

H-mode Operation in He Plasmas with pure RF-Heating and ITER-like Tungsten Divertor on EAST

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The helium plasmas have been demonstrated for the first time on EAST under the condition of pure RF-heating and ITER-like tungsten divertor, which advances physical understanding in support of the ITER non-nuclear operational phase [1]. Concentration of helium (C_{He}) in the plasma is confirmed to play a critical role in H-mode operation, as higher concentration raises the H-mode threshold power and deteriorates the energy confinement in H-mode. At lower C_{He} , EAST achieves the stationary Type-I ELMy H-mode over 80 energy confinement time with the energy confinement slightly above $H_{98,y2}$ scaling ($H_{98,y2} \approx 1.1$) by using pure RF power.

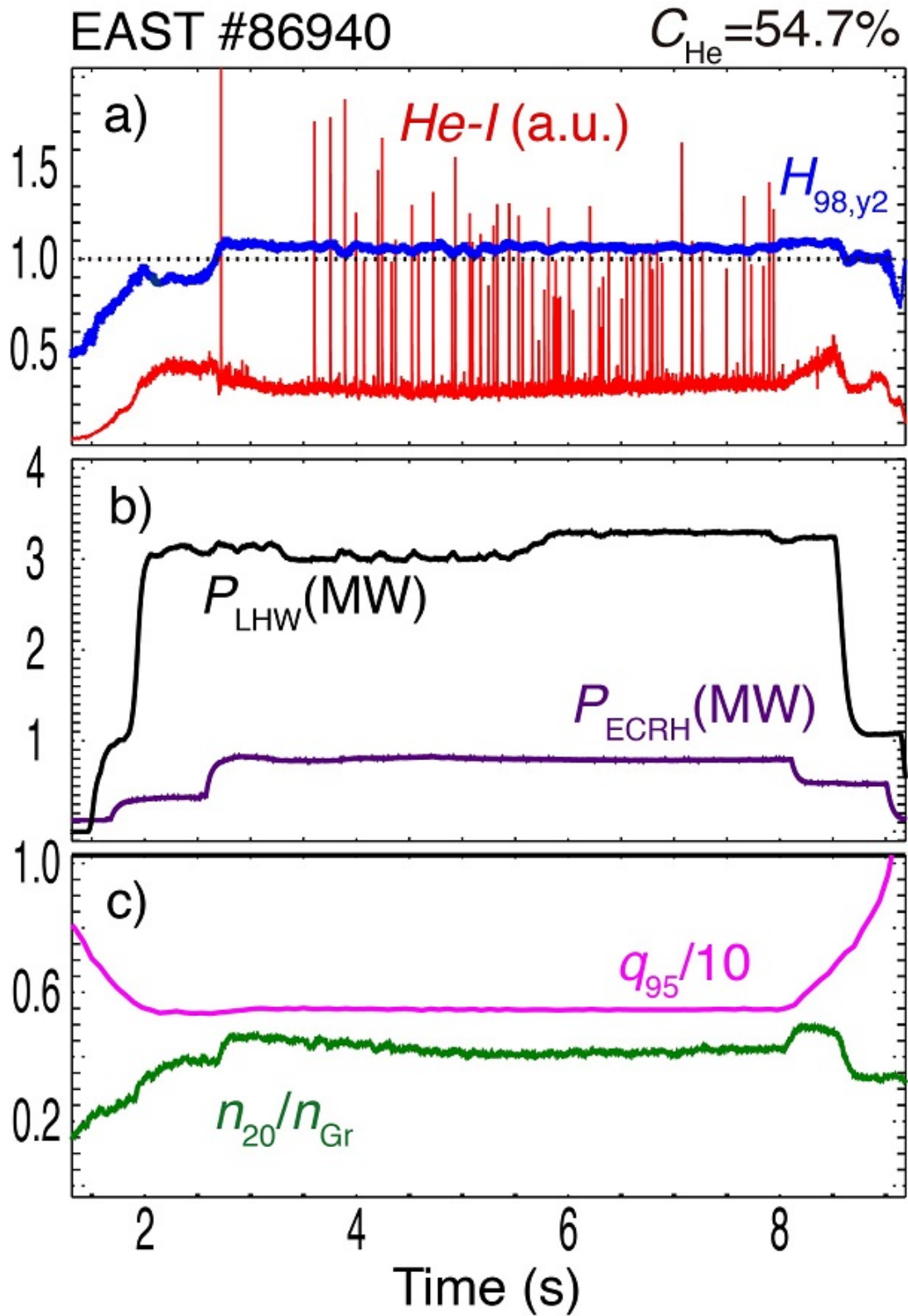


Figure 1: Time traces of plasma parameters of Type-I ELMy H-mode discharge. From top to bottom are, energy confinement factor $H_{98,y,2}$ and He-I line emission count, the injected power of LHW and ECRH, and normalized density n_{20}/n_{Gr} and q_{95} .

EAST first H-mode in helium plasma is achieved by optimizing plasma shape to improve LHW coupling and particle exhaust. Results suggest that a higher possibility of stationary Type-I ELMy H-mode comes with lower C_{He} , pedestal electron collisionality and edge safety factor q_{95} . Figure 1 shows an example of high performance H-mode discharge ($I_P = 0.5\text{MA}$, $B_T = 2.4\text{T}$, $q_{95} = 5.5$) with characteristic of Type-I ELMs behavior

($f_{ELM} \sim 5-30\text{Hz}$) from 3.5s to 8.0s. The plasma configuration is an upper single null (USN) with the strike points located on the vertical targets of tungsten divertor. With growing C_{He} under identical experimental conditions, the ELM behavior evolves into higher frequency $\sim 100\text{Hz}$, or even ELM-absent. ELM mitigation and suppression are demonstrated by n=1 resonant magnetic perturbation (RMP) coils and boron powder injection method, which shows little deleterious effect on core plasma performance for both.

Experiments on the exploration of high energy confinement operation have shown that the global energy confinement time in He is about 30% lower than in D, similar to AUG results [2]. For both ion species, the energy confinement time steadily elevate with central line-averaged density over the range of $2-5 \times 10^{19} \text{m}^{-3}$. In fixed plasma condition ($I_P = 0.5\text{MA}$, $B_T = 2.4\text{T}$, $n_e = 4.3 \times 10^{19} \text{m}^{-3}$, $P_{inj} = 4\text{MW}$), C_{He} lowered by active D gas puffing gradually improves the H-mode performance.

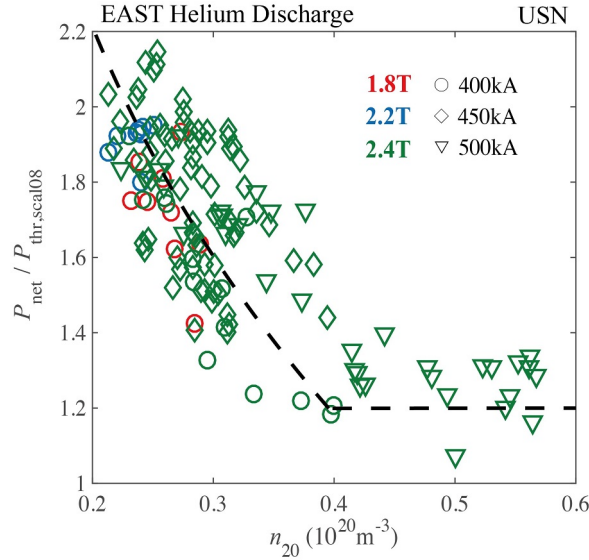


Figure 2: The net power at the L to H-mode transition normalized to the ITPA scaling law for D as a function of the line-averaged plasma density for He discharges with pure RF-heating on EAST.

Using the density scan, EAST achieves H-mode over wide density range of $2-6 \times 10^{19} \text{m}^{-3}$ with the RF heating, where H-mode threshold power (P_{thr}) indicates strong dependence on density. Figure 2 presents the statistic result of normalized threshold power (radiated power subtracted), versus density, exhibiting a minimum of P_{thr} at $n_{e,min} \approx 4 \times 10^{19} \text{m}^{-3}$. When plasma densities are below $n_{e,min}$, the required power to enter H-mode in He is 1.2-2.2 times higher than the ITPA scaling for D [3], otherwise it follows the scaling. The scatter of threshold power at one certain density is attributed to the variation of C_{He} . It is not possible to access H-mode at $C_{He} > 70\%$ with the available heating power level at the time of experiments.

In summary, recent EAST experiments demonstrates high performance H-mode operation in helium plasma with RF-only heating. More analysis of experimental data from core and edge diagnostics is ongoing, to investigate physical mechanism of C_{He} effect on H-mode operation. With the upgrades [4] in 2020, EAST will be able to further extend high performance helium H-mode towards ITER-relevant by exploiting two extra ECH gyrotrons and the condition of full metal walls.

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