

## Technological exploitation of the JET neutron environment: progress in ITER materials irradiation and nuclear analysis

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## Introduction

The planned high profile DT experiments expected in the next few years at the Joint European Torus (JET) is expected to produce large neutron yields in the region of 10<sup>21</sup>



neutrons.

The scientific objectives of the experiments are linked with a technology programme, WPJET3, to deliver the maximum scientific and technological return through exploitation via the high neutron fluxes predicted at JET.

The 'ACT' subproject is focussed on the the irradiation of ITER materials within the JET neutron environment.

Motivation: Take advantage of the large 14 MeV neutron fluence expected during JET DTE2 to irradiate samples of real ITER materials used in the manufacturing of the main in-vessel tokamak components. -> Provision of benchmark data and improved understanding through measurement of nuclide activities for each material with comparison against the predicted quantities via calculation with neutron transport and activation codes and modern nuclear data libraries.

The status of the ACT subproject is presented here with particular focus on the analysis associated with the JET C38 D-D experimental campaign, where ITER materials with diagnostic foils have been exposed to the JET neutron environment for the first time, retrieved post-irradiation and then analysed by a number of participating laboratories.







Top plot: specific activity prediction of dominant nuclides during and following JET irradiation of a EUROFER sample. The dashed vertical line denotes the time at which the samples were removed from the JET LTIS. Bottom plot: daily neutron fluence averaged over the sample volume within the LTIS. The inset plot shows the neutron energy spectrum averaged over the sample volume within the LTIS.





Activation predictions and experimental validation using dosimetry foils



Selected ITER material bulk samples that were sourced by F4E: a) PF coil jacket; b) Radial closure plate for TF coil; c) TF coil case specimen; d) In-wall shielding material; e) Inconel 718; f) Divertor material g) Divertor W monoblock; h); Vacuum vessel forging; i) Reacted TF strand; j) Vacuum vessel plate; k) CuCrZr pipes for the divertor; I) Eurofer 97-2 material.

## Loading configuration for the JET long-term irradiation station (LTIS)

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ITER



Long term irradiation station sample assembly dosimetry foil and ITER material sample arrangement by sample cavity position numbers, 1-26

Table below: ITER material description, unique LTIS position ID and other relevant details for irradiated samples exposed during the JET C38 irradiation campaign. The sample LTIS position-depth ID may be used to map to the LTIS configuration shown in the RHS figure below. The ITER materials are shown as light blue boxes with a corresponding unique position--depth identifier, which may be matched with the table to provides the full ITER material description. Other colours shown denote the institute laboratory responsible for post-irradiation analysis of various dosimetry foils

	Matarial	Monufacturer and somela datails	Ampleusia	Commis	Maggungal	Neminal	Naminal	Neminal														
Sample LTIS	Material	Manufacturer and sample details	Analysis	Sample batch ID	Measured	INOminal	Nominal	Nominal	Denth	Position 1	Position 2	Desition 2	Desition 4	Position F	Desition 6	Desition 7	Position 9	Desition 0	Peritian 10	Position 11	Desition 12	Decition 12
ID			Laboratory	Datch ID	mass (g)	(mm)	(mm)	$(g/cm^3)$	mm	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org
1-2	EUROFER	Saarschmiede Gmbh Vacuum Induction Melting	CCFE (UK)	6a.8	0.356	0.5	11	7.87	0.1													
	Lonor Lit	(VIM) + Vacuum Arc Remelting (VAR) 1.4914x3 EU-		0420	01000	010			0.2	Fe CCFE	Fe CCFE	Fe CCFE	Fe CCFE	Fe CCFE	Fe CCFE	Fe CCFE	Fe CCFE	Fe CCFE	Fe CCFE	Fe CCFE	Fe CCFE	Fe CCFE
		ROFER 97-2. order no: 8186097							0.4													
3-2	Al–Bronze	Aubert & Duval, used for the ITER inner vertical tar-	-	9_2	0.857	0.5	17.5	7.6	0.5													
		get (IVT), Copper Alloys Ltd. Cast ID: 51519051							0.7													
5-3	Tungsten	AT&M for ATMOSTAT, W monoblocks, purity 99.5,	-	13_1	0.705	0.5	10	19.3	0.8	1-2 ITER	2-2 ITER	3-2 ITER	4-2 ITER	5-2 ITER	6-2 ITER	Co CCFE	Ta CCFE	NI CCFE	TI CCFE	Y CCFE	12-2 ITER	13-2 ITER
		ref: PD-13482-999							0.9													
5-2	A660 alloy	Carpenter powder products, India DA, ITER In Wall		10a_10	0.886	0.5	17.5	7.92	1.1													
		Shield (IWS), Heat Nr 5600413							1.2													
6-4	316L(N)	Thyssen Krupp Materials France SAS, Radial plates		4c_9	0.506	0.5	13	7.93	1.3	1-3 ITER	2-3 ITER	3-3 ITER	4-3 ITER	5-3 ITER	6-3 ITER	Co CCFE	Ta CCFE	Ni CCFE	TI CCFE	Y CCFE	12-3 ITER	13-3 ITER
		for the ITER toroidal field coils, 316LN Class C2 so-							1.4													
		lution treated and quenched, stress relieved $(2500/33)$							1.6													
12-4	316L(N)	Special TF cover plate $(304757)$		$3a_1$	0.927	0.5	17.5	7.93	1.7							0. 00FF			-			
12-3	XM19	Aubert & Duval, Forgings for divertor cassette	1	14_3	0.921	0.5	17.5	7.88	1.8	1-4 ITER	Z=4 ITER	3-4 ITER	4-4 ITER	5-4 ITER	6-4 ITER	CO CCFE	Ta COFE	NI CCFE	II COFE	T COFE	12-4 ITER	13-4 ITER
12-2	316L	Salzgitter Mannesmann Stainless Tubes GmbH,		2_10	0.886	0.5	17.5	7.93	2													
14.0		Poloidal field coil jacket	ļ	0.1	0.007				2.1													
14-3	Inconel	Inconel alloy 718	-	8_1	0.967	0.5	17.5	8.2	2.2											Y CCFE		
14-2	316L	Divertor Nadege 316L	-	15_3	0.92	0.5	17.5	7.93	2.4													
3-4	316L(N)-IG	Thyssen Krupp Materials France SAS, forged block		5b_1	0.925	0.5	17.5	7.93	2.5													
		TTER grade vacuum vessel plate, specimen number							2.6													
0.0	CC-7-	Verseta Einst mell commencent Discreten ning 212601	-	111 4	0.052	0.5	175	8.0	2.8													
0.0	A660 allow	ITER divertor material	-	110_4	0.955	0.5	17.5	0.9 7.02	2.9													
2-3	216I (N) IC	Industeel Croupe Argeler, ITEP, vaguum vessel plate	ENEA (Italy)	52.0	0.925	0.5	17.5	7.02	Depth	Position 14	Position 15	Position 16	Position 17	Position 18	Position 19	Position 20	Position 21	Position 22	Position 23	Position 24	Position 25	Position 26
1-4	316L(N)-IG	Thussen Krupp Materials France SAS Badial plates	ENER (Italy)	Ja_9 4a 9	0.9238	0.5	17.5	7.93	mm 0.1	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org	Mat Org
4-4	510D(IV)	(iacket) for the ITER toroidal field coils (2500/64)		44_3	0.0755	0.5	17.5	1.30	0.2													
14-4	316L(N)	Special TF cover plate (304756)	-	3c 1	0.9258	0.5	17.5	7.93	0.3	Fe CCFE			Fe CCFE									
1-3	A660 allov	ITER divertor material	-	12 10	0.924	0.5	17.5	7.92	0.4													
6-3	Tungsten	AT&M for ATMOSTAT. W monoblocks, purity 99.5.	-	13 10	tba	0.5	10	19.3	0.6													
	8	ref: PD-13482-999							0.7				_									
13-3	XM-19	Aubert & Duval, Forgings for the ITER divertor cas-		14_1	0.9104	0.5	17.5	7.88	0.8	14-2 ITER	VERDI 1 graphite	VERDI 1 graphite	Ta NCSRI	Co NCSRD	Co NCSRD	NI NCSRD	NI NCSRE	Sc IFJ		TI IFJ		
		sette							1		Brobine	Brabinee							Ni IFJ		Ni IFJ	Ni IFJ
2-2	EUROFER	Saarschmiede GmbH, Vacuum Induction Melting	1	6b_1	0.5023	0.5	11	7.87	1.1													
		(VIM) + Vacuum Arc Remelting (VAR) 1.4914x3,							1.2	14-3 ITEP								Sc IEI				
		EUROFER 97-2, order no: 8186097							1.4	LAD HER			Tu HOSKI	LO NORD	So NORD	AT NOR		50				
6-2	SS304	Carpenter powder products, India DA, In wall shield	1	10b_2	0.9236	0.5	17.5	7.85	1.5											Ti IFJ		
		sample (IWS)							1.6										Co IFJ			
13-2	A286 alloy	Villares Metals, ITER In wall shield (IWS)	IFJ/IPPLM	7_1	0.9172	0.5	17.5	7.92	1.8	14-4 ITER			Ta NCSRI	Co NCSRD	Co NCSRD	NI NCSRD	NI NCSRE	Sc IFJ	Co IFJ		Y IFJ	Y IFJ
			(Poland)						1.9										Co IFJ			
4-3	CuCrZr	KME, First wall component, divertor pipe 212606		11a_1	1.0018	0.5	17.5	7.93	2										Co IFJ			
13-4	316L(N)	Special TF cover plate (304761)		3b_1	0.9381	0.5	17.5	7.93	2.2													
4-2	Al-Bronze	Aubert & Duval Copper Alloys Ltd. For the inner	NCSRD	9_3	0.85251	0.5	17.5	7.45	2.3													
		vertical target (IVT), Cast ID: 51519051	(Greece)						2.4													
5-4	316L(N)	Thyssen Krupp Materials France SAS, Radial plates		4b_10	0.88502	0.5	17.5	7.93	2.6													
		for the ITER toroidal field coils, 316LN Class C2 so-							2.7													
0.4	91CL (N) TO	lution treated and quenched, stress relieved (2500/68)	ļ	5 0	0.004.40		175	7.02	2.9													
2-4	316L(N)-IG	R Kind GmBh, ITER vacuum vessel plate		5c_9	0.92449	0.5	17.5	7.93	3													

## Conclusions

27 ITER material samples were exposed to neutrons from the JET plasma with a neutron yield of 3.151E19. The samples were then retrieved and distributed to a number of participating laboratories. The analysis identified a number of activation products present in each sample and corresponding modelling predictions have been compared against these observations.

The dosimetry foil measurements for 9 reactions generally show that the calculated neutron fluence for the fast neutron spectrum, including the D-T fraction due to triton burn-up, and the thermal neutron

region is close to the experimental observations, though may be revised in future analyses. For the ITER material results the full data set of C/E values per measured isotope has been presented briefly with some initial remarks in this short paper, though will be discussed in more detail in a more extensive future paper. The measurements for <sup>54</sup>Mn, <sup>58</sup>Co and <sup>57</sup>Co are observed to be close to 1 and a relatively greater spread in C/E results for<sup>60</sup>Co and <sup>59</sup>Fe results is evident. Highlighted discrepancies in C/E values have been noted for <sup>65</sup>Zn, <sup>124</sup>Sb, and <sup>182</sup>Ta.

**Preparations for future activities:** Installation of the LTIS with new samples was completed in September 2020, in readiness for the combined irradiation of the LTIS during the C40 T-T experimental campaign in 2021 followed by the DTE2 (D-T) campaign. The samples included a range of ITER materials, dosimetry foils, VERDI detectors and thin samples of W, Mo and Fe to study for radiation-induced defects. These samples are expected to be retrieved following and then measured either by low background HPGe gamma spectrometry techniques or, in the case of the thin W, Mo and Fe samples, measured by the Positron-Annihilation Lifetime Spectroscopy (PALS) technique to determine the density and size distribution of radiation-induced defects.



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