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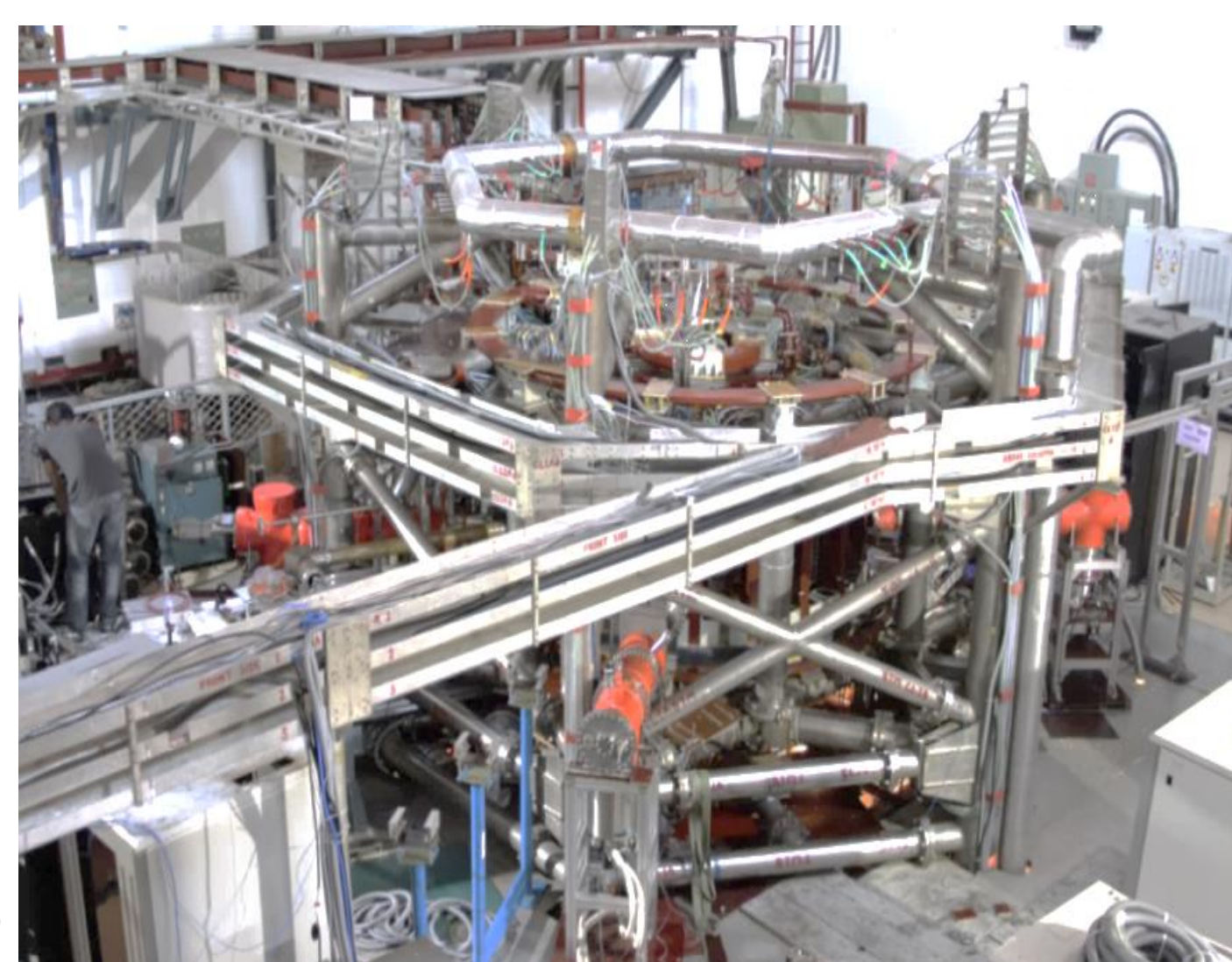


ABSTRACT/ INTRODUCTION

- Control and/or mitigation of runaway electrons (REs) is necessary for the operation of larger fusion devices including ITER
- ADITYA-U: REs are observed by monitoring the hard X-rays (HXR) produced when they interact with the limiter tiles, usually in low density discharges.
- In general, the REs are lost as soon as the plasma disrupts (~ 1 ms).
- However, in the discharges performed by placing a biased electrode inside the last closed flux surface (LCFS), the HXR flux is observed to persist even after plasma disruption.
- The REs are confined due to the $E_r \times B_\phi$ rotation induced by the external radial electric field.

ADITYA-U Tokamak

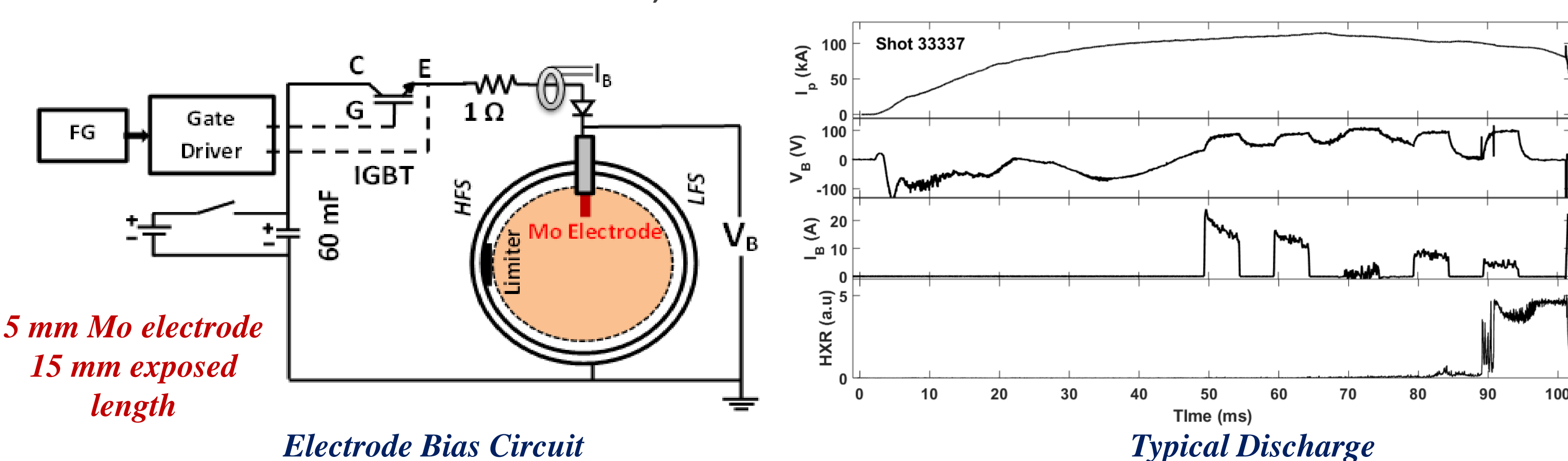
- $R = 0.75$ m; $a = 25$ cm
- Toroidal belt limiter
- 2 poloidal quarter ring limiters
- Toroidal field $B_T : 1.28$ T
- $q_{edge} \sim 3.5 - 4.0$
- Plasma current I_p : 100 - 130 kA
- Plasma duration: 50 - 100 ms
- Electron density n_e : $1.0 - 2.0 \times 10^{19}$
- Electron temperature T_e : 200 - 300 eV



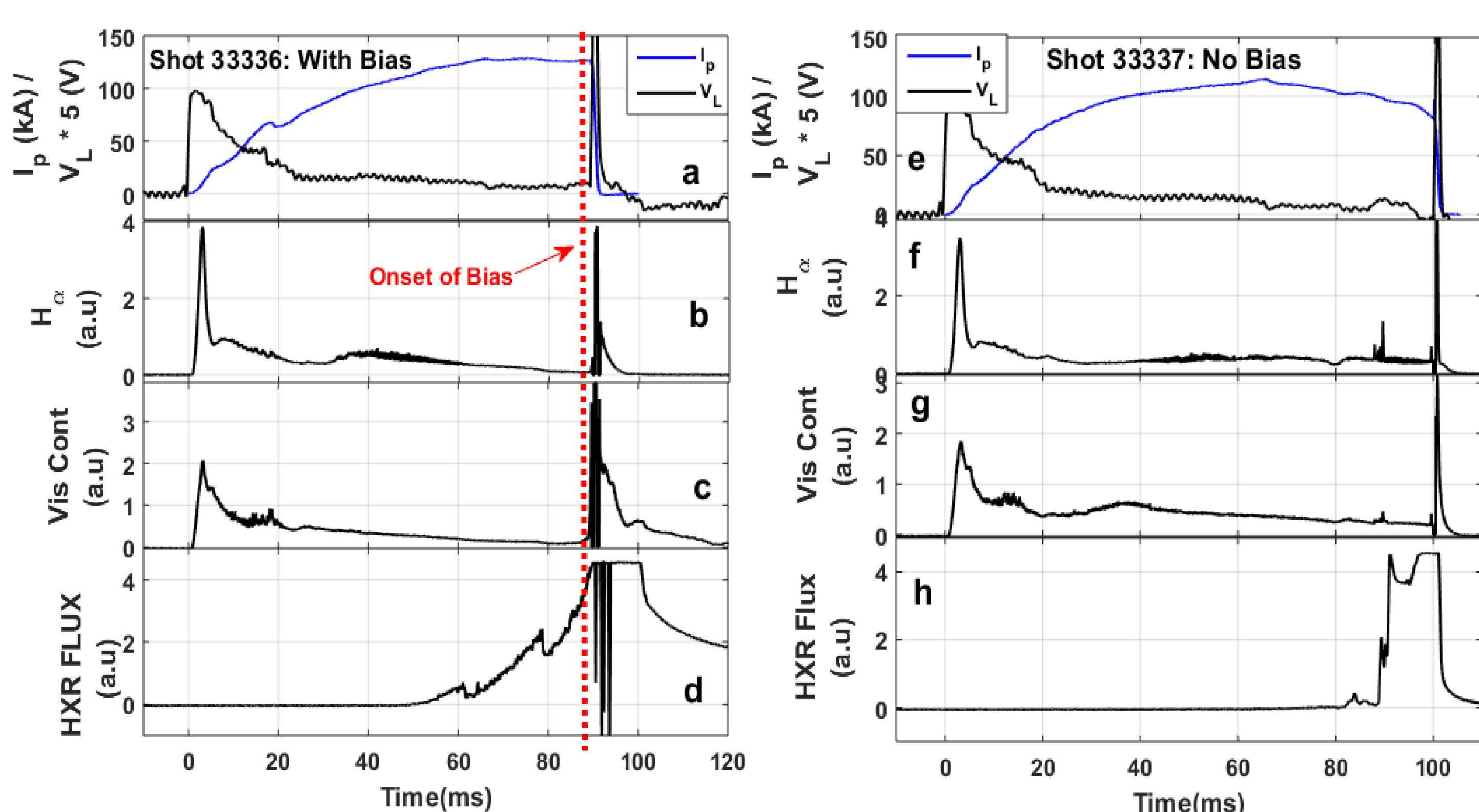
ADITYA-U tokamak

EXPERIMENTAL SET UP

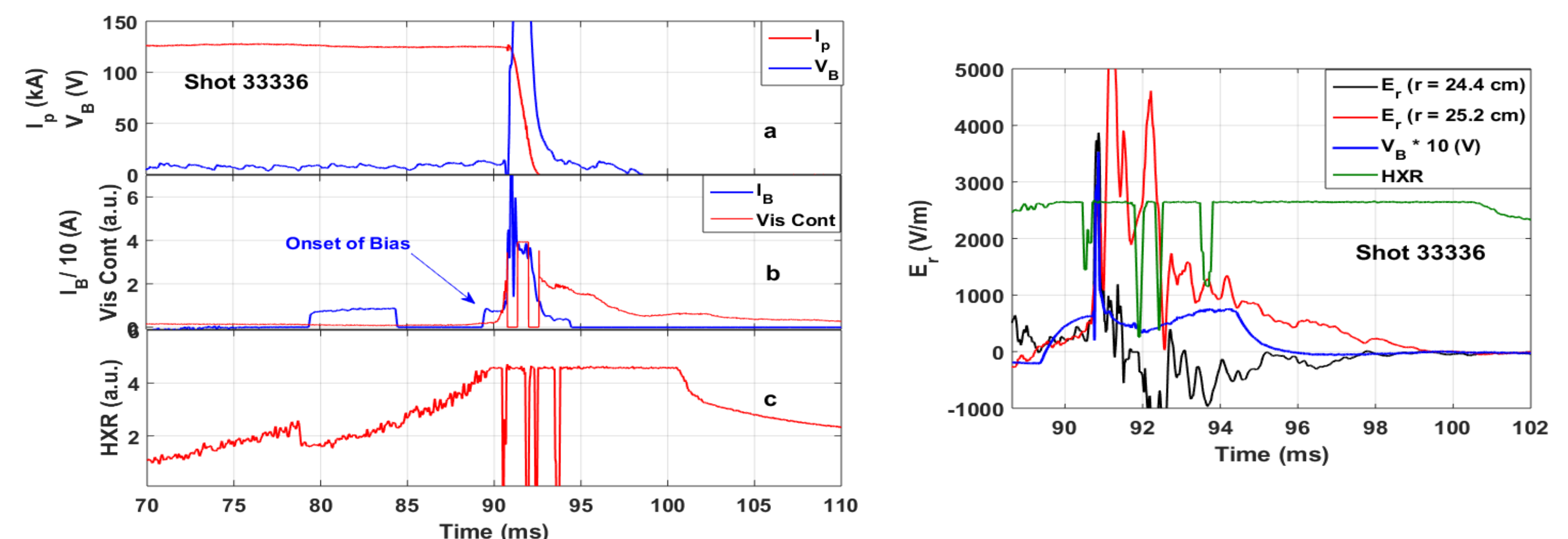
- A set of 15 Langmuir probes (LP): measure the edge plasma n_e , T_e .
- Two arrays of Rake LP (each consisting seven equidistant LP) measure the floating potential and its fluctuations.
- $E_r = \Delta V_p / \Delta r$ - Radial electric field
- 3 inch NaI (TI) scintillator detector; minimum energy 221 keV
- Plastic scintillator detector; measures HXR flux



EXPERIMENTAL OBSERVATIONS



EXPERIMENTAL OBSERVATIONS



HXR flux persists after disruption; Bias voltage ~ 100 V

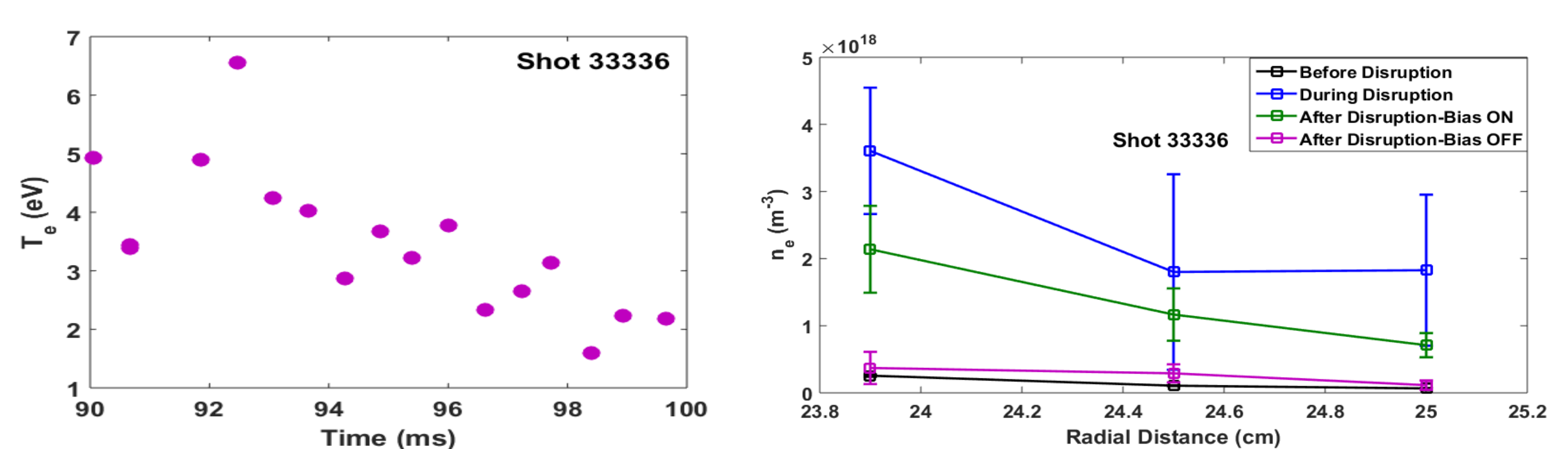
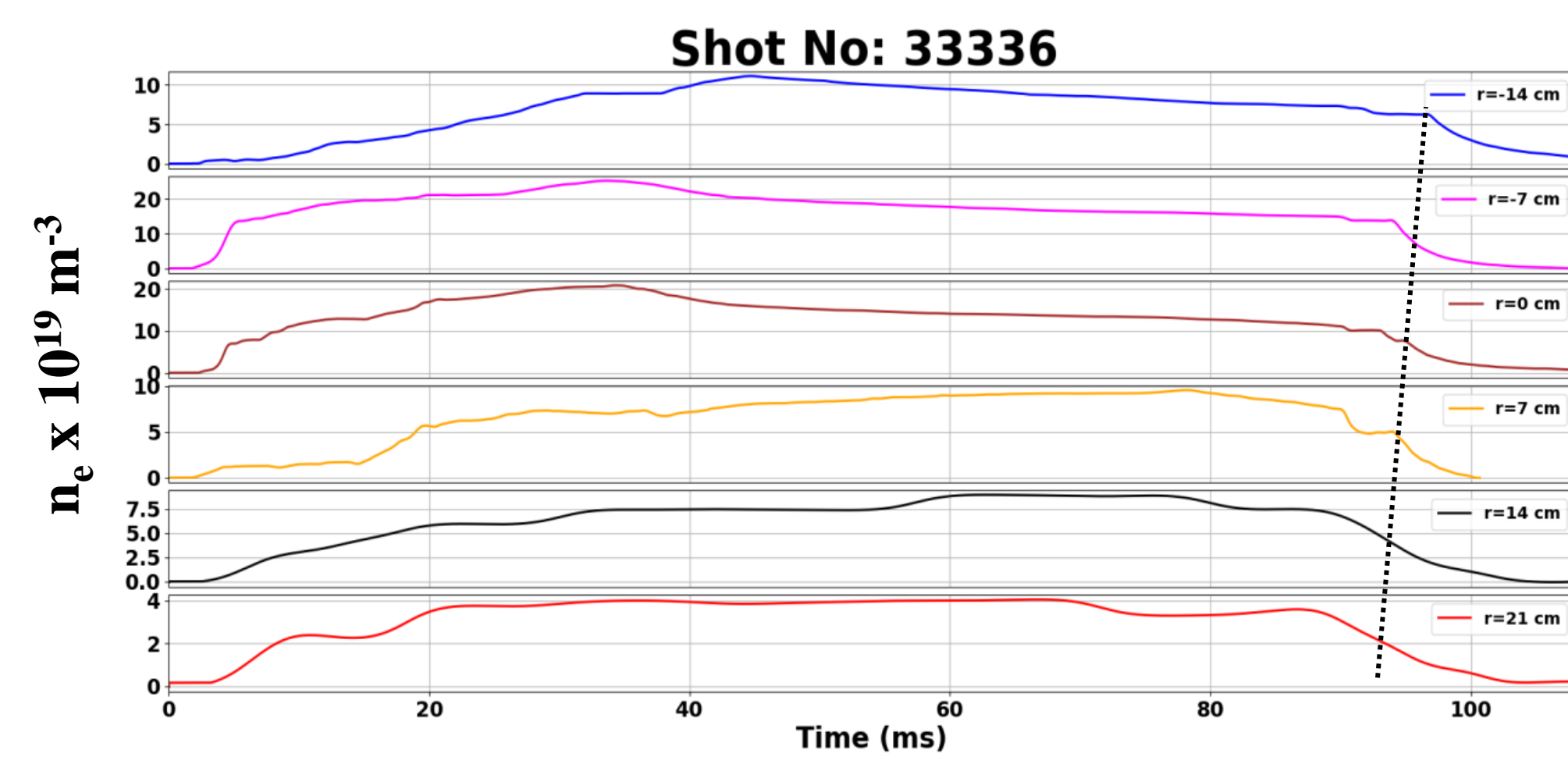
Radial Electric Field measured with LP

How are REs confined?

- I_p falls but RE flux persists
- Bias ON during disruption
- Main plasma disrupted
- Role of E_r ?
- $E_r \times B_\phi$ causes RE to rotate?
- Central n_e do not change substantially
- Why no I_p ?

- E_r reduces after bias turned off; How does it sustain after bias?
- Non-neutral plasma? Confinement improvement due to E_r ? [1]

- Density exists but no I_p ?
- Density decreases rapidly after bias is turned off
- A clear lag in the fall of density in subsequent chords observed.
- Inward propagation of RE after bias OFF?



(a) Edge temperature measured at $r = 25$ cm, (b) Density profile before, during and after disruption

- T_e measured with LP: decreases from 5 eV at the end to disruption to 2 eV at $t \sim 100$ ms
- The corresponding density profile show:
 - During disruption: Sharp profile
 - After disruption: Profile still sharp when bias is ON: Confinement of REs
 - After disruption: Profile becomes flat as soon as bias is turned off: RE loss

CONCLUSION

- REs confined using an induced radial electric field by biasing an electrode in the tokamak edge.
- $E_r \times B_\phi$ rotation of the RE helps in the confinement?
- Confinement as long as biasing is ON.
- Non-neutrality measurement important for the physics understanding.
- Parametric variation with B_T , I_p , biased voltage important

ACKNOWLEDGEMENTS / REFERENCES

- Sarasola X, Pedersen T S, First experimental studies of the physics of plasmas of arbitrary degree of neutrality, Plasma Phys. Control. Fusion 54 (2012) 124008