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# Quasi-symmetric error field correction in tokamaks

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- Background: Non-axisymmetric error field control (EFC) in tokamaks
  - Recent progress on resonant EFC
  - Issue with residual magnetic perturbations after resonant EFC
- Modeling: EFC optimization towards quasi-symmetric (QS) residuals
  - Minimization of variation in field strength and 3D neoclassical transport
  - Optimization via torque response matrix
- Experiment: Testing quasi-symmetric magnetic perturbations (QSMP)
  - In comparison with RMPs and NRMPs in KSTAR and DIII-D
  - Safety of QSMPs during transient phase
- Summary and outlook

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#### A small non-axisymmetric magnetic field can greatly change tokamak performance and thus must be under control

- A tokamak has always intrinsic nonaxisymmetric (3D) error fields (EF)
  - Due to imperfect magnets and components
- A 3D field can also be introduced on purpose
  - Mostly for instability control, as highlighted by "RMP ELM control" in tokamaks
- In either case, a 3D field as small as  $\delta B/B=10^{-3}\sim 10^{-4}$  can greatly degrade or even disrupt tokamak plasmas, if not properly controlled or judiciously used
- Any dangerous or unnecessary 3D field components must be compensated → Error Field Correction (EFC)





# Recent progress on plasma response and MHDs is offering a reliable leading-order EFC scheme



- Ideal MHD clearly shows which 3D field is most resonant with tokamak plasmas and thus must be compensated if not necessary
  - Leading to a major change in EFC approach, via "resonant overlap" field
    - Extensively validated in tokamak devices including DIII-D [LanctotPOP10, Paz-SoldanPOP14]
- Present ITER EFC strategy: Reduce overlap with dominant resonant field below "EF penetration" threshold
  - Two-fluids MHDs then can offer prediction of EF penetration threshold in practice
    - See N. Logan's poster for resonant EFC summary

[TM1 (Hu), EPEC (Fitzpatrick)]

# Residual EFs may not be disruptive in stable operating conditions but shown to be still problematic transiently

- COMPASS studies with the high-field-side proxy-EF show
  - Locked modes could indeed be avoided by resonant EFC, but large non-resonant residual EFs could still be disruptive during L-H transition [MarkovicEPS2018]

Successful EFC against locked modes but not for L-H transition



HFS EF

• NSTX-U and DIII-D also showed that NTV rotational damping by residual EFs which can eventually cause instability issues [Paz-SoldanPOP14, ParkAPS18]

Needs a complementary EFC approach

for residual EFs which often have greater non-axisymmetry and create non-linear effects



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# Minimizing all the prominent residual EF effects reminisces optimization towards a quasi-symmetry

• Eliminating all static EF effects in guiding center plasmas is in principle achievable by:

Variation in the field strength 
$$\left|\delta \vec{B}\right|_{particles} \rightarrow 0$$
 [NurenbergPLA88,BoozerPPCF95]

- Ideally, there is a linear path to perturb a tokamak while holding this condition:
- However, it is the force balance in plasma that dictates the  $\vec{\xi}$  profiles

$$\frac{\left|\delta\vec{B}\right| \sim \hat{b} \cdot \left(\vec{\nabla} \times \left(\vec{\xi} \times \vec{B}_{0}\right) + \vec{\xi} \cdot \vec{\nabla}B_{0}\right) \rightarrow 0}{\sum_{\substack{Lagrangian \\ Changes in a fixed space \\ changes with field lines}} \delta\vec{F}\left[\vec{\xi}\right] = 0$$
These two are NOT compatible in general as well known [Garron&BoozerPFB9] from more general 3D geometry

Nonetheless, quasi-symmetric optimization can be performed in average



### Self-consistent perturbed equilibria with neoclassical transport offers a unique QS optimizing scheme, via torque response matrix

• Perturbed equilibria with non-adiabatic pressure (including 3D coils):

$$\delta \vec{F} \begin{bmatrix} \vec{\xi} \end{bmatrix} = \delta \vec{F}_{ideal} \begin{bmatrix} \vec{\xi} \end{bmatrix} - \vec{\nabla} \cdot \Pi \begin{bmatrix} \vec{\xi} \end{bmatrix} = 0$$

• Neoclassical torque is also given by integrating:

$$\tau_{\varphi} = Im \left[ n \int_{plasma} dx^3 \left( \vec{\xi} \cdot \delta \vec{F}[\vec{\xi}] \right) \right]$$

 Torque minimization leads minimized 3D neoclassical particle, momentum, heat transport, although its momentum part (called NTV) is mostly pronounced in tokamaks

$$\tau_{\varphi} \propto \Gamma_{NTV} \propto Q_{NA} \sim 0$$

• Full solutions provide torque response matrices to given 3D fields or coils

$$\tau_{\varphi}(\psi) = (Fourier \ modes)^{+} \cdot T(\psi) \cdot (Fourier \ modes)$$
$$= (Coil \ currents)^{+} \cdot T_{c}(\psi) \cdot (Coil \ currents)$$

• Method above has been implemented in general perturbed equilibrium code (GPEC) which has been used as a primary tool to design QSMP configurations

### Torque response matrix contains all the information of neoclassical torque that a tokamak can drive with available coils

- All possible neoclassical torque that a tokamak (e.g. KSTAR) can drive using their 3 rows of coils are given by
  - 3x3 matrix, per each n, per a target equilibrium and its kinetic profiles

$$\tau(\psi) = (I_T e^{-i\phi_T} \quad I_M e^{-i\phi_M} \quad I_B e^{-i\phi_B}) \cdot \begin{pmatrix} T_{TT}(\psi) & T_{TM}(\psi) & T_{TB}(\psi) \\ T_{MT}(\psi) & T_{MM}(\psi) & T_{MB}(\psi) \\ T_{BT}(\psi) & T_{BM}(\psi) & T_{BB}(\psi) \end{pmatrix} \cdot \begin{pmatrix} I_T e^{i\phi_T} \\ I_M e^{i\phi_M} \\ I_B e^{i\phi_B} \end{pmatrix}$$

 Its eigenvector for the minimum eigenvalue of the torque-coil response matrix: The best possible quasi-symmetric magnetic perturbation (QSMP) in a tokamak

KSTAR n=1 QSMP

DIII-D n=1 QSMP





# QSMP is clearly contrasted to two other categories of small 3D fields in tokamaks

- RMP creates strong resonant response (at the rational surfaces)
- NRMP can drive substantial non-resonant NTV, but without resonant response
- QSMP suppressed both resonant and non-resonant response while maintaining the same power norm of field amplitudes or currents





### QSMP optimized by GPEC indeed minimizes variation in the field strength at best upon constrained by force balance and torque

- GPEC finds the best possible QSMP by minimizing total torque, within force balance
- Resulting in minimization of plasma response and variation in the field strength
- Resulting in optimization of displacement spectrum

$$\delta_L \equiv \frac{|\delta B|}{B_0} = \hat{b} \cdot \vec{\nabla} \, \vec{\xi} \cdot \hat{b} - \vec{\nabla} \cdot \vec{\xi} \sim 0$$





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#### QSMP designed and tested in KSTAR indeed did not bring any meaningful effects despite the large amplitudes

- RMP caused density pumping, confinement degradation, and rotational damping
  - Could suppress ELMs if further optimized
- NRMP induced rotational damping only (without density pump-out)
- QSMP did not show any degradation, even with the maximum currents applied (10kAt)



[S. M. Yang, 2019 KSTAR Campaign]

PPPL DIII-D KSTAR

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**OPPPL DIII-D KST**AR

# QSMP did not induce any visible effects in DIII-D either despite strong 3D response expected otherwise

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   Eventually caused a locking due to strong resonant response
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# QSMP remains also safe through early ramp-up and L-H transitions

- QSMP for a new 2020 KSTAR target is designed and applied during the ramp-up, with the maximum amplitude
- Did not leave any influence in the ramp-up and through L-H transition, compared to the reference without 3D fields
- QSMP plasma in fact showed better confinement after L-H transition which will be further investigated





# L-H transition with marginal power remained intact by QSMP, although disrupted by NRMP

- QSMP applied also to a marginal Hmode in DIII-D
- @ L-H:
  - $P_{NBI} \sim 1 MW, T_{NBI} \sim 0.83 Nm,$
  - $I_P = 1.2MA, B_T = 1.8T,$
  - $\beta_{\text{N}} = 0.24 \text{~~} 1.5, \, q_{95} \text{~~} 4.0,$
  - n<sub>e</sub>~2.2e19m<sup>-3</sup>, T<sub>e</sub>~1.7keV,  $\omega_{\phi}$ ~17krad/s
- No impact by QSMP, although NRMP disrupted plasma through L-H
  - As observed in COMPASS
  - In DIII-D, locked modes were observed before L-H transitions
    - Indicating NRMP is not entirely optimized
    - Still, showing value of QS optimization



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#### Torque response matrix offers fundamental approaches to design coils and create large quasi-symmetric tokamak deformation

- Torque mode matrix reveals the second dominant group which should be targeted in subsidiary residual EF correction
- If coils are already designed, torque-coil matrix can be used to deform EF to a quasi-symmetric residual using the correction coils



# QSMP can also be used to find and investigate the effective RMP with minimized transport



 Benefits of such near-QSMP suppressing ELMs must be confirmed by comparison study

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**OPPPL** 

DIII-D KSTAR

0

n

2

4

6 Time [s]

۵

12

KSTAR #26000

8

10

# QSMP will be used to find a 3D field that can create heat flux spreading without degrading plasma performance



optimum trade-off between heat flux spreading and performance

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DIII-D KSTAR

2

4

6 Time [s]

0

n

11

12

10

8

### Summary

- Residual non-axisymmetry after EFC against dominant resonant mode can still cause a significant impact depending on cases (e.g. NSTX-U or COMPASS)
- As a complementary approach, residual non-resonant EF can be further optimized towards quasi-symmetry
- Such a quasi-symmetric magnetic perturbation (QSMP) has been designed using GPEC torque matrix and tested in KSTAR and DIII-D using its available coils
- No negative effects were found with QSMPs in the studied cases in contrast to RMP or NRMP, despite the large overall amplitudes of perturbations
- The results indicate QSMP renders a group of safe non-axisymmetric fields, showing the feasibility of QS even in a perturbed tokamak

