ID: 838 The Electrochemical Approaches for the Development of a Liquid Blanket System J. Yagi^{1,2}, Y. Ogino², T. Okada², H. Miyagaki², K. Mukai^{1,2}, H. Noto^{3,4}, T. Tanaka^{3,4}, S. Konishi^{1,2} 1)Institute of Advanced Energy, Kyoto University, 2) Graduate School of Energy Science, Kyoto University, 3) National Institute for Fusion Science, National Institutes of Natural Sciences, 4) The Graduate University for Advanced Studies, SOKENDAL j-yagi@iae.Kyoto-u.ac.jp

ABSTRACT

The electrochemical technique is effective to develop liquid blanket system, not only for the molten salt one but also for the liquid metal ones. Based on the experiments using metallic tritium breeder and LiCI-KCI molten salt, new liquid breeding system using Li-Pb intermetallic compound and LiCl-KCl salt is proposed. Hydrogen transport behavior in the system is experimentally investigated and rough tritium and heat balance in the 3GW_{th} system is calculated as well, which is shown to be feasible.

OUTCOME

<u>H transport from Li-Pb to vacuum and to LiCl-KCl</u> -H₂ release from Li-Pb to vacuum was almost linear to the square of the time. This result indicate the diffusion limited transport. D~10⁻¹⁰m²/s (773K)

-Current correspond to the LiH in LiCl-KCl rapidly increased when H absorbed Li-Pb is introduced to the salt. The increase of the current was almost the



vacuum (diffusion limited)

BACKGROUND

•Electrochemical purification has been investigated mainly for the molten fluoride (FLiBe) system (removal of TF, Ni, Fe, H₂O etc.)

•Recently, electrochemical impurity removal for liquid metal breeder (Li, Li₁₇Pb₈₃ eutectic) is shown to be possible using compatible molten salt such as LiCl-KCl.

•High m.p. Li-Pb intermetallic compound (Li3Pb, Li10Pb3, etc.) has not been well investigated because of its difficulty in T recovery (high T solubility) while it may be possible using compatible molten salt (LiCl-KCl) and electrochemical method.

•Behavior of blanket system with Li-Pb intermetallic compound and chloride molten salt is completely unknown, so that basic "experimental investigation" and "tritium and thermal balance calculation" are required.

CHALLENGES

Chloride molten salt for liquid breeder purification

Feasibility of electrochemical purification of liquid breeder metal (Li-Pb eutectic, Li) using LiCl-KCl has been shown (O in Li-Pb, N in Li, etc.) as well as the T extraction from liquid Li.

same as that quantified by the result in vacuum.



Chloride molten salt blanket system with Li-Pb intermetallic compound

- Diffusivity in the Li-Pb is supposed to be 10^{-10} m²/s
- 1 dimensional TBR calculation is done.
- Enrichment of ⁶Li and ³⁷Cl is preferable for TBR
- As a structure material, F82H is selected.
- T flow in the Li-Pb pebble (2mm) and thermal balance are calculated.
- Tritium transport at the Li-Pb/salt surface is supposed to be smooth.

Li-Pb intermetallic compound(Li-Pb) is compatible with LiCl-KCl (unlike FLiBe). Li-Pb can function as i) neutron multiplier, ii) tritium breeder(*solid*), iii) redox reagent (reacts with TCl etc.). Furthermore, Li-Pb pebble in molten salt flow will increase the heat transference.



If generated T in Li-Pb pebble easily transported to LiCl-KCl (as LiT), tritium recovery from the system can be done with electrochemical process.

In this work, transport of H in Li-Pb and from Li-Pb to LiCl-KCl is investigated and based on the result, feasibility of Li-Pb/LiCl-KCl blanket system is also investigated.



TC7





Li-Pb solid pebble	Reflector/Shielding	
ith flowing LiCl-KCl	blanket	

-TBR is found to be enough. -Low m.p. of LiCl-KCl enables \angle T~170K using RAFM steel. -Tritium inventory in Li-Pb is not severe.

SUMMARY

2+8

plasma

•Li-Pb intermetallic compound with high melting point is prepared and its hydrogen release behavior in vacuum and in LiCl-KCl salt is experimentally investigated.

Blanket system		
Prooding coolant	LiCl-KCl	
Breeding coolant	(59-41at%)	
Bleeding multiplier	$Li_{10}Pb_3$	
⁶ Li enrichment [%]	90	
³⁷ Cl enrichment [%]	90	
local TBR	1.22	
Li-Pb/salt zone volume [m ³]	900	
Thermal balance		
thermal output [GW]	3.0	

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Coolant flowrate [10 ³ kg/s]	13.4
Coolant temp. in/out [K]	653 / 823

Tritium balance	
Tritium flow [10 ⁻³ g/s]	4.6
pebble diameter [10 ⁻³ m]	2.0
Max. T conc. in pebble $[g/m^3]$	0.1
T inventory in pebble [g]	22.5

Li-Pb in stainless steel capsule





Li-Pb after H absorption Li-Pb preparation and H charge •H release in both cases were almost similar, which indicate the H transport through Li-Pb/LiCl-KCl boundary is smooth. • Rough TBR and heat/T balance is calculated. •Li-Pb/LiCl-KCl blanket has good TBR, wider \angle T than FLiBe system •Further investigation is necessary, including activation of Cl etc., cost for Cl enrichment, fabrication of Li-Pb pebble, long time compatibility, etc.

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