# INTERACTION BETWEEN MAGNETIC GEOMETRY AND TURBULENCE D: 286 IN 3D GLOBAL FLUID SIMULATIONS

E.Serre<sup>1</sup>, P. Tamain<sup>2</sup>, H. Bufferand<sup>2</sup>, G. Ciraolo<sup>2</sup>, D. Galassi<sup>1</sup>, W. Gracias<sup>3</sup>, Ph. Ghendrih<sup>2</sup>, E. Laribi<sup>2</sup>, B. Luce<sup>1,2</sup>, Y. Marandet<sup>1</sup>, N. Nace<sup>2</sup>, F. Nespoli<sup>1,2</sup>, F. Schwander<sup>1</sup> <sup>1</sup>Aix-Marseille Univ., CNRS, M2P2-PIIM, France; <sup>2</sup>CEA IRFM France; <sup>3</sup>Univ. Carlos III de Madrid Spain Email: eric.serre@univ-amu.fr

### ABSTRACT

- Transport and turbulence properties with X-point, 3D disturbed magnetic equilibrium (reminiscent of RMPs), and with negative triangularity  $\rightarrow$  a relatively unexplored domain in numerical global simulations of plasma edge
- Full 3D fluid turbulent simulations with TOKAM3X [1].
- Changes in the fluctuations level, filaments dynamics, heat decay length and heat flux peaks at the targets depending on the configuration.
- Some key experimental features are recovered: better understanding on
- TOKAM3X diverted simulations exhibit a mild transport barrier around the separatrix induced by the magnetic shear [4].



Figure 2: Zoom on the turbulent structures in the divertor. Simu (left), MAST [5] expe (right). The red line on the outer divertor leg delimits the quiescent zone.

the edge plasma dynamics in relevant magnetic configurations for the fusion operation

## **BACKGROUND AND MOTIVATION**

- Turbulence governs the transverse transport  $\rightarrow$  heat exhaust.
- X-point plasmas and H-mode are needed to reach optimal conditions for fusion.
  - $\rightarrow$  The X-point: a highly non-linear SOL dynamics governed by an interplay of turbulence, background drifts, sources, and sinks
  - $\rightarrow$  The RMPs for controlling ELMs: their efficiency depends on different conditions, and the understanding of their complex interaction with the plasma remains a challenging task
  - $\rightarrow$  Negative triangularity: might be a solution to remain in L-mode, yet still achieving sufficient confinement for ignition but requires significant effort to be better understood.
- Progress in the last decade allow to simulate now realistic magnetic geometries that 3D edge turbulence codes have only recently been dealing with

### **THE RMPs IMPACT**

- $\vec{A} = \Psi_p (1 + \varepsilon_p) \vec{\nabla} \varphi + \Psi_t \vec{\nabla} \theta$  where  $|\varepsilon_p(\psi, \theta, \varphi)| \ll 1$  and  $\varepsilon_p = 10^{-3} a \sin(m\theta n\varphi)$ • A pump-out of the electron density (about 20%)
- Drop of the amplitude of the radial electric field inside the separatrix [6].
- Turbulence properties are only moderately impacted



Figure 3: Temporal evolution of the content in the tokamak particles depending on the perturbation. Nonisothermal simulations with (particle source at the limiter) or without (source at the core) recycling.

#### **NEGATIVE TRIANGULARITY IMPACT**

An analytical equilibrium  $R = R_0 + r(\cos(\theta) + \sin^{-1}(\delta)\sin(\theta))$  and  $Z = r \sin(\theta)$ with triangularity  $\delta$  scanned from 0.5 to -0.5.

• In the configuration with a bottom limiter (non constant limiter wetted area): of (N, Te) decay lengths + of the heat load peak value

#### THE TOKAM3X MODEL

- A 3D two-fluid non-isothermal drift-reduced electrostatic model for electrons and a single ion species based on the Braginskii's closure [1].
- Flux driven simulations with flexible axisymmetric magnetic geometries encompassing closed (CFR) and open (SOL) magnetic field lines (Figure 1).
- Typical grid resolution  $64 \times 512 \times 64$  in the (r,  $\theta$ ,  $\phi$ )
- $\eta_{\parallel}(en_0/B_0) = 10^{-5}$  and  $D_{\perp} = 10^{-2}(\rho_{\perp}^2 \omega_c)$  fixed in all simulations.



Figure 1: 3D TOKAM3X simulation in limited configuration. RMP coils in yellow.

- In the configuration with a HFS limiter (constant limiter wetted area): no impact is observed
- In more realistic TCV diverted plasmas: SOL turbulence level + steepening of average profiles in SOL for  $\delta < 0$ . Favorable distribution of the particle flux (Figure 4) with / of the wetted area on the divertor targets for  $\delta < 0$ .
- $\rightarrow$  Geometrical changes with  $\delta$  seem to dominate here the pure effects of changing  $\delta$ .



Figure 4: TCV diverted plasma configuration. Radial distributions of particle fluxes on divertor targets for  $\delta$ >0 and  $\delta$ <0.

### **CONCLUDING REMARKS**

- Clear impact of the magnetic geometry on the plasma dynamics
- A dedicated effort on the numerical efficiency allows us to address now more realistic configurations

Density (on the left) and electron temperature (on the right) fluctuations.

## MOST SIGNIFICANT OUTCOMES

#### **X-POINT IMPACT**

- Poloidal gradients + a quasi-empty and cold private flux region (PFR) lead to large amplitude steady-state radial ExB flows around the X-point: main contributors to the spreading of particles fluxes into the PFR [2].
- Filaments:
  - $\rightarrow$  get strongly elongated in r due to the flux expansion introduced by the X-point (Figure 2)
  - $\rightarrow$  disconnect from the target in the near SOL due to the magnetic + the poloidal shear of the ExB radial velocity around the X-point while they reconnect in the far SOL: lead to different radial velocity scaling [3]

• Very promising preliminary results recover some key experimental findings in TCV diverted plasmas with negative triangularity and in 3D transport simulations with ripple in WEST.

## **REFERENCES AND ACKNOWLEDGEMENTS**

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