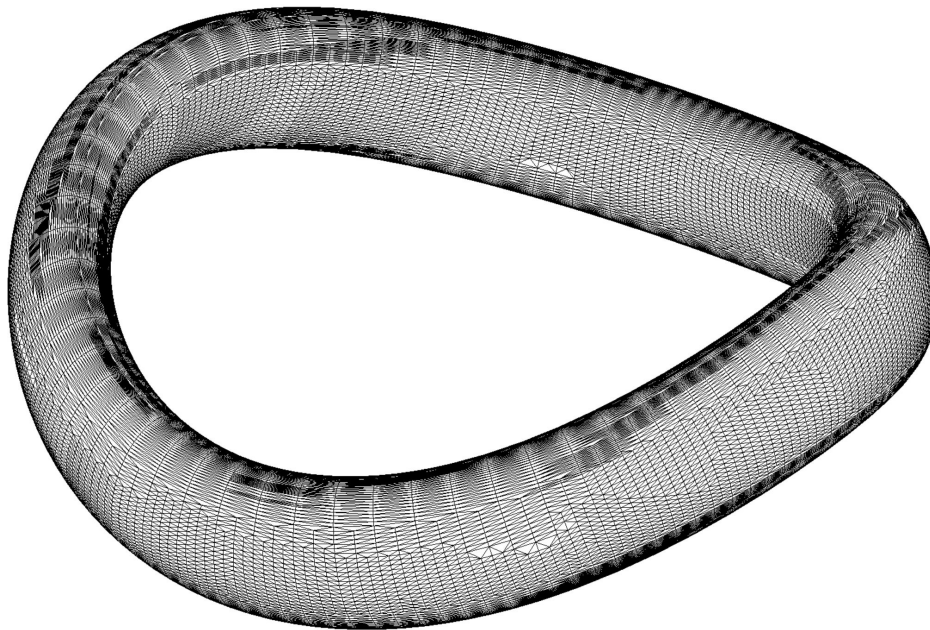
Aurora: DAGMC

Monte Carlo particle transport on surface tessellations of CAD geometry

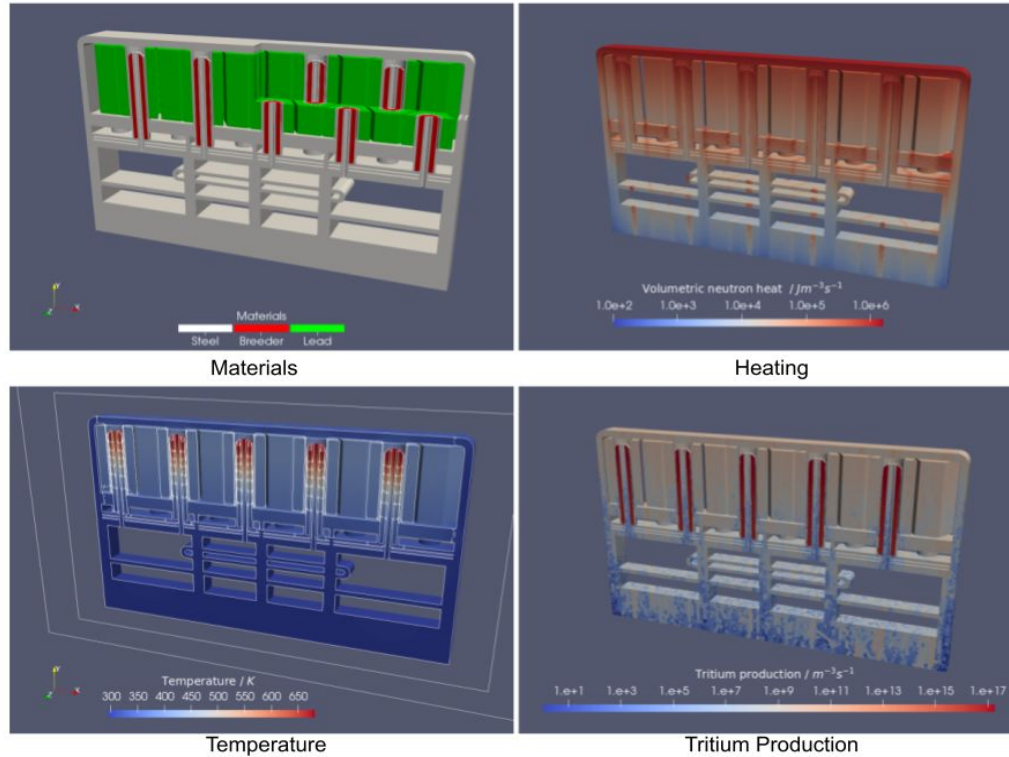


Aurora: DAGMC

Monte Carlo particle transport on surface tessellations of CAD geometry



Aurora: Application



Multiphysics Testing

OpenMC's C/C++ API now contains functions that are purely for multiphysics. Where does testing belong?

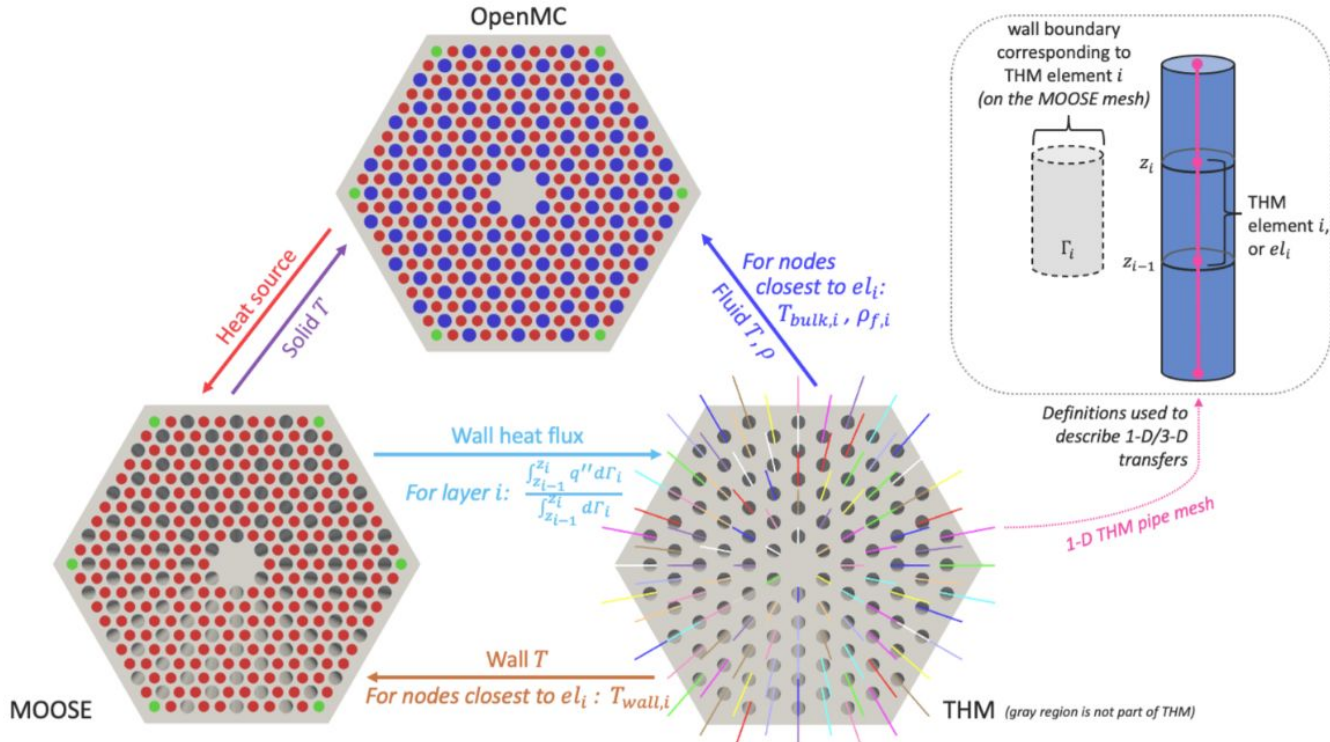
Answer is still unclear. Some are tested in OpenMC, some are tested in multiphysics apps themselves.

Acknowledgements



Thank you!

Cardinal: Information Flow



Aurora: Neutronics Model Updates

