ID: 1129 JT-60SA TF coils AC losses: acceptance tests modelling with CEA simulation codes and first extrapolations to tokamak operation A. Louzguiti¹, Q. Le Coz², S. Nicollet¹, A. Torre¹, B. Turck¹, L. Zani¹ ¹CEA, IRFM, F-13108 Saint Paul-lez-Durance, France, ²Assystem, F-84000 Pertuis, France Cea infinition EUROfusion UT-60SA alexandre.louzguiti@cea.fr

ABSTRACT

- •AC losses deposited in the magnets during JT-60SA commissioning and operation will both affect the local stability of the conductor and act as a load on the cryogenic system.
- •Our objective is thus to develop an accurate modeling of the AC losses

•From the AC losses modeling, during TFC02 fast discharge 5 kJ are generated by hysteresis, 21 kJ by coupling and 157 kJ by eddy currents in casing (total = **183 kJ**). This modeling is thus consistent with the experiment as the difference between 183 kJ and 214 kJ is about 15% only.

SIMULATIONS IN TOKAMAK CONFIGURATION

generated in the magnets by different current scenarios.

•Since the TF coils have been tested in the Cold Test Facility (CTF) and will be the first ones to be fully energized during the commissioning, we have chosen to first focus our work on the AC losses deposited in the TF

•This enables us to confront our modeling to the CTF data.

•We also present a pseudo-3D thermo-hydraulic simulations to estimate the impact on the helium temperature of the AC losses generated during fast discharge of the TF current foreseen during the commissioning

Magnetic field at center of JT-60SA TF CICC cross-section

Conductor length [m]

18 mm

120

22 mm

Tokamak central pancake

CTF lateral pancake

CTF central pancake

AC LOSSES MODELING

 Magnetic field map computed with TRAPS code [1] at 9 points of JT-Cable-In-Conduit 60SA TF Conductor (CICC) along TF coil in self-field CTF (i.e. coil configuration) and tokamak (i.e. toroidal coils configuration).

•Hysteresis losses computed with analytical formulae from [2]

thermal/thermohydraulic simulation tool developed at CEA that couples THEA (1D thermohydraulics) and Cast3m (2D thermal) [10]-[11]. •TACTICS simulations are performed considering Helium flow operating conditions to evaluate the impact of AC losses due to a fast TF discharge in tokamak configuration at nominal current (25.7 kA) •WP/casing thermal contact is unknown (WP deformation due to Lorentz forces), so two extreme cases are considered:

Helium outlet temperatures



$$P_{h} = \begin{cases} rac{\pi |\dot{B}| \Delta B^{2}}{2\mu_{0}^{2} J_{c} d_{eff}} (1 - rac{\pi \Delta B}{3\mu_{0} J_{c} d_{eff}}) \text{ if } \Delta B < B_{pen} \\ rac{2 J_{c} d_{eff} |\dot{B}| [1 + (I/I_{c})^{2} + 0.2056(I/I_{c})^{4}]}{3\pi} \text{ if } \Delta B > B_{pen} \end{cases}$$

with $B_{pen} = rac{2\mu_{0} J_{c} d_{eff}}{\pi} [1 - rac{\pi^{2}}{8} rac{I}{I_{c}} + (rac{\pi^{2}}{8} - 1)(rac{I}{I_{c}})^{2}]$

deff is measured in [3] and Jc(B,T) is measured in [4].

•Coupling losses computed with MPAS model [5] with data from [6]

 $P_c(W/m^3) = \sum_{j=1 o N} rac{nk_j au_j \dot{B}_{int,j}^2}{\mu_0}$ $nk_{1 \rightarrow N} = 0.220 / 0.254 / 0.293 / 0.340 / 2.23$ $tau_{1\to N}(ms) = 6.02 / 14.6 / 42.8 / 85.9 / 250$ $B_{int,j} + \tau_j \dot{B}_{int,j} = B_{ext}$

•Eddy currents losses in TF coil casing are computed with R,L model and data from [7],[8]. Fast discharges at nominal current (25.7 kA) generates **157 kJ in CTF configuration** (with ~8s time constant) and **609 kJ in tokamak configuration** (with ~14s time constant).

COMPARISON WITH CTF DATA

facility at CEA Saclay, •CTF France where JT-60SA tokamak TF coils have been tested in self-field configuration before their delivery to QST in Naka, Japan [9] •TFC02 coil had additional thermal sensors on the pancakes outlets for advanced tests activity, or ATAs •Calorimetric analysis from 600 Helium T, P, dm/dt sensors at 500 inlet and outlet of the winding pack (WP) and casing led to 214 kJ deposited by 200 hysteresis, coupling and 100 casing eddy current losses during TFC02 fast discharge



•AC losses in JT-60SA TF winding pack much lower than losses in casing •AC losses modeling in fair agreement with CTF experimental results •Simulations in tokamak configuration to be compared (commissioning)

ACKNOWLEDGEMENTS / REFERENCES

This work has been carried out within the framework of the EUROfusion Consortium and has received funding from the Euratom research and training programme 2014-2018 and 2019-2020 under grant agreement No 633053. The views and opinions expressed herein do not necessarily reflect those of the European Commission. The authors gratefully acknowledge members of the JT-60SA Integrated Project Team for data exchange and fruitful discussions.

References: [1] P. Hertout et al., IEEE Trans. Appl. Supercond., 2002 / [2] B. Turck, CEA Technical note, 1985 / [3] M. Chiletti et al., IEEE Trans. Appl. Supercond., 2020 / [4] L. Zani et al., IEEE Trans. Appl. Supercond., 2013 / [5] B. Turck et al., Cryogenics, 2010 / [6] M. Chiletti, PhD Dissertation, 2021 / [7] JT-60SA PID / [8] CTF test report TFC02, 2019 / [9] W. Abdel Maksoud et al., Fus. Eng. Des., 2015 / [10] Q. Le Coz et al., Fus. Eng. Des., 2017 / [11] L. Zani et al., Cryogenics, 2020